Cleanup Verification Package for the 618-8 Burial Ground

Prepared for the U.S. Department of Energy by Washington Closure Hanford

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EXECUTIVE SUMMARY

This cleanup verification package documents completion of remedial action for the 618-8 Burial Ground, also referred to as the Solid Waste Burial Ground No. 8, 318-8, and the Early Solid Waste Burial Ground. This site is located in the 300-FF-2 Operable Unit in the 600 Area of the Hanford Site in southeastern Washington State. During its period of operation from 1943 to 1954, the 618-8 site is speculated to have been used to bury uranium-contaminated waste derived from fuel manufacturing, and construction debris from the remodeling of the 313 Building.

Site excavation and waste disposal are complete, and the exposed surfaces have been sampled and analyzed to verify attainment of the remedial action goals. Results of the sampling, laboratory analyses, and data evaluations for the 618-8 site indicate that all remedial action objectives and goals for direct exposure, protection of groundwater, and protection of the Columbia River have been met for industrial land use (Table ES-1).

Because residual soil concentrations indicated that cleanup levels for more stringent land uses may have been achieved for the 618-8 site, a supplemental evaluation was performed against unrestricted land-use cleanup objectives established in the *Explanation of Significant Differences for the 300-FF-2 Operable Unit Record of Decision* (EPA 2004). Results of the evaluation (Table ES-2) demonstrate that residual contaminant concentrations do not preclude any future uses (as bounded by the rural-residential scenario) and allow for unrestricted use of shallow zone soils (i.e., surface to 4.6 m [15 ft] deep). This site does not have a deep zone; therefore, no deep zone institutional controls are required.

The site meets cleanup standards and has been reclassified as "interim closed out" in accordance with the *Hanford Federal Facility Agreement and Consent Order* (Ecology et al. 1989) and the Waste Site Reclassification Guideline TPA-MP-14 (RL-TPA-90-0001) (DOE-RL 1998). A copy of the waste site reclassification form is included as Attachment ES-1.

Table ES-1.Summary of Cleanup Verification Resultsfor the 618-8 Waste Site - Industrial Land Use. (2 pages)

Regulatory Requirement	Remedial Action Goals	Results		Remedial Action Objectives Attained?	Ref.
Direct Exposure – Radionuclides	 Attain 15 mrem/yr dose rate above background over 1,000 years. 	1.	All individual radionuclide COC detections were below background.	Yes	а
Direct Exposure – Nonradionuclides	1. Attain individual COC RAGs.	1.	All individual COC concentrations were below the direct exposure criteria.	Yes	а
Nonradionuclide Risk Requirements1. Attain hazard quotient of <1 for noncarcinogens.2. Attain cumulative hazard quotient of <1 for noncarcinogens.		1,2. The hazard quotient for selenium, the sole nonradionuclide detected above background, is 2.1 x 10 ⁻³ , which is less than 1. Because selenium is the sole nonradionuclide detected above background, the cumulative hazard quotient is also <1.		а	
	 Attain excess cancer risk of <1 x 10⁻⁵ for individual carcinogens. 	3,4. Excess cancer risk values were not calculated because all nonradionuclide carcinogenic COCs (arsenic and cadmium)			
	 Attain a total excess cancer risk of <1 x 10⁻⁵ for carcinogens. 		were detected below statistical background levels.		
Groundwater/River Protection – Radionuclides	1. Attain single-COC groundwater and river protection RAGs.	1.	All individual radionuclide COC detections were below background.		
	 Attain National Primary Drinking Water Standards: 4 mrem/yr (beta/gamma) dose rate to target receptor/organs. 	2.	All individual radionuclide COC detections were below background.	Yes	а
	 Meet drinking water standards for alpha emitters: the more stringent of the 15 pCi/L MCL or 1/25th of the derived concentration guide per DOE Order 5400.5. 	3.	All individual radionuclide COC detections were below background.		
	 Meet total uranium standard of 21.2 pCi/L.^b 	4.	Uranium statistical values are below background for this site.	NA	
Groundwater/River Protection – Nonradionuclides	 Attain individual nonradionuclide groundwater and river cleanup requirements. 	1.	Soil cleanup levels for groundwater and river protection have been attained.	Yes	a

Table ES-1. Summary of Cleanup Verification Results for the 618-8 Waste Site - Industrial Land Use. (2 pages)

Regulatory Requirement	Remedial Action Goals	Results	Remedial Action Objectives Attained?	Ref.
Other supporting	ing 1. 618-8 Burial Ground Sample Design (Appendix C)			с
mornation	2. Closeout Plan for the 618-8 Burial Ground			d

^a 618-8 Burial Ground Cleanup Verification 95% UCL Calculations, 0600X-CA-V0058, Washington Closure Hanford, Richland, Washington (Appendix C).

^b The EPA has promulgated a drinking water MCL of 30 μg/L for total uranium (40 CFR 141.66). Based on the isotopic distribution of uranium on the Hanford Site, the 30 μg/L MCL corresponds to 21.2 pCi/L. Concentration-to-activity calculations are documented in the *Calculation of Total Uranium Activity Corresponding to a Maximum Contaminant Level for Total Uranium of 30 Micrograms per Liter in Groundwater* calculation brief (BHI 2001).

° 618-8 Shallow Zone Sampling Plan, 0300X-CA-V0067, Washington Closure Hanford, Richland, Washington.

^d Closeout Plan for the 618-8 Burial Ground, CCN 126068, Washington Closure Hanford, Richland, Washington (WCH 2006b).

COC = contaminant of concern

NA = not applicable

RAG = remedial action goal

MCL = maximum contaminant level (drinking water standard)

UCL = upper confidence limit

Table ES-2. Summary of Cleanup Verification Results for the 618-8 Waste Site - Unrestricted Land Use. (2 Pages)

Regulatory Requirement	Remedial Action Goals	Results	Remedial Action Objectives Attained?	Ref.
Direct Exposure – Radionuclides	 Attain 15 mrem/yr dose rate above background over 1,000 years. 	 All individual radionuclide COC detections were below background. 	Yes	а
Direct Exposure – Nonradionuclides	1. Attain individual COC RAGs.	1. All individual COC concentrations were below the direct exposure criteria.	Yes	а
Nonradionuclide Risk Requirements	 Attain hazard quotient of <1 for noncarcinogens. 	1,2. The hazard quotient for selenium, the sole nonradionuclide detected		
	 Attain cumulative hazard quotient of <1 for noncarcinogens. 	which is less than 1. Because selenium is the sole nonradionuclide detected above background, the cumulative hazard quotient is also <1.	Yes	а
	 Attain excess cancer risk of <1 x 10⁻⁶ for individual carcinogens. 	3,4. Excess cancer risk values were not calculated because all nonradionuclide carcinogenic		
	 Attain a total excess cancer risk of <1 x 10⁻⁵ for carcinogens. 	were detected below statistical background levels.		
Groundwater/River Protection – Radionuclides	1. Attain single-COC groundwater and river protection RAGs.	 All individual radionuclide COC detections were below background. 	Yes	а

Table ES-2. Summary of Cleanup Verification Results for the 618-8 Waste Site - Unrestricted Land Use. (2 Pages)

Regulatory Requirement	Remedial Action Goals	Results Results Action Objectives Attained?	Ref.
	 Attain National Primary Drinking Water Standards: 4 mrem/yr (beta/gamma) dose rate to target receptor/organs. 	2. All individual radionuclide COC detections were below background.	
	 Meet drinking water standards for alpha emitters: the more stringent of the 15 pCi/L MCL or 1/25th of the derived concentration guide per DOE Order 5400.5. 	3. All individual radionuclide COC detections were below background.	
	 Meet total uranium standard of 21.2 pCi/L.^b 	4. Uranium statistical values are below background for this site.	
Groundwater/River Protection – Nonradionuclides	 Attain individual nonradionuclide groundwater and river cleanup requirements. 	1. Soil cleanup levels for groundwater and river protection Yes have been attained.	a
Other supporting	1. 618-8 Burial Ground Sample Design (Appendix C)		
Information	2. Closeout Plan for the 618-8 Burial Ground		

^a 618-8 Burial Ground Cleanup Verification 95% UCL Calculations, 0600X-CA-V0058, Washington Closure Hanford, Richland, Washington (Appendix C).

^b The EPA has promulgated a drinking water MCL of 30 μg/L for total uranium (40 CFR 141.66). Based on the isotopic distribution of uranium on the Hanford Site, the 30 µg/L MCL corresponds to 21.2 pCi/L. Concentration-to-activity calculations are documented in the Calculation of Total Uranium Activity Corresponding to a Maximum Contaminant Level for Total Uranium of 30 Micrograms per Liter in Groundwater calculation brief (BHI 2001).

[°] 618-8 Shallow Zone Sampling Plan, 0300X-CA-V0067, Washington Closure Hanford, Richland, Washington. ^d Closeout Plan for the 618-8 Burial Ground, CCN 126068, Washington Closure Hanford, Richland, Washington (WCH 2006a).

COC = contaminant of concern

NA = not applicable

RAG = remedial action goal

MCL = maximum contaminant level (drinking water standard)

UCL = upper confidence limit

Attachment ES-1 Waste Site Reclassification Form

Date Submitted: 7/10/06	<u>Operable Unit(s)</u> : 300-FF-2		Control Number: 2006-036	
<u>Originator</u> : L. M. Dittmer	Waste Site ID: 618-8		Lead Agency: EPA	
Dhanes 272 0664	Type of Reclassification Action]:		
<u>Pnone</u> : 372-9664	Rejected Closed Out Closed Out No Action]]]		
This form documents agree rejected, closed out, or no a Priorities List of no action or	ment among the parties listed below action and authorizing backfill of the r closed-out sites will occur at a futu	w authorizing classifica site, if appropriate. Fi ure date.	tion of the subject unit as nal removal from the National	
Description of current wa	ste site condition:	·		
Remedial actions at this site established by the U.S. Env Office, in concurrence with (1) excavating the site to the excavation materials at the (3) backfilling the site with c	e have been performed in accordan ironmental Protection Agency and the Washington State Department of e extent required to meet specified Environmental Restoration Disposa lean soil to adjacent grade elevatio	nce with remedial action the U.S. Department o of Ecology. The select soil cleanup levels, (2) al Facility in the 200 Aro ons.	n objectives and goals f Energy, Richland Operations ed remedial action involved disposing of contaminated ea of the Hanford Site, and	
Basis for reclassification:				
The 618-8 waste site has been remediated to meet the cleanup standards specified in the <i>Interim Action Record of Decision for the 300-FF-2 Operable Unit</i> , U.S. Environmental Protection Agency, Region 10, Seattle, Washington. Remedial actions were performed to support future industrial land use and to protect groundwater and the Columbia River. Further, the residual contaminant concentrations achieved do not preclude any future uses (as bounded by the rural-residential scenario) and allow for unrestricted use of shallow zone soils (i.e., surface to 4.6 m [15 ft] deep). This site has no deep zone; therefore, no deep zone institutional controls are required. The basis for reclassification is described in detail in the <i>Cleanup Verification Package for the 618-8 Burial Ground</i> (CVP-2006-00006), Washington.				
D. C. Smith DOE-RL Project Manager	Signatur	- An	<u>0/7/16</u> Date	
NA Ecology Project Manager	Signatu	re	Date _	
<u>A. L. Boyd</u> EPA Project Manager	A.J. Signatur	Boyd	<u>8/7/06</u> _{Date}	

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ACRONYMS AND ABBREVIATIONS

COC	contaminant of concern
CVP	cleanup verification package
DQA	data quality assessment
EPA	U.S. Environmental Protection Agency
ERDF	Environmental Restoration Disposal Facility
GPR	ground-penetrating radar
MTCA	Model Toxics Control Act
RAG	remedial action goal
RAO	remedial action objective
RDR/RAWP	remedial design report/remedial action work plan
RESRAD	RESidual RADioactivity (dose assessment model)
ROD	record of decision
SAP	sampling and analysis plan
UCL	upper confidence limit
WAC	Washington Administrative Code

1.0 INTRODUCTION

This cleanup verification package (CVP) documents that the 618-8 waste site was remediated in accordance with the *Interim Action Record of Decision for the 300-FF-2 Operable Unit, Hanford Site, Benton County, Washington* (ROD) (EPA 2001). Remedial action objectives (RAOs) and remedial action goals (RAGs) for this site are documented in the ROD (EPA 2001) and the *Remedial Design Report/Remedial Action Work Plan for the 300 Area* (RDR/RAWP) (DOE-RL 2004a). The ROD provides the U.S. Department of Energy, Richland Operations Office, the authority, guidance, and objectives to conduct this remedial action.

