JOINT U.S. -EURATOM

EUR/AEC Report

1191

JOINT UNITED STATES
ATOMIC ENERGY COMMISSION
RESEARCH AND DEVELOPMENT PROGRAM

PROGRESS REPORT

MASTER

THE DEVELOPMENT AND TESTING OF UO₂
FUEL SYSTEMS FOR
WATER REACTOR APPLICATIONS

Period April 1 to July 31, 1964

> PATENT CLEARANCE OBTAINED. RELEASE TO THE CUBIO IS APPROVED. PROCEDURES PAR ON THE AN THE RECEIVING SECTION.

Issued August 1964

NUCLEAR DIVISION

COMBUSTION ENGINEERING, INC.

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency Thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

DISCLAIMER

Portions of this document may be illegible in electronic image products. Images are produced from the best available original document.

CEND-2863-217

JOINT U.S. -EURATOM

JOINT UNITED STATES

COMIC ENTRY OF THE

MAGELLE I LE LOS DESIGNATIONS -

THE TENTH OF THE PROPERTY OF T

CONTROL IN THE PROPERTY OF THE PARTY OF THE

2000000

A Commence of the Albertain

-LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission.

A. As tee any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, method, or process disclosed in this report may not infringe privately owned rights; or

B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method, or process disclosed in this record

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares disseminates, or provides access to, any information pursuant to bis employment or contract with the Corumission, or his employment with such contractor.

PROGRESS REPORT

THE DEVELOPMENT AND TESTING OF UO₂ FUEL SYSTEMS FOR WATER REACTOR APPLICATIONS

Period April 1 to July 31, 1964

Prepared by:

W. P. Chernock

C. E. Burdg

E. I. Veil

G. Zuromsky

J. J. Koziol

Contract AT(30-1)-2863
U. S. Atomic Energy Commission

Issued August 1964

Classification /

Authorized Classifier

Date

NUCLEAR DIVISION
COMBUSTION ENGINEERING, INC.
WINDSOR, CONNECTICUT

EXTERNAL DISTRIBUTION:	No. of Copies
Mr. M. Plisner, Contract Administrator U. S. Atomic Energy Commission New York Operations Office 376 Hudson Street New York, New York 10014	22
Including copies for:	
U.SEuratom Joint Research and Development Board 20 51 Rue Belliard Brussels, Belgium	
Mr. J. M. Simmons, Chief Fuels and Materials Development Branch U. S. Atomic Energy Commission Division of Reactor Development Washington, D. C. 20545	2
Mr. W. R. Voigt Water Reactors Branch U. S. Atomic Energy Commission Washington, D. C. 20545	3
Dr. R. C. Dalzell, Assistant Director for Foreign Activities	2
U. S. Atomic Energy Commission Division of Reactor Development Washington, D. C. 20545	
U. S. Atomic Energy Commission Division of Technical Information Extension Post Office Box 62	3
Oak Ridge, Tennessee 37831	
Mr. Harmon S. Potter, Chief New York Patent Group Brookhaven National Laboratory Upton, L.I., New York 11973	1

EXTERNAL DISTRIBUTION (Continued):	No. of Copies
Dr. J. Frye Metallurgy Division Oak Ridge National Laboratory	1
Post Office Box X Oak Ridge, Tennessee	
Dr. Stephen Lawroski Argonne National Laboratory 9700 South Cass Avenue Argonne, Illinois	4
Mr. T. Pashos General Electric Company Atomic Power Equipment Department Post Office Box 1.131 San Jose, California	1
Dr. R. Allio Westinghouse Electric Corporation Atomic Power Department Post Office Box 1075 Pittsburgh, Pennsylvania	1
Dr. Ersel A. Evans General Electric Company Hanford Atomic Products Operation Richland, Washington	1
Mr. L. R. Weissert Babcock and Wilcox Company Research and Development Division Nuclear Development Center Post Office Box 1260 Lynchburg, Virginia	. 1

THE DEVELOPMENT AND TESTING OF UO₂ FUEL SYSTEMS FOR WATER REACTOR APPLICATIONS

TABLE OF CONTENTS

		Page No.
List	of Figures	v
List	of Tables	vi
Intro	oduction	vii
Abst	ract	viii
I.	SUMMARY	1
II.	THERMAL PERFORMANCE CHARACTERISTICS OF UO ₂	3
III.	COLLAPSE CRITERIA	14
IV.	PROJECTION WELDING	20
REF	ERENCES	25

THE DEVELOPMENT AND TESTING OF UO₂ FUEL SYSTEMS FOR WATER REACTOR APPLICATIONS

LIST OF FIGURES

		Following
Figure No.	<u>Title</u>	Page No.
1	Sight Holes and Post-Test Sections	4
2	Comparison of Other Data With CEND Data	5 ,
. 3	Centerline Temperature History	6
4	Section 39-4 - Post Test Microstructures at Four Locations in a Section	6
5	Post Test Microstructures - Run No. 38	7
6	Post Test Microstructures - Run No. 39	. 7
7	Run 38 - Width of Grain Growth Region	7
8	Run 39 - Width of Grain Growth Region	7
9	Radial Temp. Distribution - Run No. 's 38 and 39	9
10	Typical Pellet ID Surface Temperature Determination	10
11	Typical Determination of Temperature for Onset of Grain Growth	12
12	Temperature for Onset of Grain Growth	10
13	Variation of Actual From Predicted Room Temperature Collapse Values for 3.175 cm Annealed Tubing	14
14	Collapse of Annealed Stainless Steel Tubes at	17

V

THE DEVELOPMENT AND TESTING OF UO₂ FUEL SYSTEMS FOR WATER REACTOR APPLICATIONS

LIST OF TABLES

Table No.	Title	Page No.		
· I	Temperature for Onset of Grain Growth	11		
II	Collapse Data	15		
III	Variation of Poisson's Ratio With Tangent Modulus	16		
IV	Dimensional Inspection of Tube-to-Tube Spacings	. 22		

INTRODUCTION

The United States and the European Atomic Energy Community (EURATOM) on May 29 and June 18, 1958, signed an agreement which provides a basis for cooperation in programs for the advancement of the peaceful applications of atomic energy. This agreement, in part, provides for the establishment of a Joint U.S.-Euratom research and development program which is aimed at reactors to be constructed in Europe under the Joint Program.

The work described in this report represents the Joint U.S.-Euratom effort which is in keeping with the spirit of cooperation in contributing to the common good by the sharing of scientific and technical information and minimizing the duplication of effort by the limited pool of technical talent available in Western Europe and the United States.

THE DEVELOPMENT AND TESTING OF UO₂ FUEL SYSTEMS FOR WATER REACTOR APPLICATIONS

Вy

W. P. Chernock, C. E. Burdg, E. I. Veil, G. Zuromsky and J. J. Koziol

ABSTRACT

The thermal conductivity of UO_2 at temperatures up to $2400^{\circ}C$ is reported together with an analysis of the temperature for onset of different microstructural constituents. Collapse testing of 3.175 cm diameter tubing with t/R_0 values of 0.046 to 0.119 was completed. An analysis of collapse criteria indicates that use of the Tangent Modulus approach provides conservative estimates within 20% of actual collapse pressures within the range of diameters and tube wall thickness tested. Further refinements in the projection welding process have demonstrated dimensional control of tube-to-tube spacings within specified requirements. Tube-to-spacer wire welds are of acceptable metallurgical quality.

I. SUMMARY

Thermal conductivity of UO₂ was determined for temperatures up to 2400°C. A minimum in the conductivity curve was found at 1400°C with values ranging from 0.025 to 0.033 watts/cm-°C. A distinct increase in the thermal conductivity was found for temperatures in excess of 1400°C.

