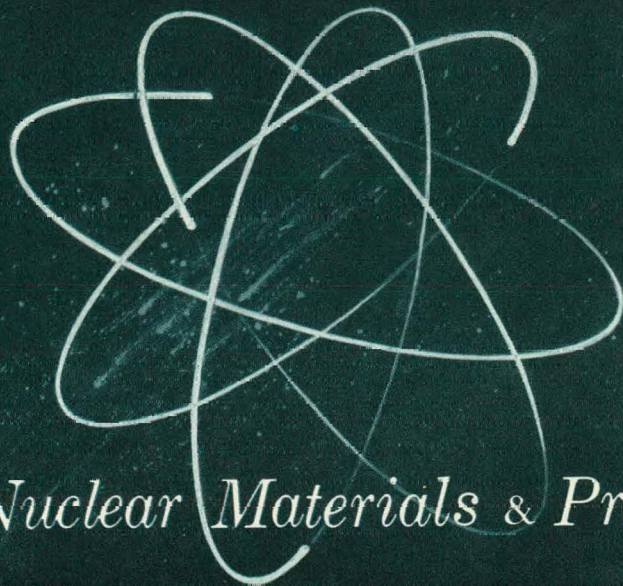


APR 29 1964

DTIC

UNCLASSIFIED
GEMP - 278
(INFORMAL)

MASTER



Nuclear Materials & Propulsion Operation

Gamma Ray Production Cross Sections

W. E. Edwards

ADVANCED TECHNOLOGY SERVICES

GENERAL  ELECTRIC

UNCLASSIFIED

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency Thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

DISCLAIMER

Portions of this document may be illegible in electronic image products. Images are produced from the best available original document.

LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission:

- A. Makes any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, material, method, or process disclosed in this report may not infringe privately owned rights; or
- B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, material, method, or process disclosed in this report.

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares, disseminates, or provides access to, any information pursuant to his employment or contract with the Commission or his employment with such contractor.

UNCLASSIFIED

GEMP-278
(INFORMAL)

Gamma Ray Production Cross Sections

W. E. Edwards

April 7, 1964

Facsimile Price \$ 3.60

Microfilm Price \$ 1.25

Available from the
Office of Technical Services
Department of Commerce
Washington 25, D. C.

United States Atomic Energy Commission

Contract No. AT(40-1)-2847

NUCLEAR MATERIALS and PROPULSION OPERATION
ADVANCED TECHNOLOGY SERVICES

GENERAL  ELECTRIC

Cincinnati 15, Ohio

UNCLASSIFIED

DISTRIBUTION

EXTERNAL

C. L. Karl, AEC, CAO
R. G. Oehl (3), AEC-DRD Headquarters
J. Pidkowitz (3), AEC-ORO
J. F. Weissenberg, AEC, CAO
D. Bugart, NASA, Lewis Research Center
W. Lowenstein, Argonne National Laboratory
DTIE

INTERNAL

H. L. Bermanis
R. W. Brisken
J. M. Brunner
C. L. Chase
E. B. Delson
W. E. Edwards (5)
M. B. Goldstein
W. B. Henderson
E. M. Livingston
R. E. Motsinger
W. E. Niemuth
A. Prince
W. Z. Prickett
R. E. Reid
C. S. Robertson
R. J. Spera
J. D. Simpson
F. L. Sims (2)
C. L. Storrs
W. P. Walsh
J. W. Zwick
Library (15)

ABSTRACT

Gamma ray production cross sections are presented in this report as a function of 16 neutron lethargy groups and 11 photon energy groups for 13 elements and isotopes. They were computed by Program GAMMA-P and account for the production of gamma rays by neutron radiative capture, neutron inelastic scattering, neutron induced fissioning, and (n, α) reactions. The production cross sections are available on IBM decimal cards for use with transport theory Program S-X.

UNCLASSIFIED

CONTENTS

	Page
1. Introduction	5
2. Discussion of Production Cross Sections	7
3. Tabulation of Gamma Ray Production Cross Sections	11
Appendix: Gamma - P Input Listing	13
References	39

1. INTRODUCTION

Gamma ray production cross sections are presented in this report as a function of 16 neutron lethargy groups and 11 photon energy groups for 13 elements and isotopes. They were computed by Program GAMMA-P1* and account for the production of gamma rays by neutron radiative capture, neutron inelastic scattering, neutron induced fissioning, and (n, α) reactions. The production cross sections are available on IBM decimal cards for use with transport theory Program S-X.

Several assumptions are made in Program GAMMA-P to simplify the numerical calculation of production cross sections. First, separation of variables is assumed so that

$$\sigma_{n, \gamma}(\bar{u}_n, \bar{E} \gamma) = \sigma_{n, \gamma}(\bar{u}_n) \cdot S_{n, \gamma}(\bar{u}_n, \bar{E} \gamma),$$

where $\sigma_{n, \gamma}(\bar{u}_n, \bar{E} \gamma)$ is the cross section for the production of gamma rays in gamma energy group $\bar{E} \gamma$ by radiative capture of neutrons in lethargy group \bar{u}_n , $\sigma_{n, \gamma}(\bar{u}_n)$ is the cross section for radiative capture of neutrons in lethargy group \bar{u}_n , and $S_{n, \gamma}$ is the gamma spectrum resulting from neutron radiative capture. Similar assumptions are made for other production reactions.

Second, the gamma energy spectrum from thermal neutron capture is normalized to the binding energy of the neutron. All unmeasured gamma energy is assumed to be released in the lowest energy group because, in general, gamma rays with low energy have not been measured. The third assumption is that the gamma energy from radiative neutron capture equals the neutron binding energy plus kinetic energy. Corrections for internal conversion are possible; however, they were not considered in this analysis.

*Superscripts refer to reference list at end of the report.

2. DISCUSSION OF PRODUCTION CROSS SECTIONS

The cross sections and spectra used in the computation of these production cross sections are discussed in the same format used by M. J. Ferry.¹ As in her compilation, experimental data supplemented by extrapolation when justified by the available data were used.

In all cases, the gamma spectrum from epithermal neutron capture was assumed to have the same shape as that from thermal neutron capture. All references to gamma spectra from radiative neutron capture are for the capture of thermal neutrons.

No attempt was made to account for all the energy available from inelastic neutron scattering. The upper limit of the gamma energy from inelastic scattering is the kinetic energy of the neutron. However, an appreciable portion of the kinetic energy stays with the scattered neutron. Only experimental data were used for the gamma rays from inelastic scattering. Therefore, these production cross sections underestimate the gamma energy released in inelastic neutron scattering.

Atomic weights needed in binding energy calculations were taken from W. H. Sullivan's "Trilinear Chart of Nuclides."² The atomic weights for Mo¹⁰¹, W¹⁸¹, W¹⁸⁵, and Re¹⁸⁵ were computed by equation 8 from reference 1. Percentage abundances of each isotope also were taken from Sullivan. Thermal neutron absorption cross sections were obtained from BNL-325.^{3, 4}

BERYLLIUM

Radiative Capture

The energies and intensities of discrete gamma rays were taken from a tabulation by Troubetzkoy and Goldstein,⁵ and the radiative capture cross section was assumed to vary inversely as the neutron velocity from $\sigma_n, \gamma = 0.010 \text{ barns}^3$ at 0.0253 ev.

Inelastic Scattering

Hall and Bonner⁶ reported that there are no detectable gamma rays from inelastic scattering in beryllium.

BORON - 10

Radiative Capture

The energies and intensities of discrete gamma rays were taken from Troubetzkoy and Goldstein,⁵ and the radiative capture cross section was assumed to vary inversely as the neutron velocity from $\sigma_n, \gamma = 0.5 \text{ barns}^3$ at 0.0253 ev.

Inelastic Scattering

The cross section for the production of 0.717 Mev gamma rays by inelastic scattering measured by Day and Walt⁷ for $1.0 \leq E_n \leq 5.5 \text{ Mev}$ was used. It was extrapolated to 70 mb at $E_n = 6 \text{ Mev}$ and assumed constant at 70 mb above 6 Mev.

