



INEEL/CON-02-00671  
PREPRINT

## Construction Of The WSU Epithermal-Neutron Filter

J. R. Venhuizen  
D. W. Nigg  
G. E. Tripard

September 9, 2002

10<sup>th</sup> International Congress On Neutron Capture Therapy

*This is a preprint of a paper intended for publication in a journal or proceedings. Since changes may be made before publication, this preprint should not be cited or reproduced without permission of the author.*

*This document was prepared as a account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, or any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for any third party's use, or the results of such use, of any information, apparatus, product or process disclosed in this report, or represents that its use by such third party would not infringe privately owned rights. The views expressed in this paper are not necessarily those of the U.S. Government or the sponsoring agency.*

# Construction of the WSU Epithermal-Neutron Filter

J. R. Venhuizen<sup>1</sup>, D. W. Nigg<sup>1</sup>, and G. E. Tripard<sup>2</sup>

<sup>1</sup>Idaho National Engineering and Environmental Laboratory, Idaho Falls, Idaho, USA

<sup>2</sup>Washington State University, Pullman, Washington, USA

**SUMMARY** Moderating material has been installed in the original thermal-neutron filter region of the Washington State University (WSU) TRIGA™ type reactor to produce an epithermal-neutron beam.

**INTRODUCTION** Attention has been focused upon the development of a convenient, local, epithermal-neutron beam facility at WSU for collaborative Idaho National Engineering and Environmental Laboratory (INEEL)/WSU boron neutron capture therapy (BNCT) preclinical research and boronated pharmaceutical screening in cell and animal models. The design of the new facility was performed in a collaborative effort<sup>1,2</sup> of WSU and INEEL scientists. This paper summarizes the physical assembly of this filter.

## **FACILITY DESCRIPTION**

In this design, the original graphite has been removed from the thermal column region and replaced with an epithermal-neutron filtering, moderating, and collimating assembly, as shown in Figure 1. The spectrum is tailored in this region such that most neutrons emerge with energies in the epithermal energy range (0.5 eV – 10 keV). Downstream of the filtering and moderating region is a cadmium thermal-neutron absorber, a bismuth and lead gamma shield, and a conical neutron collimator composed of bismuth surrounded by borated polyethylene. A lead gamma shield and lithiated polyethylene thermal-neutron absorber complete the assembly. Provision is made for several different exit port aperture sizes, as shown in Figure 1. A key distinguishing feature of the WSU facility is the incorporation of a high-efficiency, neutron moderating and filtering material, FLUENTAL™, developed by the Technical Research Centre of Finland<sup>3</sup>. FLUENTAL is manufactured by hot isostatic pressing of a powdered mixture of 69% (by weight) aluminum fluoride, 30% aluminum, and 1% lithium fluoride. A block of this material, having a thickness in the beam propagation direction of 0.64 m and transverse dimensions of approximately 0.6 m, is surrounded by aluminum oxide to produce the neutron filtering and moderating region. MCNP<sup>4</sup> and DORT<sup>5</sup> radiation transport design calculations for the coupled core and filter-collimator assembly indicate that an epithermal-neutron flux of approximately  $10^9$  n/cm<sup>2</sup>-s at a reactor power of 1 MW will be

produced at the exit port of the collimator (with the reactor core optimally loaded). The background neutron KERMA (a measure of the fast-neutron contamination) for the beam is calculated to be in the range of approximately  $2.0 \times 10^{-11}$  to  $4.0 \times 10^{-11}$  cGy/n-cm<sup>2</sup>. The computational methods used for this design were previously validated against INEEL measurements performed for a similar neutron beam facility that is in operation at the FiR 1 TRIGA research reactor in Finland<sup>6</sup>. An additional key feature of the WSU beam facility design is the provision for adjustable filter-moderator thickness (in 4 cm steps) to systematically explore the radiobiological consequences of increasing the fast-neutron contamination above the nominal value associated with the baseline system described above.

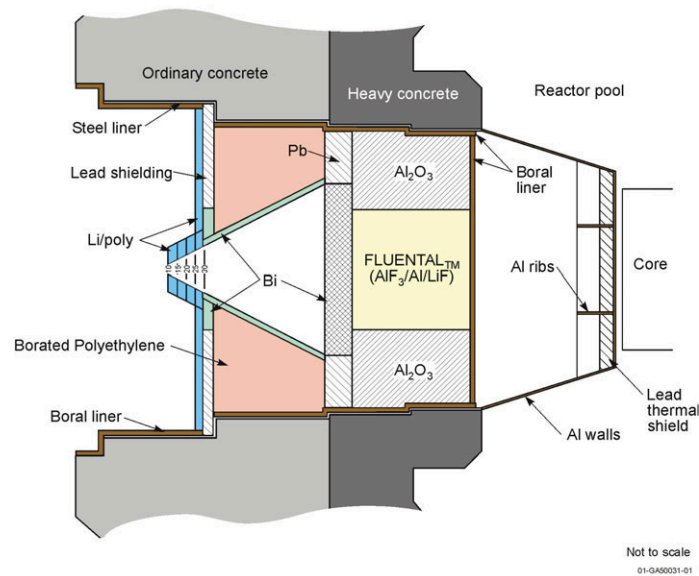


Figure 1. WSU column assembly, with epithermal-neutron filter in place.