The range of values at 2000 and 2400°C are 0.034 to 0.046 and 0.048 to 0.059 watts/cm-°C. Onset of small equiaxed grain growth (with grain diameters greater than 15 \(\mu \)) was observed at temperatures between 1400 and 1700°C.

A difference in radial thermal gradient by a factor of approximately two had little influence on the temperature of onset of small equiaxed grain growth.

The onset of massive, blocky grain growth occurred at 2030 to 2190°C and 1940 to 2080°C for the lower and higher radial thermal gradient samples.

Although an overlap in the data from these elements is evident, the lower temperatures for onset with the higher radial gradient may be significant.

The collapse test program on 3.175 cm diameter tubing was completed and results have been compared with those previously obtained for smaller diameter tubing. In the range of $t/R_{\rm O}$ (thickness to radius) and tube diameters tested, the Tangent Modulus Criterion consistently predicted conservative collapse pressures within 20% of the actual values. There appears to be a trend toward better agreement of actual and predicted collapse on the basis of the Inelastic Model at higher $t/R_{\rm O}$ as the tube diameter increases.

Additional 2-row, 17-tube subassemblies of Type 347 stainless steel were fabricated utilizing improved electrode tooling and fixturing. Tube-to-tube spacings in the two rows of the subassemblies were maintained within

the required range of 1.587 ±0.008 cm (0.625 ±0.003 in.). Lateral displacement or skewing of rows of tubes has been eliminated. However, the diagonal spacings between tubes in the two rows were on the high side of the nominal value of 1.587 cm. A reduction in this spacing dimension will be accomplished by an increase in spacer wire setdown at the weld joints from 0.015 to 0.020 cm (0.006 to 0.008 in.). The control and reproducibility of tube-to-tube dimensions will be demonstrated by the fabrication of a final evaluation subassembly prior to projection welding of the full size 61-tube cluster.

II. THERMAL PERFORMANCE CHARACTERISTICS OF UO₂ - C. E. Burdg J. J. Koziol

A. Introduction

The objectives of this Task are to determine the thermal conductivity of UO₂, the effective thermal conductivity of a typical fuel element system, and the relationships between microstructure and thermal parameters. Thermal performance testing attempts to simulate, out-of-pile, thermal conditions in a fuel element during irradiation. Successful attainment of these objectives will provide more confidence in UO₂ conductivity values and in the utilization of microstructural analysis as a means of deducing the thermal history of an element.

The last Quarterly Report¹ contained the results of thermal testing of four elements and a composite curve for thermal conductivity which exhibited a distinct upturn at UO₂ temperatures above about 1500°C. A further interesting result was that the "effective conductivity" for the fuel element system did not exhibit significant change when the centerline temperatures were above 500°C; at lower temperatures, the effect of reduced pellet/clad contact pressure resulted in low effective conductivity for the system.

Two compound test elements, consisting of a UO₂ pellet (with its axial heater) surrounded by an insulating sleeve of ZrO₂, were tested to determine the influence of temperature gradient on the system. A third compound element has been prepared but was not tested.

A major effort during this reporting period has been the analysis of thermal and microstructural data with the intent of determining the relationships between microstructure and thermal parameters. This has involved

making detailed measurements on over 250 photomicrographs, and development of interpretative procedures. The results of these analyses are discussed.

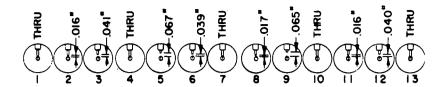
B. Test Conditions

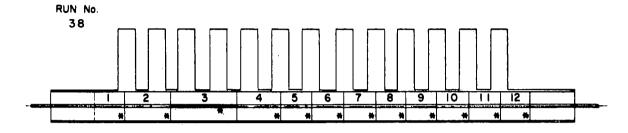
1. Run #39

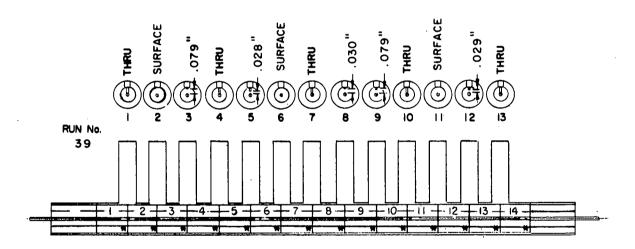
Run #39 was made with a compound element, consisting of a UO₂ pellet surrounded by a ZrO₂ insulating sleeve. As described in the last Quarterly Report¹, the purpose of this configuration was to provide a radial temperature profile which would be significantly different in shape from that existing in a simple all-UO₂ element. Because ZrO₂ has a thermal conductivity of about 0.015 watts/cm-^oC, which is only about half that of UO₂, the profile would be expected to be less steep in the UO₂ than in previous tests; the reduction of the gradient is further enhanced by the larger overall diameter of the system. Thus, the effect, if any, of a lower temperature gradient on the microstructure would be demonstrated; and second, the radial position of a given isotherm would be farther from the heater.

Figure 1 shows the compound element configuration, and the sight hole and sectioning plan for element #39. Typical dimensions of the various parts of the compound elements are summarized below:

Component	Material	Dimensions
Clad	Type 304 SS	1.905 cm (0.750 in.) OD x
•		0.038 cm (0.015 in.) wall
Insulating	ZrO_2	1.826 cm (0.719 in.) OD x
sleeve		0.866 cm (0.341 in.) ID
Pellet	UO ₂	0.864 cm (0.340 in.) OD x
•	_	0.193 cm (0.076 in.) ID
Heater	W-26 Re	0.190 cm (0.075 in.) OD







*-THIS SURFACE PREPARED FOR METALLOGRAPHY

SIGHT HOLES AND POST-TEST SECTIONS

2. Run #40

This run was with an element essentially identical to #39.

The run was unsuccessful, in that the axial temperature variation as compared to the temperature variation of the other elements tested was too large to be considered acceptable. An analysis of test conditions failed to establish the cause of this anomalous behavior.

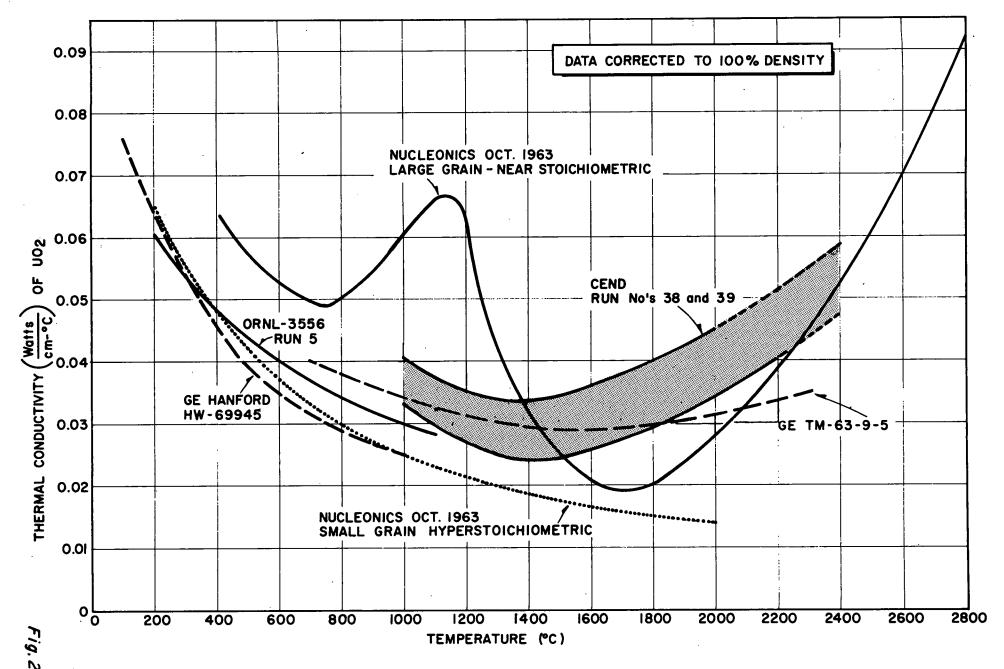
C. Results

1. Thermal Conductivity of UO2

Knowledge of the behavior of the thermal conductivity of UO₂ as a function of temperature is imperative for the development of a radial temperature distribution for the UO₂ pellets. Having established the temperature variation as a function of radial position in the UO₂, an attempt was made to determine the influence of temperature and thermal gradient on the resulting microstructure.

