Scaled Experimental Modeling of VHTR Plenum Flows

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Abstract

The Very High Temperature Reactor (VHTR) is the leading candidate for the Next Generation Nuclear Power (NGNP) Project in the U.S. which has the goal of demonstrating the production of emissions free electricity and hydrogen by 2015. Various scaled heated and unheated gas and water flow facilities were investigated for modeling VHTR upper and lower plenum flows during the decay heat portion of a pressurized conduction-cooldown scenario and for modeling thermal mixing and stratification ("thermal striping") in the lower plenum during normal operation. It was concluded, based on phenomena scaling and instrumentation and other practical considerations, that a heated water flow scale model facility is preferable to a heated gas flow facility and to unheated facilities which use fluids with ranges of density to simulate the density effect of heating. For a heated water flow lower plenum model, both the Richardson numbers and Reynolds numbers may be approximately matched for conduction-cooldown natural circulation conditions. Thermal mixing during normal operation may be simulated but at lower, but still fully turbulent, Reynolds numbers than in the prototype. Natural circulation flows in the upper plenum may also be simulated in a separate heated water flow facility that uses the same plumbing as the lower plenum model. However, Reynolds number scaling distortions will occur at matching Richardson numbers due primarily to the necessity of using a reduced number of channels connected to the plenum than in the prototype (which has approximately 11,000 core channels connected to the upper plenum) in an otherwise geometrically scaled model.

Experiments conducted in either or both facilities will meet the objectives of providing benchmark data for the validation of codes proposed for NGNP designs and safety studies, as well as providing a better understanding of the complex flow phenomena in the plenums.

Introduction

The Very High Temperature Reactor (VHTR) is the leading candidate for the Next Generation Nuclear Power (NGNP) Project in the U.S., according to MacDonald et al., 2003. The NGNP project has the goal of demonstrating the production of emissions free electricity and hydrogen by 2015. The NGNP reactor core could be either a prismatic graphite block type core or a pebble bed core. The use of a molten-salt coolant is also being evaluated. One of the great challenges of studying, designing, and licensing the NGNP is to verify that the intended NGNP computer code analysis tools can be used with confidence to make decisions and to assure all that the reactor systems are safe and meet
their performance objectives. Comparison of computer code calculations with experimental data is the key component of code verification. This paper addresses experimental modeling of lower and upper plenum flow and thermal mixing phenomena of importance during normal operation and during a loss of forced reactor cooling (pressurized conduction cooldown) scenario. A prismatic core gas-cooled reactor following the Gas Turbine-Modular Helium Reactor (GT-MHR) design from General Atomics [GA 1996] is assumed in this study. VHTR vessel thermal-hydraulic phenomena that are of importance during normal, reduced power, and accident operation were identified by McEligot and McCreery, 2004, and are indicated in Figure 1. The objectives of the experiments are, 1), provide benchmark data for assessment and improvement of codes proposed for NGNP designs and safety studies, and, 2), obtain a better understanding of related phenomena, behavior and needs.

Various scaled heated gas and water flow facilities were investigated for modeling VHTR upper and lower plenum flows during the decay heat portion of a pressurized conduction-cooldown scenario and for modeling thermal mixing and stratification (“thermal striping”) in the lower plenum during normal operation. The choice of facilities depends not only on scaling fidelity but on the practical considerations of instrumentation, flow visualization, power requirements, environmental and safety requirements, and cost.

A number of previously conducted analyses and experiments that employed a variety of methods are relevant to the present study. Thermal fluctuations in the lower plenum of a high-temperature gas-cooled reactor of the French CEA design were investigated analytically by Tauveron, 2002. The calculations illustrate the complexity of the flow and the thermal loading imposed on internal structures. Scaled experiments were conducted at the INL which investigated off-normal and accident conditions in the upper plenum of a Savannah River Site nuclear reactor (McCreery et al., 1991). The experiments illustrate the complexity of flow and mixing in a plenum containing a large number of cylindrical components (as does the VHTR lower plenum design). Many studies of cross-flow in tube bundles, single and multiple jets mixing in confined spaces are reported in the literature and the references were compiled by King, 2004. Experiments which modeled natural circulation during PWR severe accidents were conducted at Westinghouse Electric Corporation (Westinghouse Electric Corporation, 1990) using sulfur hexafluoride (SF6) in a 1/7 geometrically scaled model facility. Flow in the reactor vessel connected to two loops containing model steam generators were simulated in the electrically heated facility.