The remedy specified in the ROD for the 618-8 waste site included (1) excavating the site to the extent required to meet specified soil cleanup levels, (2) disposing of contaminated excavation materials at the Environmental Restoration Disposal Facility (ERDF) in the 200 Area of the Hanford Site, and (3) backfilling the site with clean soil to average adjacent grade elevation. Excavation was driven by RAOs for direct exposure, protection of groundwater, and protection of the Columbia River. For the respective points of compliance, RAGs summarized in Table 1 were established for the contaminants of concern (COCs) in the RDR/RAWP (DOE-RL 2004b). Preliminary waste site COCs were identified in the *300 Area Remedial Action Sampling and Analysis Plan* (SAP) (DOE-RL 2004a). Following excavation of the site, final COCs were identified in the *Closeout Plan for the 618-8 Burial Ground* (WCH 2006a) and are presented in Table 1.

Contaminants of Concern	Soil Cleanup Level for Direct Exposure ^a (pCi/g)	Soil Cleanup Level for Groundwater Protection ^b (pCi/g)	Soil Cleanup Level for River Protection (pCi/g)	
	Radionuclia	les		
Uranium-233/234	169 ^c	129 ^d	129 ^d	
Uranium-235	17 ^c	13 ^d	13 ^d	
Uranium-238	165°	125 ^d	125 ^d	
Contaminants of Concern	Soil Cleanup Level for Direct Exposure (mg/kg)	Soil Cleanup Level for Groundwater Protection ^b (mg/kg)	Soil Cleanup Level for River Protection ^b (mg/kg)	
Nonradionuclides				
Arsenic	58 ^e	NA ^f	NA ^f	
Barium	4,900 ^e	NA ^f	NA ^f	
Cadmium	139 ^e	NA ^f	NA ^f	

Table 1. Summary of Remedial Action Goals – Industrial Land Use.(2 Pages)

Contaminants of Concern	Soil Cleanup Level for Direct Exposure (mg/kg)	Soil Cleanup Level for Groundwater Protection ^b (mg/kg)	Soil Cleanup Level for River Protection ^b (mg/kg)	
Nonradionuclides (cont.)				
Chromium	>1,000,000	NA ^f	NA [†]	
Lead	1,000	NA ^f	NA ^f	
Selenium	400	5 ⁹	1 ⁿ	
Silver ⁱ	400	8	0.73	
Uranium (total)	505 ⁱ	385 ^j	385 ^j	

Table 1. Summary of Remedial Action Goals – Industrial Land Use.(2 Pages)

^a Listed values represent a 15 mrem/yr dose for the industrial exposure scenario.

^b Groundwater protection values represent soil concentrations that will be protective of groundwater. River protection values represent soil concentrations that will not cause applicable river cleanup standards to be exceeded as contaminants migrate through the soil column to the river. Listed values are calculated by RESRAD, based on applicable river cleanup standards in the RDR/RAWP (DOE-RL 2004b).

^c Direct exposure lookup values for uranium isotopes calculated using secular equilibrium isotopic ratios and the selected soil cleanup level for total uranium under the industrial exposure-scenario (350 pCi/g), as provided in the RDR/RAWP (DOE-RL 2004b).

^d Soil lookup values for the protection of groundwater and the Columbia River for uranium isotopes calculated using secular equilibrium isotopic ratios and the selected soil cleanup level for total uranium under the industrial exposure-scenario (267 pCi/g), as provided in the RDR/RAWP (DOE-RL 2004b).

^e Cleanup limit based on the inhalation exposure pathway per WAC 173-340-750(4)(b)(ii)(a) or (b).

^f The RESRAD model predicts that the constituent will not reach groundwater within 1,000 years based on a generic site profile (4.6 m [15 ft] contaminated zone and 6m [19.6 ft] uncontaminated zone).

- ⁹ Soil RAG for groundwater protection calculated from WAC 173-303-740(3)(a)(ii)(A), 1996 ("100 times rule") and the MCL (40 CFR 141).
- ^h Soil RAG for river protection calculated from WAC 173-340-740(3)(a)(ii)(A), 1996 ("100 time rule") and a dilution attenuation factor of 2, using the ambient water quality criteria provided in WAC 173-201A-040, 1995.

ⁱ No value is provided for this constituent in the RDR/RAWP (DOE-RL 2004b); values for silver are determined based on WAC 173-340-740, 1996.

^j Based on the calculated isotopic distribution of uranium in the 300 Area and cleanup levels of 350 pCi/g and 267 pCi/g for total uranium, the corresponding uranium concentration is 505 mg/kg and 385 mg/kg, respectively (BHI 2002) for the 300-FF-2 Operable Unit sites.

MCL = maximum contamination level (drinking water standard)

NA = not applicable

BAG	= remedial action or	bal

RDR/RAWP = remedial design report/remedial action work plan

RESRAD = RESidual RADioactivity (dose assessment model)

WAC = Washington Administrative Code

Soil cleanup levels were established in the interim action ROD based on a limited ecological risk assessment. Although not required by the interim action ROD, a comparison against ecological risk screening levels has been made for the site COCs. Screening values were not exceeded for the COCs for this site, with the exception of selenium. Exceedance of screening values does not necessarily indicate the existence of risk to ecological receptors. It is believed that the presence of selenium at this site does not pose a risk to ecological receptors because the statistical concentration of selenium is within the range of natural site background (Ecology 1994). A baseline risk

assessment for the river corridor portion of the Hanford Site began in 2004, which includes a more complete quantitative ecological risk assessment. That baseline risk assessment will be used as part of the final closeout decision for this site.

2.0 SITE DESCRIPTION AND SUPPORTING INFORMATION

The 618-8 waste site is part of the 300-FF-2 Operable Unit in the 300 Area. The site was reportedly located north of the 300 Area, beneath a portion of the 300 Area North Parking Lot (in the vicinity of Washington State Plane coordinates E 593820, N 116410), and in an area immediately north of the parking lot (in the vicinity of Washington State Plane coordinates E 593820, N 116480) (Figure 1). The burial ground was reported to be a rectangular shaped area approximately 183 m (600 ft) long by 31 m (100 ft) wide. The waste site was believed to include the area under the parking lot, because when the North Parking Lot was constructed (sometime in the early 1950s), brass medallions were placed in the asphalt to delineate the presence of an Underground Contaminated Material Area. The waste site was also believed to include the area under the area north of the parking lot because this area was delineated by "post and chain" boundary markers with signs identifying it as an Underground Contaminated Material Area (WCH 2006a).

Information on dates of operation and burial ground inventory are limited. However, it has been speculated that the segment located under the parking lot may have been used in 1943 and 1944 to bury uranium-contaminated waste derived from uranium fuel manufacturing. The area north of the parking lot was speculated to have been used in the mid 1950s to bury uranium-contaminated waste and construction debris from the 1954 remodeling of the 313 Building (WCH 2006a).

Early characterization investigations at the 618-8 Burial Ground included a series of *in situ* gamma spectral measurements, performed in September 1980. These measurements were performed to determine the nature and extent of radiological contamination at the waste site. Results of the investigation indicated that the radiological contamination was found primarily within established "post and chain" boundaries associated with the area located north of the parking lot. The radiological contamination identified in this investigation consisted exclusively of uranium isotopes, primarily uranium-235 and uranium-238.

Additional characterization included geophysical, ground-penetrating radar (GPR) surveys performed in 1987. These GPR surveys identified two distinct areas containing substantial amounts of buried waste in the areas located north of the parking lot (inside the area previously delineated by the "post and chain" boundaries). The GPR surveys could not identify a traditional trench configuration or evidence of buried waste under the area delineated by the medallions in the parking lot. Subsequent test excavations and trenching performed at the parking lot area did not expose any evidence of buried waste or debris in that area. A total of eight soil samples were collected from soil





removed during the trenching and test excavations (WCH 2006b). These samples were analyzed for *Resource Conservation and Recovery Act of 1976* metals and a wide range of radionuclides. Analytical results showed all results were either undetected or detected below lookup values. As a result, the area under the parking lot was not excavated further. Results of the parking lot test pit/trenching investigation are documented in WCH (2006b).

3.0 REMEDIAL ACTION FIELD ACTIVITIES

3.1 EXCAVATION AND DISPOSAL

Remedial action activities at the 618-8 waste site began on November 1, 2004, and were completed on November 8, 2004. Remediation involved excavation and staging of clean overburden material and removal of contaminated soil to the extent required to satisfy the RAOs and corresponding RAGs. Excavated material consisted of soil and a wide variety of construction-type debris. Some land disposal restricted materials (primarily three metal drums) were identified and separated from the bulk soil and debris during excavation and sorting operations. Sorting and sampling of the excavated soil and debris was performed in a designated staging pile area. The staging pile area used for the 618-8 waste site also supported the 618-2 and 618-3 Burial Ground excavations and will be closed out with the 618-2 waste site. Subsequent to sorting and receipt of sampling results, the "released" material was loaded into ERDF containers and transported to the ERDF for disposal. Segregated land disposal restricted materials were subsequently loaded out and transported to the ERDF under a separate waste profile. In December 2004, load-out operations at the 618-8 Burial Ground staging piles were suspended, due to the discovery of plutonium-contaminated waste at the 618-2 Burial Ground. As part of mitigation actions to stabilize excavated waste and debris from 618-2 Burial Ground, a soil cover was placed on all stockpiled material from the 618-8 Burial Ground.

Load-out of the 618-8 stockpiled material resumed on September 1, 2005, and was completed on September 13, 2005. Approximately 6,462 metric tons (7,125 U.S. tons) of material from the site was removed and disposed at the ERDF. Pre- and post-remediation topographic maps are shown in Figures 2 and 3.

There was no indication of bulk liquid waste disposal observed during excavation of the 618-8 waste site.

3.2 FIELD SCREENING

Post-excavation radiological surveys of the 618-8 waste site floor were performed in December 2005. The field radiological measurements survey results did not identify any residual radiological contamination above background levels. The radiological survey maps are included as Figures 4 and 5.



Figure 2. Pre-Remediation Topographic Plan for the Western Portion of the 618-8 Waste Site.



Figure 3. Post-Remediation Topographic Plan for the Western Portion of the 618-8 Waste Site.

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3.3 CLEANUP VERIFICATION SAMPLING AND ANALYSIS

Final cleanup verification sampling was conducted on January 31, 2006. The verification samples were submitted to offsite laboratories for analysis using approved U.S. Environmental Protection Agency analytical methods as required per the SAP (DOE-RL 2004a). Each verification sample was composed of a composite sample formed by combining soil collected at the required number of randomly selected locations within each sampling area (excluding the quality assurance/quality control samples).

The 618-8 waste site excavation has only a shallow zone decision unit, as shown on the sample design figure (Appendix C), inclusive of the excavation sidewalls and floors. Direct exposure, groundwater protection, and river protection RAGs are all applicable to shallow zone soils (i.e., soils within 4.6 m [15 ft] of the ground surface). Based on the size of the remediation footprint, the 618-8 shallow zone decision unit is comprised of one decision subunit, divided into four sampling areas (DOE-RL 2004a). All sampling areas were further divided into 16 sampling nodes, each as shown in the sample design methodology and sample location figures presented in the calculation brief in Appendix C. The analytical results for the samples were used for the purposes of statistical calculations, as described in the cleanup verification calculation brief (Appendix C).

4.0 CLEANUP VERIFICATION DATA EVALUATION

This section presents the evaluation and modeling of the 618-8 cleanup verification data for comparison with the data quality criteria and RAGs.

4.1 DATA QUALITY ASSESSMENT PROCESS

A data quality assessment (DQA) is performed to compare the verification sampling approach and resulting analytical data with the sampling and data quality requirements specified by the project objectives and performance specifications.

The DQA for the 618-8 waste site determined that the data are of the right type, quality, and quantity to support site verification decisions within specified error tolerances. All analytical data were found to be acceptable for decision-making purposes. The evaluation also verified that the sample design was sufficient to support clean site verification. The analytical data are stored in the Environmental Restoration database prior to being transferred to the Hanford Environmental Information System and are summarized in Appendix A. The detailed DQA is presented in Appendix B.