(n, α) Reaction

It was assumed that each $B^{10}(n, \alpha)Li^7$ reaction was accompanied by a 0.478 Mev gamma ray,⁸ and it was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

CARBONRadiative Capture

The energies and intensities of discrete gamma rays were taken from Troubetzkoy and Goldstein,⁵ and it was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.

Inelastic Scattering

An average $\sigma_{n, n'\gamma}$ of 0.27 barn for 3 to 10 Mev neutrons for the production of 4.43 Mev gamma rays was determined from data in BNL - 325. Data are presented there for $4.9 \leq E_n \leq 9.2$ Mev.

OXYGENRadiative Capture

The radiative capture cross section of oxygen is less than 0.2 mb at 0.0253 ev,³ and no reference to measured gamma spectra from neutron radiative capture was found. Therefore, radiative capture was omitted from the gamma production cross sections for oxygen.

Inelastic Scattering

Cross sections for the production of a 6.1 Mev gamma ray and for the production of unseparated 6.9 and 7.1 Mev gamma rays were obtained from BNL - 325. An average energy of 7.0 Mev was assumed for the 6.9 and 7.1 Mev gamma rays.

CHROMIUMRadiative Capture

The energies and intensities of discrete gamma rays were taken from Troubetzky and Goldstein,⁵ and it was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

Inelastic Scattering

The cross section for the production of the 1.44 Mev gamma ray by inelastic scattering of neutrons with energies between 1.4 and 2.8 Mev was obtained from BNL - 325. Data from Tralli¹⁰ were used for the energy range 2.8 to 10 Mev.

IRONRadiative Capture

The energies and intensities of discrete gamma rays were taken from Troubetzkoy and Goldstein,⁵ and it was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

Inelastic Scattering

The cross section for the production of 0.845 Mev gamma rays by inelastic scattering of neutrons in the energy range 1.4 to 3.0 Mev was obtained from BNL - 325. The cross section was assumed a constant 1.4 barns above 3 Mev. Cross sections for the production of gamma rays with energies 1.26, 1.41, 1.81, 2.1, 2.26, 2.6, 3.0, and 3.3 Mev were obtained directly from or extrapolated from data received from Montague and Paul.¹¹

NICKEL

Radiative Capture

The energies and intensities of discrete gamma rays were taken from Troubetzkoy and Goldstein,⁵ and it was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

Inelastic Scattering

Cross sections for the production of 1.33 and 1.47 Mev gamma rays by inelastic scattering were taken from BNL - 325 for neutron energies from threshold to 2.7 Mev. Constant cross sections were extrapolated above 2.7 Mev.

MOLYBDENUM

Radiative Capture

Values of the energy-integrated spectrum of gamma rays from radiative capture were taken from the tabulation of Troubetzkoy and Goldstein,⁵ and it was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

Inelastic Scattering

Cross sections at $E_n = 3.2$ Mev were obtained from Price, et al.,¹² for the production of 0.73, 1.4, and 2.5 gamma rays by inelastic scattering. These cross sections were extrapolated at constant values above 3.2 Mev.

TANTALUM

Radiative Capture

Data defining the photon spectrum from radiative capture were read at discrete energy points from a curve in the atlas of Groshev, et al.¹³ It was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

Inelastic Scattering

The cross section for the production of 0.137 Mev gamma rays by inelastic scattering was obtained for $0.3 \leq E_n \leq 1.8$ Mev from BNL - 325. The cross section was extrapolated at essentially constant value above 1.8 Mev.

TUNGSTEN

Radiative Capture

Data defining the photon spectrum from the radiative capture were read at discrete energy points from a curve in the atlas of Groshev. It was assumed that $\sigma_{n,\gamma}$ equaled σ_a from the tabulation of Robertson and Benson.⁹

Inelastic Scattering

Spectra calculated by Troubetzkoy¹⁴ for the gamma rays from neutron inelastic scattering in tungsten were used in this analysis. Troubetzkoy's spectra at $E_n = 3$ and 7 Mev were used for W¹⁸³, and his spectra for W¹⁸⁴ at the same neutron energies were used for all even isotopes. The spectra for $E_n = 3$ Mev were used for all neutron energies below 7 Mev, and the spectra for $E_n = 7$ Mev were used for all other energies.

The inelastic scattering cross section for $0.1 \leq E \leq 1$ Mev was read from a curve published by Howerton.¹⁵ For neutron energies between 1 and 10 Mev, the inelastic scattering cross section was assumed equal to the non-elastic scattering cross section, and values were read from a curve in Troubetzkoy's report.¹⁴

RHENIUMRadiative Capture

Data defining the photon spectrum from radiative capture were read at discrete energy points from a curve in the atlas of Groshev.¹³ It was assumed that σ_n, γ equaled σ_a , and σ_a was taken from the tabulation of Robertson and Benson⁹ for neutron energies above 61.44 ev. The absorption cross section for neutron energies below 61.44 ev was obtained from a report by Zwick.¹⁵

Inelastic Scattering

The cross section for the production of 0.135 Mev gamma rays by inelastic scattering were obtained for $0.135 \leq E_n \leq 1.9$ Mev from BNL - 325. The cross section was extrapolated at a constant 0.8 barn above 1.9 Mev.

URANIUM - 235Fission

Data compiled by J. G. Carver¹⁷ for prompt-plus-delayed gamma radiation from the fissioning of U²³⁵ were used to compute the spectrum for an operating time of 15 minutes. The group-averaged U²³⁵ fission cross sections that were used are listed in the Appendix. Although they have been used extensively in NMPO, they have not been documented.

Radiative Capture

The energy-integrated gamma spectrum from radiative capture was assumed equal to that tabulated by Carver¹⁷ for prompt gamma rays from U²³⁵ fission. The radiative capture cross section was assumed equal to the absorption cross section tabulated by Robertson and Benson⁹ minus the fission cross section.

Inelastic Scattering

No inelastic scattering data were found.

URANIUM - 238Fission

The U²³⁵ fission gamma spectrum was assumed identical to that used for U²³⁵. The group-averaged U²³⁸ fission cross sections that were used are listed in the Appendix. Again, these data have not been documented.

Radiative Capture

The energies and intensities of discrete gamma rays were taken from the tabulation of Troubetzkoy and Goldstein.⁵ The radiative capture cross section was assumed equal to the absorption cross section tabulated by Robertson and Benson⁹ minus the fission cross section.

Inelastic Scattering

No inelastic scattering data were found.

3. TABULATION OF GAMMA RAY PRODUCTION CROSS SECTIONS

Cross sections for the production of gamma rays by radiative neutron capture, neutron inelastic scattering, neutron induced fissioning, and (n, α) reactions are tabulated on the following pages for 13 elements and isotopes as a function of ascending photon and neutron energies (descending lethargy).

The 16 neutron energy and lethargy groups are given in Table 1. The 11 photon energy groups are given in Table 2.

TABLE 1
CROSS SECTION ENERGY LATTICE

Group Index	Energy Level	Lethargy Level	Lethargy Interval
1	10.0 Mev	0.0	1.204
2	3.0 Mev	1.204	0.762
3	1.4 Mev	1.996	0.442
4	0.9 Mev	2.408	0.811
5	0.4 Mev	3.219	1.386
6	0.1 Mev	4.605	1.772
7	17.0 Kev	6.377	1.623
8	3.354 Kev	8.0	2.0
9	0.454 Kev	10.0	2.0
10	61.44 ev	12.0	2.0
11	8.315 ev	14.0	1.5
12	1.855 ev	15.5	1.0
13	0.6826 ev	16.5	1.0
14	0.2511 ev	17.5	1.0
15	0.09237 ev	18.5	1.055
16	0.03216 ev	19.555	0.24
	00.0	∞	

TABLE 2
GAMMA PHOTON ENERGY GROUPS

Photon Energy Group	Group Energy Range, Mev
1	0.01 to 0.4
2	0.4 to 0.9
3	0.9 to 1.35
4	1.35 to 1.8
5	1.8 to 2.2
6	2.2 to 2.6
7	2.6 to 3.0
8	3.0 to 4.0
9	4.0 to 5.0
10	5.0 to 7.0
11	7.0 to 10.0