Thermal conductivity data for sintered polycrystalline UO₂ as a function of temperature up to approximately 1000°C is well documented with close agreement among investigators. Agreement on the findings for conductivity versus temperature above 1000°C is not nearly as good. This is illustrated in the composite curve (Figure 2) constructed from data published by Daniel² et al, Godfrey³ et al, de Halas⁴, and Feith⁵, along with the CEND data for Runs 38 and 39.

The thermal conductivity curves for runs 38 and 39 result from calculations based on a constant interface gap (between heater and fuel) conductance of 2 watts/cm²-OC and optical temperature measurements made



COMPARISON OF OTHER DATA WITH CEND DATA

by sighting into holes at various radial distances from the heater. The data points obtained from these calculations for runs 38 and 39 fall into the band shown in Figure 2.

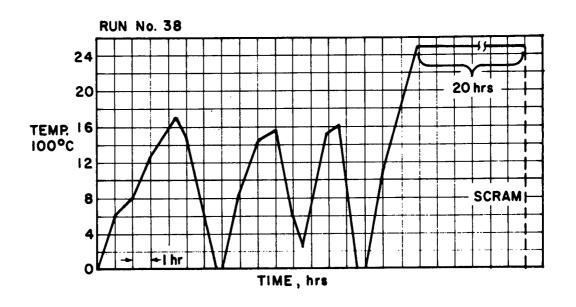
This Figure indicates reasonable agreement among investigators at temperatures near 1000°C. At higher temperatures (above 1400°C), CEND's values for the thermal conductivity of UO₂ increase at a greater rate than do the calculated values published by de Halas⁴ for small grain hyperstoichiometric UO₂ (no change to positive slope) but do not increase at as great a rate as do the values for large-grain, near-stoichiometric UO₂. The CEND values also increase at a greater rate than do Feith's values which show a minimum at approximately 1600°C.

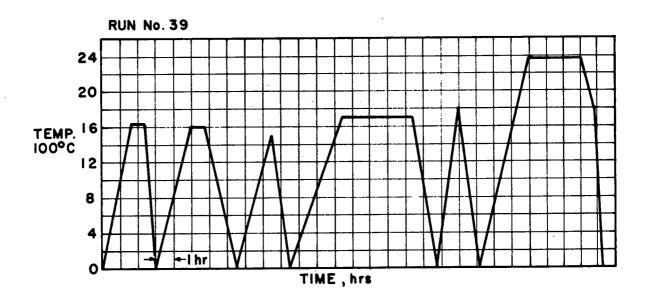
2. Metallographic Analysis

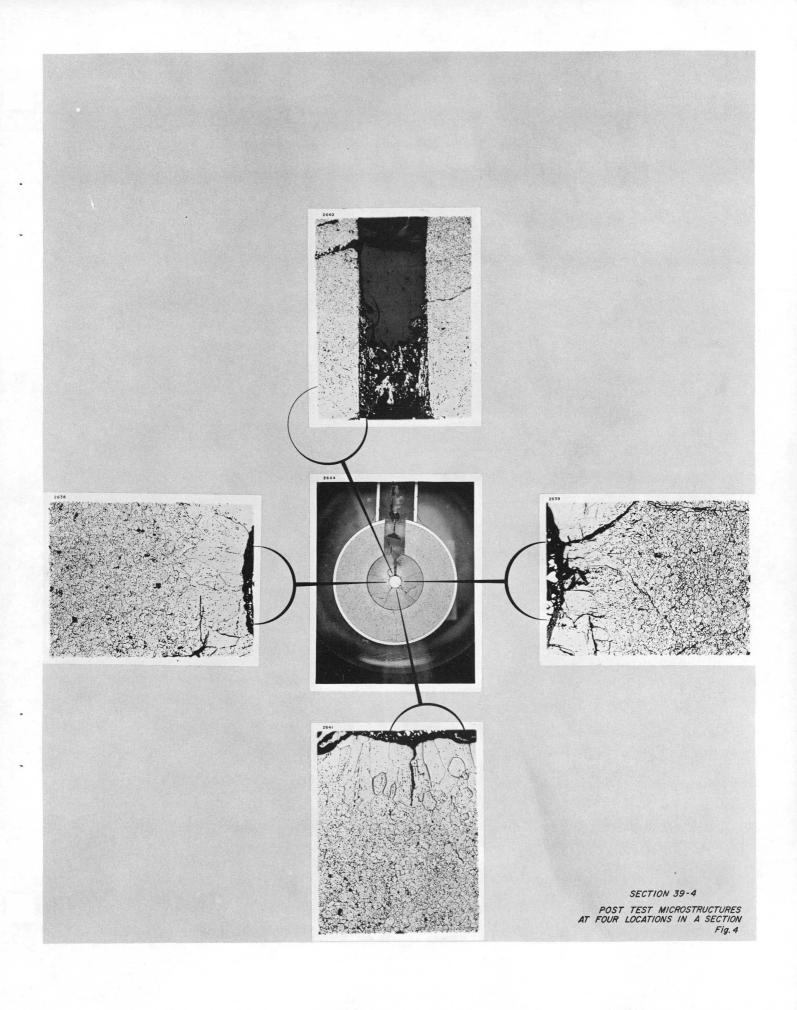
Elements 38 (uninsulated UO₂ pellets) and 39 (compound element with insulated pellets) were run in the thermal test apparatus for times and temperatures shown in Figure 3. Each element was then sectioned for metallographic examination in accordance with the sectioning plan shown in Figure 1.

Photomacrographs were taken of the entire section to show the post-test appearance of the UO₂ pellet and the tungsten-rhenium heater rod. Photomicrographs were then taken at four positions around the heater rod, 90° apart, as shown in Figure 4.

Analysis of the photomicrographs showed the anticipated large changes in grain size of the UO₂. These changes resulted in the formation, adjacent to the heater rod, of massive grains which, in some instances, were as large as 100 to 200 microns in diameter. Typically, the grains decreased







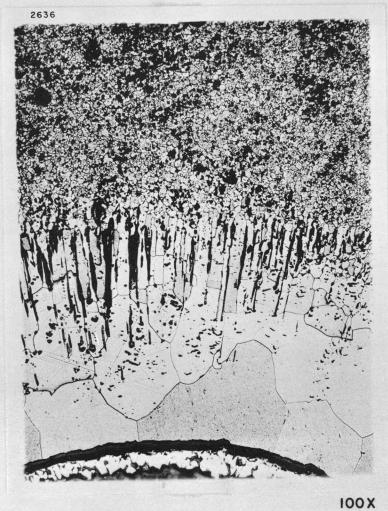
in size with increasing radial distance from the heater until the as-sintered grain size (approximately 10 microns diameter) was reached. Occasionally columnar grains of various sizes were encountered. These seemed to occur more frequently in element 39 (compound element with insulated pellets) than in element 38 (uninsulated pellets), but in general, were not present in all sections. Photomicrographs of typical structures from elements 38 and 39 are shown in Figures 5 and 6.