4.2 CONTAMINANTS OF CONCERN 95% UPPER CONFIDENCE LIMIT

The primary statistical calculation to support cleanup verification is the 95% upper confidence limit (UCL) on the arithmetic mean of the data. The 95% UCL values for each COC are computed for the decision unit. Prior to calculating the 95% UCL, the individual sample results are reviewed and, as appropriate, adjusted per the SAP (DOE-RL 2004a). This process is summarized below.

4.2.1 Radionuclides

For radionuclides, the laboratory-reported value is used in the calculation of the 95% UCL. In cases where the laboratory does not report a value for data qualified with a "U" (i.e., less than the detection limit), one-half of the minimum detectable activity is used in the calculation of the 95% UCL.

4.2.2 Nonradionuclides

For nonradionuclides, a value equal to one-half the practical quantitation limit is used for data flagged with a "U" (i.e., less than the detection limit) in the calculation of the 95% UCL, as required by *Washington Administrative Code* (WAC) 173-340-740[7][g]. If greater than half of the sample results for a given nonradionuclide COC are below detection, the statistical value is set equal to the maximum concentration detected (i.e., versus computing a 95% UCL).

Statistical calculations for 618-8 verification data are presented in the 95% UCL calculation brief (Appendix C), with results shown in Table 2. The columns on the left side of Table 2 are the COCs and the 95% statistical values before subtraction of background. The third column of Table 2 presents the background, and the last column presents the statistical values adjusted for background, if appropriate, which becomes the cleanup verification data set used for evaluation against RAGs. All of the COCs for the 618-8 site were detected below background in the verification samples, with the exception of selenium, which was detected slightly above background. Individual sample cleanup verification results are presented in Appendix A.

COCs	95% UCL Statistical Values ^ª	Hanford Site Background	Shallow Zone Cleanup Verification Data Set ^b						
Radionuclide Concentration (pCi/g)									
Uranium-233/234	1.04	1.1 [°]	0 (< BG)						
Uranium-235	0.228 U	0.11 ^c	0.118 U						
Uranium-238	0.643	1.1 ^c	0 (< BG)						

 Table 2. Cleanup Verification Data Set. (2 Pages)

COCs	95% UCL Statistical Values ^a	Hanford Site Background	Shallow Zone Cleanup Verification Data Set⁵							
Nonradionuclide Concentration (mg/kg)										
Arsenic	3.7	6.5	3.7							
Barium	93.3	132	93.3							
Cadmium	0.07 U	0.81 [°]	0.07 U							
Chromium	11.8	18.5	11.8							
Lead	4.8	10.2	4.8							
Selenium	0.83	0.78 ^c	0.83							
Silver	0.15 U	0.73	0.15 U							
Uranium (total)	1.7	3.21	1.7							

Table 2. Cleanup Verification Data Set. (2 Pages)

^a Laboratory data, including the minimum detectable activity or practical quantitation limit for the individual cleanup

verification samples, are included in Appendix A and the 95% UCL calculation brief in Appendix C. ^b For shallow zone decision units, background is subtracted only for naturally occurring radionuclides (e.g., uranium). Nonradionuclide background levels are considered in direct evaluation of the cleanup verification data set.

^c Hanford Site Background: Part 2, Soil Background for Radionuclides (DOE-RL 1996).

^d Hanford Site-specific background not available. Value is from Ecology publication 94-11 (Ecology 1994).

BG = background

COC = contaminant of concern

U = undetected (in all samples)

UCL = upper confidence limit

4.3 SITE-SPECIFIC CLEANUP VERIFICATION MODEL

A site-specific vadose zone model was not developed for the 618-8 waste site. For the statistical cleanup verification data set, all COCs were either undetected, or the statistical values were determined to be below statistical background levels, with the exception of selenium, which was detected slightly above background, as shown in Table 2.

4.4 **RESRAD MODELING**

A site-specific RESidual RADioactivity (RESRAD) model was not developed for the 618-8 waste site, because the statistical values for radionuclides were determined to be below the statistical background levels as reported in Hanford Site Background: Part 2, Soil Background for Radionuclides (DOE-RL 1996).

5.0 EVALUATION OF REMEDIAL ACTION GOAL ATTAINMENT FOR INDUSTRIAL LAND USE

This section demonstrates that remedial action at the 618-8 waste site has achieved the applicable RAGs developed to support industrial land use. Sections 5.1, 5.2, and 5.3 address attainment of direct exposure RAGs, groundwater protection RAGs, and Columbia River protection RAGs, respectively. Section 5.4 documents application of the

WAC 173-340-740(7)(e) three-part test, which is required for nonradionuclide COCs only.

5.1 DIRECT EXPOSURE SOIL REMEDIAL ACTION GOALS ATTAINED

5.1.1 Radionuclides

5.1.1.1 Direct Comparison to RAGs. Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2. All applicable RAGs have been met.

5.1.1.2 Radionuclide Risk. Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2.

5.1.2 Nonradionuclides

5.1.2.1 Direct Comparison to RAGs. All nonradionuclide COCs in the statistical verification data set were either not detected or detected below background levels, with the exception of selenium, which was detected slightly above background, as shown in Table 2. Table 3 compares the nonradionuclide cleanup verification statistical values for selenium, presented in Table 2, to the direct exposure RAG presented in Table 1. The statistical value is less than the corresponding RAG.

Table 3.	Attainment of Nonradionuclide Direct Exposure
	Standards – Industrial Land Use.

Nonradionuclides	RAG (mg/kg)	Shallow Zone Verification Data Set Values (mg/kg)	Direct Exposure RAG Attained? ^a
Selenium	17,500	0.83	Yes

^aCriterion is comparison to direct exposure RAG.

RAG = remedial action goal

5.1.2.2 Noncarcinogenic Hazard Quotient RAG Attained. For noncarcinogenic COCs, WAC 173-340-740(5)(a) and (b) specify the evaluation of the hazard quotient, which is given as daily intake divided by a reference dose. This evaluation is shown in the 95% UCL calculation brief (Appendix C). The calculated hazard quotient for statistical residual selenium concentrations (the only nonradionuclide COC detected above background) at the 618-8 waste site is 2.1×10^{-3} . This value is below the individual and cumulative RAGs (a hazard quotient of 1.0 in both cases).

5.1.2.3 Carcinogenic Risk RAG Attained. For individual nonradionuclide carcinogenic COCs, the WAC 173-340-745(4)(a)(iii) Method C cleanup limits are based

on an industrial land-use incremental cancer risk of 1×10^{-5} . The cumulative excess cancer risk for all nonradionuclide carcinogenic COCs must also be less than 1×10^{-5} (WAC 173-340). Excess cancer risk was not calculated because the nonradionuclide carcinogenic COCs identified for the 618-8 waste site (arsenic and cadmium) were either not detected or detected below the statistical background levels.

5.2 GROUNDWATER REMEDIAL ACTION GOALS ATTAINED

5.2.1 Radionuclides

Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2. Therefore, the groundwater protection RAGs have been attained.

5.2.2 Nonradionuclides

All nonradionuclide COCs in the statistical verification data set were either not detected or detected below background levels, with the exception of selenium. Selenium was detected slightly above background, but below the applicable RAGs. Table 4 compares the nonradionuclide cleanup verification statistical values for selenium, presented in Table 2, to the soil RAG for groundwater protection presented in Table 1. Residual concentrations of selenium, the sole nonradionuclide COC detected above background for the 618-8 waste site, are less than the soil RAG for groundwater protection.

Table 4.	Attainment of Nonradionuclide Groundwater and
River	Protection Standards for the 618-8 Waste Site.

Contaminant of Concern	Cleanup Verification Data Set (mg/kg)	Soil RAG for Groundwater Protection (mg/kg)	Soil RAG for River Protection (mg/kg)	Cleanup Criteria Attained? ^a
Selenium	0.83	5 ^b	1 [°]	Yes

^a Criterion is comparison to soil RAGs for groundwater and river protection.

^b Soil RAG for groundwater protection calculated from WAC 173-303-740(3)(a)(ii)(A), 1996 ("100 times rule") and the maximum contaminant level (drinking water standard) (40 CFR 141).

^c Soil RAG for river protection calculated from WAC 173-340-740(3)(a)(ii)(A), 1996 ("100 time rule") and a

dilution attenuation factor of 2, using the ambient water quality criteria provided in WAC 173-201A-040, 1995. RAG = remedial action goal

5.3 COLUMBIA RIVER REMEDIAL ACTION GOALS ATTAINED

5.3.1 Radionuclides

Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2, therefore, the river protection RAGs have been attained.

5.3.2 Nonradionuclides

All nonradionuclide COCs in the statistical verification data set, with the exception of selenium, were either not detected or detected below background levels and are, therefore, below applicable soil RAGs for protection of the Columbia River. Selenium was detected slightly above background. Residual concentrations of selenium are less than the applicable soil RAG for protection of the Columbia River (Table 4).

5.4 WAC 173-340 THREE-PART TEST FOR NONRADIONUCLIDES

Although performed for all nonradionuclide COCs in the 95% UCL calculation (Appendix C, the WAC 173-340-740(7)(e) three-part test is required only for nonradionuclide statistical verification data sets with detections above background. The three-part test consists of the following criteria: (1) the cleanup verification statistical value must be less than the cleanup level, (2) no single detection within the data set can exceed two times the cleanup criteria, and (3) the percentage of samples in the data set exceeding the cleanup criteria must be less than 10%.

Selenium was the only nonradionuclide detected above background in the statistical verification data set. Table 5 summarizes the results of the WAC 173-340-740[7][e] three-part test for the 618-8 cleanup verification nonradionuclide data set for selenium in comparison to the most restrictive applicable RAG. The table lists the most restrictive RAG (from Table 1), the maximum detected value, the total number of samples collected, and the percentage of samples exceeding the RAG. The final column of the table describes the result of applying the three criteria using the values listed in the preceding columns.

Contaminant of Concern	Most Restrictive Applicable RAG ^a	Statistical Cleanup Verification Value (mg/kg) ^b	Maximum Detected Cleanup Verification Value (mg/kg) ^c	Total Number of Samples ^d	Percentage of Cleanup Verification Data Set Exceeding RAG ^e	Cleanup Criteria Attained?
Selenium	1 ^f	0.83	0.85	4	0	Yes

Table 5. Application of the WAC 173-340 Three-Part Test.

^a From Table 1, the most restrictive RAG is the soil RAG for protection of the Columbia River.

^b Criterion is statistical value cannot exceed most restrictive applicable RAG.

^c Criterion is no single detection can exceed two times the most restrictive applicable RAG.

^d Total number of samples in the decision unit includes field duplicate samples, which are included in the evaluation as separate samples.

³ Criterion is percentage of data set exceeding the most restrictive applicable RAG cannot exceed 10%.

^f Soil RAG for river protection calculated from WAC 173-340-740(3)(a)(ii)(A), 1996 ("100 time rule") and a dilution attenuation factor of 2, using the ambient water quality criteria provided in WAC 173-201A-040, 1995.

RAG = remedial action goal

WAC = Washington Administrative Code

As demonstrated in Table 5, residual shallow zone concentrations of selenium (the sole nonradionuclide COC exceeding background) at the 618-8 site pass the three-part test in comparison to the most restrictive applicable RAG, therefore, the RAOs for protection of groundwater and the river have been attained.

6.0 EVALUATION OF REMEDIAL ACTION GOAL ATTAINMENT FOR UNRESTRICTED LAND USE

The information presented in the previous section demonstrates that the cleanup objectives established in the ROD (EPA 2001) for industrial land use have been achieved. In addition, residual soil concentrations indicated that cleanup levels for more stringent land uses may have been achieved for the 618-8 waste site. The information presented in this section evaluates the remedial action results against cleanup criteria established for unrestricted land use to be implemented at selected sites in the 300-FF-2 Operable Unit through the *Explanation of Significant Differences for the 300-FF-2 Operable Unit Record of Decision* (ESD) (EPA 2004).

The 300 Area unrestricted land-use scenario is represented by an individual in a rural-residential setting. The exposure pathways considered in estimating dose from radionuclides in soil are inhalation; soil ingestion; ingestion of crops, meat, fish, drinking water, and milk; and external gamma exposure. This individual is conservatively assumed to spend 80% of his/her lifetime onsite. It is assumed that drinking water and irrigation water are obtained from groundwater, as impacted by the waste site.