Several previously conducted experiments investigated hot-streaking in specific gas-cooled reactor core outlets which, although the lower plenum geometries are different from the GA design, have some relevance to the present study. Experiments were conducted to characterize thermal mixing and hot streaking in the lower plenum (core bottom structure, CBS) of the gas-cooled high temperature engineering test reactor (HTTR) developed by the Japan Atomic Energy Institute (JAERI). Initial experiments (Inagaki, et al., 1990) were carried out using a one-seventh scale model of the CBS including a plenum and outlet hot gas duct with water as the test fluid. Hot and cold water were injected into the model and the temperature distributions of the mixed water
were measured. Follow-on experiments were conducted using a full-scale model of the vessel (Inagaki, et al., 1991), including the CBS, in the heated helium engineering demonstration test loop (HENDL). It was determined that, with the inclusion of a mixing promoter, mixing was sufficient to prevent significant thermal striping (greater than a 15°C variation according to their definition). Core outlet temperature mixing in the outlet plenum (hot gas header) of the helium cooled modular high temperature reactor (HTR) developed by Interatom and Siemens was investigated in a 1:2.9 scaled plastic model using heated air flow (Damm and Wehrlein, 1990). Colder gas leakages into the plenum were also simulated in the model. The favorably high mixing rate allowed the plenum design to be simplified by changing a complex network of mixing channels into simple straight channels and by reducing the volume of the plenum. Thermal mixing in the lower plenum (hot gas chamber) of the high temperature gas-cooled pebble-bed reactor test module (HTR-10) located at Tsinghua University, China, was investigated in a 1:1.5 scale model using heated air flow (Yao, et al., 2002). Gas mixing takes place in cavities between eight supporting ribs and then flows out radially into a circumferential channel and then into a horizontal outlet pipe. Four different flow mixing arrangements were investigated. All arrangements except an empty plenum provided acceptable mixing. Although these reactors are much smaller than the proposed VHTR and none of the lower plenum geometries investigated have much similarity to the VHTR lower plenum, the experimental methods and results help give confidence to the experimental modeling methods proposed in this document.

Normal operation at full or partial loads
- Mixing of hot jets in the reactor core lower plenum ("hot streaking")
- Coolant flow and temperature distributions through reactor core channels ("hot channel")

Loss of forced reactor core cooling (LOFA or "pressurized cooldown")
- Mixing of hot plumes in the reactor core upper plenum
- Coolant flow and temperature distributions through reactor core channels (natural circulation, "hot channel")
- Rejection of heat by natural convection and thermal radiation at the vessel outer surface

Loss of forced reactor core cooling and loss of coolant inventory (LOCA or "depressurized cooldown")
- Prediction of reactor core depressurized cooldown - conduction and thermal radiation
- Rejection of heat by natural convection and thermal radiation at the vessel outer surface

Figure 1. Important VHTR thermal-hydraulic phenomena.
Model Conceptual Design and Scaling Approach

Code predictions of VHTR core flow during a pressurized-conduction-cooldown scenario (Bayless, 2006) indicate that channel-to-channel flow is important with upflow occurring in the more central channels and downflow occurring in the peripheral channels. An experimental apparatus capable of simulating three-dimensional (laminar) natural circulation flows in the upper and lower plenums can either generate the channel flows by heating the fluid in simulated core channels or provide the simulated channel flows from an external source; blowers in the case of gas flow or pumps in the case of water flow. In addition, the apparatus design must be capable of delivering higher turbulent flow to the lower plenum to simulate thermal mixing during normal operation forced circulation in the lower plenum (the “thermal striping” or “hot streaking” problem). A suitable apparatus needs to be geometrically scaled to the prototype so that velocity ratios, an important scaling criterion for mixing, are preserved. Employing the one-half symmetry of the prototype lower plenum permits improved instrumentation and visibility and reduced total flow rate (by ½) as compared with a full cylindrical model. A one-half symmetric lower plenum apparatus is shown schematically in Figure 2. The upper plenum may be scaled to either complete, one-half, or one-quarter of the prototype. A one-quarter symmetric upper plenum model is shown in Figure 3. The one-quarter symmetric upper plenum model is preferred because of simplicity and because for a water-filled model a one-quarter symmetric upper plenum (as shown schematically in Figure 3) permits laser light sheets to illuminate the plenum for side-views without the added distortions of a water-filled rectangular box surrounding the plenum (top views of the plenum will still require a water filled box). A water-filled rectangular box is required to eliminate distortion caused by non index-matching surfaces (water, plastic, and air) by providing plane-surface windows.