BORON 10

9	2.3683-01	8.5517-01	2.2128+00	0.	0.
9	0.	0.	0.	0.	1.7000+03
9	1.7731+00				
9	1.8444-01	6.6599-01	1.7233+00	0.	0.
9	0.	0.	0.	0.	1.3197+03
9	1.3809+00				
9	1.1013-01	3.9766-01	1.0290+00	0.	0.
9	0.	0.	0.	0.	7.7202+02
9	8.2452-01				
9	6.6825-02	2.4130-01	6.2437-01	0.	0.
9	0.	0.	0.	0.	4.7843+02
9	5.0031-01				
9	4.0523-02	1.4632-01	3.7862-01	0.	0.
9	0.	0.	0.	0.	2.9020+02
9	3.0339-01				
9	2.1972-02	7.9339-02	2.0529-01	0.	0.
9	0.	0.	0.	0.	1.5219+02
9	1.6450-01				
9	9.3021-03	3.3589-02	8.6913-02	0.	0.
9	0.	0.	0.	0.	7.2264+01
9	6.9644-02				
9	3.4322-03	1.2393-02	3.2068-02	0.	0.
9	0.	0.	0.	0.	2.3406+01
9	2.5697-02				
9	1.2618-03	4.5563-03	1.1790-02	0.	0.
9	0.	0.	0.	0.	8.7508+00
9	9.4470-03				
9	5.0339-04	1.8177-03	4.7034-03	0.	0.
9	0.	0.	0.	0.	3.5354+00
9	3.7689-03				
9	2.1732-04	7.8471-04	2.0305-03	0.	0.
9	0.	0.	0.	0.	1.5780+00
9	1.6270-03				
9	9.8966-05	3.5736-04	9.2468-04	0.	0.
9	0.	0.	0.	0.	4.9973-01
9	7.4095-04				
9	5.7755-05	2.0855-04	5.3963-04	0.	0.
9	0.	0.	0.	0.	2.1269-01
9	4.3241-04				
9	4.3691-05	1.5777-04	4.0823-04	0.	0.
9	0.	0.	0.	0.	9.6628-02
9	3.2711-04				
9	3.4919-05	1.2609-04	3.2626-04	0.	0.
9	0.	0.	0.	0.	1.5253-01
9	2.6144-04				
9	2.6552-05	9.5878-05	2.4809-04	0.	0.
9	0.	0.	0.	0.	9.3927-02
9	1.9879-04				

CHROMIUM

9	7.8855+00	1.5159+00	9.9382-02	3.4671-01	0.
9	1.1934-01	9.6707-02	9.0020-02	0.	5.1671-01
9	8.9118+00				
9	6.1215+00	1.1768+00	7.7150-02	2.6915-01	0.
9	9.2644-02	7.5074-02	6.9882-02	0.	4.0112-01
9	6.9182+00				
9	3.6587+00	7.0335-01	4.6111-02	1.6086-01	0.
9	5.5371-02	4.4870-02	4.1767-02	0.	2.3974-01
9	4.1348+00				
9	2.2190+00	4.2658-01	2.7966-02	9.7563-02	0.
9	3.3582-02	2.7213-02	2.5332-02	0.	1.4540-01
9	2.5078+00				
9	1.3462+00	2.5878-01	1.6966-02	5.9187-02	0.
9	2.0373-02	1.6509-02	1.5367-02	0.	8.8209-02
9	1.5213+00				
9	7.0620-01	1.3576-01	8.9003-03	3.1050-02	0.
9	1.0688-02	8.6608-03	8.0619-03	0.	4.6275-02
9	7.9811-01				
9	3.3520-01	6.4438-02	4.2245-03	1.4738-02	0.
9	5.0729-03	4.1108-03	3.8266-03	0.	2.1964-02
9	3.7882-01				
9	1.8687-01	3.5924-02	2.3552-03	8.2163-03	0.
9	2.8282-03	2.2918-03	2.1333-03	0.	1.2245-02
9	2.1119-01				
9	1.4020-01	2.6952-02	1.7670-03	6.1642-03	0.
9	2.1218-03	1.7194-03	1.6005-03	0.	9.1869-03
9	1.5845-01				
9	3.7561-01	7.2208-02	4.7339-03	1.6515-02	0.
9	5.6846-03	4.6065-03	4.2879-03	0.	2.4613-02
9	4.2450-01				
9	6.3385-02	1.2185-02	7.9884-04	2.7869-03	0.
9	9.5927-04	7.7734-04	7.2359-04	0.	4.1534-03
9	7.1634-02				
9	5.1305-02	9.8628-03	6.4660-04	2.2557-03	0.
9	7.7645-04	6.2920-04	5.8569-04	0.	3.3618-03
9	5.7982-02				
9	5.0470-02	9.7023-03	6.3607-04	2.2190-03	0.
9	7.6381-04	6.1895-04	5.7615-04	0.	3.3071-03
9	5.7038-02				
9	4.8900-02	9.4006-03	6.1629-04	2.1500-03	0.
9	7.4006-04	5.9971-04	5.5824-04	0.	3.2043-03
9	5.5264-02				
9	6.5596-02	1.2610-02	8.2671-04	2.8841-03	0.
9	9.9273-04	8.0445-04	1.6279+00	0.	4.2983-03
9	7.4132-02				
9	1.0055-01	1.9330-02	1.2672-03	4.4209-03	0.
9	1.5217-03	1.2331-03	1.8011+00	0.	6.5887-03
9	1.1364-01				

IRON

9	6.0510+00	1.7232+00	4.6340-01	1.0774+00	3.7131-01
9	0.	1.6033-01	5.3483-01	4.1604-02	4.1770-02
9	6.9753+00				
9	4.6980+00	1.3379+00	3.5979-01	8.3648-01	2.8828-01
9	0.	1.2448-01	4.1524-01	3.2301-02	3.2430-02
9	5.4156+00				
9	2.8062+00	7.9915-01	2.1491-01	4.9966-01	1.7220-01
9	0.	7.4355-02	2.4803-01	1.9294-02	1.9371-02
9	3.2349+00				
9	1.7020+00	4.8469-01	1.3034-01	3.0304-01	1.0444-01
9	0.	4.5097-02	1.5043-01	1.1702-02	1.1749-02
9	1.9620+00				
9	1.0336+00	2.9435-01	7.9157-02	1.8403-01	6.3425-02
9	0.	2.7907-02	9.1357-02	7.1066-03	7.1349-03
9	1.1915+00				
9	5.4313-01	1.5467-01	4.1595-02	9.6706-02	3.3328-02
9	0.	1.4391-02	4.8006-02	3.7343-03	3.7492-03
9	6.2609-01				
9	2.5749-01	7.3328-02	1.9720-02	4.5847-02	1.5800-02
9	0.	6.8226-03	2.2759-02	1.7704-03	1.7775-03
9	2.9682-01				
9	2.7848+00	7.9304-01	2.1327-01	4.9583-01	1.7088-01
9	0.	7.3786-02	2.4614-01	1.9147-02	1.9223-02
9	3.2101+00				
9	3.5142-02	1.0008-02	2.6913-03	6.2571-03	2.1564-03
9	0.	9.3114-04	3.1061-03	2.4162-04	2.4258-04
9	4.0510-02				
9	6.9374-02	1.9756-02	5.3129-03	1.2352-02	4.2570-03
9	0.	1.8382-03	6.1317-03	4.7698-04	4.7889-04
9	7.9971-02				
9	1.3348-01	3.8012-02	1.0222-02	2.3767-02	8.1908-03
9	0.	3.5368-03	1.1798-02	9.1775-04	9.2142-04
9	1.5387-01				
9	5.0336-02	1.4334-02	3.8549-03	8.9624-03	3.0888-03
9	0.	1.3337-03	4.4490-03	3.4609-04	3.4747-04
9	5.8024-02				
9	4.3512-02	1.2391-02	3.3323-03	7.7475-03	2.6701-03
9	0.	1.1529-03	3.8459-03	2.9917-04	3.0037-04
9	5.0159-02				
9	3.9205-02	1.1165-02	3.0025-03	6.9806-03	2.4058-03
9	0.	1.0388-03	3.4652-03	2.6956-04	2.7063-04
9	4.5194-02				
9	5.1180-02	1.4575-02	3.9195-03	9.1126-03	3.1406-03
9	0.	4.1176-02	6.0924-02	7.3432-02	7.7775-01
9	5.8997-02				
9	8.3034-02	2.3646-02	6.3590-03	3.1178-01	1.8510-01
9	5.9900-01	7.4220-01	1.2014-01	2.5257-01	1.1836+00
9	9.5717-02				