Unrestricted land-use cleanup levels for chemicals or nonradionuclides are based on WAC 173-340-740(3), which assumes that the exposure pathway for residual contamination will be from ingestion of contaminated soil. Soil cleanup levels are calculated using the equations provided by WAC 173-340-740(3) for carcinogens and for noncarcinogens. For both carcinogens and noncarcinogens, the calculations assume that a resident with an average body weight 16 kg (35 lb) over the period of exposure ingests soil at a rate of 200 mg/day (73 g/yr [2.6 oz/yr]), with a frequency of contact of 100% and a gastrointestinal absorption rate of 100%. For carcinogens, the calculation is based on achieving a lifetime cancer risk goal of 1 in 1,000,000 (1 x 10^{-6}) for an exposure duration of 6 years and a lifetime of 75 years. For noncarcinogens, the calculation is based on achieving a hazard quotient of 1.

The key assumptions in the 300 Area unrestricted land-use scenario that affect groundwater protection are irrigation at agronomic rates (76 cm/yr [30 in./yr]), surface vegetation resulting in an evapotranspiration coefficient of 91%, and inclusion of drinking water ingestion as an exposure pathway. Details of this land-use scenario and associated RAGs are documented in the ESD (EPA 2004).

A comparison of the 618-8 waste site cleanup verification data set to the cleanup objectives for unrestricted land use as established in the ESD (EPA 2004) is presented in the following sections.

6.1 DIRECT EXPOSURE SOIL REMEDIAL ACTION GOALS ATTAINED

6.1.1 Radionuclides

6.1.1.1 Direct Comparison to RAGs. Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2. All applicable RAGs have been met.

6.1.1.2 Radionuclide Risk. Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2.

6.1.2 Nonradionuclides

6.1.2.1 Direct Comparison to RAGs. All nonradionuclide COCs in the statistical verification data set were either not detected or detected below background levels, with the exception of selenium, which was detected slightly above background, as shown in Table 2. Table 6 compares the nonradionuclide cleanup verification statistical values for selenium, presented in Table 2, to the direct exposure RAG presented in Table 1. The statistical value is less than the corresponding RAG.

Table 6. Attainment of Nonradionuclide Direct ExposureStandards – Unrestricted Land Use.

Nonradionuclides	Nonradionuclides (mg/kg)		Direct Exposure RAG Attained? ^a
Selenium	400	0.83	Yes

^aCriterion is comparison to direct exposure RAG. RAG = remedial action goal

6.1.2.2 Noncarcinogenic Hazard Quotient RAG Attained. For noncarcinogenic COCs, WAC 173-340-740(5)(a) and (b) specify the evaluation of the hazard quotient, which is given as daily intake divided by a reference dose. This evaluation is shown in the 95% UCL calculation brief (Appendix C). The calculated hazard quotient for statistical residual selenium concentrations (the only nonradionuclide COC detected above background) at the 618-8 waste site is 2.1×10^{-3} . This value is below the individual and cumulative RAGs (a hazard quotient of 1.0 in both cases).

6.1.2.3 Carcinogenic Risk RAG Attained. For individual nonradionuclide carcinogenic COCs, the WAC 173-340-745(4)(a)(iii) Method B cleanup limits are based on an unrestricted land-use incremental cancer risk of 1×10^{-6} . The cumulative excess cancer risk for all nonradionuclide carcinogenic COCs must also be less than 1×10^{-5} (WAC 173-340). Excess cancer risk was not calculated because the nonradionuclide carcinogenic COCs identified for the 618-8 waste site (arsenic and cadmium) were either not detected or detected below the statistical background levels.

6.2 GROUNDWATER REMEDIAL ACTION GOALS ATTAINED

6.2.1 Radionuclides

Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2. Therefore, the groundwater protection RAGs have been attained.

6.2.2 Nonradionuclides

All nonradionuclide COCs in the statistical verification data set were either not detected or detected below background levels, with the exception of selenium. Selenium was detected slightly above background, but below the applicable RAGs. Table 7 compares the nonradionuclide cleanup verification statistical values for selenium, presented in Table 2, to the soil RAG for groundwater protection presented in Table 1. Residual concentrations of selenium, the sole nonradionuclide COC detected above background for the 618-8 waste site, are less than the soil RAG for groundwater protection.

COC	COC Cleanup Verification Data Set (mg/kg)		Soil RAG for River Protection (mg/kg)	Cleanup Criteria Attained?
Selenium	0.83	5 ^a	1 ^b	Yes

Table 7. Attainment of Nonradionuclide Groundwater andRiver Protection Standards for the 618-8 Waste Site.

^a Soil RAG for groundwater protection calculated from WAC 173-303-740(3)(a)(ii)(A), 1996 ("100 times rule") and the maximum contaminant level (drinking water standard) (40 CFR 141).

^b Soil RAG for river protection calculated from WAC 173-340-740(3)(a)(ii)(A), 1996 ("100 time rule") and a dilution attenuation factor of 2, using the ambient water quality criteria provided in WAC 173-201A-040, 1995.
 RAG = remedial action goal

6.3 COLUMBIA RIVER REMEDIAL ACTION GOALS ATTAINED

6.3.1 Radionuclides

Radionuclide COCs were either not detected or detected below background levels for the statistical verification data set at the 618-8 waste site, as shown in Table 2, therefore, the river protection RAGs have been attained.

6.3.2 Nonradionuclides

All nonradionuclide COCs in the statistical verification data set, with the exception of selenium, were either not detected or detected below background levels and are, therefore, below applicable soil RAGs for protection of the Columbia River. Selenium was detected slightly above background. Residual concentrations of selenium are less than the applicable soil RAG for protection of the Columbia River (Table 7).

6.4 WAC 173-340 THREE-PART TEST FOR NONRADIONUCLIDES

Although performed for all nonradionuclide COCs in the 95% UCL calculation (Appendix C, the WAC 173-340-740(7)(e) three-part test is required only for nonradionuclide statistical verification data sets with detections above background. The three-part test consists of the following criteria: (1) the cleanup verification statistical value must be less than the cleanup level, (2) no single detection within the data set can exceed two times the cleanup criteria, and (3) the percentage of samples in the data set exceeding the cleanup criteria must be less than 10%.

Selenium was the only nonradionuclide detected above background in the statistical verification data set. Table 8 summarizes the results of the WAC 173-340-740[7][e] three-part test for the 618-8 cleanup verification nonradionuclide data set for selenium in comparison to the most restrictive applicable RAG. The table lists the most restrictive RAG (from Table 1), the maximum detected value, the total number of samples collected, and the percentage of samples exceeding the RAG. The final column of the table describes the result of applying the three criteria using the values listed in the preceding columns.

Contaminant of Concern	Most Restrictive Applicable RAG ^a	Statistical Cleanup Verification Value (mg/kg) ^b	tical nup ationMaximum Detected Cleanup Verification Value (mg/kg)cS		Percentage of Cleanup Verification Data Set Exceeding RAG ^e	Cleanup Criteria Attained?
Selenium	1 ^f	0.83	0.85	4	0	Yes

Table 8. Application of the WAC 173-340 Three-Part Test.

^a From Table 1, the most restrictive RAG is the soil RAG for protection of the Columbia River.

^b Criterion is statistical value cannot exceed most restrictive applicable RAG.

^c Criterion is no single detection can exceed two times the most restrictive applicable RAG.

^d Total number of samples in the decision unit includes field duplicate samples, which are included in the evaluation as separate samples.

^e Criterion is percentage of data set exceeding the most restrictive applicable RAG cannot exceed 10%.

^f Soil RAG for river protection calculated from WAC 173-340-740(3)(a)(ii)(A), 1996 ("100 time rule") and a dilution attenuation factor of 2, using the ambient water quality criteria provided in WAC 173-201A-040, 1995.

RAG = remedial action goal

WAC = Washington Administrative Code

As demonstrated in Table 8, residual shallow zone concentrations of selenium (the sole nonradionuclide COC exceeding background) at the 618-8 site pass the three-part test in comparison to the most restrictive applicable RAG, therefore, the RAOs for protection of groundwater and the river have been attained.

7.0 STATEMENT OF PROTECTIVENESS

This CVP demonstrates that remedial action at the 618-8 waste site has achieved the RAOs and corresponding RAGs established for the industrial land-use scenario in the ROD (EPA 2001), EPA (2004), and the RDR/RAWP (DOE-RL 2004b). The contaminated materials from the site have been excavated and disposed at ERDF. The remaining soil at the 618-8 waste site has been sampled, analyzed, and evaluated. Results indicate that the site supports future land uses that can be represented (or bounded) by the industrial land-use scenario and poses no threat to groundwater or the Columbia River. Consequently, the 618-8 waste site is verified to be remediated in accordance with the ROD.

Because residual soil concentrations indicated that cleanup levels for more stringent land uses may have been achieved for the 618-8 waste site, a supplemental evaluation was performed against the unrestricted land-use RAGs established for the 300 Area in the ESD (EPA 2004). This evaluation demonstrated that the results of verification sampling do not preclude any future uses (as bounded by the rural-residential scenario) and allow unrestricted use of shallow zone soils. In consideration of this and because the site has no deep zone, no institutional controls are required at the 618-8 waste site.

8.0 REFERENCES

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- WCH, 2006b, Parking Lot Test Pit/Trenching Investigation for the 618-8 Burial Ground, Interoffice Memorandum from L. M. Dittmer to M. J. Haass, CCN 124817, dated June 28, 2006, Washington Closure Hanford, Richland, Washington.

APPENDIX A

SUMMARY OF VERIFICATION SOIL SAMPLING AND ANALYTICAL RESULTS

Sampling	HEIS	Sample	Ş	Silver			Arsenic			Barium			Cadmium		
Area	Number	Date	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	
A1	J11271	1/31/2006	0.15	U	0.15	3.4		0.36	81.3		0.02	0.07	U	0.07	
Duplicate of J11271	J11272	1/31/2006	0.15	U	0.15	4.1		0.36	79.4		0.02	0.07	U	0.07	
A2	J11273	1/31/2006	0.15	U	0.15	3.8		0.34	97.6		0.02	0.07	U	0.07	
A3	J11274	1/31/2006	0.15	U	0.15	2.4		0.36	87.0		0.02	0.07	U	0.07	
A4	J11275	1/31/2006	0.15	U	0.15	2.4		0.36	69.1		0.02	0.07	U	0.07	
Split of J11271	J11277	1/31/2006	1.1	U	1.1	3.5		1.1	94.9		21.3	0.53	υ	0.53	

Table A-1. 618- 8 Verification Sampling Results.

Sampling	HEIS	Sample	Ch	romi	um		Lead	1	Se	leni	um	Tota	Ura	nium
Area	Number	Date	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL
A1	J11271	1/31/2006	10.4		0.17	4.5		0.33	0.81		0.38	1.28		0.017
Duplicate of J11271	J11272	1/31/2006	11.7		0.17	4.2		0.33	0.76		0.38	1.13		0.017
A2	J11273	1/31/2006	12.4		0.17	5.1		0.32	0.51		0.21	1.67		0.017
A3	J11274	1/31/2006	10.2		0.17	4.4		0.33	0.85		0.38	1.72		0.017
A4	J11275	1/31/2006	9.4	İ –	0.17	3.9		0.32	0.69		0.38	1.35		0.017
Split of J11271	J11277	1/31/2006	8.3		1.1	3.9		1.1	0.33	в	1.6	3.28		0.020

Sampling	HEIS	Sample	Uraniu	ım-2	33/234	Ura	nium	-235	Urai	nium	-238
Area	Number	Date	pCi/g	Q	MDA	pCi/g	Q	MDA	pCi/g	Q	MDA
A1	J11271	1/31/2006	0.540		0.27	0	U	0.31	0.439		0.26
Duplicate of J11271	J11272	1/31/2006	0.736		0.26	0.081	U	0.31	0.401	·	0.26
A2	J11273	1/31/2006	0.780		0.35	0.056	U	0.42	0.734		0.35
A3	J11274	1/31/2006	1.19		0.40	0.314	U	0.48	0.363	U	0.40
A4	J11275	1/31/2006	0.778		0.25	0.078	U	0.30	0.518		0.25
Split of J11271	J11277	1/31/2006	1.14		0.053	0.078		0.030	1.10		0.053
B= analyte found in method blankMDA= minimum detectable activityQ= 0								Q = qualit			

PQL = practical quantitation limit

ier U = undetected

APPENDIX B

DATA QUALITY ASSESSMENT

B1.0 DATA QUALITY ASSESSMENT FOR 618-8 BURIAL GROUND WASTE SITE

B1.1 OVERVIEW

The data quality assessment (DQA) completes the data life cycle (i.e., planning, implementation, and assessment) that was initiated by the data quality objectives process. The DQA includes a review of the field logbook information (WCH 2006c) to verify sample location, date, and time. It also involves a scientific and statistical evaluation of the data to determine if they are of the right type, quality, and quantity to support their intended use for closeout decisions.