Elements 35, 36 and 37, previously reported¹, along with elements 38 and 39 were sectioned and photographed. Measurements of more than 250 photomicrographs were made to the radial distance at which onset of small equiaxed massive grains (greater than about 100 microns diameter) occurred. Element 38 was typical of the elements made from uninsulated UO₂ pellets (elements 35 to 38). Data from this element are compared with those from element 39, a compound element made from ZrO₂ insulated UO₂ pellets. Results of these measurements for elements 38 and 39 as a function of axial position are shown in Figures 7 and 8. The inside diameter of the pellet was used as the reference for measurements.

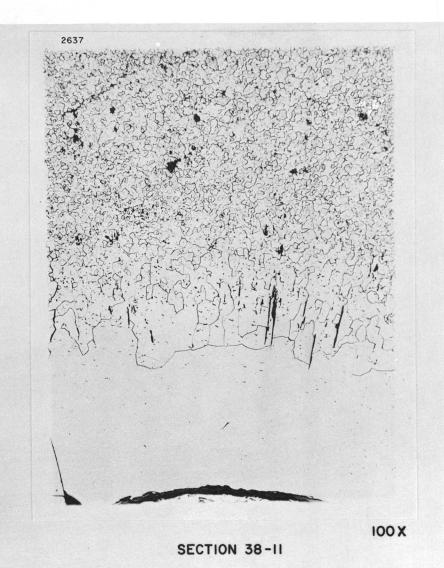
3. Temperature for Onset of Grain Growth

a. Radial Temperature Distribution

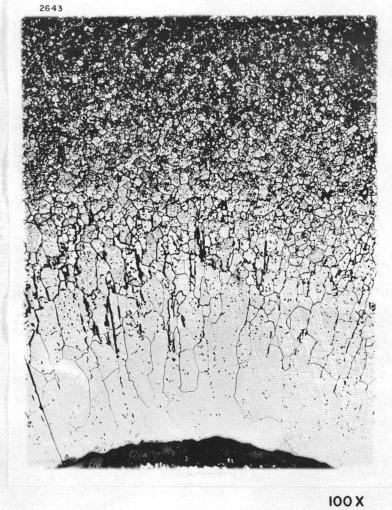
In an attempt to determine the temperature for the onset of small and massive grain growth, a radial temperature distribution for the uninsulated and insulated UO₂ elements was determined. Values from Figure 2 for thermal conductivity as a function of temperature were used and the following assumptions were made:



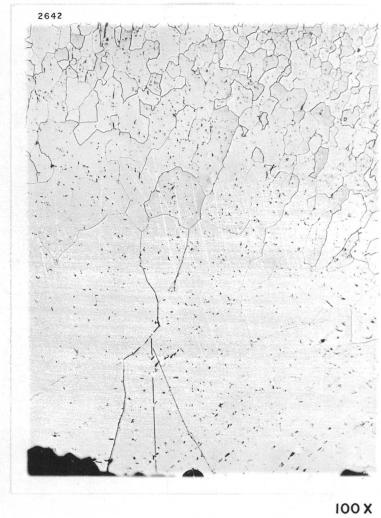
SECTION 38-8



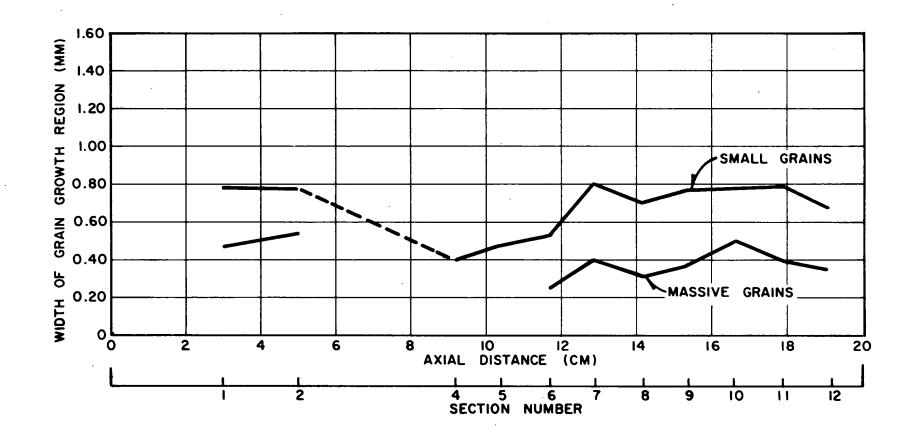
POST TEST MICROSTRUCTURES - RUN No. 38

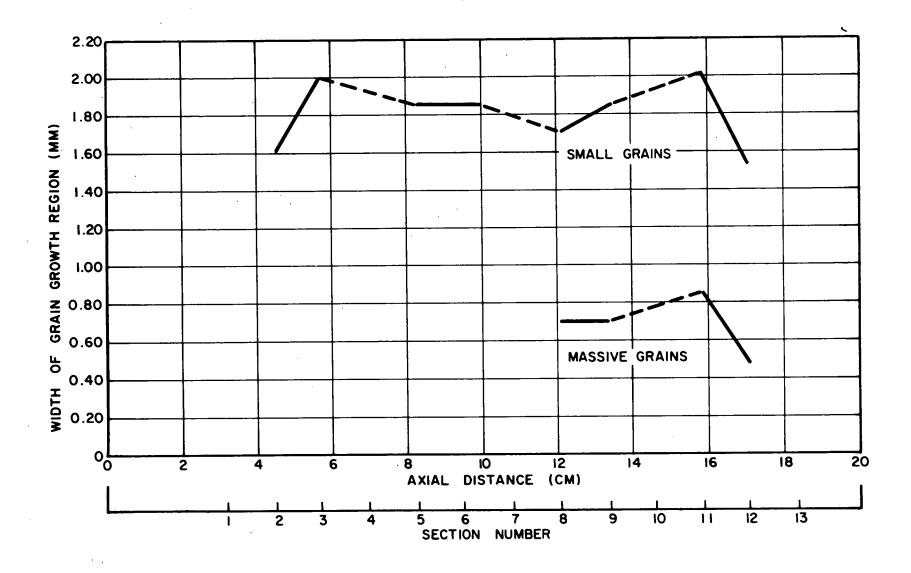


SECTION - 39-8



SECTION - 39-II





- (1) The UO₂ pellets were in contact with the tungsten-rhenium heater rod.*
- (2) The gap (between heater rod and fuel pellet)

 conductance was a constant value of 2 watts/
- (3) The contribution of cracks and voids to the thermal resistance of the UO₂ was insignificant.

In addition, the outer surface of the UO_2 was assumed to be in contact with the clad for the uninsulated elements and in contact with the ZrO_2 insulator for the compound element.

Knowing the average temperature of the heater, the geometry of the element and the power required to maintain a given temperature within the element, the ID surface temperature of the UO₂ pellet was derived.

Calculations of the temperature distribution within the UO2 pellet were made by applying the integrated form of Fourier's law for heat transmission by conduction across a homogeneous material.

$$Q = \frac{k2\pi 1\Delta T}{\ln \frac{R_2}{R_1}}$$
 (1)

where Q = heat flux

k = thermal conductivity

1 = length of heat transfer surface

^{*} An analysis of post-test metallography indicated, in some sections, mechanical attachment of the heater rod surface to the pellet inside diameter. In addition, there was some evidence of a change in heater rod shape resulting from heater-fuel mechanical interaction.

△ T = temperature drop across a given segment of the material

 R_1 and R_2 = radii of inner and outer surface within which a temperature drop, ΔT , occurs.

Since the heat flow across any radial segment is constant and the temperature drop is proportional to the thermal resistance of that segment, equation (1) can be put in a form from which the temperature drop across an individual radial segment can be determined.