The several models considered in this study are all geometrically (linearly) scaled to the prototype except for the core channels, which are too small, 7.9 mm in diameter, and too numerous, on the order of 11,000, to geometrically scale. The flows modeled are assumed to be quasi steady-state which computer code calculations indicate is a reasonable assumption for the normal power and decay heat conditions considered. Fast transients, such as LOCA’s, are not considered. All models are assumed to have the same geometric scaling ratio, S, of 1/6.55, which is the same scaling ratio as employed for the INL isothermal Matched-Index of Refraction (MIR) lower plenum model (McElroy et al., 2006). The model employs a reduced number of jets (4) and a reduced number of posts (5 plus 10 half-posts at walls) in a geometrically scaled facility to study lower plenum flow. The scaling ratio was chosen for practical reasons including the availability of materials, optical access, and pumping requirements, as well as the ease of implementing experimental methods developed for the MIR experiments and comparing the present experimental results with MIR experimental data.
Several circulation methods and fluids were investigated for implementation in the model. Results of scaling calculations and practical considerations are summarized in Table 1. The method most conceptually similar to the prototype would be to employ electrically heated tubes with atmospheric pressure nitrogen or another gas, such as sulfur hexafluoride, to induce natural convection in the core. An external heat exchanger would be necessary to remove heat generated in the simulated core. An external blower would be necessary to drive forced convection flow. This concept is shown in Figure 4. Rather than using heated gas flow, buoyancy forces might be simulated by injecting a heavier gas, such as argon or sulfur hexafluoride, into channels which represent lower heat transfer channels in the prototype and a lighter gas into channels which represent higher heat transfer. The gas mixture flowing through the upper plenum would be removed as it enters the outer, simulated low heat transfer, channels and replaced with the heavier gas. Of the two gas flow methods, the scaling consideration of providing closer Reynolds numbers at matching Richardson numbers to prototypical values and construction and operation and other practicality considerations favor the heavy-gas injection method. However, the disposal of or recapturing and separating large volumes of a heavy gas is a major difficulty with the method.

Rather than using gas flow, water flow may be used to simulate forced and natural circulation flow in the prototype. Buoyancy forces within the plenums can be simulated by either heating the water or by adding a dissolved substance, such as salt, to increase the water density in channels representing lower heat transfer channels in the prototype. However, concerns about the disposal of large quantities of salt water make this latter approach impractical. The heated water flow method provides the closest match of Reynolds numbers at prototypical Richardson numbers, especially for lower plenum flow. Comparing gas flow and water flow methods; the venting of large quantities of gas and increased measurement difficulties compared to water flow, favor the heated water flow method. Heated water flow is therefore the preferred choice. Although natural circulation is to be simulated, the power requirements of heating water in a simulated core to drive the flow are excessive. Therefore, flow will be delivered by pumps from two reservoirs which contain heated water in one reservoir and unheated water in the other (Figure 5). Reservoir sizes will be chosen to provide sufficient time to collect data once flow and temperatures are at steady-state operating conditions. Approximately five minutes is sufficient time (including start-up) to obtain PIV and thermocouple measurements. Reservoir sizes will therefore need to be approximately 1,000 gallons in order to sustain the 200 GPM needed for maximum flow (Table 1). Flow rates and temperature boundary conditions will be provided by ATHENA/RELAP5-3D calculations. The 134 channels are subdivided into nine heat transfer regions which correspond to the nine core regions used in the VHTR ATHENA/RELAP5-3D model of Bayless, 2006 (Figure 6).

**Scaling Relationships and Distortions**

The general approach to scaling experiments that simulate natural circulation in the prototype reactor plenums in the water-flow facility is to match Richardson number, the
ratio of buoyant to inertial forces, and, if possible, Reynolds number, the ratio of inertial to viscous forces. The approach to scaling experiments that simulate turbulent forced circulation in the lower plenum during normal operation is to insure that flow is fully turbulent in each component. This is insured if Reynolds number for flow in the smallest nozzle entering the lower plenum is greater than approximately 4,000. Reynolds numbers will necessarily be lower than in the prototype. For fully turbulent flow, buoyant forces will be much lower than inertial forces and Richardson number scaling may therefore be ignored. Instead, a small but easily measurable temperature range (e.g. 10 °C) will be employed to quantify mixing.