This DQA was performed in accordance with ENV-1, *Environmental Monitoring and Management*. Specific data quality objectives for the site are found in the *300 Area Remedial Action Sampling and Analysis Plan* (SAP) (DOE-RL 2004a). The DQA is based on the guidelines presented in *Guidance for Data Quality Assessment* (EPA 2000). Statistical tests used in this DQA were performed as specified in the SAP and the *Remedial Design Report/Remedial Action Work Plan for the 300 Area* (RDR/RAWP) (DOE-RL 2004b).

Prior to performing statistical tests, the field logbook (WCH 2006c), sample designs (WCH 2006b), and sample analytical data are evaluated. A portion of the cleanup verification sample analytical data is validated for compliance requirements (DOE-RL 2004a). Data evaluation is performed to determine if the laboratory carried out all steps required by the SAP and the laboratory contract governing the conduct of analysis and reporting of the data. This evaluation also examines the available laboratory data to determine if an analyte is present or absent in a sample and the degree of overall uncertainty associated with that determination. Data validation is done in accordance with validation procedures (BHI 2000a, 2000b). After data evaluation and validation, the appropriate statistical analyses are performed on the adjusted raw analytical data (Appendix C) to determine statistical values for each contaminant. The cleanup verification sample analytical data are stored in the Environmental Restoration database prior to being transferred to the Hanford Environmental Information System and are summarized in Appendix A.

B1.2 LABORATORY QUALITY MEASURES

All verification samples are subject to laboratory-specific quality assurance (QA) requirements, including instrument procurement, maintenance, calibration, and operation. Additional laboratory quality control (QC) checks are performed, as appropriate, for the analytical method at a rate of one per sample delivery group (SDG), or 1 in 20, whichever is more frequent. Laboratory internal QC checks include the following:

- <u>Laboratory Contamination</u>. Each analytical batch contains a laboratory (method) blank (material of similar composition as the samples with known/minimal contamination of the analytes of interest) carried through the complete analytical process. The method blank is used to evaluate false-positive results in samples, due to contamination during handling at the laboratory.
- <u>Analytical Accuracy</u>. For most analyses, a known quantity of representative analytes of interest (matrix spike/matrix spike duplicate [MS/MSD]) is added to a separate aliquot of a sample from the analytical batch. The recovery percentage of the added MS is used to evaluate analytical accuracy. For analyses not amenable to MS techniques (e.g., gamma energy analysis) or where analytical recovery is corrected via internal standards (e.g., alpha spectral analyses), accuracy is evaluated from recovery of the QC reference sample (e.g., laboratory control spike or blank spike sample).
- <u>Analytical Precision</u>. Separate aliquots removed from the same sample container (replicate samples) are analyzed for each analytical batch. The replicate sample results (evaluated as relative percent differences [RPDs]) are used to assess analytical precision.
- <u>QC Reference Samples</u>. A QC reference sample is prepared from an independent standard at a concentration other than that used for calibration, but within the calibration range. Reference samples provide an independent check on analytical technique and methodology.

Laboratories are also subject to periodic and random assessments of the laboratory performance, systems, and overall program. These assessments are performed by the Washington Closure Hanford QA group to ensure that the laboratories are performing within laboratory contract requirements.

B1.3 DATA VALIDATION

After sampling was completed, all of the fixed-base laboratory data from SDG K0204 were submitted for third-party validation to Level C per ENV-1-2.12, "Data Package Validation." Level C validation procedures are specified in *Data Validation Procedure for Radiochemical Analysis* (BHI 2000b) and *Data Validation Procedure for Chemical Analysis* (BHI 2000a).

Use of Level C validation procedures was included in the review of the following items, as appropriate, for each analytical method:

- Sample holding times
- Method blanks
- MS/MSD recovery
- Surrogate recovery

- Sample replicates
- Associated batch laboratory control sample results
- Data package completeness
- Achievement of required (or contractual) detection limits (RDLs).

Data flagged by the validator as estimated (i.e., "J") indicate that the associated concentration is an estimate, but that the data may be used for decision-making purposes. Data flagged as below detection limits (i.e., "U") indicate the contaminant was analyzed for but not detected, and the concentration is below the minimum detectable activity (MDA) for radionuclides or the practical quantitation limit (PQL) (i.e., reporting limit) for nonradionuclides. For nonradionuclides, nondetects are reported at the PQL. For radionuclides, nondetects report the actual value obtained from analysis (positive or negative but less than the MDA) except for limited analyses where no value can be calculated. In these cases, the MDA is reported. This situation is applicable for sample results that are below detection limits. All other validated results are considered to be accurate within the standard errors associated with the methods.

The adequacy of laboratory QA/QC was evaluated for precision, accuracy, completeness, and RDLs pursuant to the SAP (DOE-RL 2004a). The organization performing the data validation reported that, of the data given formal validation, the laboratory met the standards for performance for precision (\pm 30%), accuracy (\pm 30%), and completeness (>90%). Comparison of the RDL with the respective MDA or PQL is discussed in Section B1.4.

The validated SDG K0204 contains six samples (J11271, J11272, J11273, J11274, J11275, J11276) from the 618-8 Burial Ground. Sample J11276 is the equipment blank and sample J11272 is a duplicate of sample J11271. A summary of deficiencies noted during validation follows.

• Radionuclides. The validation DQA noted no major deficiencies.

Total uranium was found in the equipment blank (J11276) at 0.496 mg/kg. The Hanford Site-specific background value for total uranium is 3.21 mg/kg. This result is probably carryover in the analytical equipment but does not represent a significant source of contamination. The field sample data are useable for decision making purposes.

• Nonradionuclides. The validation DQA noted no major deficiencies.

Barium and lead were found in the equipment blank (J11276) at 1.8 mg/kg and 0.72 mg/kg, respectively.

Third party validation did not qualify any of the data in SDG K0204.

B1.4 LABORATORY DATA EVALUATION

The following paragraphs include a data evaluation of two verification sample SDGs for the 618-8 Burial Ground, SDG K0204 and SDG J00056. SDG K0204 consists of six total samples (J11271 to J11276) from the 618-8 site: four statistical samples, an equipment blank (J11276) and a field duplicate (J11272). SDG K0204 was also subjected to third party verification, as mentioned above. SDG J00056 consists of one split sample (J11277).

The context for assessing the data includes evaluating the sample data using the statistical methodology from the SAP (DOE-RL 2004a) (included in the calculation briefs in Appendix C) and a comparison of analytical results to the parameters as specified in the SAP. This section summarizes the results of the comparison and presents an evaluation of the affected data.

MAJOR DEFICIENCIES

Any data anomaly that causes final data to be qualified as rejected (R flagged) is considered a major deficiency. No major deficiencies were identified in the data.

MINOR DEFICIENCIES

Sample Holding Times. All of the method-specific holding times were met for all samples in the 618-8 data set.

<u>Method Blanks.</u> The method blank is used to evaluate false-positive results in samples due to contamination during handling at the laboratory.

Radionuclides. In the radionuclide analyses, low-level positive results were observed for uranium-233/234 at 0.035 pCi/g. The Hanford Site-specific background activity for uranium-233/234 is 1.1 pCi/g. Most radiological analytical techniques are counting methods. Due to the nature of this type of analysis, positive, but insignificant, results are not uncommon in the method blank and are not considered contamination.

Nonradionuclides. Barium was detected in the method blank at 0.09 mg/kg. The lowest applicable remedial action goal (RAG) for barium is the river protection RAG at 132 mg/kg. All of the nonradiological method blanks associated with 618-8 were found acceptable.

MS/MSD Recoveries. Recovery of spiked analytes in the MS/MSD pair is used to evaluate method efficiency and the effect of the matrix on an environmental sample.

Radionuclides. All MS/MSD recoveries for radionuclide analytes were within acceptance criteria.

Nonradionuclides. In the inductively coupled plasma metals analysis of SDG J00056, the percent recoveries for barium and chromium were 148% and 128%, respectively. The barium result is outside of the project-specific acceptance criteria of +/- 30%. The chromium result is outside of the laboratory acceptance criteria of +/- 20%, but is within the project-specific acceptance criteria. Both results are attributed to natural heterogeneity in the soil sample used as the matrix. The data are useable for the intended purpose.

RDL Comparison. Reported analytical detection levels for nondetected analytes were compared to the RDLs specified in the SAP (DOE-RL 2005a). When detected results were obtained, evaluation of detection limits was not performed. The data validation and supplemental data evaluation noted any analyses in which the detection limit (MDA or PQL) was above the SAP RDLs for nondetected analytes.

Radionuclides. All of the reported MDAs are sufficiently low for decision-making purposes. All values meet the site cleanup criteria as demonstrated in the calculation briefs (Appendix C) and discussed in this cleanup verification package.

Nonradionuclides. All of the reported method detection limits are less than applicable RAGs, and the data are of sufficient quality for decision-making purposes.

Precision and Accuracy Evaluation. Analytical accuracy and precision were evaluated by examination of the RPD of the main, duplicate, and split samples. Only the contaminants of concern (COCs) detected at five times the target detection limit (or greater) are used for data analysis with respect to accuracy and precision.

Radionuclides. RPDs for the radionuclide analytes were not calculated because an evaluation of the data shows none of the analytes were detected in both the main and duplicate (or split) sample at more than five times the target detection limit.

Nonradionuclides. RPDs for the nonradionuclide analytes barium and chromium were calculated for both the duplicate and split samples. In the duplicate sample analysis, both barium and chromium were within acceptance criteria with RPDs of 2.4% and 12%, respectively. The RPDs for the split sample analysis of barium and chromium were 15% and 22%, respectively. The acceptance criteria for split sample RPDs is 30%. It was previously discussed that barium and chromium also had high matrix spike recoveries. All of the sample extracts associated with these results were produced using field-collected materials. The natural heterogeneity of field collected soils and materials adds to elevated RPDs. This variability is expected and does not indicate a problem with the analytical system. RPDs of analytes detected at low concentrations (less than five times the detection limit) are also not considered to be indicative of the analytical system performance. The data are useable for decision-making purposes.

B1.5 FIELD QUALITY ASSURANCE/QUALITY CONTROL

Field QA/QC measures were used to assess potential sources of error and crosscontamination of soil samples that could bias results. Field QA/QC samples listed in the field logbook (WCH 2006c) are summarized in Table B-1. All main and QA/QC sample results are presented in Appendix A.

Equipment Blank	Main Sample	Duplicate	Split
J11276	J11271	J11272	J11277

 Table B-1. Summary of Field Quality Control Samples.

Field duplicate samples were collected to provide a relative measure of the degree of local heterogeneity in the sampling medium, unlike laboratory duplicates that are used to evaluate precision in the analytical process. The field duplicates are evaluated by computing the RPD of the duplicate samples for each COC. Only analytes with values above five times the detection limits for both the main and duplicate samples are compared. The 95% upper confidence limit (UCL) calculation brief in Appendix C provides details on duplicate pair evaluation and RPD calculation. The data are suitable for the intended purpose of cleanup verification.

Split samples were collected to provide a relative measure of the degree of variability in the sampling, sample handling, and analytical techniques used by commercial laboratories. The field main and split samples are evaluated by computing the RPD of the split samples for each COC to determine the usability of the verification data. The U.S. Environmental Protection Agency Contract Laboratory Program duplicate sample comparison methodology, *USEPA Contract Laboratory Program National Functional Guidelines for Inorganic Data Review* (EPA 1994), is used as an initial test of the data from the splits. Only analytes that had values above five times the contractual RDL for both the main and split sample were compared. The 95% UCL calculation brief in Appendix C provides details on split pair RPD calculation. These results are typical of the heterogeneity found in the sample matrices and do not indicate a problem in the analytical systems.

B1.6 SUITABILITY OF DATA

The DQA for the 618-8 Burial Ground determined that the data are of the right type, quality, and quantity to support site cleanup verification decisions within specified error tolerances. The evaluation verified that the sample design was sufficient for the purpose of clean site verification. All analytical data were found to be acceptable for decision-making purposes.