Thus
$$\Delta T_{\text{total}} = \Delta T_1 + \Delta T_2 + \Delta T_3 \dots$$
 etc. (2)

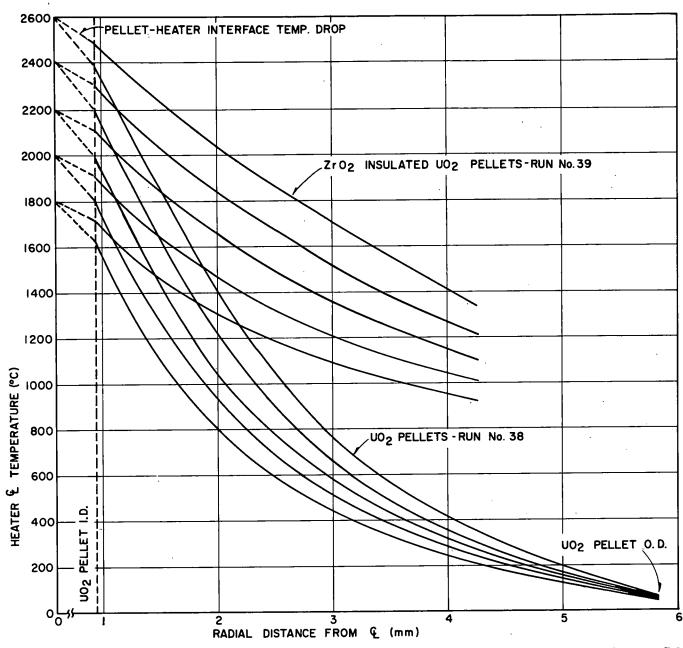
and $\Delta T_1 = \frac{\ln \frac{R_2}{R_1}}{\ln \frac{R_2}{R_1}} + \frac{\ln \frac{R_3}{R_2}}{\ln \frac{R_4}{R_3}} + \frac{\ln \frac{R_4}{R_3}}{\ln \frac{R_4}{R_3}} \dots$ etc.

where k_{m_1} = mean thermal conductivity for the average temperature of a segment having radii, R_1 and R_2 , as physical boundaries.

The radial temperature distribution curves were then determined from an assumed temperature profile within a UO₂ pellet. A series of iterations were performed until the assumed radial position for a given temperature drop was identical to the calculated radial position for the same temperature drop.

The results of these operations are shown in Figure 9.

It can be seen that the thermal gradient for the insulated pellets (Run 39) at any



RADIAL TEMP. DISTRIBUTION - RUN No's 38 and 39

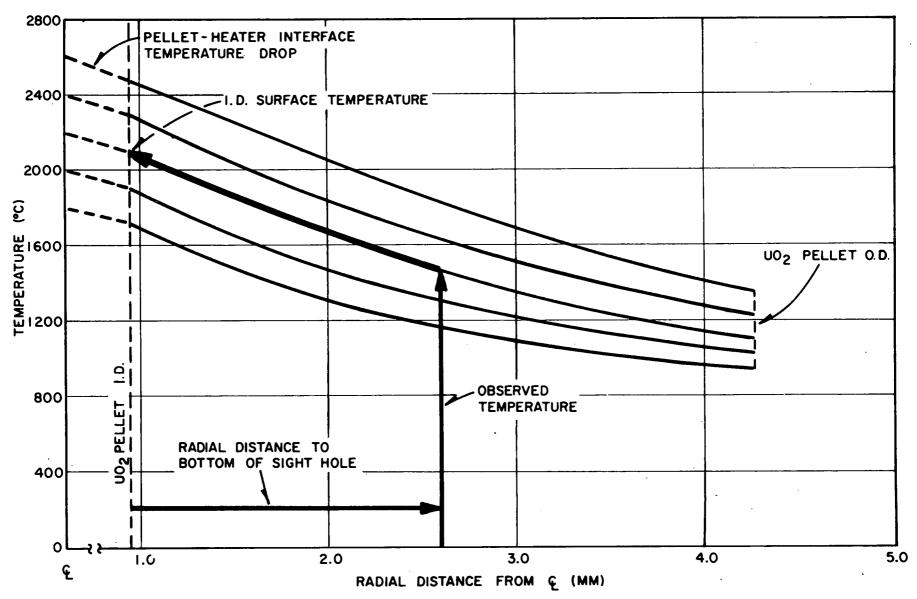
Fig. 9

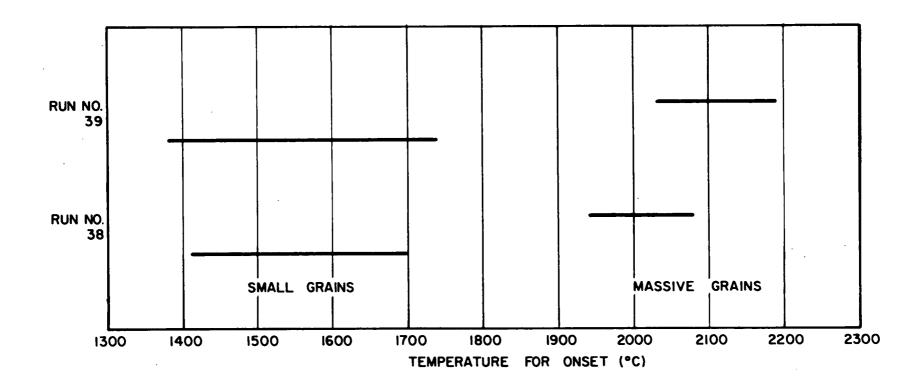
point is approximately one-half the gradient in the uninsulated pellets (Run 38). The lower thermal gradient of element 39 results in higher temperatures at any given radial location within the UO₂ pellet. This gradient also more closely approximates the in-pile thermal gradient for UO₂ and, as seen in Figures 7 and 8, the grain growth region in element 39 (insulated) is about twice as wide as the grain growth region in element 38 (uninsulated). Thus, measurements of the locations of microstructural change were less difficult.

b. Determination of Temperature for Onset of Grain Growth

Using the radial temperature distribution curves (Figure 9), the measurements of distance from pellet edge to the microstructural change (Figures 7 and 8), and temperature prior to final cooldown for the section of interest, the temperature for onset of grain growth (grains with an approximate grain diameter of greater than 15 microns) and for onset of massive grains (approximate grain diameter greater than 100 microns) was determined.

The following procedure was used in determining the range of temperatures shown in Figure 12 for onset of grain growth constituents. Actual data used are shown in Table I. The optically measured temperatures (sightings to various radial positions within the UO₂) were used to enter the radial temperature distribution curves. Using the radial position of the sighting and the observed temperature for that radial position as input data (Figure 10), the intersection of these two values fell on a radial temperature profile curve from which the temperature of the pellet ID surface was obtained by following the curve back to the ordinate.





TEMPERATURE FOR ONSET OF GRAIN GROWTH

TABLE I
TEMPERATURE FOR ONSET OF GRAIN GROWTH

	Sighthole Number	Section Number	Dist. from	Observed Temperature (^O C)	Pellet I Size	from D to Grain Change m) Massive Grains	Tempe for C (^O) Small Grains	erature Onset C) Massive Grains
RUN 38	6 9 12	4 7 10	1.97 2.60 1.97	956 1060 1485	0.40 0.80 0.78	0.40 0.50	1410 1700 1680	 2080 1940
RUN 39	2 3 5 6 8 9 11	2 3 5 6 8 9 11	4.27 2.96 1.69 4.27 1.69 2.96 4.27	1176 1460 1740 1068 2011 1690 1411	1.61 2.00 1.85 1.85 1.70 1.85 2.15	 0.70 0.70 0.85	1610 1460 1380 1380 1680 1740 1740	 2030 2150 2190

. |-|This temperature determined a specific temperature profile within the pellet. The temperature of onset of a particular microstructure was determined by moving along the temperature profile curve determined by the pellet ID surface temperature for the section (Figure 11), to the intersection of the measured radial distance for the microstructure of interest. The intersection of these two values established the temperature for onset of the microstructure of interest.