## CONSTRUCTION

### Thermocouples and Nitrogen Vent Line

Prior to installation of the filter-moderator components, thermocouples were placed at selected locations on the lead primary gamma shielding at the far end of the thermal column cone housing. Five thermocouple leads were attached to the lead thermal shield; the sixth lead is attached to the center of the boron liner facing the cone. Also included with the thermocouple leads was a 0.635 cm diameter nylon tube that vents into the bottom of the cone area to supply a nitrogen-purge in the cone area.

### **Aluminum Oxide and FLUENTAL Assembly**

The primary filter moderator-filter assembly was built up from blocks of aluminum oxide and FLUENTAL. A sheet of boral prevents the blocks from falling into the cone area. The FLUENTAL material consists of 17 blocks of nominal dimensions 63.5 x 22 x 8 cm, 8 blocks of 63.5 x 11 x 8 cm, and 6 blocks of 63.5 x 22 x 4 cm. This allows stacking the blocks in a cube arrangement of 8 rows measuring 63.5 x 64 x 66 cm when complete. The 4 cm thick blocks make it possible to shorten the filter in 4 cm increments if desired. The alumina blocks were produced in 8 x 8 x 16 cm size by Coors Ceramics (Golden, CO., USA) to mate with the FLUENTAL. The blocks were stacked with an offset in both directions to minimize streaming paths. Cadmium sheets, 30 mil thick, were 'hung' over the end of the assembly, providing a thermal neutron absorber.

### **Gamma Shield**

The gamma shield consists of 10.16 cm of lead bricks covering the outside area of the alumina blocks, and 10.16 cm of bismuth covering the FLUENTAL region. The lead bricks are stacked on edge, with an overlapping design to prevent streaming. The four interlocking bismuth shield blocks, designed to mate with the lead bricks, were cast at the INEEL.

### **Collimator Design**

The design of the cone-shaped bismuth collimator was based on a tradeoff of cone height to maximize the neutron collimation versus intensity loss of the neutrons. The final design selected was a truncated right circular cone with a base opening diameter of 91.4 cm, a height of 38.1 cm, and a wall thickness of 3 cm. The angle of the cone is 45 degrees, giving an opening of 25.4 cm. The final 5 cm of the cone is fitted with a collar, designed so that various shaped apertures can be fabricated from lithiated polyethylene and inserted in a flanged opening. The bismuth cone is supported by borated polyethylene (Reactor Experiments, 5% Borated Polyethylene, Catalog Number 201) for the first 27.9 cm, and two rows (10.16 cm) of lead bricks. The last five cm of the bismuth staves are fitted with a bismuth collar designed to provide a transition from the cone-shape to mate with the lead bricks while providing shielding for the capture gammas generated from the boron in the polyethylene. The collar was formed in a mold also, in two pieces due to the final weight (~45 kg each). This bismuth collar will accommodate lithiated polyethylene final neutron beam delimiters.

### **Collimator Assembly**

The collimator was assembled in two halves. The poly sheets have the corresponding cone shape cut in them, are bolted together with nylon bolts. With the lower half of the poly in position, the front lead bricks

were assembled up to the level of the collimator collar. The lower half of the collar was placed on the lead, and the bismuth staves put into position. The bottom seven staves are pinned to the collar with aluminum rod. A carbon fiber ring was put in place at the rear of the assembly, and a lexan ring inserted at the front of the assembly, providing support for the top half of the collimator staves. With the top half of staves and poly in place, the upper bismuth collar was positioned, and lead bricks continued up to the level of the top of the collar. The top of the collar and accompanying lead bricks are used to support the final large bismuth lintel piece. With the lintel positioned, the remainder of the lead bricks were added to complete the lead wall. The lead wall was covered with lithiated polyethylene, held in place with nylon bolts.

### **Conclusions**

A new epithermal-neutron beam is available for research in the U.S. The beam is being characterized, with a measured source plane (defined at the lexan opening) useful for treatment planning calculations. A beam monitor system remains to be developed and installed. When completed, beam measurements will be made and compared to the FiR 1 neutron source and to calculated values.

### **REFERENCES**

1. F.J. WHEELER, D.W. NIGG, "Feasibility Study for an Epithermal-Neutron Beam Facility at the Washington State University Radiation Center, EGG-NRE-11296, Idaho National Engineering Laboratory, USA, 1994.
2. T.D. BURNS Jr., "A Monte Carlo Model System for Core Analysis and Design at the Washington State University Radiation Center", INEL-95/0458, Idaho National Engineering Laboratory, 1996.
3. I. AUTERINEN, P. HIISMAKI, "Epithermal BNCT Neutron Beam Design for a Triga II Reactor", Advances in Neutron Capture Therapy, Plenum Press, New York, 1993, Pages 81-84.
4. BREISMEISTER, J.F. (1993) MCNP – A General Monte Carlo N-Particle Transport Code, Version 4A, LA-12625-M, Los Alamos National Laboratory, USA.
5. W.A. RHOADES, et. al., "TORT-DORT: Two and Three-Dimensional Discrete-Ordinates Transport", Radiation Shielding Information Center, Oak Ridge National Laboratory, CCC-543, 1993.
6. D.W. NIGG, et al., "Collaborative Neutronic Performance Characterization of the FiR-1 Clinical Epithermal Neutron Beam Facility for BNCT", In: J.R. Venhuizen (ed.), "INEEL BNCT Research Program Annual Report – 1998", INEEL-EXT-99-00293, April, 1999. pp 13-38.