The primary forces involved in scaling convective flows involving a temperature gradient are inertia, buoyancy, and viscous dissipation (Turner, 1973). Temperature variations within a convective flow give rise to variations in properties of the fluid. The mass, momentum and energy equations describing the flow are commonly used in a form known as the Boussinesq approximation, where variations of fluid properties other than density as it gives rise to buoyancy force are ignored. The Boussinesq approximation for density change is,

\[ \rho = \rho_0 + \Delta \rho \]

With this approximation, the Navier-Stokes equation becomes (Tritton, 1977),

\[
\frac{D\vec{V}}{Dt} = -\frac{\nabla P}{\rho} + \nu^2 \vec{V} + \frac{\Delta \rho}{\rho} \bar{g}
\]

Where, \( \vec{V} \) is (vector) velocity, \( t \) is time, \( P \) is pressure, \( \nu \) is kinematic viscosity, and \( \bar{g} \) is the gravitational acceleration.

For a density variation due to temperature, the dependence of \( \rho \) on \( T \) may be expressed as,

\[ \Delta \rho = -\alpha \rho_0 \Delta T \]

Where, \( \alpha \) is the thermal expansion coefficient (= 1/T for a perfect gas).

The ratio of buoyant to inertia forces is defined as the Richardson number, which may be expressed in terms of either a density difference or a temperature difference, and a relevant length, \( D \), as,

\[ Ri = \frac{g(\rho_0 - \rho)D}{\nu^2} = \frac{g \alpha \Delta T D}{\nu^2} = \frac{g(\Delta T / T)D}{\nu^2} \]

The ratio of Richardson numbers of the model (m) and prototype (p) is then,
\[
\frac{R_i_m}{R_i_p} = \frac{\Delta \rho}{\rho} \left( \frac{V_m^2 D_m}{V_p^2 D_p} \right)
\]

The Reynolds number, the ratio of inertial to viscous forces, is,

\[
Re = \frac{VD}{\nu}
\]

If, for the natural circulation experiments, Richardson number is matched in the model and prototype, then the Reynolds number ratio for model and prototype may be expressed as,

\[
\frac{Re_m}{Re_p} = \left( \frac{\Delta \rho}{\rho} \right)^{1/2} \left( \frac{D_m}{D_p} \right)^{3/2} \frac{V_p}{V_m}
\]

Calculations using the above equations indicate that, for gas flow, model Reynolds number will be a fraction of that of the prototype for realistically achievable temperatures and density ratios in a model facility operated at close to atmospheric pressure (see Table 1). However, for water flow, calculations indicate that both Richardson and Reynolds numbers of the lower plenum model may be approximately matched to the prototype for reasonable cold and hot water temperatures (approximately 20 C cold and 46 C maximum hot respectively, as shown in Figure 7). This matching is possible in a scale model facility primarily because of the considerably higher kinematic viscosity of helium compared with water. It is more important to match Reynolds number in laminar flow natural circulation experiments than in turbulent flow experiments, since quantities such as fluid mixing, drag coefficients and eddy dimensions are a much stronger function of Reynolds number in laminar flow.

The above equations apply to both the upper and lower plenums. However, the Reynolds numbers for the channels connecting to the upper plenum will be distorted because the large number of channels in the prototype (approximately 11,000 total, or 2,750 connecting to one-quarter of the plenum) will necessarily be modeled using fewer channels in the model. Reducing the number of channels and locations to the corresponding distribution in the lower plenum (67 channels for a ¼ representation) will permit the use of the same plumbing system for either experiment. The connection of core channels to lower plenum nozzles (and therefore the correspondence of core channels in the prototype with the choice of channels connected to the upper plenum in the model) is shown in figure 8. Because the temperature differences entering the upper
plenum are greater than those entering the lower plenum (Figure 7), and because the maximum temperature difference between channels in the model is limited to approximately 60 °C or less, Reynolds numbers in the model upper plenum will be further distorted at matching Richardson numbers.