Figure 12 shows that the temperature range for onset of grain growth for the UO₂ in the uninsulated (element 38) and insulated (element 39) elements are approximately the same, the range for the former element being slightly narrower. The temperature range for onset of massive grains appears to be lower in the element with the higher temperature gradient, although overlap of data exists.

D. Conclusions

It is concluded that the thermal conductivity of sintered UO₂ pellets decreases with temperature to a value between 0.025 and 0.033 watts/cm²-°C at about 1400°C and then rises to about 0.034 to 0.046 watts/cm²-°C at 2000°C and 0.048 to 0.059 watts/cm²-°C at 2400°C. These results, compared to the low temperature results reported by Daniel² et al, and Godfrey³ et al, show reasonable agreement at around 1000°C. At higher temperatures, the shape of the CEND conductivity versus temperature curve is similar to that reported by Feith⁵, although the increase in conductivity with temperature after passing through a minimum is greater for the CEND data.

It was found that grain growth initiated at temperatures in the range of 1400 to 1700° C and that growth of massive grains occurred at temperatures

around 2000°C. A trend toward lower temperatures for the onset of massive grain growth was observed at higher thermal gradients, although overlap in the data existed. The temperature ranges reported for the onset of small and massive grain growth are important benchmarks for post-irradiation evaluation of UO₂ irradiations under typical present-day power reactor conditions where the specific power in the element is not sufficient to produce large columnar grains or melting.

E. Future Work

An additional element of the compound (element 39) configuration will be tested in an attempt to verify the shape and position of the thermal conductivity curves and to more closely establish the temperature for onset of grain growth. The latter objective will be achieved by providing more sight holes in the UO₂ at radial positions near those calculated for the onset of microstructural constituents.

III. COLLAPSE CRITERIA

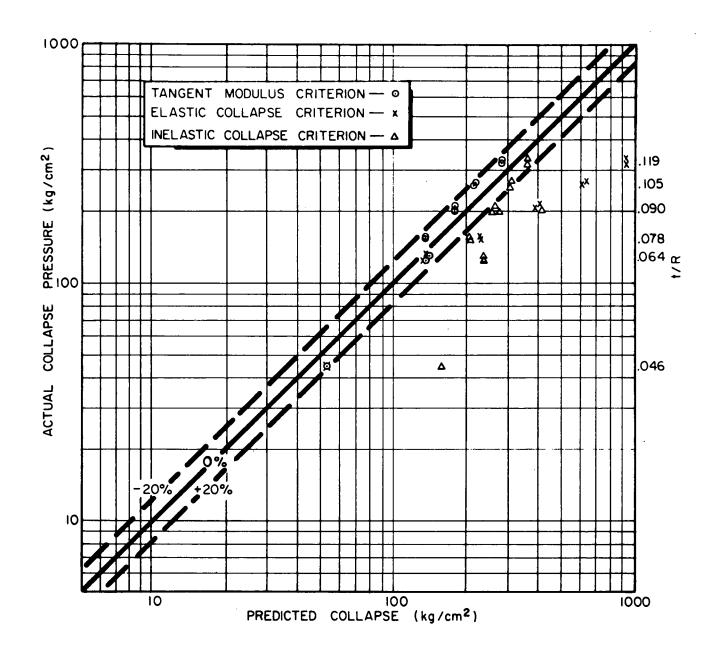
A. Introduction

In previous work at CEND the Tangent Modulus Criterion closely approximated the collapse pressure in stainless steel tubing with diameters up to 1.9 cm (0.750 in.) and tube thickness to radius ratios (t/R) up to 0.012. The present program was initiated to determine whether this criterion can be applied to tubing with diameters up to 3.175 cm (1.250 in.). Type 347 weldrawn annealed tubing, 3.175 cm (1.250 in.) in outside diameter with six different wall thicknesses in the range of 0.07 to 0.19 cm (0.028 to 0.075 in.) was procured for this investigation. The experimental effort and results are discussed below.

B. Experimental Effort

Two collapse test specimens were fabricated for each of the six different wall thicknesses (0.028 to 0.075 in.) of the 3.175 cm diameter tubing. The specimens were 63.5 cm in length which represented a length to diameter ratio of 20. The collapse data are shown in Table II.

Figure 13 is a plot of the actual collapse pressure versus the predicted collapse pressure for the Tangent Modulus, Elastic, and Inelastic Criteria. In this Figure the solid line indicates complete agreement between actual and predicted values; a range of $\pm 20\%$ from this line is also shown. In using the Tangent Modulus Criterion, the value of Poisson's ratio was set as a function of the tangent modulus (E_t), since Poisson's ratio is greater for plastic deformation than for elastic strain. As E_t decreased, Poisson's ratio increased from 0.3 to 0.5 as indicated in Table III.



VARIATION OF ACTUAL FROM PREDICTED ROOM TEMPERATURE COLLAPSE VALUES FOR 3.175 Cm. ANNEALED TUBING

TABLE II - Collapse Data

												Actual Collapse
			Avg.						Collapse	Predicted	Predicted	Pressure
			Wall					Actual	by	Collapse	Collapse	Normal-
		•	Thick-	Avg.			Avg. 0.2%		Tangent	by Elastic	by Inelastic	ized
	Sample	Mate-	ness	OD	t (a)	Oval-	Offset Y.S.	Pressure	Modulus (c)	Criterion(d)	Criterion ^(e)	to 2660 ₂
	No.	rial	(cm)	(cm)	$\overline{R_o}$	ity(b)	(kg/cm^2)	(kg/cm^2)	(kg/cm^2)	(kg/cm^2)	(kg/cm ²)	(kg/cm²)
					:							
	CT-A-28-1	347	.073	3.177	.046	.10	3197	45.5	53.2	53.2	154.5	(f) ··
	CT-A-28-2	347	.072	3.175	.045	.10	3197	44.8	5 3. 2	53.2	154.5	(f)
	CT-A-40-l	347	.101	3.172	.064	.04	3358	131.3	139.7	135.5	232.1	104.0
	CT-A-40-2	347	.101	3.172	.064	.04	3358	127.8	139.7	135.5	232.1	101.2
	CT-A-49-1	347	.123	3.177	.078	.20	2511	157.5	135.1	225.3	202.1	166.8
	CT-A-49-2	347	.124	3.176	.078	.28	2511	155.8	135.1	225.3	200.7	165.0
	CT-A-56-1	347	.144	3.178	.091	.04	2688	211.8	176.8	404.3	263.4	209.7
	CT-A-56-2	347	.143	3.176	.090	.10	2688	199.5	175.0	391.0	256.3	197.5
	CT-A-56-3	347	.143	3.176	.090	.02	2688	204.8	175.0	391.4	265.6	202.8
_	CT-A-65-1	347	.166	3.175	.105	.02	2657	266.0	213.5	619.2	_, 303.1	266.0
ינ	CT-A-65-2	347	.166	3.176	.104	.06	2657	264.3	211.1	601.7	297.2	264.3
	CT-A-75-l	347	.188	3.170	.119	.04	2717	329.0	274.8	904.4	347.8	322.1
	CT-A-75-2	347	.188	3.171	.119	.06	2717	325.5	274.8	904.4	343.3	318.7

- (a) t/R_0 = Average wall thickness + Radius.
- (b) Dmax Dmin/Dmin · 100%; where D = Outside diameter.
- (c) $P_c = \frac{E_t}{4(1-a^2)} (t/R_o)^3$; $E_t = Tangent Modulus$, a = Poisson's RatioReference: "The Development and Testing of UO₂ Fuel Systems for Water Reactor Applications," CEND-157, June 1962.
- (d) $P_c = \frac{E}{4(1-a^2)} (t/R_0)^3$; where a = 0.284, and E = Young's Modulus.