NICKEL

9	1.7634+01	4.8617+00	2.8519-01	4.0270-01	0.
9	0.	1.7544-01	1.3342-01	8.9760-02	1.6769-01
9	1.1405+01				
9	1.3688+01	3.7737+00	2.2137-01	3.1258-01	0.
9	0.	1.3618-01	1.0356-01	6.9673-02	1.3016-01
9	8.8531+00				
9	8.1796+00	2.2552+00	1.3229-01	1.8679-01	0.
9	0.	8.1379-02	6.1886-02	4.1636-02	7.7783-02
9	5.2905+00				
9	4.9625+00	1.3682+00	8.0258-02	1.1333-01	0.
9	0.	4.9372-02	3.7546-02	2.5260-02	4.7191-02
9	3.2097+00				
9	3.0067+00	8.2896-01	4.8627-02	6.8663-02	0.
9	0.	2.9914-02	2.2748-02	1.5305-02	2.8592-02
9	1.9447+00				
9	1.5770+00	4.3479-01	2.5505-02	3.6014-02	0.
9	0.	1.5690-02	1.1932-02	8.0274-03	1.4997-02
9	1.0200+00				
9	7.4970-01	2.0670-01	1.2125-02	1.7121-02	0.
9	0.	7.4588-03	5.6722-03	3.8161-03	7.1292-03
9	4.8490-01				
9	2.7057-01	7.4596-02	4.3758-03	6.1788-03	0.
9	0.	2.6919-03	2.0471-03	1.3772-03	2.5729-03
9	1.7500-01				
9	1.2705-01	3.5027-02	2.0547-03	2.9013-03	0.
9	0.	1.2640-03	9.6121-04	6.4669-04	1.2081-03
9	8.2172-02				
9	1.4703+00	4.0535-01	2.3778-02	3.3575-02	0.
9	0.	1.4628-02	1.1124-02	7.4839-03	1.3981-02
9	9.5095-01				
9	2.8070-01	7.7389-02	4.5397-03	6.4101-03	0.
9	0.	2.7926-03	2.1237-03	1.4288-03	2.6692-03
9	1.8155-01				
9	1.1670-01	3.2174-02	1.8873-03	2.6650-03	0.
9	0.	1.1610-03	8.8292-04	5.9401-04	1.1097-03
9	7.5479-02				
9	8.5860-02	2.3672-02	1.3886-03	1.9607-03	0.
9	0.	8.5422-04	6.4961-04	4.3705-04	8.1648-04
9	5.5534-02				
9	8.0814-02	2.2281-02	1.3070-03	1.8455-03	0.
9	0.	8.0402-04	6.1143-04	4.1136-04	7.6850-04
9	5.2270-02				
9	8.6949-02	2.3972-02	1.4062-03	1.9856-03	0.
9	0.	8.6506-04	9.8556-01	3.9944-01	8.2683-04
9	5.6238-02				
9	1.2524-01	3.4528-02	2.0254-03	2.8599-03	0.
9	0.	1.2460-03	1.1769+00	5.3264-01	1.1909-03
9	8.1001-02				

MOLYBDENUM

9	4.6561-01	3.4588+00	3.1263+00	4.2792+00	0.
9	0.	0.	0.	0.	0.
9	5.1870+00				
9	3.6146-01	2.6851+00	2.4269+00	3.3220+00	0.
9	0.	0.	0.	0.	0.
9	4.0267+00				
9	2.1600-01	1.6046+00	1.4503+00	1.9852+00	0.
9	0.	0.	0.	0.	0.
9	2.4063+00				
9	1.3104-01	9.7342-01	8.7983-01	1.2043+00	0.
9	0.	0.	0.	0.	0.
9	1.4598+00				
9	7.9481-02	5.9043-01	5.3366-01	7.3047-01	0.
9	0.	0.	0.	0.	0.
9	8.8543-01				
9	3.5271-01	2.6202+00	2.3682+00	3.2416+00	0.
9	0.	0.	0.	0.	0.
9	3.9293+00				
9	1.0605+00	7.8783+00	7.1208+00	9.7468+00	0.
9	0.	0.	0.	0.	0.
9	1.1815+01				
9	1.1193+00	8.3150+00	7.5155+00	1.0287+01	0.
9	0.	0.	0.	0.	0.
9	1.2470+01				
9	4.2008-03	3.1206-02	2.8205-02	3.8607-02	0.
9	0.	0.	0.	0.	0.
9	4.6798-02				
9	0.	0.	0.	0.	0.
9	0.	0.	0.	0.	0.
9	0.				
9	2.1131-03	1.5697-02	1.4188-02	1.9420-02	0.
9	0.	0.	0.	0.	0.
9	2.3540-02				
9	0.	0.	0.	0.	0.
9	0.	0.	0.	0.	0.
9	0.				
9	0.	0.	0.	0.	0.
9	0.	0.	0.	0.	0.
9	0.				
9	0.	0.	0.	0.	0.
9	4.3482-03	3.2301-02	2.9195-02	3.9962-02	0.
9	0.	0.	0.	0.	0.
9	4.8440-02				
9	0.	0.	0.	0.	0.
9	0.	0.	0.	0.	0.
9	0.				
9	0.	0.	0.	0.	0.
9	1.4750-01	0.	4.6200-01	0.	4.8180-01
9	0.				

TANTALUM

9	0.	5.8344+00	1.4169+01	2.3584+01	1.1477+01
9	1.2542+01	1.3021+01	1.3220+01	9.9492+00	8.8548+00
9	1.3633+01				
9	0.	4.8428+00	1.1761+01	1.9576+01	9.5261+00
9	1.0411+01	1.0808+01	1.0973+01	8.2582+00	7.3498+00
9	1.1316+01				
9	0.	2.7458+00	6.6683+00	1.1099+01	5.4012+00
9	5.9026+00	6.1280+00	6.2214+00	4.6823+00	4.1673+00
9	6.4158+00				
9	0.	1.7164+00	4.1684+00	6.9382+00	3.3764+00
9	3.6898+00	3.8307+00	3.8891+00	2.9270+00	2.6050+00
9	4.0106+00				
9	0.	1.6758+00	4.0698+00	6.7740+00	3.2964+00
9	3.6025+00	3.7400+00	3.7970+00	2.8577+00	2.5434+00
9	3.9157+00				
9	0.	9.3737+01	2.2764+02	3.7891+02	1.8439+02
9	2.0151+02	2.0920+02	2.1239+02	1.5985+02	1.4226+02
9	2.1903+02				
9	0.	3.2420+01	7.8733+01	1.3105+02	6.3772+01
9	6.9693+01	7.2354+01	7.3457+01	5.5285+01	4.9203+01
9	7.5752+01				
9	0.	9.8969+00	2.4035+01	4.0005+01	1.9468+01
9	2.1275+01	2.2088+01	2.2424+01	1.6877+01	1.5020+01
9	2.3125+01				
9	0.	2.5958+00	6.3040+00	1.0493+01	5.1061+00
9	5.5802+00	5.7932+00	5.8815+00	4.4265+00	3.9396+00
9	6.0653+00				
9	0.	1.1240+00	2.7296+00	4.5433+00	2.2109+00
9	2.4162+00	2.5084+00	2.5467+00	1.9167+00	1.7058+00
9	2.6262+00				
9	0.	3.5739-01	8.6793-01	1.4446+00	7.0301-01
9	7.6828-01	7.9760-01	8.0977-01	6.0944-01	5.4240-01
9	8.3507-01				
9	0.	9.9475-02	2.4158-01	4.0210-01	1.9568-01
9	2.1384-01	2.2201-01	2.2539-01	1.6963-01	1.5097-01
9	2.3243-01				
9	0.	7.2710-02	1.7658-01	2.9391-01	1.4303-01
9	1.5630-01	1.6227-01	1.6475-01	1.2399-01	1.1035-01
9	2.7264-01				
9	0.	5.7459-02	1.3954-01	2.3226-01	1.1303-01
9	1.2352-01	1.2823-01	1.3019-01	9.7982-02	8.7204-02
9	2.9866-01				
9	0.	4.2204-02	1.0249-01	1.7060-01	8.3018-02
9	9.0726-02	9.4189-02	9.5625-02	7.1969-02	6.4052-02
9	3.4521-01				
9	0.	2.7450-02	6.6663-02	1.1096-01	5.3995-02
9	5.9009-02	6.1261-02	6.2195-02	4.6809-02	4.1660-02
9	3.2444-01				