B2.0 REFERENCES

- BHI, 2000a, *Data Validation Procedure for Chemical Analysis*, BHI-01435, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.
- BHI, 2000b, *Data Validation Procedure for Radiochemical Analysis*, BHI-01433, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.
- DOE-RL, 2004a, *300 Area Remedial Action Sampling and Analysis Plan*, DOE/RL-2001-48, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 2004b, *Remedial Design Report/Remedial Action Work Plan for the 300 Area*, DOE/RL-2001-47, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- ENV-1, *Environmental Monitoring & Management*, Washington Closure Hanford, Richland, Washington.
- EPA, 1994, USEPA Contract Laboratory Program National Functional Guidelines for Inorganic Data Review, EPA 540/R-94/013, U.S. Environmental Protection Agency, Washington, D.C.
- EPA, 2000, *Guidance for Data Quality Assessment*, EPA QA/G-9, QA00 Update, U.S. Environmental Protection Agency, Office of Environmental Information, Washington, D.C.
- WCH, 2006a, 618-8 Burial Ground Cleanup Verification 95% UCL Calculations, Calculation Number 0600X-CA-V0058, Washington Closure Hanford, Richland, Washington.
- WCH, 2006b, *Closeout Plan for the 618-8 Burial Ground*, CCN 126068, Washington Closure Hanford, Richland, Washington.
- WCH, 2006c, *Remedial Sampling*, Logbook EL-1395-11, Washington Closure Hanford, LLC., Richland, Washington.

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APPENDIX C

CALCULATION BRIEF EXCERPTS

DISCLAIMER FOR CALCULATIONS

The calculations that are provided in the following appendix have been generated to document compliance with established cleanup levels. These calculations should be used in conjunction with other relevant documents in the administrative record.

CALCULATION BRIEFS

The following calculation briefs have been prepared in accordance with ENG-1, *Engineering Services*, ENG-1-4.5, "Project Calculations," Washington Closure Hanford, Richland, Washington.

- 618-8 Shallow Zone Sampling Plan, Calculation No. 0300X-CA-V0067, Rev. 0, Washington Closure Hanford, Richland, Washington.
- 618-8 Burial Ground Cleanup Verification 95% UCL Calculations, Calculation No. 0600X-CA-V0058, Rev. 0, Washington Closure Hanford, Richland, Washington.

NOTE: The calculation briefs referenced in this appendix are kept in the active Washington Closure Hanford project files and are available upon request. When the project is completed, the files will be stored in a U.S. Department of Energy, Richland Operations Office repository.

CALCULATION COVER SHEET

Project Title:	618-8 Burial Ground Sample Design		Job No.	14655
Area	300 Area			
Discipline	Environmental Engineering	Calc. No.	0300X-CA-V0067	
Subject	618-8 Shallow Zone Sampling Plan			
Computer Program	Excel	Program No.	Excel 2003	

The attached calculations have been generated to document compliance with established cleanup levels. These documents should be used in conjuction with other relevent documents in the administrative record.

Commit	tted Calculation	X	Preliminary	Superseded	Voided	
Rev.	Sheet Numbers	Originator	Checker	Reviewer	Approval	Date
0	Cover = 1 Sht Calc = 1 Shts Attach1 = 1 Sht Attach2 = 1 Sht Attach3 = 1 Shts Total = 5 Shts	JA G. Cruz 02/06/06	B C.A. Bentz 2/6/06	<u>Р.В.Кечк</u> <u>Р.В.К.</u> R.B. Kerkow 2/6/06	M.J. Haass	216/06
	T		SUMMARY OF RE	VISIONS		•

* Obtain calc no. from DIS

DE01437.03 (12/09/2004)

Washington Giospré Nanford	ashington Closure	Hanford CALCULATION SH	EET					
Originato	G. Cruz	Date 2/2/2006	Calc. No.	0300X-0	CA-V0067	Rev. No.	0	
Project	618-8 Burial Gro	und Sample Design	Job No.	14655	Checked	ab	Date	2/6/06
Subject	618-8 Shallow Zo	one Sampling Plan				Sheet No.	1 of 1	

Problem:	Calculate a	and display I	required	sampling nodes in concurr	ence with 30) Area		T		
	SAP DOE/	RL-2001-48	Rev. 01	for verification and closure.						
Given:	-SAP (DOF	-/RL-2001-4	8 Rev ()) requirements						
	-Shallow S	ampling Are	a (Surfa	ce area of each zone dete	rmined from (CAD progra	m.			
	Attachmon	t 3 Sht 1of		10 3X 0202068 618-8 Pur	al Ground Sh	allow Zone	Sampling	Plan)		
	Allacimen		, 070 1	10 07.1020200D, 010-0 Dul			camping	1		
	-								· · · · · · · · · · · · · · · · · · ·	
SAP Requir	rements:									
	-Develop a	a 16 node sa	ampling g	grid for the sampling area						
Shallow Zor	ne-Use table	3-2 of the S	AP to de	etermine which four of the	sixteen nodes	s will be sar	npled			
	to collect c	lean up veri	fication :	samples						
	-Develop a	a 16 node sa	ampling g	grid for the sampling area						
Overburden	: -Use table	3-2 of the S	SAP to de	etermine which four of the	sixteen nodes	s will be sar	npled			
	to collect o	lean up veri	fication	samples						
2]								
1	-Develop a	a 16 node sa	ampling	grid for the sampling area						
Deep Zone:	-Use table	3-2 of the S	SAP to de	etermine which four of the	sixteen node	s will be san	npled			
5	to collect o	lean up ver	fication	samples						
3		1		· · · · · · · · · · · · · · · · · · ·						
Determinat	ion of Shall	ow Zone Sa	mpling	Grid:						
- stannia		1	9							
Shallow Zor	e Sampling	Grid Area d	etermine	d from Table 3-2, SAP						
Attachment	2 Number of	of Decision	Subunite	Based on Area (Converter	to Sa Meter	s)				
in machinem						-/				
Total Ar	-					1767 04	m ²		-	
Total Area:		1 · · · ·	l			4707.24	2			
Area of Dec	sision Subuni	its (total are	a 1 subu			1/6/.24	in			
1		1	L	L			2			
Decision Su	bunit divide	d into 4 Sam	pling Ar	eas:		441.81	m [*]			
3										
Sampling A	reas divided	into a 16 no	de grid	(node numbers 1-16):		27.61	m²			
3		T								
Nodes to be	Sampled (a	as determine	ed from A	Attachment 1, Table A-1. S	ample Grid P	oint Lookur	o Table)			
	See Attac	hment 3. Sh	t 10f1.6	18-8 Burial Ground Shallo	w Zone Sam	ling Plan,				T
1	for Sample	e Location T	able							
2		1	T							
1										
·I		1	1		L					

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Washington Closure Nacional	hington Closure Hanford							
Originator	G. Cruz Date	2/2/2006	Calc. No.	0300X-CA	-V0067	Rev. No.	0	
Project	618-8 Burial Ground San	nple Design	Job No	14655 (Checked_	ass	Date	2/6/06
Subject	618-8 Shallow Zone Sam	pling Plan	*****			Sheet No	1of1	

1 ATTACHMENT 1

3 Sample Grid Point Lookup Table.

6	Default Plan	Sampling Area 1	Sampling Area 2	Sampling Area 3	Sampling Area 4	Sampling Area 5	Sampling Area 6	Sampling Area 7	Sampling Area 8	Sampling Area 9	Sampling Area 10
7	Closeout	3	6	1	4	5	1	3	3	4	16
8	Closeout	4	7	11	3	15	15	5	13	10	10
9	Closeout	16	3	2	7	7	10	11	4	3	14
10	Closeout	10	15	4	12	1	13	4	8	16	4
11	Not Sampling	2	14	5	9	13	12	8	2	14	8
12	Not Sampling	13	10	9	13	2	16	1	12	5	3
13	Not Sampling	6	1	10	8	14	4	16	5	8	6
14	Not Sampling	1	9	13	1	10	5	12	1	1	15
15	Not Sampling	9	12	7	5	6	2	6	7	15	9
16	Not Sampling	15	16	15	14	16	6	2	15	11	1
17	Not Sampling	8	13	8	10	12	11	13	14	2	12
18	Not Sampling	5	2	3	11	4	3	9	10	7	11
9	Not Sampling	7	11	14	15	11	14	14	6	13	2
20	Not Sampling	11	4	6	2	9	7	7	11	9	7
21	Not Sampling	12	8	16	16	3	8	15	9	6	.13
22	Not Sampling	14	5	12	6	8	9	10	16	12	5
23 24	** Note: Grid nodes for each sampling area in each waste site should be numbered consistently, e.g., begin numbering the nodes in the northwesternmost node. Then number consecutively left to right.										

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Washington Closure Hauford	ashington Closure	Hanfor	d					
Originato	r G. Ćruz	Date	2/2/2006	Calc. No.	0300X-C	A-V0067	Rev. No.	0
Project	618-8 Burial Grou	- und Sa	mple Design	Job No.	14655	Checked	CIB	Date 2/6/06
Subject	618-8 Shallow Zo	one Sa	mpling Plan				Sheet No.	. 1of1

1 ATTACHMENT 2

3 Number of Decision Subunits Based on Area.

Decision Unit [*]	Waste Site Size ^b	Decision Subunits	Blocks ^c	Discrete Samples	Composite Samples				
Shallow zone -	Small: $< 100.000 \text{ ft}^2$	1	4	16	4				
0 10 15 ft	Medium: $>100,000 \text{ ft}^2 < 400,000 \text{ ft}^2$	4	16	64	16				
	Large: >400.000 ft ²	8	32	128	32				
Deep Zone -	Small: $< 100.000 \text{ ft}^2$	1	4	16	4				
>15 ft	Medium: >100,000 ft^2 < 400,000 ft^2	4	16	64	16				
	Large: >400,000 ft^2	8	32	128	32				
Overburen/layback	Small: $< 100.000 \text{ ft}^2$	1	4	16	4				
stockpiles	Medium: $>100,000 \text{ ft}^2 < 400,000 \text{ ft}^2$	4	16	64	16				
•	Large: >400,000 ft^2	8	32	128	32				
Staging pile areas residual soil)	Small: $< 100,000 \text{ ft}^2$	1	4	16	4				
	Medium: $>100,000 \text{ ft}^2 < 400,000 \text{ ft}^2$	4	16	64	16				
	Large: $>400.000 \text{ ft}^2$	8	32	128	32				

The shallow zone, deep zone, overburden stockpile, and staging pile areas each represent single decision units. The total number of decision units will vary because individual waste sites may not have a deep zone, overburden stockpile, and/or staging pile areas.
 A rea of exposed surface after excavation or area of stockpile base (as applicable)
 Decision subunits are divided into four blocks to ensure that random sampling locations are not bunched together in one area

C-8



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C-9/C-10

	r								
		NOT	TES						
	1. SHALLOW ZONE SQUARE METERS	NODE AREAS AF	RE APPROXIMA	TELY 27.61					
4,	2. SAMPLES ARE T OF EACH NODE	AKEN FROM THE	APPROXIMATE	CENTER					
· ~~,	3. THE SHALLOW 2 A2, A3, & A4	ONE CONSISTS	OF SAMPLING SUBUNIT 1.	AREAS A1,					
			END						
	c	LEAN UP VERIFIC	CATION SAMPL	ING NODE					
	······································	SAMPLE LOC	ATION TAB	_ <u>L</u>					
	DECISION SUBUNIT	SAMPLING AREA	SAMPLE NODE	NORTHING	EASTING				
	1	A1	S-A1-3	116521.03	593820.18				
íx,			S-A1-4	116496.32	593806.40				
×.			S-A1-16	116499.63	593823.61				
X		A2	S-A2-3	116492.61	593816.10				
			S-A2-6	116485.98	593810.90				
			S-A2-7	116490.31	593818.33				
			S-A2-15	116484.36	593821.04				
		A3	S-A3-1	116473.31	593810.36				
			S-A3-2	116477.27	593816.75				
			S-A3-4	116485.69	593829.35				
			S-A3-11	116475.28	593827.88				
		A4	S-A4-3	116457.15	593831.39				
			S-A4-4	116450.81	593838.83				
			S-A4-12	116450.74	503848.02				
	L	1	5-44-12	110430.74	595646.0Z				
	(Pathagas	3		Churchile	1 08 1				
	Aractment Sheet No. 1 of 1 Originetor Date OZ/05/06 Chiró By Date Z/6/06 Chiró By OZ/05/06 Chiró By OZ/06/06 Chiró By OZ/06/06 Chiró By OZ/06/06 Otto No. 03000x-CA-V0067 Rev. No. 0								
+		300							
	300 AREA 300 AREA REMEDIAL DESIGN 618-8 BURIAL GROUND SHALLOW ZONE SAMPLING PLAN								
	SHALLOW ZONE SAMPLING PLAN								