(e)
$$P_{C} = \frac{(1 - B) \sigma_{y} (1 - R_{i}^{2}/R_{o}^{2})}{\sqrt{3}}; \text{ where } B = \text{Percentage reduction in collapse strength due to out-of-roundness.}$$

$$\sigma_{y} = 0.2\% \text{ offset yield strength of the material.}$$

$$R_{i} = \text{Inside radius.}$$

(f) Collapse is elastic, therefore, independent of yield strength in this range.

TABLE III

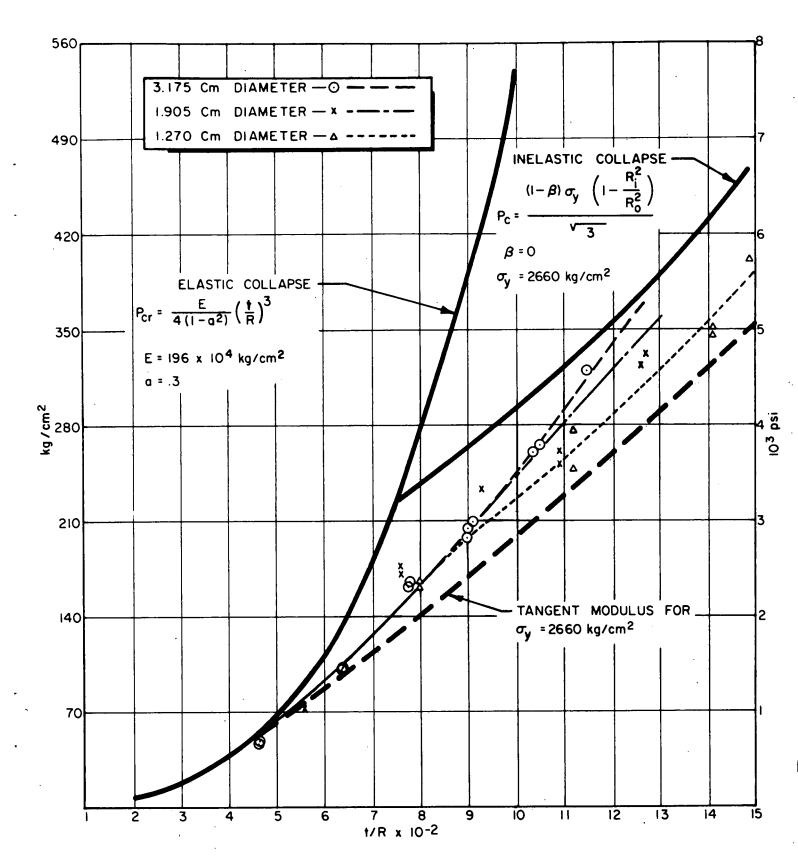
Variation of Poisson's Ratio With Tangent Modulus

Poisson's Ratio	Tangent Modulus (10 ⁴ kg/cm ²)
0. 20	210
0, 30	210
0,315	203
0.33	1 96
0.345	189
0.36	182
0,37	175
0.38	168
0.39	161
0.40	154
0.41	147
0.42	140
0,43	133
0.44	126
0.45	119
0.475	112
0.50	7 - 105

The following conclusions were based upon data presented in Figure 13.

- 1. With the exception of data points represented by the Tangent Modulus Criterion, all data fell below the indicated straight line relationship between actual and predicted collapse pressures. Thus, collapse pressures predicted by either the Elastic or Inelastic Collapse Criteria are greater than those actually observed.
- 2. For the range of t/R investigated, data points represented by the Tangent Modulus Criterion consistently fell within the band of ±20% surrounding the indicated straight line relationship. With one major exception at a t/R of 0.046, all deviations from the solid line were in the direction of larger measured collapse pressures than those calculated. Thus, values obtained through application of the Tangent Modulus Criterion are considered to be conservative.
- 3. The Elastic Criterion provides a reasonably close relationship between actual and predicted collapse up to a t/R of 0.060. As anticipated (see Figure 14) in the region beyond the transition between Elastic and Inelastic behavior, the Inelastic Criterion becomes more accurate at t/R is increased.
- 4. Except for the range of t/R from 0.060 to 0.115*, the combined use of the Elastic and Inelastic Criteria, in which the lowest calculated collapse pressure is employed, provides a reasonably accurate prediction of actual collapse pressures. However, as noted above, all deviations are in the direction of predicting lower than actual collapse pressures.

^{*} By extrapolation from Figure 14.



COLLAPSE OF ANNEALED STAINLESS STEEL TUBES AT ROOM TEMPERATURE

5. For the entire range of t/R investigated, the Tangent Modulus Criterion provides the most conservative estimate of collapse pressures. In all cases the difference between actual and predicted collapse, using this Criterion, was less than 20%.

Figure 14 shows a plot of the collapse pressures for the room temperature 3.175 cm outside diameter annealed tubing normalized to a yield strength of 2660 kg/cm² (38,000 psi) as a function of t/R₀ (thickness to outside radius). Included for comparison, are data points generated on an earlier program^{6,7} for 1.270 and 1.905 cm (0.500 and 0.750 in.) outside diameter annealed tubing. Data points were not corrected for ovality since the influence of ovality on tube collapse was not observed until the ovality exceeded 1% in 1.270 and 1.905 cm diameter tubing and 0.3% in 3.175 cm tubing. (The greatest ovality observed in the 3.175 cm tubing was 0.3%.) Also plotted are the predicted collapse curves based on the three collapse criteria.

Data shown in Figure 14 not only substantiate the conclusions made above for the 3.175 cm diameter tubing, but also indicates that these conclusions are generally applicable to the range of tube diameters investigated.

The Tangent Modulus Criterion provides a close approximation of actual collapse for the 1.270 cm diameter tubing. As the tube diameter increases, there is poorer agreement between actual collapse pressures and those predicted from the Tangent Modulus Criterion. As the cladding diameter increases, actual collapse pressures approach those predicted by the Inelastic Model at lower t/R values. With only one exception (t/R = 0.046), all measured collapse pressures fell below those predicted by either the Elastic or Inelastic Collapse Criteria. On the other hand, the Tangent Modulus Criterion provided

a reasonably accurate, but conservative, indication of collapse pressure; the agreement for the smaller tubing (typical of boiling- and pressurized-water cladding) was particularly good.

IV. PROJECTION WELDING

A. Introduction

This task has the following objectives:

- 1. To refine the projection welding method for the fabrication of 61-tube hexagonal clusters, 7 feet in length, of Type 347 stainless steel and Zircaloy-2. The feasibility of the method was established earlier in this program.
- 2. To test and evaluate full size clusters fabricated from the two materials for differential thermal expansion behavior to establish overall structural soundness and dimensional integrity of the weldments.

Current program activity has centered on improvements in the fabrication process to develop optimum quality tube-to-spacer wire welds and closer dimensional control of tube-to-tube spacings. Further modifications were made in electrode tooling and cluster fixturing with emphasis on control of alignment and positions of assembled tubes during welding.

B. Experimental Effort

Two additional subassemblies, each consisting of two rows of eight and nine tubes of Type 347 stainless steel, were fabricated at Sciaky Brothers, Inc. and evaluated at CEND. The tubes were 1.2700 cm ±0.0013 cm (0.5000 ±0.0005 inch) diameter by 0.051 ±0.005 cm (0.020 ±0.002 inch) wall thickness and 45.7 cm (18 inches) in length. The spacer wires joining the two rows of tubes were 0.1575 ±0.0013 cm (0.0620 ±0.0005 inch) in diameter. Both materials were in the bright annealed condition.