TUNGSTEN

9	9.7737+01	1.6525+01	1.3229+01	1.8691+01	8.9441+00
9	9.9525+00	9.1644+00	8.9756+00	6.8092+00	1.0525+01
9	3.0395+00				
9	7.9729+01	1.3480+01	1.0791+01	1.5247+01	7.2961+00
9	8.1188+00	7.4758+00	7.3218+00	5.5546+00	8.5858+00
9	2.4794+00				
9	4.6688+01	7.8939+00	6.3193+00	8.9284+00	4.2725+00
9	4.7542+00	4.3777+00	4.2875+00	3.2527+00	5.0277+00
9	1.4519+00				
9	2.8419+01	4.8050+00	3.8465+00	5.4347+00	2.6006+00
9	2.8939+00	2.6647+00	2.6098+00	1.9799+00	3.0603+00
9	8.8377-01				
9	1.7254-01	2.9173+00	2.3354+00	3.2996+00	1.5789+00
9	1.7570+00	1.6178+00	1.5845+00	1.2021+00	1.8581+00
9	5.3657-01				
9	1.2252+00	2.0715+01	1.6583+01	2.3430+01	1.1212+01
9	1.2476+01	1.1488+01	1.1251+01	8.5358+00	1.3194+01
9	3.8101+00				
9	1.1710+00	1.9799+01	1.5849+01	2.2393+01	1.0716+01
9	1.1924+01	1.0980+01	1.0754+01	8.1580+00	1.2610+01
9	3.6415+00				
9	2.9820-01	5.0419+00	4.0361+00	5.7026+00	2.7288+00
9	3.0365+00	2.7961+00	2.7385+00	2.0775+00	3.2112+00
9	9.2734-01				
9	3.5508-02	6.0037-01	4.8061-01	6.7904-01	3.2494-01
9	3.6158-01	3.3294-01	3.2609-01	2.4738-01	3.8238-01
9	1.1042-01				
9	2.7080-02	4.5786-01	3.6653-01	5.1786-01	2.4781-01
9	2.7575-01	2.5391-01	2.4869-01	1.8866-01	2.9162-01
9	8.4214-02				
9	1.1428-02	1.9322-01	1.5468-01	2.1854-01	1.0458-01
9	1.1637-01	1.0715-01	1.0494-01	7.9615-02	1.2306-01
9	3.5538-02				
9	5.7807-03	9.7740-02	7.8243-02	1.1055+01	7.4040-02
9	1.2477-01	1.6967-01	2.4293-01	2.3200-01	1.4790-01
9	7.1527-02				
9	3.8355-03	6.4851-02	5.1914-02	7.3349-02	1.0758-01
9	2.6502-01	4.3184-01	6.8610-01	6.8408-01	3.3494-01
9	1.9553-01				
9	3.0527-03	5.1615-02	4.1319-02	5.8379-02	1.6082-01
9	4.4535-01	7.5440-01	1.2213+00	1.2264+00	5.7121-01
9	3.4609-01				
9	2.2513-03	3.8064-02	3.0471-02	4.3053-02	1.7583-01
9	5.0686-01	8.6895-01	1.4146+00	1.4235+00	6.5312-01
9	4.0021-01				
9	2.0516-03	3.4688-02	6.0197-01	2.1597+00	1.3619+00
9	1.7764+00	2.1329+00	2.6503+00	1.9866+00	7.0171-01
9	4.0731-01				

RHENIUM

9	0.	2.4593+01	5.5096+01	7.4600+01	3.6773+01
9	4.1066+01	4.3502+01	4.1069+01	3.3616+01	3.0601+01
9	3.1003+02				
9	0.	1.4608+01	3.2726+01	4.4311+01	2.1843+01
9	2.4392+01	2.5839+01	2.4394+01	1.9968+01	1.8176+01
9	1.8415+02				
9	0.	1.0762+01	2.4110+01	3.2645+01	1.6092+01
9	1.7970+01	1.9036+01	1.7971+01	1.4710+01	1.3391+01
9	1.3567+02				
9	0.	2.5913+01	5.8054+01	7.8605+01	3.8747+01
9	4.3270+01	4.5837+01	4.3273+01	3.5421+01	3.2243+01
9	3.2667+02				
9	0.	2.7100+02	6.0711+02	8.2204+02	4.0521+02
9	4.5251+02	4.7936+02	4.5255+02	3.7043+02	3.3720+02
9	3.4163+03				
9	0.	6.9238+01	1.5511+02	2.1002+02	1.0353+02
9	1.1561+02	1.2247+02	1.1562+02	9.4642+01	8.6151+01
9	8.7283+02				
9	0.	8.2138+00	1.8401+01	2.4915+01	1.2282+01
9	1.3715+01	1.4529+01	1.3716+01	1.1227+01	1.0220+01
9	1.0354+02				
9	0.	8.1156+00	1.8181+01	2.4618+01	1.2135+01
9	1.3551+01	1.4355+01	1.3553+01	1.1093+01	1.0098+01
9	1.0231+02				
9	0.	2.9195+00	6.5406+00	8.8560+00	4.3655+00
9	4.8750+00	5.1643+00	4.8754+00	3.9907+00	3.6327+00
9	3.6804+01				
9	0.	1.0822+00	2.4245+00	3.2828+00	1.6182+00
9	1.8071+00	1.9143+00	1.8072+00	1.4793+00	1.3466+00
9	1.3643+01				
9	0.	3.8597-01	8.6468-01	1.1708+00	5.7712-01
9	6.4449-01	6.8272-01	6.4454-01	5.2758-01	4.8025-01
9	4.8656+00				
9	0.	1.5288-01	3.4250-01	4.6374-01	2.2860-01
9	2.5528-01	2.7042-01	2.5530-01	2.0897-01	1.9023-01
9	1.9610+00				
9	0.	5.4831-02	1.2284-01	1.6632-01	8.1987-02
9	9.1557-02	9.6988-02	9.1564-02	7.4949-02	6.8225-02
9	7.7221-01				
9	0.	3.8013-02	8.5159-02	1.1531-01	5.6839-02
9	6.3474-02	6.7239-02	6.3478-02	5.1960-02	4.7298-02
9	5.8045-01				
9	0.	4.2496-02	9.5202-02	1.2890-01	6.3542-02
9	7.0959-02	7.5169-02	7.0964-02	5.8087-02	5.2876-02
9	6.4371-01				
9	0.	2.4667-02	5.5260-02	7.4823-02	3.6883-02
9	4.1188-02	4.3632-02	4.1191-02	3.3717-02	3.0692-02
9	4.1895-01				