CALCULATION COVER SHEET

Project Title:	300 Area Field Rer	nediation		Job No.	14655
Area	600 Environmental		*Calc No	0600X-CA-V0058	
Subject	618-8 Burial Grour	d Cleanup Verification	n 95% UCL Calculations		
Computer Program	Excel		Program No.	Excel 2003	
The attached calculation should be used in conju	s have been generate	ed to document compl ant documents in the	iance with established c administrative record.	leanup levels. These	calculations
Committed Calculation	X	Preliminary 🔲	Superseded	Voideo	· 🖸
Rev. Sheet Numbers	Originator	Checker	Reviewer	Approval	Date
Cover = 1 0 Sheets = 5	, BS. Wigman 5/9/06	J.M.Blahly 5/9/06	Ambitmer 5/11/06	MJ H m.J. Haass PBKWK	5/15/06
Total = 6	B. S. Wiegman	T. M. Blakley	L. M. Dittmer	R. B. Kerkow	
		SUMMARY OF REV	ISIONS		
			*Obtain Calo No from	B&DC and Form fro	m Intranet

Washington Closure Hanford	CALCULATION SHE	ET	
Originator B. S. Wiegman Project 300 Area Field Remediation	Date 05/09/06 Job No. 14655	Calc. No. <u>0600X-CA-V0058</u> Checked T. M. Blakley Jmrs	Rev. No. 0 Date 05/09/06
Subject 618-8 Burial Ground Cleanup Verification 9	35% UCL Calculations		Sheet No. 1 of 5
Summary			
Calculate the 95% upper confidence limit (UCL) to evalua nonradionuclide analytes, perform the <i>Washington Admin</i> , difference (RPD) for each contaminant of concern (COC).	te compliance with cleanup standards for t <i>istrative Code</i> (WAC) 173-340 (Model Tox	he subject site. Also, calculate the carcinogeni ics Control Act [MTCA]) 3-part test, if required,	c risk for applicable and calculate the relative perc
Table of Contents: Sheets 1 to 2 - Calculation Sheet Summary Sheet 3 - Calculation Sheet Metals Sheet 4 - Calculation Sheet Radionuclides		,	
Sheet 5 - Calculation Sheet Split-Duplicate Analysis			
 Given/References: 1) Sample Results. 2) All lookup values, remedial action goals (RAGs), and 3) DOE-RL, 2001, Hantord Site Background: Part 1, Soil Office, Richland, Washington. 4) DOE-RL, 2004a, 300 Area Remedial Action Sampling Richland, Washington. 5) DOE-RL, 2004b, Remedial Design Report/Remedial A Constructions Office 	background values are taken from DOE-R Background for Nonradioactive Analytes, and Analysis Plan (SAP), DOE/RL-2001- Action Work Plan for the 300 Area (RDR/R	L (2001), DOE-RL (2004b), and Ecology (1994 DOE/RL-92-24, Rev. 4, U.S. Department of En 48, Rev. 1, U.S. Department of Energy, Richlar (AWP), DOE/RL-2001-47, Rev. 1, U.S. Departm	unless stated otherwise. ergy, Richland Operations Id Operations Office, nent of Energy, Richland
 Cology, 1992, Statistical Guidance for Ecology Site N Values (Censored Data Sets), Publication #92-54, Wa Secology, 1994, Natural Background Soil Metals Conc Washington. 	Aanagers, Publication #92-54, Washingtor Aanagers, Supplement S-6, Analyzing Site Ishington State Department of Ecology, Ol entrations in Washington State, Publication	n State Department of Ecology, Olympia, Wash or Background Data with Below-Detection Limi ympia, Washington. n No. 94-115, Washington State Department of	ington. t <i>or Below-PQL</i> Ecology, Olympia,
 Ecology, 1996, Model Toxics Control Act Cleanup Let Washington. EPA, 1994, USEPA Contract Laboratory Program Na Washington, D.C. WAC 173-340, 1996, "Model Toxics Control ActCle 	vels and Risk Calculations (CLARC II), Pu ational Functional Guidelines for Inorganic anup," Washington Administrative Code.	blication #94-145, Washington State Departme Data Review, EPA 540/R-94/013, U.S. Environ	nt of Ecology, Olympia, mental Protection Agency,
Solution: Calculation methodology is described in Ecology Pub. #9: calculate the 95% UCL, hazard quotients, excess carcino duplicate and primary-split sample pairs.	2-54 (Ecology 1992, 1993), below, and in t ggenic risk, perform the WAC 173-340 3-pa	he RDR/RAWP (DOE-RL 2004b). Use data fro art test for nonradionuclides, and calculate the F	m the attached worksheets to IPD for each COC in the prima
Calculation Description: The subject calculations were performed on data from so performed by utilizing the built-in spreadsheet functions a RL 2004b) is documented by this calculation. Split and d this site.	I verification samples from the 618-8 wast nd/or creating formulae within the cells. Ti uplicate RPD results are used in evaluation	e site. The data were entered into an EXCEL 2 he statistical evaluation of data for use in accorr n of data quality and are presented in the clean	003 spreadsheet and calculati dance with the RDR/RAWP (D up verification package (CVP)
Methodology: For nonradioactive analytes with <50% of the data below 95% UCL. For nonradioactive analytes with >50% of the reported as being below detection limits are set to ½ the done on the reported value. In cases where the laborato statistical evaluation of primary-duplicate sample pairs, th	detection limits and all radionuclide analyti data below detection limits, the maximum detection limit value for calculation of the s y does not report a value below the minim ne samples are averaged before being incl	es, the statistical value calculated to evaluate th value for the data set is used instead of the 95° tatistics (Ecology 1993). For radionuclide data, al detectable activity (MDA), half of the MDA is uded in the data set, after adjustments for cens	the effectiveness of cleanup is to & UCL. All nonradionuclide de calculation of the statistics wa used in the calculation. For the ored data as described above.
For nonradionuclides, the WAC 173-340 statistical guidar distribution using Ecology software. For nonradionuclide so no test for distribution is performed. For nonradionucl Background is subtracted for applicable radionuclides on	nce suggests that a test for distributional fo small data sets (n < 10) and all radionuclio ide data sets of ten or greater, distributiona ly. Comparison against background levels	orm be performed on the data and the 95% UCL de data sets, the calculations are performed ass at testing is done using Ecology's MTCAStat so for nonradionuclides is included within the CVf	calculated on the appropriate suming nonparametric distribut tware (Ecology 1993).
The hazard quotient (for shallow zone nonradionuclide C cleanup limit. The excess nonradionuclide carcinogenic 10 ⁻⁶ .	OCs) is determined by dividing the statistic risk is determined by dividing the statistica	cal value (derived in this calculation) by the WA I value by the WAC 173-340 carcinogenic clear	C 173-340 non-carcinogenic sup limit and then multiplying b
The WAC 173-340 3-part test is performed for nonradion 1) the 95% UCL value exceeds the most stringent cleanu 2) greater than 10% of the raw data exceed the most strii 3) the maximum value of the raw data set exceeds two ti	uclide analytes only and determines if: IP limit for each non-radionuclide COC, ngent cleanup limit for each non-radionucli mes the most stringent cleanup limit for ea	de COC, Ich non-radionuclide COC.	
The RPD is calculated when both the primary value and The TDL is a laboratory detection limit pre-determined for formula: RPD ={ [M-S]/((M+S)/2])*100	either the duplicate or split values are above r each analytical method, listed in Table II-	ve detection limits and are greater than 5 times 1 of the SAP (DOE-RL 2004a). The RPD calcu	the target detection limit (TDL lations use the following
where, M = Main Sample Valu	Je S = Split (or duplicate) Sample V	/alue	
For quality assurance/quality control (QA/QC) split and d threshold of 35% is used (EPA 1994). If the RPD is grea Additional discussion as necessary is provided in the dat	uplicate RPD calculations, a value less tha ter than 30% (or 35% for regulatory split da a quality assessment section of the applic:	an +/- 30% indicates the data compare favorably ata), further investigation regarding the usability able CVP.	 For regulatory splits, a of the data is performed.
If regulator split comparison is required, an additional par than 5 times the TDL and above detection. In the case w Therefore, the following calculation is performed as part of greater than +/- 2 times the TDL, then further investigatio	ameter is evaluated. A control limit of +/- : rhere only one result is greater than 5 time of the evaluation for these two cases invol- n regarding the usability of the data is perf	2 times the TDL shall be used if either the main s the TDL and the other is below, the +/- 2 time ving regulator split data: difference = main - regi ormed and presented in the applicable CVP dat	or regulator split value is less s the TDL criteria applies. ulator split. If the difference is a quality assessment section
No regulatory split samples were collected for this site.			

Washington Closure Hanford

CALCULATION SHEET

Originator B. S. Wiegman

 Date
 05/09/06
 Calc. No.
 0600X-CA-V0058

 Project
 300 Area Field Remediation
 Job No.
 14655
 Checked
 T. M. Blakley

 Subject
 618-8 Burial Ground Cleanup Verification 95% UCL Calculations
 6000X-CA-V0058
 Checked
 T. M. Blakley

Rev. No. 0 Date 05/09/06 Sheet No. 2 of 5

Summary (continued)

Results: 1 2

The results presented in the summary tables that follow are for use in RESidual RADioactivity dose/risk analysis and the CVP for this site.

3	Recult	e Summaru		A
5	nesur	Shallo	w Zone	11-11-
6	Analyte	Result	Qualifier	Units
7	Silver	0.15	U	mg/kg
8	Arsenic	3.7		mg/kg
9	Barium	93.3		mg/kg
10	Cadmium	0.07	U	mg/kg
11	Chromium	11.8		mg/kg
12	Lead	4.8		mg/kg
13	Selenium	0.83		mg/kg
14	Uranium-233/234	0 (< BG)		pCi/g
15	Uranium-235	0.118	U	pCi/g
16	Uranium-238	0 (< BG)		pCi/g
17	Total Uranium	1.7		mg/kg
18	WAC 173-340 Evaluation	ו		
19				
20	3-Part Test:			
21	95% UCL > Cleanup Limit	t?	NO	
22	> 10% above Cleanup Lin	nit?	NO	
23	Any sample > 2x Cleanup	Limit?	NO	
24				
25	Risk Estimate:			
26	Nonrad noncarcinogenic i			
27	Nonrad carcinogenic risk:		0	
28				
27				
28				
29	Relative Percent Different	nce Results	* QA/QC A	nalysis
~ ~	T	01-11-		3

30		Shallo	w Zone
	Analyte	Duplicate	Split
31		Analysis**	Analysis**
32	Silver		
33	Arsenic		
34	Barium	2.4%	15%
35	Cadmium		
36	Chromium	12%	22%
37	Lead		
38	Selenium		
39	Uranium-233/234		
40	Uranium-235		
41	I tranium 238		

44 QA/QC = quality assurance/quality control

45 RPD = relative percent difference

46 U = undetected

CALCULATION SHEET

Washington Closure Hanford

Originator	B. S. Wiegman	BN
Project	300 Area Field Re	mediation

Subject 618-8 Burial Ground Cleanup Verification 95% UCL Calculations

Date 05/09/06 Job No. 14655

Calc. No. 0600X-CA-V0058 Checked T. M. Blakley JM3

1 Shallow Zone Sample Data

	Silanow Long	Sample Data										-															
2	Sampling	HEIS	Sample		Silve	r	A (Arseni	с	E	Bariur	m	Ca	admiu	ım	Ch	romiu	ım		Lead		Se	leniun	า	Total	Uran	ium
3	Area	Number	Date	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL
4	A1	J11271	1/31/2006	0.15	U	0.15	3.4		0.36	81.3		0.02	0.07	U	0.07	10.4		0.17	4.5		0.33	0.81		0.38	1.28		0.017
5	Duplicate of J11271	J11272	1/31/2006	0.15	υ	0.15	4.1		0.36	. 79.4		0.02	0.07	U	0.07	11.7		0.17	4.2		0.33	0.76		0.38	1.13		0.017
6	A2	J11273	1/31/2006	0.15	U	0.15	3.8		0.34	97.6		0.02	0.07	U	0.07	12.4		0.17	5.1		0.32	0.51		0.21	1.67		0.017
7	A3	J11274	1/31/2006	0.15	U	0.15	2.4		0.36	87.0		0.02	0.07	U	0.07	10.2		0.17	4.4		0.33	0.85		0.38	1.72		0.017
8	A4	J11275	1/31/2006	0.15	U	0.15	2.4		0.36	69.1		0.02	0.07	U	0.07	9.4		0.17	3.9		0.32	0.69		0.38	1.35		0.017
9																											

0	Statistical Co	omputation Input L	Jata								
1	Sampling	HEIS	Sample	Silver	Arsenic	Barium	Cadmium	Chromium	Lead	Selenium	Total Uranium
2	Area	Number	Date	mg/kg	mg/kg	mg/kg	mg/kg	mg/kg	mg/kg	mg/kg	pCi/g
3	A1 [.]	J11271/J11272	1/31/2006	0.08	3.8	80.4	0.04	11.1	4.4	0.79	1.21
4	A1	J11273	1/31/2006	0.08	3.8	97.6	0.04	12.4	5.1	0.51	1.67
5	A2	J11274	1/31/2006	0.08	2.4	87.0	0.04	10.2	4.4	0.85	1.72
6	A4	J11275	1/31/2006	0.08	2.4	69.1	0.04	9.4	3.9	0.69	1.35

17 Statistical Computations

.

17	Statistical computations																		
18		Silver			Arsenic		Barium		Cadmium			Chromium		Lead		Selenium		Total Uraniu	m
		Small d	lata se	et. Use	Small data	set. Use	Small data	set. Use	Small d	ata set	Use	Small data	set. Use	Small data	a set. Use	Small dat	ta set. Use	Small dat	a set. Use
19	95% UCL value based on	nonparam	netric z	z-statistic.	nonparametri	c z-statistic.	nonparametric	c z-statistic.	nonparam	etric z-	statistic.	nonparametric	z-statistic.	nonparametr	ic z-statistic.	nonparame	tric z-statistic.	nonparamet	ric z-statistic.
20	N	4			4		4		4			4	1	4		4		4	
21	% < Detection limit	100%			0%		0%		100%			0%		0%		0%		0%	
22	mean	0.08			3.1		83.5		0.04			10.8		4.4		0.7		1.5	
23	st. dev.	0.00			0.79		11.9		0.00			1.3		0.5		0.15		0.249	
24	Z-statistic	1.645			1.645		1.645		1.645			1.645		1.645		1.645		1.645	
25	95% UCL on mean	0.08			3.7		93.3		0.04			11.8		4.8		0.83		1.69	
26	max value	0.15	U		4.1		97.6		0.07	U		12.4		5.1		0.85		1.72	
27	Statistical value	0.15	U		3.7		93.3		0.07	U		11.8		4.8		0.83		1.69	
28	Background	0.73			6.5		132		0.81			18.5		10		0.78		3.21	
29	Statistical value above background	0.15	U		3.7		93.3		0.07	U		11.8		4.8		0.83		1.69	
	Most Stringent Cleanup Limit for	0.70	B	G/River	00	DC	1000	DC	12.0		DC	100000	DC	050	DC	1	River	50	GW/
30	nonradionuclide and RAG type	0.75	Pro	otection ^a	20	00	1000	DC	13.9		00	120000	DC		00	1	Protection ^a	- 55	aw
31	WAC 173-340 3-PART Test																		
32	95% UCL > Cleanup Limit?	NO			NO		NO		NÖ			NO		NO		NO		NO	
33	> 10% above Cleanup Limit?	NO			NO		NO		NO			NO		NO		NO		NO	
34	Any sample > 2X Cleanup Limit?	NO			NO		NO		NO			NO		NO		NO		NO	
35	EXCESS RISK EVALUATION																		
36	WAC 173-340 Non-Carcinogenic Cleanup:	400			24		5600		80			120000		353		400		240	
37	Hazard quotient for each nonradionuclide:	0			0		0		0			0		0		2.1E-03		0	
38	WAC 173-340 Carcinogenic Cleanup:	NA			0.7		NA		13.9			NA		NA		NA		NA	
39	Risk for each carcinogenic nonradionuclide:	0			0		0		0			0		0		0		0	
	WAC 173-340 3-Part-Test								Deserve		-l1	Descuse all		Dessures	la a d a l a			Deseures	-11
40	Compliance? NO	Because	all silv	er values	Because all a	senic values	Because all ba	arium values	Becaus	e all ca	amium	Because all	chromium	Because all	lead values	Because all s	selenium values	Because	all uranium
		are belo	w bac	kground	are below bac	kground (6.5	are below back	kground (132	value	s are D		values an		(0.10 mg/kg		are below ba	ckground (0.78	values a	
	Nonrad noncarcinogenic	(0.73 mg/k	g), cal	lcualtion of	mg/kg), cal	cualtion of	mg/kg), cald	cualtion of	Dackgrour	iu (0.8	ring/kg),	background (10.5 mg/kg),	(0.10 mg/kg	, calcualtion	mg/kg), c	alcualtion of	background	(3.21 mg/kg),
41	index sum: 2.1.E-03	excess ris	k is no	ot required.	excess risk is	not required.	excess risk is	not required.	realculation	or exc	ass nSK IS	calcualtion or e	wired	or excess	lisk is flot	excess risk	is not required.	calcualtion o	oguirod
42	Nonrad carcinogenic risk: 0								not	require		not req	uirea.	requ	ineu.			IS NOT I	equireu.

43 ^a No value is provided for this constituent in the 300 Area RDR/RAWP (DOE-RL 2004b), values for selenium and silver are determined based on WAC 173-340-740, 1996. 44 BG = background

45 DC = direct contact

46 GW = groundwater 47 HEIS = Hanford Environmental Information System

48 NA = not applicable

49 PQL = practical quantitation limit

50 Q = qualifier

51 RAG = remedial action goal

· 52 U = undetected

53 UCL = upper confidence limit 54 WAC = Washington Administrative Code

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Date	05/09/06
Sheet No.	3 of 5

CALCULATION SHEET

Washington Closure Hanford

Originator B. S. Wiegman	Date 05/09/06	Calc. No.	0600X-CA-V0058	Rev. No.	0
Project 300 Area Field Remediation	Job No. 14655	Checked	T. M. Blakley Jung	Date	05/09/06
Subject 618-8 Burial Ground Cleanup Verification 95% UCL	Calculations			Sheet No.	4 of 5

1 Shallow Zone Sample Data (continued)

2	Sampling	HEIS	Sample	Uraniu	ım-2	33/234	Ura	nium-	-235	Urar	nium	-238
3	Area	Number	Date	pCi/g	Q	MDA	pCi/g	Q	MDA	pCi/g	Q	MDA
4	A1	J11271	1/31/2006	0.540		0.27	0	U	0.31	0.439		0.26
5	Duplicate of J11271	J11272	1/31/2006	0.736		0.26	0.081	υ	0.31	0.401	4	0.26
6	A2	J11273	1/31/2006	0.780		0.35	0.056	U	0.42	0.734		0.35
7	A3	J11274	1/31/2006	1.19		0.40	0.314	U	0.48	0.363	U	0.40
8	A4	J11275	1/31/2006	0.778		0.25	0.078	U	0.30	0.518		0.25
9												

10 Statistical Computation Input Data

11	Sampling	HEIS	Sample	Uranium-233/234	Uranium-235	Uranium-238
12	Area	Number	Date	pCi/g	pCi/g	pCi/g
13	A1	J11271/J11272	1/31/2006	0.638	0.041	0.420
4	A1	J11273	1/31/2006	0.780	0.056	0.734
5	A2	J11274	1/31/2006	1.19	0.314	0.363
6	A4	J11275	1/31/2006	0.778	0.078	0.518
7						

18 Statistical Computations

19		Uranium-233/234			Uran	ium-2	235	Uran	ium-238	
		Radionuo	clide	data set.	Radionucli	de da	ta set. Use	Radionuclide data set		
		Use no	npai	rametric	non	param	etric	Use nonparametric		
20	Statistical value based on	Z-8	tatis	tíc.	Z-	statist	ic.	z-statistic.		
21	N	4			4			4		
22	% < Detection limit	0%			100%			25%		
23	mean	0.847			0.122			0.509 ·		
24	st. dev.	0.238			0.129			0.163		
25	Z-statistic	1.645			1.645			1.645		
26	95% UCL on mean	1.04			0.228			0.643		
27	max value	1.19			0.314	U		0.734		
28	Statistical value	1.04			0.228	U		0.643		
29	Background	1.1			0.11			1.1		
30	Statistical value above background	0 (< BG)			0.118	U		0 (< BG)		

32 HEIS = Hanford Environmental Information System

33 MDA = minimum detectable activity

RAG = remedial action goal

etectable activity

U = undetected

34 NA = not applicable

UCL = upper confidence limit

35 Q = qualifier

WAC = Washington Administrative Code

CALCULATION SHEET

Washington Closure Hanford

Originator B. S. Wiegman BSO	Date 05/09/06	Calc. No. 0600X-CA-V0058	Rev. No. 0
Project 300 Area Field Remediation	Job No. 14655	Checked T. M. Blakley Jm.B	Date 05/09/06
Subject 618-8 Burial Ground Cleanup Verification 95% UCL Calculations			Sheet No. 5 of 5

Split-Duplicate Analysis

1 Shallow Zone Sample Results:

2	Sampling			Silve	r	A	Arsenic		Barium			Cadmium			Chromium			Lead		
3	Area	HEIS NUMBER	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL	mg/kg	Q	PQL
4	A1	J11271	0.15	U	0.15	3.4		0.36	81.3		0.02	0.07	U	0.07	10.4		0.17	4.5		0.33
5	Duplicate of J11271	J11272	0.15	U	0.15	4.1		0.36	79.4		0.02	0.07	U	0.07	11.7		0.17	4.2		0.33
6	Split of J11271	J11277	1.1	U	1.1	3.5		1.1	94.9		21.3	0.53	U	0.53	8.3		1.1	3.9		1.1
	Challow Zone Analys	lo.		المشمط																

- 1	Shahow Zone Analysis;										
8	TDL		0.2	10	2	0.2	1	5			
9		Both > MDA?	No-Stop (acceptable)	Yes (continue)	Yes (continue)	No-Stop (acceptable)	Yes (continue)	Yes (continue)			
10	Duplicate Analysis	Both > 5xTDL?		No-Stop (acceptable)	Yes (calc RPD)		Yes (calc RPD)	No-Stop (acceptable)			
11		RPD		······	2.4%		12%				
12		Both > MDA?	No-Stop (acceptable)	Yes (continue)	Yes (continue)	No-Stop (acceptable)	Yes (continue)	Yes (continue)			
13	Split Analysis	Both > 5xTDL?	I I I	No-Stop (acceptable)	Yes (calc RPD)		Yes (calc RPD)	No-Stop (acceptable)			
14		RPD			15%		22%				
15											

16 Shallow Zone Sample Results (continued)

17	Sampling		Se	leni	um	Tota	Ura	nium	Urani	um-23	33/234	Ura	nium	-235	Ura	nium	-238
18	Area	HEIS Number	mg/kg	Q	PQL	mg/kg	Q	PQL	pCi/g	Q	MDA	pCi/g	Q	MDA	pCi/g	Q	MDA
19	A1	J11271	0.81		0.38	1.28		0.017	0.540		0.270	0.000	U	0.310	0.439		0.260
20	Duplicate of J11271	J11272	0.76		0.38	1.13		0.017	0.736		0.260	0.081	U	0.310	0.401		0.260
21	Split of J11271	J11277	0.33	В	1.6	3.28		0.020	1.14		0.053	0.078		0.030	1.10		0.053

22 Shallow Zone Analysis

23	TDL		1	1.0	1.0	0.5	1.0
24		Both > MDA?	Yes (continue)	Yes (continue)	Yes (continue)	No-Stop (acceptable)	Yes (continue)
25	Duplicate Analysis	Both > 5xTDL?	No-Stop (acceptable)	No-Stop (acceptable)	No-Stop (acceptable)		No-Stop (acceptable)
26		RPD					
27		Both > MDA?	No-Stop (acceptable)	Yes (continue)	Yes (continue)	No-Stop (acceptable)	Yes (continue)
28	Split Analysis	Both > 5xTDL?		No-Stop (acceptable)	No-Stop (acceptable)		No-Stop (acceptable)
29	. ,	RPD		······································			

30 Note: The significance of the reported RPD values, including values greater than 30%, is addressed within the Data Quality Assessment section of the Cleanup Verification Package for this site.

31 B = analyte found in method blank

32 HEIS = Hanford Environmental Information System

33 MDA = minimum detectable activity

34 PQL = practical quantitation limit

35 Q = qualifier

36 RPD = relative percent difference

37 TDL = target detection limit

38 U = undetected

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