The results of dimensional measurements of tube center-to-tube center distances are included in Table IV. The inspection data show that distances between adjacent tubes in both rows of the two subassemblies were maintained within the required range of 1.587 ±0.008 cm (0.625 ±0.003 inch). Spacings for the end tubes on both sides of the second subassembly were out of specification. Cross wires had not been welded to the outside of this subassembly and it is probable that the unrestrained side tubes may have rolled during shipment.

Lateral displacement or skewing between rows of tubes, reported for previous subassemblies¹, has been essentially eliminated by further improvements in fixturing of rows of tubes. However, the diagonal spacing between adjacent tubes was consistently on the high side of the nominal dimension of 1.587 cm (0.625 inch). This diagonal tube center to tube center spacing may be expressed as follows from data of Table IV.

Spacings at one end
$$1.597^{+0.005}_{-0.003}$$
 cm or $0.629^{+0.002}_{-0.001}$ inch

Spacings at opposite end
$$1.597^{+0.010}_{-0.003}$$
 cm or $0.629^{+0.004}_{-0.001}$ inch

In order to attain the desired 1.587 ±0.008 cm (0.620 ±0.003 inch) dimension, a reduction of about 0.010 cm (0.004 inch) is required in the spacing between tube rows. This will be accomplished by increasing the spacer wire setdown, i.e., reduction in the diameter of the spacer wire at the tube-spacer wire joints as a result of weld metal expulsion. Spacer wire setdown for these evaluation subassemblies has been limited to 0.015 cm (0.006 inch) for all

TABLE IV - Dimensional Inspection of Tube-to-Tube Spacings

	•	First Subassembly	Second Subassembly								
TUBES WITHIN	I ROWS										
One End	8-tube row	1.587 $^{+0.008}_{-0.008}$ cm (0.625 $^{+0.003}_{-0.003}$ inch)	1.587 $^{+0.000}_{-0.008}$ cm (0.625 $^{+0.000}_{-0.003}$ inch)								
	9-tube row	1.587 $^{+0.003}_{-0.008}$ cm (0.625 $^{+0.001}_{-0.003}$ inch)	1.587 $^{+0.003}_{-0.008}$ cm (0.625 $^{+0.001}_{-0.003}$ inch)								
Opposite End	8-tube row	1.587 +0.003 cm (0.625 +0.001 inch)	1.587 +0.000 cm (0.625 +0.000 inch)								
	9-tube row	1.587 $^{+0.003}_{-0.003}$ cm (0.625 $^{+0.001}_{-0.001}$ inch)	1.587 +0.008 cm (0.625 +0.003 inch)								
))											
•											
TUBES BETWE	TUBES BETWEEN ROWS (DIAGONAL SPACINGS)										
One End		1.587 +0.020 cm (0.625 +0.008 inch)	1.587 $^{+0.020}_{-0.000}$ cm (0.625 $^{+0.008}_{-0.000}$ inch)								
•											
Opposite End		1.587 $^{+0.033}_{-0.003}$ cm (0.625 $^{+0.013}_{-0.001}$ inch)	1.587 $^{+0.015}_{-0.000}$ cm (0.625 $^{+0.006}_{-0.000}$ inch)								

joints. Earlier work on the program had demonstrated that setdown of 0.020 cm (0.008 inch) had produced welds of suitable quality. Thus, an increase in spacer wire setdown from 0.015 to 0.020 cm should reduce the spacing between tube rows by 0.010 cm without seriously affecting weld quality.

A final 2-row, 17-tube subassembly of Type 347 stainless steel will be welded and evaluated to demonstrate the capability of the fabrication process to provide the required dimensional and structural integrity.

. C. Thermal Tests

Upon complete verification of the projection welding process, a 61-tube hexagonal cluster, 7 feet in length, will be fabricated from Type 347 stainless steel. This cluster will represent the highest attainable quality welds as well as control of tube-to-tube dimensions to within ±0.008 cm of the nominal value of 1.587 cm. The cluster will be tested to demonstrate the ability of spacer wires to accommodate differential thermal expansion of adjacent tubes without causing failures at the tube-spacer wire joints. Performance capability will be determined by pre-test and post-test evaluations involving dimensional measurements, metallographic and mechanical tests (tensile and shear) of selected welds.

All of the components required to assemble the apparatus for the differential thermal elongation tests have been either fabricated or purchased. Modifications have been incorporated which will ensure free relative motion of tubes within the manifolds. The system contains hot and cold water lines which are channeled alternately through rows of tubes of the test assembly. These tests will evaluate the capability of full scale clusters, fabricated from Type 347 stainless steel and Zircaloy-2, to withstand cyclic temperature

changes of 10°C (20°F) between rows of tubes for 3000 temperature reversals.

D. Future Work

The final phase of the program includes the fabrication of two 61-tube hexagonal clusters, 2.13 meters (7 feet) in length, of Type 347 stainless steel and Zircaloy-2. The Zircaloy cluster will be projection welded in an inert atmosphere. Both clusters will be similarly tested for differential thermal expansion behavior. Post-test evaluations will establish the overall structural soundness and dimensional integrity of the clusters.

REFERENCES

- 1. Chernock, W. P., Burdg, C. E., Veil, E. I., and Zuromsky, G., CEND-2863-211 (March 1964), "The Development and Testing of UO₂ Fuel Systems for Water Reactor Applications. Progress Report."
- 2. Daniel, J. L., Matolick, J., Jr., and Deem, H. W., "Thermal Conductivity of UO₂," USAEC Report HW-69945, p. 3, Hanford Laboratories, September 1962.
- 3. Godfrey, T. G., Fulkerson, W., Kollie, T. G., Moore, J. P., McElroy, D. L., "Thermal Conductivity of Uranium Dioxide and Armco Iron by Improved Radial Heat Flow Technique," USAEC Report ORNL-3556, pp. 16-17, Oak Ridge National Laboratory, June 1964.
- 4. de Halas, D. R., Understanding UO₂ Thermal Conductivity, "Nucleonics," 21, No. 10, 92, 1963.
- 5. Feith, A. D., "Thermal Conductivity of UO₂ by a Radial Heat Flow Method," Report TM-63-9-5, General Electric Company.
- 6. Chernock, W. P. and Murtha, B. E., CEND-157 (June 1962), "The Development and Testing of UO₂ Fuel Systems for Water Reactor Applications Summary Report."
- 7. Chernock, W. P., Burdg, C. E., Andrews, M. G., Veil, E. I., and Mordarski, W. J., CEND-199 (September 1963), "The Development and Testing of UO₂ Fuel Systems for Water Reactor Applications Progress Report."

This document was prepared under the sponsorship of the Euratom Commission pursuant to the Joint Research and Development Program established by the Agreement for Cooperation signed November 8, 1958 between the Government of the United States of America and the European Atomic Energy Community (Euratom). Neither the United States, the U.S. Atomic Energy Commission, the European Atomic Energy Community, the Euratom Commission, nor any person acting on behalf of either Commission:

- A. Makes any warranty or representation, express or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this document, or that the use of any information, apparatus, method, or process disclosed in this document may not infringe privately owned rights; or
- B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method or process disclosed in this document.

As used in the above, "person acting on behalf of either Commission" includes any employee or contractor of either Commission or employee of such contractor to the extent that such employee or contractor or employee of such contractor prepares, handles, disseminates, or provides access to, any information pursuant to this employment or contract with either Commission or his employment with such contractor.