URANIUM 235

9	9.0369+00	1.0834+02	2.4921+02	5.8797+02	3.7313+02
9	4.9936+02	6.3980+02	8.7294+02	1.2006+03	1.6005+03
9	7.3891+02				
9	6.8751+00	8.2764+01	1.9087+02	4.5052+02	2.8586+02
9	3.8247+02	4.8998+02	6.6862+02	9.1903+02	1.2245+03
9	5.6480+02				
9	3.7480+00	4.4904+01	1.0324+02	2.4357+02	1.5457+02
9	2.0687+02	2.6506+02	3.6164+02	4.9745+02	6.6319+02
9	3.0622+02				
9	2.1755+00	2.5982+01	5.9617+01	1.4060+02	8.9240+01
9	1.1946+02	1.5307+02	2.0882+02	2.8738+02	3.8328+02
9	1.7710+02				
9	9.4934-01	1.1166+01	2.5367+01	5.9730+01	3.7932+01
4	5.0822+01	6.5154+01	8.8838+01	1.2256+02	1.6376+02
9	7.5937+01				
9	1.1730+00	1.2779+01	2.7506+01	6.4191+01	4.0899+01
9	5.5076+01	7.0794+01	9.6253+01	1.3461+02	1.8170+02
9	8.5870+01				
9	1.3698+00	1.4914+01	3.2088+01	7.4879+01	4.7711+01
9	6.4251+01	8.2590+01	1.1229+02	1.5706+02	2.1201+02
9	1.0021+02				
9	3.8513-01	4.3582+00	9.6438+00	2.2611+01	1.4382+01
9	1.9316+01	2.4795+01	3.3761+01	4.6885+01	6.2954+01
9	2.9465+01				
9	1.6756-01	1.9145+00	4.2649+00	1.0011+01	6.3648+00
9	8.5430+00	1.0962+01	1.4932+01	2.0702+01	2.7762+01
9	1.2963+01				
9	8.2950-02	9.6664-01	2.1825+00	5.1341+00	3.2616+00
9	4.3724+00	5.6071+00	7.6429+00	1.0560+01	1.4127+01
9	6.5648+00				
9	4.7533-02	5.5981-01	1.2729+00	2.9977+00	1.9036+00
9	2.5502+00	3.2693+00	4.4579+00	6.1489+00	8.2145+00
9	3.8079+00				
9	2.5095-02	3.0227-01	6.9738-01	1.6461+00	1.0445+00
9	1.3974+00	1.7902+00	2.4429+00	3.3575+00	4.4733+00
9	2.0630+00				
9	2.0520-02	2.4914-01	5.7767-01	1.3646+00	8.6561-01
9	1.1576+00	1.4826+00	2.0237+00	2.7780+00	3.6978+00
9	1.7023+00				
9	2.0282-02	2.4843-01	5.7919-01	1.3694+00	8.6836-01
9	1.1607+00	1.4862+00	2.0292+00	2.7818+00	3.6991+00
9	1.6996+00				
9	1.9926-02	2.4544-01	5.7420-01	1.3583+00	8.6117-01
9	1.1508+00	1.4732+00	2.0118+00	2.7556+00	3.6620+00
9	1.6805+00				
9	1.9608-02	2.4213-01	5.6733-01	1.3424+00	8.5099-01
9	1.1370+00	1.4555+00	1.9877+00	2.7217+00	3.6159+00
9	1.6584+00				

URANIUM 238

9	0.	0.	6.9345-01	3.5304-01	0.
9	0.	0.	0.	0.	7.2852-01
9	9.6816+00				
9	0.	0.	5.3921-01	2.7452-01	0.
9	0.	0.	0.	0.	5.6648-01
9	7.5282+00				
9	0.	0.	3.1989-01	1.6286-01	0.
9	0.	0.	0.	0.	3.3606-01
9	4.4661+00				
9	0.	0.	1.9619-01	9.9884-02	0.
9	0.	0.	0.	0.	2.0612-01
9	2.7392+00				
9	0.	0.	1.1942-01	6.0798-02	0.
9	0.	0.	0.	0.	1.2546-01
9	1.6673+00				
9	0.	0.	3.7005+01	1.8840+01	0.
9	0.	0.	0.	0.	3.8877+01
9	5.1665+02				
9	0.	0.	4.0092+01	2.0411+01	0.
9	0.	0.	0.	0.	4.2120+01
9	5.5975+02				
9	0.	0.	3.1279+00	1.5924+00	0.
9	0.	0.	0.	0.	3.2861+00
9	4.3670+01				
9	0.	0.	5.7170-01	2.9106-01	0.
9	0.	0.	0.	0.	6.0061-01
9	7.9818+00				
9	0.	0.	1.9939-01	1.0151-01	0.
9	0.	0.	0.	0.	2.0948-01
9	2.7838+00				
9	0.	0.	1.2922-01	6.5789-02	0.
9	0.	0.	0.	0.	1.3576-01
9	1.8042+00				
9	0.	0.	4.7581-02	2.4224-02	0.
9	0.	0.	0.	0.	4.9988-02
9	6.6431-01				
9	0.	0.	4.5010-02	2.2915-02	0.
9	0.	0.	0.	0.	4.7287-02
9	6.2842-01				
9	6.6000-04	8.3600-03	5.5143-02	6.5117-02	2.9876-02
9	3.9864-02	5.0996-02	6.9696-02	9.5084-02	1.6301-01
9	5.4968-01				
9	7.2750-03	9.2150-02	2.3976-01	5.3038-01	3.2931-01
9	4.3941-01	5.6211-01	7.6824-01	1.0481+00	1.4101+00
9	9.2023-01				
9	9.2400-03	1.1704-01	2.7843-01	6.6035-01	4.1826-01
9	5.5810-01	7.1394-01	9.7574-01	1.3312+00	1.7636+00
9	8.0450-01				

APPENDIX: GAMMA-P INPUT LISTING

Input data used in the preparation of these gamma production cross sections are listed on the following pages. The data defining gamma spectra from the various sources could be used in GAMMA-P calculations for other neutron or gamma energy groups.

GAMMA-P INPUT DATA

BERYLLIUM

3Z 4.0
4N 16 J 11 I 2 L 0 NOELEM 13 K 1
3D .5 .75
3E 3.41 6.797
3SIGNG .0089 .0069 .0041 .0025 .0015 .0008 .0003 .0001
0 0 0 0 0 0 0
3AMNEUT 1.008986
3AMISO1 9.015060 AMISO2 10.016716
3C1 931.162
3P 1.
3SIGA .01
3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
.4 .9 1.4 3. 10.
3RME .510984
3C 2.997923/10
3TK 2.5/-8
4KL 0
-KTROL +1 0 0 0 0 0

BORON 10

3Z 5.0
 4N 16 J 11 I 6 L 1 K 1
 3D .8 .3 .1 .18 .06 .008
 3E 4.47 4.73 6.74 6.98 8.91 11.43
 3SFG 0 .478 0 0 0 0 0 0 0 0
 3SNNG 420= 0 .717
 3SNNG 450= 0 .717
 3SIGNG .443 .345 .206 .125 .0758 .0411 .0174 .00642 .00236
 .000941 .000405 .000182 .000103 .000075 .000056 .000034
 3FSIG 3556.4 2760.9 1615.1 1000.9 607.11 318.39 151.18 48.9668
 18.3072 7.39614 3.30121 1.04546 .44496 .20215 .26961 .0915
 3SIGNN 14= .033 .070
 3AMNEUT 1.008986
 3AMISO1 10.016119 AMISO2 11.012795
 3C1 931.162
 3P 1. SIGA 4017.
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL +1 0 +1 +1 0 0

CARBON

3Z 6.
 4N 16 J 11 I 2 L 1 K 2
 3D .3 .7
 3E 3.68 4.948
 3SNNG 458= 4.43
 3SIGNG .0028 .00216 .00128 .00078 .00048 .00032 .00014
 0 0 0 0 0 0 0 0
 3SIGNN 15= .27
 3AMNEUT 1.008986
 3AMISO1 12.003816 13.007478
 3AMISO2 13.007478 14.007687
 3C1 931.162
 3P .989 .111 SIGA .0033 .0007
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL +1 0 +1 0 0 0

OXYGEN

3Z 8.
4N 16 J 11 I 0 L 2 K 3
3SNG 459= 6.1
3SNG 1659= 7.
3SIGNN 15= .196
3SIGNN 55= .05
3AMNEUT 1.008986
3AMISO1 16.000000 17.004537 18.004855
3AMISO2 17.004537 18.004855 19.00959
3C1 931.162
3P .99759 .00037 .00204 SIGA 0 .4 .00021
3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
.4 .9 1.4 3. 10.
3RME .510984
3C 2.997923/10
3TK 2.5/-8
4KL 0
-KTROL 0 0 +1 0 0 0