If Richardson numbers based on channel-to-channel temperature differences and on plenum diameter and internal components other than the channels are matched, and if the sum total of channel flow areas is scaled as $1/S^2$, which insures that velocity ratios of the channels and upper plenum components are maintained, then channel Reynolds number will be high by approximately a factor of three (the flow will still be laminar) and Reynolds numbers for internal components will be low by approximately a factor of three. These results assume that the maximum water temperature is limited to 46 °C, the same as for the lower plenum model.

Another variation on the scaling approach for the upper plenum is to match Richardson numbers based on channel dimensions and temperatures, which will then cause distortions in $R_i$ and $R_e$ for other components. A series of experiments will need to be conducted over ranges of Richardson and Reynolds numbers to characterize these distortions.

**Other Scaling Considerations**

Although the majority of phenomena of interest are properly scaled with Richardson and Reynolds numbers, several effects, which include wall heat transfer, jet inlet conditions, jet entrainment and eddy shedding require further consideration. These include the following:

The heat transfer effects of hot or cold surfaces are not modeled. However, calculations by Bayless, 2004, indicate that the plenum walls during a conduction-cooldown scenario may be considered to be adiabatic. Since internal components have less mass than the walls, they may also be considered adiabatic.

Jet flow, as it enters the lower plenum, is influenced by the jet boundary conditions. Turbulent jet flow downstream of the entry is strongly influenced by entry geometry, according to Nobes and Nathan, 2001. Entry nozzle geometry and upstream development length/diameter therefore need to be similar in the model and prototype. For natural circulation conditions, the jets need to be well-developed (with a parabolic velocity profile).

Entrainment of ambient fluid into free jets issuing from nozzles was investigated by McCreery and Condie, 2006. It was concluded that jet entrainment and jet-to-jet interaction in a geometrically scaled facility are scaled properly and should be approximately independent of Reynolds number. However, since the channels entering
the upper plenum are not geometrically scaled to the prototype, there will be distortions in entrainment and jet interactions. Because of this, the jets will merge at greater scaled distance from the channel entrances to the upper plenum than in the prototype. The distance for jets to merge will be a small fraction of the upper plenum radius in both prototype and model and so, perhaps, this will not be a significant distortion (this distortion will need to be further investigated both experimentally and analytically).

Scaling of eddy shedding frequency for cross-flow past posts may be important if either the prototype or model exhibit regular shedding frequencies. The prototype appears to have flow during normal operation over much of the lower plenum in a Reynolds number (based on post diameter) range where eddy shedding for uniform flow across a cylinder is irregular \( (3 \times 10^5 < \text{Re} < 3 \times 10^6) \) (Huang and Lin, 2000). Reynolds number, based on post diameter, in the prototype ranges from approximately \( 5 \times 10^4 \) to \( 10^6 \). Eddy-shedding frequency \((f)\) is characterized by non-dimensional Strouhal number \( \text{St} = f D/V \). Frequency, in general, is not matched in a scaled system when Reynolds number is matched (since \( \text{Re} \) depends on the product rather than the division of \( D \) and \( V \)). Flow visualization in the MIR experiments (using air bubbles) shows that no visually apparent frequency is associated with eddy shedding. This may be due to the wide range of (vector) velocities impinging on each of the cylinders. However, detailed velocity measurements and analyses of the MIR experiments may show a peaked frequency spectrum that is not apparent visually.
Table 1. Comparison of working fluids for VHTR scale models. VHTR natural circulation decay-heat conditions based on RELAP5/ATHENA calculations at 144,000 s after scram (Bayless, 2006).