CHROMIUM

3Z 24.
 4N 16 J 11 I 26 L 1 K 4
 3D .02 .03 .2 .02 .02 .02 .02 .02 .008 .01 .02 .01
 .008 .009 .004 .03 .005 .024 .002 .03 .002 .002
 .06 .05 .14 .05
 3E .52 .75 .84 1.75 1.88 2.32 3.02 3.72 4.83 5.26
 5.61 6. 6.12 6.28 6.358 6.644 6.872 7.097 7.21
 7.364 7.54 7.67 7.929 8.499 8.881 9.716
 3SNNG 423= 1.44
 3SNNG 453= 1.44
 3SIGNG 2.572 1.99664 1.19334 .72376 .43907 .23034 .10933
 .06095 .04572 .12238 .02055 .01628 .01525 .01392 .01684 .01875
 3SIGNN 14= 1.13 1.25
 3AMNEUT 1.008986
 3AMISO1 49.96164 51.95699 52.95446 53.95602
 3AMISO2 50.96084 52.95746 53.95602 54.95843
 3C1 931.162
 3P .0431 .8376 .0955 .0238 SIGA 15.9 .76 18.2 .38
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL +1 0 +1 0 0 0

IRON

3Z 26.
 4N 16 J 11 I 33 L 9 K 4
 3D .032 .067 .041 .015 .019 .061 .064 .023 .014 .01
 .029 .021 .021 .029 .039 .014 .011 .02 .007 .02 .01
 .01 .006 .006 .06 .06 .003 .03 .29 .006 .003 .02 .0006
 3E .313 .364 .454 1.236 1.53 1.626 1.72 1.802 2.143
 2.672 2.73 2.837 3.146 3.24 3.43 3.552 3.725 3.43
 3.86 4.21 4.44 4.81 4.968 5.51 5.914 6.015 6.369
 7.285 7.639 8.345 8.872 9.298 10.16
 3SNNG 421= .845
 3SNNG 451= .845
 3SNNG 1622= 1.26
 3SNNG 1652= 1.26
 3SNNG 2823= 1.41
 3SNNG 2853= 1.41
 3SNNG 4024= 1.81
 3SNNG 4054= 1.81
 3SNNG 5254= 2.1
 3SNNG 6455= 2.26
 3SNNG 7655= 2.6
 3SNNG 8856= 3.
 3SNNG 10057= 3.3
 3SIGNG 2.244 1.74224 1.04069 .63118 .38331 .20142 .09549
 1.0327 .01303 .0257 .04921 .01817 .01497 .01272 .015 .01775
 3SIGNN 14= .92 1.4
 3SIGNN 54= .058 .2
 3SIGNN 94= .04 .08
 3SIGNN 134= .022 .2
 3SIGNN 175= .18
 3SIGNN 215= .15
 3SIGNN 255= .1
 3SIGNN 295= .06
 3SIGNN 335= .09
 3AMNEUT 1.008986
 3AMISO1 53.95664 55.95264 56.95342 57.95147
 3AMISO2 54.95565 56.95342 57.95147 58.9536
 3C1 931.162
 3P .0584 .9168 .0217 .0031 SIGA 2.3 2.7 2.5 2.5
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL +1 0 +1 0 0 0

NICKEL

3Z 28.
 4N 16 J 11 I 28 L 2 K 5
 3D .035 .07 .03 .06 .02 .01 .01 .02 .01 .01 .01 .01 .007
 .013 .006 .03 .004 .013 .01 .02 .09 .006 .004 .04 .06
 .025 .11 .26
 3E .28 .33 .436 .467 1.1 1.53 1.74 2.15 3.03 3.17 3.67
 4.05 4.20 5.31 5.7 5.82 5.99 6.1 6.34 6.58 6.839
 7.05 7.22 7.528 7.817 8.119 8.532 8.997
 3SNG 422= 1.33
 3SNG 452= 1.33
 3SNG 1623= 1.47
 3SNG 1653= 1.47
 3SIGNG 4.08 3.16695 1.89254 1.14819 .69567 .36488 .17346 .0626
 .02939 .33985 .0646 .02635 .01856 .01656 .01623 .01742
 3SIGNN 14= .3 .4
 3SIGNN 54= .67 .8
 3AMNEUT 1.008986
 3AMISO1 57.9538 59.949824 60.9497 61.9476 63.9481
 3AMISO2 58.9531 60.9497 61.9476 62.9495 64.9506
 3C1 931.162
 3P .678 .2623 .0125 .0366 .0115
 3SIGA 4.4 2.6 2. 15. 1.52
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL +1 0 +1 0 0 0

MOLYBDENUM

3Z 42.
 4N 16 J 11 I 0 L 3 K 7
 3SNG 7= 1.93 1.41 1.56 .21
 3SNNG 451= .73
 3SNNG 1653= 1.4
 3SNNG 2855= 2.5
 3SIGNG 2.2172 1.72124 1.02859 .62399 .37848 1.67959 5.05016
 5.33 .02 0 .01 0 0 .018 0 0
 3SIGNN 15= .66
 3SIGNN 55= .33
 3SIGNN 95= .059
 3AMNEUT 1.008986
 3AMISO1 91.9352 93.9343 94.9357 95.9349 96.9365
 97.9366 99.9383
 3AMISO2 92.9357 94.9357 95.9349 96.9365 97.9366
 98.9400 101.0173
 3C1 931.162
 3P .1586 .0912 .157 .165 .0945 .2375 .0962
 3SIGA .3 0 13.9 1.2 2.2 .4 .5
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL 0 0 +1 0 0 0

TANTALUM

3Z 73.
 4N 16 J 11 I 31 L 1 NOELEM 3 K 1
 3F .1/4 .3/4 .4/4 .82/4 .27/4 .4/4 .43/4 .54/4 .66/4
 .64/4 .63/4 .59/4 .525/4 .38/4 .29/4 .19/4
 .32/4 .16/4 .17/4 .09/4 .19/4 .08/4 .11/4
 .05/4 .08/4 .03/4 .07/4 .07/4 .23/4 .075/4 0
 3E .29 .4 .44 .5 .6 .9 1. 1.35 1.8 2.2 2.3 2.6 3. 4.
 4.48 4.58 4.68 4.8 4.88 5. 5.13 5.24 5.27 5.34 5.42 5.56 5.66
 5.7 5.8 6. 6.04
 3SNGG 360= .137
 3SNGG 390= .137
 3SNGG 420= .137
 3SNGG 450= .137
 3SIGNG 18.6135 15.4499 8.7599 5.47596 5.34633 299.05 103.429 31.5732
 8.2796 3.58136 1.1324 .3076 .21274 .15723 .10296 .04728
 3SIGNN 12= .75 1.2 1.8 1.9
 3AMNEUT 1.008986
 3AMISO1 181.0033 AMISO2 182.005
 3CI 931.162
 3P 1. SIGA 21.
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL -1 0 +1 0 +1

TA
 TA2
 FTA1
 FTA2
 FTA3
 FTA4
 ETA1
 ETA2
 ETA3
 SNGTA13
 SNGTA14
 SNGTA15
 SNGTA16
 XNGTA1
 XNGTA2
 XNNTA

ATA

PXTATA
 EG
 EN1
 EN2
 EN3

TA

TUNGSTEN

3Z 74.
 4N 16 J 11 I 42 L 2 K 5
 3F .2/4 .4/4 .68/4 .36/4 .8/4 .38/4 .55/4 .32/4
 .38/4 .33/4 .39/4 .45/4 .48/4 .53/4 .55/4
 .53/4 .43/4 .47/4 .4/4 .59/4 .31/4 .33/4 .28/4
 .37/4 .28/4 .48/4 .06/4 .12/4 .3/4 .96/4 .05/4
 .01/4 .07/4 .03/4 .3/4 .81/4 .04/4 .05/4
 0 0 .08/4 0
 3E .2 .4 .5 .56 .65 .78 .86 .9 1.06 1.17 1.35 1.5 1.8 2.2 2.4
 2.6 3. 3.1 3.34 3.45 3.6 3.75 4. 4.2 4.36 4.52 4.76 4.9 5.
 5.15 5.3 5.4 5.7 5.75 6. 6.1 6.2 6.3 6.7 7. 7.3 7.5
 3SNNG 330= .1014 .2082 .491 .473 .28 .172 .0557
 3SNNG 360= .1014 .2082 .491 .473 .28 .172 .0557
 3SNNG 390= .1014 .2082 .491 .473 .28 .172 .0557
 3SNNG 420= .1014 .2082 .491 .473 .28 .172 .0557
 3SNNG 450= .1013 .225 .678 .909 .724 .599 .459 .728 .195
 3SNNG 1530= .0516 .0365 .0568 .0694 .0499 .0163 .0047
 3SNNG 1560= .0516 .0365 .0568 .0694 .0499 .0163 .0047
 3SNNG 1590= .0516 .0365 .0568 .0694 .0499 .0163 .0047
 3SNNG 1620= .0516 .0365 .0568 .0694 .0499 .0163 .0047
 3SNNG 1650= .0535 .0374 .0835 .107 .0921 .0788 .0596 .0907 .0267
 3SIGNG 16.852 13.747 8.05 4.9 2.97498 21.125 20.1901 5.1414 .6121
 .4663 .1956 .0964 .0603 .0447 .0292 .0185
 3SIGNN 11= .35 1.2 2.2 2.57 2.59
 3SIGNN 51= .35 1.2 2.2 2.57 2.59
 3AMNEUT 1.008986
 3AMISO1 180.0017 182.0039 183.005 184.006 186.010
 3AMISO2 181.0140 183.005 184.006 185.0214 187.012
 3C1 931.162
 3P .002 .264 .144 .306 .284 SIGA 60. 20. 11. 2. 35.
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL -1 0 +1 0 +1 0

W
 W
 FW1
 FW2
 FW3
 FW4
 FW5
 FW6
 EW1
 EW2
 EW3
 SNNGW112
 SNNGW113
 SNNGW114
 SNNGW115
 SNNGW116
 SNNGW212
 SNNGW213
 SNNGW214
 SNNGW215
 SNNGW216
 XNGW1
 XNGW2
 XNNW1
 XNNW2
 AW
 AW+1
 PXTAW
 EG
 EN1
 EN2
 EN3

W

RHENIUM

3Z,75.
 4N 16 J 11 I 27 L 1 K 2
 3F .25/4 .68/4 .33/4 .42/4 .47/4 .52/4 .59/4 .62/4
 .6/4 .53/4 .47/4 .38/4 .38/4 .35/4 .29/4
 .35/4 .3/4 .15/4 .17/4 .05/4 .11/4 .09/4
 .22/4 .05/4 .06/4 .05/4 0
 3E .4 .46 .5 .9 1.35 1.6 1.8 2. 2.2 2.6 3. 3.53 3.75
 4. 4.64 4.96 5. 5.12 5.18 5.35 5.46 5.64 5.8 5.9 5.96 6. 6.12
 3SNG 330= .135
 3SNG 360= .135
 3SNG 390= .135
 3SNG 420= .135
 3SNG 450= .135
 3SIGNG 66.5 39.5 29.1 70.07 732.78 187.22 22.21 21.9443 7.89335
 2.924 1.039 .405 .14 .09274 .09568 .04285
 3SIGNN 11= .25 .6 .75 .8 .8
 3AMNEUT 1.008986
 3AMISO1 185.0209 187.011 AMISO2 186.011 188.016
 3C1 931.162
 3P .3707 .6293 SIGA 104. 66.
 3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.
 3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6
 8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1
 .4 .9 1.4 3. 10.
 3RME .510984
 3C 2.997923/10
 3TK 2.5/-8
 4KL 0
 -KTROL -1 0 +1 0 +1

REF
 REF
 REF1
 REF2
 REF3
 REF4
 REF1
 REF1
 SNGREF12
 SNGREF13
 SNGREF14
 SNGREF15
 SNGREF16
 XNGREF1
 XNGREF2
 XNNREF
 ARE
 PXTARE
 EG
 EN1
 EN2
 EN3
 RE

URANIUM 238

3Z 92.

4N 16 J 11 I 10 L 0 K 1

3D .09 .1 .08 .06 .02 .08 .08 .02 .02 .07

3E .54 .552 .577 .589 .611 .628 .637 3.576 3.662 4.062

3SFG 1.306 2.863 2.161 1.584 1.159 .906 .679 1.072 .452 .19 .015

3SIGNG 2.4388 1.89636 1.12501 .69 .41999 130.145 .141. 11. 2.01
.7 .45 .16 .14 .1 .05 0

3FSIG 13= .044 .485 .616

3AMNEUT 1.008986

3AMISO1 238.12522 AMISO2 239.12916

3C1 931.162

3P 1. SIGA 2.71

3BNDGAM .01 .4 .9 1.35 1.8 2.2 2.6 3. 4. 5. 7. 10.

3BNDNEU 0 3.216/-8 9.237/-8 2.511/-7 6.826/-7 1.855/-6

8.315/-6 6.144/-5 4.54/-4 3.355/-3 1.7/-2 .1

.4 .9 1.4 3. 10.

3RME .510984

3C 2.997923/10

3TK 2.5/-8

4KL 0

-KTROL +1 0 0 +1 0 0

REFERENCES

1. Ferry, M. S., "Production Cross Sections for Gamma Rays (Program GAMMA-P)," GE-ANPD, XDC 60-11-138, November 30, 1960.
2. Sullivan, W. H., "Trilinear Chart of Nuclides," ORNL, U. S. Government Printing Office, January 1957.
3. Hughes, D. J., and Schwartz, R. B., "Neutron Cross Sections," BNL, BNL - 325, Second Edition, July 1, 1958.
4. Hughes, D. J., Magurno, B. A., and Brussel, M. K., "Neutron Cross Sections," BNL, BNL - 325, Second Edition, Supplement No. 1, January 1, 1960.
5. Troubetzkoy, E., and Goldstein, H., "A Compilation of Information on Gamma-Ray Spectra Resulting from Thermal-Neutron Capture," NDA, ORNL - 2904, January 18, 1961.
6. Hall, H. E., and Bonner, T. W., "Gamma Radiations from Inelastic Scattering of Fast Neutrons in C^{12} , H^{14} , and O^{16} ," Nuclear Physics, Vol. 14, No. 2, 1959.
7. Day, R. B., and Walt, M., "Gamma Rays from Neutron Inelastic Scattering in B^{10} , F^{19} , and Fe^{56} ," Physical Review, Vol. 117, No. 5, March 1960.
8. Goldstein, H., "Fundamental Aspects of Reactor Shielding," Reading, Massachusetts, Addison-Wesley Publishing Company, Inc., 1959.
9. Robertson, C. S., and Benson, E. M., "Nine and Sixteen Group Cross Sections for Reactor Analysis," GE-NMPO, GEMP-173, January 7, 1963.
10. Tralli, N., "Some Neutron Cross Sections for Multigroup Calculations," NDA, APEX-467, June 30, 1958.
11. Montague, J. H., and Paul, E. B., "Cross Sections for the Production of Gamma Rays by the Inelastic Scattering of Neutrons from Iron," AERE, Harwell, England, Private Communication, 1960.
12. Price, B. T., Horton, C. C., and Spinney, K. T., "Radiation Shielding," Pergamon Press, 1957.
13. Groshev, L. V., Lutsenko, V. H., Demidov, A. M., and Pelekhov, V. L., "Atlas of γ -Ray Spectra from Radiative Capture of Thermal Neutrons," Pergamon Press, 1959.
14. Troubetzkoy, E. S., "Continuous Gamma Ray Spectra Following the Inelastic Scattering of High Energy Neutrons from Tungsten," NDA, NDA Memo 2129-1, April 4, 1960.
15. Howerton, R. J., "Semi-Empirical Neutron Cross Sections, 0.5 - 15 Mev, Part II," Lawrence Radiation Laboratory, UCRL - 5351, November 1958.
16. Zwick, J. W., "Rhenium Cross Sections," GE-ANPD, DCL 59-5-108, May 8, 1959.
17. Carver, J. G., " U^{235} Fission Gamma Ray Source Data," GE-ANPD, XDC 60-11-63, November 9, 1960.

ADVANCED TECHNOLOGY SERVICES

GENERAL  ELECTRIC