<table>
<thead>
<tr>
<th>Model</th>
<th>Re$_{m}/$Re$_p$ for lower plenum N.C. if $Ri_m = Ri_p$</th>
<th>Flow visualization, PIV and PLIF</th>
<th>Max. flow rate (Re$_{L.P.}$ noz. = 7,000)</th>
<th>Significant problems</th>
</tr>
</thead>
<tbody>
<tr>
<td>Heavy-gas Injection (Ar)</td>
<td>0.16</td>
<td>Good in U.P. Mediocre in L.P.</td>
<td>315 CFM</td>
<td>Vented of Argon gas</td>
</tr>
<tr>
<td>Heavy-gas Injection (SF6)</td>
<td>0.25</td>
<td>Good in U.P. Mediocre in L.P.</td>
<td>51.3 CFM</td>
<td>Vented or separating SF6 from N2</td>
</tr>
<tr>
<td>Heated gas$^1$ (air)</td>
<td>0.079</td>
<td>Good in U.P. Mediocre in L.P.</td>
<td>315 CFM</td>
<td>Thermal stress</td>
</tr>
<tr>
<td></td>
<td></td>
<td></td>
<td></td>
<td>Low Re</td>
</tr>
<tr>
<td></td>
<td></td>
<td></td>
<td></td>
<td>Non-transparent components</td>
</tr>
<tr>
<td>Heated water$^2$</td>
<td>1.0</td>
<td>Good in both L.P. and U.P.</td>
<td>200 GPM</td>
<td>Index-of-refraction variations</td>
</tr>
<tr>
<td>Water plus dissolved salt$^3$</td>
<td>0.76</td>
<td>Good in both L.P. and U.P.</td>
<td>200 GPM</td>
<td>Disposal of large volumes of salt water</td>
</tr>
</tbody>
</table>

1. Assumes 20 °C cold gas and 80 °C hot gas.
2. Assumes 20 °C cold water and 46 °C hot water for simulated natural circulation flow and 20 °C and 30 °C for turbulent flow in lower plenum.
3. Assumes $(\Delta \rho / \rho)_{model} = (\Delta \rho / \rho)_{prototype}$
Reynolds Number Independence

The modeling methods workshop conducted in conjunction with the ICONE conference in Miami, July, 2006, brought up the issue of scaling very high Reynolds number flows in the point design VHTR in a small-scale facility which will have, as a practical necessity, flows with lower Reynolds numbers. Although Reynolds numbers for natural circulation can be matched in the plenum experiments, forced circulation Reynolds numbers will be considerably lower. For example, for full-power operation the lower plenum nozzle Reynolds number will be approximately 200,000, while the maximum achievable model Reynolds number will be about 7,000 to 8,000.

The problem of modeling very high Reynolds number (turbulent and subsonic) flow in a small-scale facility at lower, but still turbulent, Reynolds number is a commonly encountered on in wind tunnel and other flow tunnel experiments. The usual argument is that if the flow is fully turbulent and if the phenomena in question vary only slowly with Reynolds number, and asymptotically approach an infinite Reynolds number value (the flow field of which, for incompressible flow, corresponds to potential flow outside of recirculation regions), then the experimental results are representative of the higher Reynolds number flow. Common examples include the velocity profile and wall friction factor for internal flow and the flow field and coefficients of drag and lift for flow over airplane wings and other components. Examples of comparisons of complex mixing flows in lower Reynolds number experiments with full-scale prototypes that are relevant for our plenum experiments include MacDonald et al., 1998 and Snyder, 1981. The guidelines that these two authors and others provide is that for flow over objects, such as the lower plenum posts, Reynolds numbers must be over a threshold value of 4,000 in order to provide large-scale flow similarity and Reynolds number independence. If nozzle Reynolds number is a minimum of 4,000 then jets entering the lower plenum will have a fully-turbulent profile, the jets within the plenum will be fully turbulent and flow across posts and in areas of the plenum away from posts will be fully turbulent and large and small eddy flow structures will be scaled (except when there is regular eddy shedding, as discussed above in “other scaling considerations”). Reynolds number independence can be tested in experiments by varying the flow through the maximum achievable range for turbulent flow and comparing any differences, for example by varying lower-plenum Reynolds numbers for the larger nozzles from 3,000 to 7,000.

Instrumentation

The primary instrumentation for the experiments will consist of flow meters, thermocouples, pressure transducers, Particle Image Velocimeter (PIV), and Planar Laser Induced Fluorescence (PLIF). Flow meters will be employed to measure overall flow rate plus individual flow rates delivered to the nine manifolds that feed the individual tubes. Flows delivered to the individual tubes will be calibrated (and adjusted) as functions of manifold flow rates.
The primary measurements of velocities and velocity fluctuations will be performed using a LaVision 3D PIV that is presently being employed for the MIR experiments (McIlroy et al., 2006). The PIV system employs a laser light sheet for illumination. The vector velocity field is measured in the plane of the light sheet plus the velocity component transverse to the light sheet. The light sheet may be positioned between rows of tubes or along the center-lines of tubes, as shown in Figure 9, or perpendicular to and intersecting the tubes. Because the indices-of-refraction for water and the (plastic or glass) tube material are not exactly matched, PIV may be restricted to the two-dimensions of the laser light sheet when the light sheet is parallel to the tubes. The restriction to two dimensions is due to the requirement of two cameras for the stereoscopic view required for 3D PIV and because a clear view is probably limited to one direction for this geometry. When the light sheet is perpendicular to the tube axes and viewed from above or below the plenums, 3D PIV may be possible. The method of viewing between rows of tubes is to employ a cylindrical concave mirror as shown in Figure 9. Parallax divergence of view of the tube rows may be eliminated by placing the camera at the infinity focal position of the mirror (defined as a “telecentric” optical set-up). This optical method was used for flow visualization in water-flow experiments that simulated core flow and dye injection in a reactor core bundle (McCreery et al., 1990). The method was successfully employed to visualize flow and map dye concentration in the majority of the bundle cross-section, such as in the view shown in Figure 10.

Temperature distribution may be measured by the conventional method of using thermocouples, including using thermocouple rakes and thermocouples attached to traversing rods, and by Planar Laser Induced Fluorescence (PLIF) (Nash et al., 1995). PLIF is an extension of PIV in that it uses the same laser and video recording devices. For PLIF, tracer particles will be used that fluoresce with an intensity that is a function of temperature. Two-dimensional temperature fields may thereby be measured and mapped.

**Concluding Remarks**

Various scaled heated gas and water flow facilities were investigated for modeling VHTR upper and lower plenum flows during the decay heat portion of a pressurized conduction-cooldown scenario and for modeling thermal mixing and stratification (“thermal striping”) in the lower plenum during normal operation. It was concluded, based on phenomena scaling and choice of instrumentation and other practical considerations, that a heated water flow scale model facility is preferable to a heated gas flow facility and to unheated facilities which use fluids with ranges of density to simulate the density effect of heating. For a heated water flow lower plenum model, both the Richardson numbers and Reynolds numbers may be approximately matched for conduction-cooldown natural circulation conditions. Thermal mixing during normal operation may be simulated but at lower, but still fully turbulent, Reynolds numbers than in the prototype. Natural circulation flows in the upper plenum may also be simulated in a separate heated water flow facility that uses the same plumbing as the lower plenum model. However, Reynolds number scaling distortions will occur at matching Richardson numbers due primarily to the necessity of using a reduced number of channels connected to the
plenum than in the prototype (which has approximately 11,000 core channels connected to the upper plenum) in an otherwise geometrically scaled model. These scaling distortions may be characterized by running series of experiments with varying flow rates and temperatures (thereby varying Reynolds and Richardson numbers). Experiments conducted in either or both facilities will meet the objectives of providing benchmark data for the assessment and improvement of codes proposed for NGNP designs and safety studies, as well as providing better understanding of the complex flow phenomena in the plenums.
Figure 2. Isometric schematic of lower plenum model and view showing construction details including posts, outlet nozzle, top and bottom sheets, windows, and inlet tubes.
Figure 3. Upper plenum prototype and schematic of model shown without water-filled rectangular box enclosure or internal components.
Figure 4. Possible geometrically scaled one-half symmetric model of VHTR vessel employing gas flow and an electrically-heated core (this is not the preferred apparatus).
Figure 5. Plumbing schematic for water-flow apparatus. Water flow rates and temperatures delivered to each of the nine manifolds are controlled by metering flows from the hot and cold water storage tanks. Individual lines (with flow-control valves) connect the manifolds to the nozzles.
Figure 6. Nine region flow map of lower plenum and corresponding RELAP5/ATHENA model components. Corresponding cross-sections of the prototype core and the water-flow model are shown.
Plenum entrance (core exit) temperatures (K) in prototype at 144,000 s following scram for conduction-cooldown.

Upper plenum

Lower plenum

Scaled model plenum entrance temperatures (C).

Figure 7. Plenum entrance temperatures in prototype predicted by RELAP5/ATHENA calculations and corresponding scaled model water temperatures (°C).
Figure 8. Prototype geometry of core channels merging into a lower plenum nozzle.
Figure 9. PIV and PLIF telecentric optical instrumentation set-up for lower plenum.
Figure 10. Dye injection in 1/6 sector model core with water flow. The dye plume is mostly visible between and through water-filled tubes.
References:


