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SUR, A Program to Generate Error Covariance Files

F. C. Difilippo

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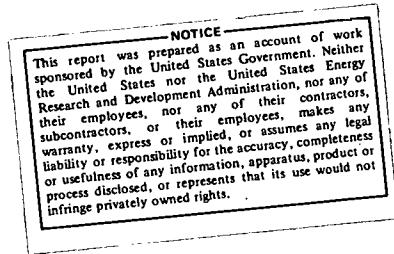
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Neutron Physics Division

SUR, A PROGRAM TO GENERATE ERROR COVARIANCE FILES

F. C. Difilippo*



*On assignment from Comisión Nacional de Energía Atómica, Argentina.

MARCH 1976

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Abstract

Covariance matrices were calculated for the ^{238}U , ^{241}Pu , and ^{239}Pu fission cross sections and for the ^{238}U , ^{240}Pu , ^{241}Pu , and ^{239}Pu capture cross sections. A computer program was written which uses the evaluated ENDF/B data files and the measured or evaluated (from other evaluations) cross sections for the calculation of the uncertainty files. An effort has been made to make the output of the program consistent with the ENDF/B error files format. A user's manual for the present code and references utilized in the covariance matrix calculations are given.

I. INTRODUCTION

The economic impact of nuclear data uncertainties in fast breeder as well as in thermal reactors has been the subject of many studies.¹⁻⁵

To evaluate the effect of group cross-section uncertainties in sensitivity calculations, the correlation both within the group structure and among different isotopes has to be calculated. Extensive discussions of this procedure are given in refs. 6-8.

II. CALCULATION OF RELATIVE COVARIANCE MATRICES

The relative covariance matrix elements are defined here as:

$$\text{COV}(I, J) = \left\langle \left(\frac{x_i^\ell - \bar{x}_i}{\bar{x}_i} \right) \left(\frac{x_j^\ell - \bar{x}_j}{\bar{x}_j} \right) \right\rangle, \quad (1)$$

where x_i^ℓ and x_j^ℓ are the group average cross sections for the i^{th} and j^{th} energy groups, respectively, for the ℓ^{th} data set. The cross sections \bar{x}_i and \bar{x}_j are the reference ("true") values, which by convention are taken to be the corresponding ENDF/B values. The bracket denotes a weighted average over the ensemble of data sets, having common data in the i^{th} and j^{th} groups, where weights, w_ℓ , have been assigned to each data set.

The ENDF/B-IV files, used as the reference data, were energy-weighted by a flat weighting function in producing the group-average cross sections. Details of calculations are given in Sect. III.

The present study of the error covariance matrix implies an "external" analysis of the data. Each data set, whether the results of a measurement or of an evaluation, is assigned an arbitrary weight and is compared with a reference line taken to be the ENDF/B-IV evaluation.

III. BRIEF DESCRIPTION OF THE PROGRAM

The calculations were done by the code SUR, which, in its present form, only handles error correlation for a given cross-section data set within a given energy range. A description of the main steps in the program follows.

Step 1: The program sets the energy limits of the group structure for several options: (1) constant lethargy interval, (2) decimal interval, or (3) a mixture of the first two.

Step 2: For the ratio measurements, the standard reference cross section was always the ENDF/B-IV evaluation.

Step 3: The measurements corresponding to the data set ℓ , inside a group j , are reduced to the mean energy of the group. This is implemented by approximating the shape of the cross section with a parabola determined by the evaluated cross sections in groups $j - 1$, j , $j + 1$. Then, all the measurements corresponding to the data set ℓ , inside group j , are averaged.

Step 4: Whenever there are not measurements in the ℓ^{th} set of data, inside group j , the program makes a linear interpolation of data set, ℓ , values corresponding to the neighboring groups. Before implementing the interpolation, the program searches whether or not the data points for the interpolation procedure are within energy limits preset by the user. If not, the interpolation is not made.

Step 5: Calculation of $\text{COV}(I, J)$ is made. This is done by the formula

$$\text{COV}(I, J) = \frac{1}{RR} \sum_{K=1}^{NA} \left[\frac{\sigma(K, I) - \sigma(I)}{\sigma(I)} \right] \left[\frac{\sigma(K, J) - \sigma(J)}{\sigma(J)} \right] W(K) , \quad (2)$$

where

$$RR = \sum_{K=1}^{NA'} W(K) , \quad (3)$$

$\sigma(K, I)$, $\sigma(K, J)$ = measured or interpolated cross sections of data set K , in groups I and J ,

$\sigma(I)$, $\sigma(J)$ = evaluated cross sections of groups I and J ,

$W(K)$ = weight of data set K ,

NA = number of data sets with common, given or

interpolated, measurement in groups I and J ,

NA' = number of data sets with measured or interpolated cross sections in group I or J , whichever is greater.

If there are not common data sets in groups I and J, the covariance is set to zero. If there are not data inside a group, the standard deviation for that group is set to the average of the corresponding values in neighboring groups.

Step 6: The program also calculates the correlation matrix [COR(I,J)] according to the formula

$$\text{COR}(I,J) = \frac{\text{COV}(I,J)}{\sqrt{\text{COV}(I,I) \text{ COV}(J,J)}} . \quad (4)$$

IV. DESCRIPTION OF THE INPUT-OUTPUT SPECIFICATIONS

A. Input

Card 1: NG,NAS,NAR,NL,IOP,MAT,MT,IP,IP0,EMA,RLER,RLET,DELT,ECA, ID

[FORMAT(9I4,5E7.0,I1)]

NG: number of group;

NAS: number of cross-section data sets included in the error file evaluation;

NAR: number of ratio data sets included in the error file evaluation;

NL: number of energy limits preset to make linear interpolation between measurements in a given data set;

IOP: option

= 0, the word FISSION is printed;

= 1, the word CAPTURE is printed;

= 3, the word ALFA is printed;

MAT: material number according to ENDF/B specifications;

MT: reaction-type number according to ENDF/B specifications;
 IP: option
 = 0, part of the input is a cross-section ratio;
 ≠ 0, otherwise;
 IP0: = 1, the output includes punched cards of the covariance matrix
 according to ENDF/B format;
 ≠ 1, otherwise;
 EMA: higher energy limit corresponding to group 1 (MeV) (group 1
 is the highest energy group);
 RLER: lethargy corresponding to the lower limit of group 1 (it is
 assumed that lethargy zero corresponds to 10 MeV);
 RLET: group lethargy interval;
 DELT: (MeV) increment corresponding to the first energy decade in the
 decimal group structure;
 ECA: (MeV) higher energy limit of the decimal group structure; if
 all the groups are decimal, ECA must be greater than EMA; if
 all the groups are of constant lethargy ECA must be lower than
 the lower limit of the last group;
 ID: ≠ 0, the output includes punched cards of the correlation
 matrix;
 = 0, otherwise.

Cards 2: NPE(ℓ), $\ell = 1, NA$; NA = NAS+NAR [FORMAT (1415)]

Number of experimental points of set ℓ .

Cards 3: FORMAT (7E10.0)

Cross sections corresponding to the data set ℓ .

Cards 4: FØRMAT (7E10.0) (MeV)

Energies corresponding to the previous data set.

Cards 3 and Cards 4 must be repeated in a sequential order from $\lambda = 1$ to $\lambda = \text{NAS}$. Following these cards, one has to place a set of cards in the same format but now for the ratio measurements (only present if IP = 0).

Cards 5: ENDF/B evaluated cross section for group J (J = 1,NG)

[FØRMAT(6E12.0)]

Cards 6: (Only present if IP = 0.) FORMAT (6E12.0), ENDF/B evaluated cross sections for group J (J = 1,NG) of the reference standard cross section corresponding to the ratio measurement. (It is assumed that all ratio measurements in the input are relative to the same cross section.)

Cards 7: FORMAT (7E10.0)

Weight corresponding to each data set λ , ($\lambda = 1, \text{NA}$).

Cards 8: FØRMAT (7E10.0). Energy limits for the interpolation procedure (see step 4 in Sect. III). These limits are preset by the user, based on his judgment on the validity of the interpolation scheme. The limits must be given in MeV and in decreasing order [EL(J),J = 1,NL].

B. Output

Printed output:

- (1) All the measured cross sections are ordered and displayed according to authors and group limits.
- (2) The evaluated and the measured cross sections (now reduced to the mean energy of the group interval) are ordered and displayed according to authors and group limits.
- (3) The covariance matrix.
- (4) The relative standard deviation.
- (5) The information contained in each punched card corresponding to the covariance matrix in the ENDF/B format.
- (6) The information contained in each punched card corresponding to the correlation matrix. The formats are the following:
 First NG cards (three values in each card):
 Lower energy limit (MeV) in group I, and the corresponding relative standard deviation in percent and reference cross sections (1X,E11.4,11X,E11.4,F11.4)
 The rest of the cards:
 COR(I,J), for each I, J varies from 1 to NG format 7E10.4.
 These cards are suitable for a three-dimensional plot of the correlation matrix.
- (7) The correlation matrix is printed as can be seen in Appendix B.

Punched card:

With the option (IPO = 1), the covariance matrix is punched out according to the ENDF/B format.

The correlation matrix output is punched out, under option.

V. LISTING OF THE PROGRAM AND SAMPLE PROBLEM

Data for the sample problem: [COV(I,J) for the ^{240}Pu capture cross section].

```
NG = 45; NAS = 4; NAR = 0; NL = 2; IOP = 1; MAT = 1265; MT = 102;
IP = 1; IPO = 1; EMA = 10.; RLER = 0.; KLET = 0.; DELT = 1.
ECA = 11.; ID = 1
NPE( $\ell$  = 1,4) : 29;6;36;36
EL(J = 1,2) = 11., .0001
```

```

**FTN,L,G,E,Y.
C *****LISTING OF THE PROGRAM SUR*****
C
C THIS PROGRAM CALCULATES THE COVARIANCE AND CORRELATION MATRICES OF
C CROSS SECTIONS. THE COVARIANCE MATRIX FILE IS PUNCHED ACCORDING TO THE
C END/R FORMAT. THE CORRELATION MATRIX IN A FORMAT SUITABLE FOR 3-D PLOT
C
C
C DIMENSION SFE28(120),SFE25(120),SFM28(35,150),E(35,150),ER(35),NPE
C 1(35),EV(120),SFM(35,120),NM(120),COV(120,120),NMAR(35,120),EL(12),
C 2W(35),AUX(120,120),TITLE(18),STA(100)
C DIMENSION LABEL(120)
C DIMENSION NDER(28)
857 DC 125 I=1,35
DC 125 J=1,150
125 SFM28(I,J)=CA
C DATA INPUT
READ 1,NG,NAS,NAF,NL,IOP,MAT,MT,IP,IPC,EMA,RLER,RLET,DELT,ECA,ID
1 FORMAT(914.5E7.0,I1)
IF(IP.EQ.0) CALL EXIT
NA=NAS+NAF
READ 2,(NPE(I),I=1,NA)
2 FCRMAT(1415)
CO_3_I=1,NA
JM=NPE(I)
READ 4,(SFM28(I,J),J=1,JM)
3 READ 4,(E(I,J),J=1,JM)
4 FORMAT(10E8.0)
READ 6,(SFE28(I),I=1,NG)
6 FORMAT(10E8.0)
IF(IP.NE.0) GO TO 253
READ 6,(SFE25(I),I=1,NG)
253 READ 4,(W(I),I=1,NA)
READ 4,(EL(I),I=1,NL)
READ 400,(TITLE(I),I=1,18)
400 FCRMAT(18AA)
C RATIO MEASUREMENT ARE REDUCED TO ABSOLUTE ONES, AND LOCATED IN THE SAME
C LOCATION, THEN WE USE THE RATIOS. EV(I) ARE THE LIMITS OF GROUP I.
EV(1)=EMA
DELT=DELT
NE=NG+1
DO 243 I=2,NE
IF(ECAL.EVA) GO TO 245
EV(I)=10.*EXP(-RLER)
IF(EV(I).GT.ECA) GO TO 244
EV(K)=ECA-DEL
IF(I.FQ.K) GC TO 243
245 CONTINUE
IF(ABS(EV(I-2)/EV(I-1)-2.).LT.1.E-03) DELT=DELT*.1
EV(I)=EV(I-1)-DELT
GO TO 243
244 RLER=RLER+RLET
K=I+1
243 CONTINUE
IF(IP.NE.0) GC TO 254
NR=NAS+1
DO 8 I=NR,NA
JM=NPE(I)
DO 8 J=1,JM
K=1
9 CCNTINUE
IF(E(I,J).LE.EV(K).AND.E(I,J).GT.EV(K+1)) GO TO 10
K=K+1
IF(K.EQ.NE) GC TO 8
GO TO 9
10 IF(K.EQ.1) GC TO 845
IF(K.EQ.NG) GO TO 846
EK1=(EV(K+1)+EV(K+2))/2.
EK2=(EV(K)+EV(K+1))/2.
EK3=(EV(K)+EV(K-1))/2.
AC=(SFE25(K+1)-SFE25(K))/((EK1-EK2)*(EK2-EK3))-(SFE25(K+1)-SFE25(K-1))/((EK1-EK3)*(EK2-EK3))
EC=(SFE25(K+1)-SFE25(K))/(EK1-EK2)-AC*(EK1+EK2)
CC=SFE25(K+1)-AC*EK1*EK1-BC*EK1
SF25=AC*E(I,J)**2+BC*E(I,J)+CC
GO TO 87
846 EK=(EV(K)+EV(K+1))/2.
EKMI=(EV(K-1)+EV(K))/2.
SF25=SFE25(K)+(SFE25(K-1)-SFE25(K))/(EKMI-EK)*(E(I,J)-EK)
GO TO 87
845 E1=(EV(1)+EV(2))/2.
E2=(EV(2)+EV(3))/2.
PE=(SFE25(1)-SFE25(2))/(E1-E2)
SF25=SFE25(2)*PE*(E(I,J)-E2)
E7 SFM28(I,J)=SFM28(I,J)+SF25
8 CCNTINUE
C OUTPUT OF MEASURED DATA
254 PRINT 11
11 FORMAT(1H1,6H GROUP,3X,20H ENERGY LIMITS (MEV),8X,16H MEASURED VAL
1UES//)
DC 12 J=1,NG
IF(IOP.EQ.0) PRINT 13,J,EV(J),EV(J+1)
IF(IOP.EQ.1) PRINT 53,J,EV(J),EV(J+1)
IF(IOP.EQ.3) PRINT 325,J,EV(J),EV(J+1)

```

```

13 FORMAT(16.3X,1PE11.4,3X,1PE11.4,3X,7H AUTHOR,3X,7H ENERGY,7X,11H S
11GFISSION,7H WEIGHT)
53 FORMAT(16.3X,1PE11.4,3X,1PE11.4,3X,7H AUTHOR,3X,7H ENERGY,7X,11H S
11GCAPTURE,7H WEIGHT)
325 FORMAT(16.3X,1PE11.4,3X,1PE11.4,3X,7H AUTHOR,3X,7H ENERGY,7X,11H
1 ALFA,7H WEIGHT)
L=0
DO 14 I=1,NA
KM=NFE(I)
DO 14 K=1,KM
IF(E(I,K).GT.EV(J).OR.E(I,K).LE.EV(J+1)) GO TO 14
PRINT 16,I,E(I,K),SFM28(I,K),W(I)
16 FORMAT(43X,12.3X,1PE11.4,3X,1PE11.4,2X,F5.2)
L=1
14 CONTINUE
IF(L.EQ.0) PRINT 17
17 FORMAT(43X,15H NC MEASUREMENT)
12 CONTINUE
C THE MEASUREMRNT OF AUTHOR I INSIDE GROUP J IS RECUCED TO THE MEAN ENERGY OF
C THE GROUP AND THEN AVERAGED. THE RESULTS ARE LOCATED IN SFM(I,J)
DO 18 J=1,12C
NM(J)=0
DO 18 I=1,35
18 SFM(I,J)=0.
DO 19 I=1,NA
KM=NFE(I)
DC 19 J=1,NG
HR=0.
DO 20 K=1,KM
IF(E(I,K).GT.EV(J).OR.E(I,K).LE.EV(J+1)) GO TO 20
IF(J.EQ.1) GO TO 364
IF(J.EQ.NG) GO TO 355
EJ1=(EV(J+1)+EV(J+2))/2.
EJ2=(EV(J)+EV(J+1))/2.
EJ3=(EV(J)+EV(J-1))/2.
AC=(SFE28(J+1)-SFE28(J))/((EJ1-EJ2)*(EJ2-EJ3))-($FE28(J+1)-$FE28(J
1-1))/((EJ1-EJ3)*(EJ2-EJ3))
BC=(SFE28(J+1)-SFE28(J))/(EJ1-EJ2)-AC*(EJ1+EJ2)
CC=SFE28(J+1)-AC*EJ1*EJ1-BC*EJ1
RK=SFM28(I,K)/(AC*E(I,K)**2+BC*E(I,K)+CC)
AC=RK*AC
BC=RK*BC
CC=RK*CC
SFM(I,J)=SFM(I,J)+AC*EJ2**2+BC*EJ2+CC
RR=RR+1.
GO TO 20
355 EJ=(EV(J+1)+EV(J))/2.
EJM1=(EV(J)+EV(J-1))/2.
PE=(SFE28(J-1)-SFE28(J))/(EJM1-EJ)
SFM(I,J)=SFM(I,J)+ SFM28(I,K)-PE*(E(I,K)-EJ)
RR=RR+1.
GO TO 20
354 E1=(EV(1)+EV(2))/2.
E2=(EV(2)+EV(3))/2.
PE=(SFE28(1)-SFE28(2))/(E1-E2)
SFM(I,1)=SFM(I,1)+SFM28(I,K)-PE*(E(I,K)-E1)
RH=RR+1.
20 CONTINUE
IF(HR.GT.0.) SFM(I,J)=SFM(I,J)/RR
19 CONTINUE
C NUMBER OF REAL MEASUREMENTS NM(J) INTG EACH GROUP
DO 21 J=1,NG
DC 21 I=1,NA
NMAR(I,J)=1
IF(SFM(I,J).GT.0.) NM(J)=NM(J)+1
21 CONTINUE
C THOSE GROUP AND AUTHORS IN WHICH THERE ARE NO MEASUREMENT ARE MARKED
DO 457 J=1,NG
DC 457 I=1,NA
IF(SFM(I,J).EQ.0.) NMAR(I,J)=0
457 CONTINUE
C ASIGNATION OF AUTHOR I MEASUREMENT IN GROUP J BY INTERPOLATION
C (IF POSSIBLE)
DC 110 J=1,NG
IF(J.EQ.1.OF.J.EQ.NG) GO TO 110
DC 110 I=1,NA
IF(SFM(I,J).NE.0.) GO TO 110
L=1
111 CONTINUE
IF(EV(J).LE.EL(L).AND.EV(J).GT.EL(L+1)) GO TO 112
L=L+1
GO TO 111
112 KAR=J-1
113 CONTINUE
IF(SFM(I,KAR).NE.0.) GO TO 114
KAR=KAR-1
IF(KAR.EG.0) GO TO 110
IF(EV(KAR).LE.EL(L)) GO TO 113
114 IF(EV(KAR).GT.EL(L)) GO TO 110
KAB=J+1
115 CONTINUE
IF(SFM(I,KAB).NE.0.) GO TO 116
KAB=KAB+1
IF(KAB.GT.NG) GO TO 110
IF(EV(KAB).GT.EL(L+1)) GO TO 115

```

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116 IF(EV(KAB+1).LT.EL(L+1)) GO TO 110
EAR=(EV(KAR)+EV(KAR+1))/2.
EAB=(EV(KAB)+EV(KAB+1))/2.
EJ=(EV(J)+EV(J+1))/2.
IF(SFM(I,KAR).EQ.0..OR.SFM(I,KAB).EQ.0..) GO TO 110
SFM(I,J)=SFM(I,KAB)+(SFM(I,KAR)-SFM(I,KAB))/(EAR-EAB)*(EJ-EAB)
NM(J)=NM(J)+1
110 CONTINUE
C CALCULATION OF THE DIAGONAL TERMS OF COV(I,J)
DO 22 I=1,NG
DO 22 J=1,NG
22 COV(I,J)=0.
DO 23 J=1,NG
IF(NM(J).EQ.0.) GO TO 23
RR=0.
DO 25 I=1,NA
IF(SFM(I,J).EQ.0.) GO TO 25
COV(J,J)=CCV(J,J)+(SFM(I,J)-SFE28(J))*2*W(I)
RR=RR+W(I)
25 CONTINUE
COV(J,J)=CCV(J,J)/RR
23 CONTINUE
DO 28 J=1,NG
IF(COV(J,J).NE.0.) GO TO 28
KAB=J+1
29 CONTINUE
IF(COV(KAB,KAB).EQ.0.) GO TO 30
GC TO 31
30 KAB=KAB+1
GO TO 29
31 KAR=J-1
32 CONTINUE
IF(COV(KAR,KAR).EQ.0.) GO TO 33
GC TO 34
33 KAR=KAR-1
GO TO 32
34 COV(J,J)=(CCV(KAB,KAB)+CCV(KAR,KAR))/2.
28 CONTINUE
C OUTPUT OF DATA
PRINT 35
35 FORMAT(1H1,6H GROUP,3X,10H EVALUATED,3X,6H MEASURES AND INTERPOLA
TED VALUES REDUCED TO THE MESH ENERGY//)
DO 36 J=1,NG
IF(IOP.EG.0) PRINT 37,J,SFE28(J)
IF(IOP.EG.1) PRINT 54,J,SFE28(J)
IF(IOP.EG.3) PRINT 326,J,SFE28(J)
37 FORMAT(16.3X,1PE11.4,3X,5H SIGF,8X,7H AUTHOR,18X,7H WEIGHT//)
326 FORMAT(16.3X,1PE11.4,3X,5H ALFA,8X,7H AUTHOR,18X,7H WEIGHT//)
54 FORMAT(16.3X,1PE11.4,3X,5H SIGC,8X,7H AUTHOR,18X,7H WEIGHT//)
IF(NM(J).EQ.0) PRINT 38
38 FORMAT(23X,1PE11.4,5X,I4,3X,13H INTERPOLATED,2X,F5.2)
40 FORMAT(23X,1PE11.4,5X,I4,3X,9H MEASURED,6X,F5.2)
36 CONTINUE
C CALCULATION OF CCV(I,J) WITH I.NE.J
CO 41 I=1,NG
CO 41 J=1,NG
RRI=0.
RRJ=0.
IF(I.EQ.J) GO TO 41
DO 42 K=1,NA
IF(SFM(K,I).EQ.0..OR.SFM(K,J).EQ.0..) GO TO 142
COV(I,J)=COV(I,J)+(SFM(K,I)-SFE28(I))*(SFM(K,J)-SFE28(J))*W(K)
142 IF(SFM(K,I).NE.0.) RRI=RRI+W(K)
IF(SFM(K,J).NE.0.) RRJ=RRJ+W(K)
42 CONTINUE
IF(RRI.GE.RRJ) RR=RRI
IF(RRI.LE.RRJ) RR=RRJ
IF(RR.NE.0.) COV(I,J)=CCV(I,J)/RR
41 CONTINUE
C OUTPUT OF DATA
PRINT 499
499 FORMAT(1H1)
DO 357 I=1,NG
CO 357 J=1,NG
357 COV(I,J)=CCV(I,J)/(SFE28(I)*SFE28(J))
DO 43 I=1,NG
PRINT 44,I,NG
44 FORMAT(9H,COV(I,J),4X,3H,I=,I3.4X,I2H,J,FROM,1,TO,I3//)
PRINT 45,(COV(I,J),J=1,NG)
45 FFORMAT(1P10E12.4)
43 CONTINUE
NG1=NG+1
DO 210 I=1,NG1
210 EV(I)=10000*CO.*EV(I)
EO=.00001
EM=20000.000
MI=0
RI=0.
MIA=33
RIA=1.
MT1=MT
C THE ORDER IN THE GROUPS ARE INVERTED

```

```

MX=NG1/2
DO 212 I=1,MX
  EVT=EV(I)
  EV(I)=EV(NG1+1-I)
  EV(NG1+I-1) =EVT
212 MX=NG/2
DO 213 I=1,MX
  SFT=SFE28(I)
  SFE28(I)=SFE28(NG+1-I)
213   SFE28(NG+1-I)=SFT
  EV(NG1+1)=200C0000.
  IF(EV(NG1).EQ.EV(NG1+1))  EV(NG1)=EV(NG1)-.001*EV(NG1)
  DO 214 I=1,NG
  DO 214 J=1,NG
214 AUX(NG+1-I,NG+1-J)=COV(I,J)
  DO 215 J=1,NG
  AUX(NG+1,J)=0.
215 AUX(NG+2,J)=0.
  PRINT 46,NG
46 FORMAT(1H1,31H RELATIVE ERROR FROM GROUP 1 TO,13)
  DO 47 I=1,NG
  COV(I,I)=SQRT(COV(I,I))
47 CCNTINUE
  PRINT 48,(COV(I,I),I=1,NG)
48 FORMAT(10F10.5)
  PRINT 211
211 FORMAT(1H1,63H CARDS OUTPUT. WARNING THE ORDER IN THE GROUPS ARE NOW
1OW INVERTED//35H COV(I,J) CARDS IN THE ENDF/B FORMAT//)
  N3=NG+3
  LB=3
  N4=2*NG+14
  N7=NG+7
  DO 216 J=1,NG
    IF(J.EQ.1) PRINT 217,MI,MT1,MI,NG,MAT,MIA,MT
    IF(J.EQ.1.AND.IPO.EQ.1) PUNCH 217,MI,MT1,MI,NG,MAT,MIA,MT
    IF(J.GT.1) PRINT 217,N3,LB,N14,N7,MAT,MIA,MT
    IF(J.GT.1.AND.IPO.EQ.1) PUNCH 217,N3,LB,N14,N7,MAT,MIA,MT
217 FORMAT(22X,4I11,14,I2,13)
  PRINT 218,E0,RI,EV(J),RIA,EV(J+1),RI,MAT,MIA,MT
  IF(IPO.EQ.1) PUNCH 218,E0,RI,EV(J),RIA,EV(J+1),RI,MAT,MIA,MT
218 FORMAT(6E11.4,I4,I2,I3)
  PRINT 219,EM,RI,E0,RI,EV(1),AUX(1,J),MAT,MIA,MT
  IF(IPO.EQ.1) PUNCH 219,EM,RI,E0,RI,EV(1),AUX(1,J),MAT,MIA,MT
219 FORMAT(6E11.4,I4,I2,I3)
  NG2=NG+2
  I=2
221 CCNTINUE
  PRINT 220,EV(I),AUX(I,J),EV(I+1),AUX(I+1,J),EV(I+2),AUX(I+2,J),MAT
  1, MIA, MT
  IF(IPO.EQ.1) PUNCH 220,EV(I),AUX(I,J),EV(I+1),AUX(I+1,J),EV(I+2),A
  UX(I+2,J),MIA,MT
220 FORMAT(6E11.4,I4,I2,I3)
  I=I+3
  IF((NG2-I).GE.2) GO TO 221
  IF((NG2-I).EQ.1) GO TO 222
  IF((NG2-I).EQ.0) GO TO 223
  GO TO 230
222 PRINT 224,EV(I),AUX(I,J),EV(I+1),AUX(I+1,J),MAT,MIA,MT
  IF(IPO.EQ.1) PUNCH 224,EV(I),AUX(I,J),EV(I+1),AUX(I+1,J),MAT,MIA,MT
  1
224 FORMAT(4E11.4,22X,I4,I2,I3)
  GO TO 230
223 PRINT 225,EV(I),AUX(I,J),MAT,MIA,MT
  IF(IPO.EQ.1) PUNCH 225,EV(I),AUX(I,J),MAT,MIA,MT
225 FORMAT(2E11.4,44X,I4,I2,I3)
230 CCNTINUE
216 CCNTINUE
  IF(ID,NE,.0) PUNCH 309, TITLE, NG
309 FORMAT(1E4/15)
  PRINT 310
310 FORMAT(1H1,41H CORRELATION MATRIX CARDS FOR THE DRAWING//)
  DC 300 I=1,NG
  ST=100.*SQRT(AUX(I,I))
  ENE=.000001*EV(I)
  PRINT 301,ENE,ST,SFE28(I)
  IF(ID,NE,.0) PUNCH 301,ENE,ST,SFE28(I)
301 FORMAT(1X,E11.4,11X,E11.4,F11.4)
300 CCNTINUE
  ENE=EV(NE)/10**6
  IF(ID,NE,.0) PUNCH 301,ENE
  DO 311 I=1,NG
  DO 311 J=1,NG
311 COV(I,J)=AUX(I,J)/SQRT(AUX(I,I)*AUX(J,J))
  DO 302 I=1,NG
  PRINT 305,(CCV(I,J),J=1,NG)
  IF(ID,NE,.0) PUNCH 303,(COV(I,J),J=1,NG)
303 FORMAT(8F9.5)
305 FORMAT(1X,7E10.4)
302 CCNTINUE
  DO 391 I=1,NG
391 STA(I)=100.*SQRT(AUX(I,I))
  PRINT 402,TITLE
402 FORMAT(1H1,18A4)
  DC 399 N=2,28
399 NDER(N)=4H
  DC 400,L=1,NG

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```
400 LABEL(L)=L
    L1=1
401 LM=L1+27
    IF(LM.GT.NG),LM=NG
    PRINT 405,(LABEL(L),L=L1,LM)
405 FCRMAT(29H0 I ELC(KEV) RSD CERR*100 /14X3HPCT,1X10I4,1X10I4,
    1 IX8I4)
    PRINT 430
430 FORMAT(1H )
    DO 421 I=L1,NG
    ENE=EV(I)/1000.
    DG 405,L=L1,LM
    TEMP= COV(I,L)*100.
409 LABEL(L)=TEMP
410 PRINT 406, I,ENE,STA(I),(LABEL(L),L=L1,LM)
    IL=I+1-L1
    IF(IL.GT.28) GO TO 421
    NDER(IL)=4H
    PRINT 420,NDER
420 FORMAT(1H+,17X10A4,1X10A4,1X8A4)
    NDER(IL)=4H
421 CONTINUE
    ENE= EV(NE)/1000.
    PRINT 406, NE, ENE
    L1=LM+1
    IF(L1.LE.NG) GO TO 401
406 FORMAT(1F 13,F8.2,F5.1,1X10I4,1X10I4,1X8I4)
408 CONTINUE
    GC TO 897
END
```

GROUP	ENERGY LIMITS (MEV)		MEASURED VALUES		
1	1.0000E 01	9.0000E 00	AUTHOR 3	ENERGY 9.5000E 00	SIGCAPTURE 4.7000E-03
			AUTHOR 4	9.5000E 00	WEIGHT 1.00
2	9.0000E 00	8.0000E 00	AUTHOR 3	ENERGY 8.5000E 00	SIGCAPTURE 5.7000E-03
			AUTHOR 4	8.5000E 00	WEIGHT 1.00
3	8.0000E 00	7.0000E 00	AUTHOR 3	ENERGY 7.5000E 00	SIGCAPTURE 5.3000E-03
			AUTHOR 4	7.5000E 00	WEIGHT 1.00
4	7.0000E 00	6.0000E 00	AUTHOR 3	ENERGY 6.5000E 00	SIGCAPTURE 7.0000E-03
			AUTHOR 4	6.5000E 00	WEIGHT 1.00
5	6.0000E 00	5.0000E 00	AUTHOR 3	ENERGY 5.5000E 00	SIGCAPTURE 1.1000E-02
			AUTHOR 4	5.5000E 00	WEIGHT 1.00
6	5.0000E 00	4.0000E 00	AUTHOR 3	ENERGY 4.5000E 00	SIGCAPTURE 1.6000E-02
			AUTHOR 4	4.5000E 00	WEIGHT 1.00
7	4.0000E 00	3.0000E 00	AUTHOR 3	ENERGY 3.5000E 00	SIGCAPTURE 1.1000E-02
			AUTHOR 4	3.5000E 00	WEIGHT 1.00
8	3.0000E 00	2.0000E 00	AUTHOR 3	ENERGY 2.5000E 00	SIGCAPTURE 2.4000E-02
			AUTHOR 4	2.5000E 00	WEIGHT 1.00
9	2.0000E 00	1.0000E 00	AUTHOR 3	ENERGY 1.5000E 00	SIGCAPTURE 1.7000E-02
			AUTHOR 4	1.5000E 00	WEIGHT 1.00
10	1.0000E 00	9.0000E-01	AUTHOR 3	ENERGY 9.5000E-01	SIGCAPTURE 1.1000E-01
			AUTHOR 4	9.5000E-01	WEIGHT 1.00
11	9.0000E-01	8.0000E-01	AUTHOR 3	ENERGY 8.5000E-01	SIGCAPTURE 1.2000E-01
			AUTHOR 4	8.5000E-01	WEIGHT 1.00
12	8.0000E-01	7.0000E-01	AUTHOR 3	ENERGY 7.5000E-01	SIGCAPTURE 1.2000E-01
			AUTHOR 4	7.5000E-01	WEIGHT 1.00
13	7.0000E-01	6.0000E-01	AUTHOR 3	ENERGY 6.5000E-01	SIGCAPTURE 1.4000E-01
			AUTHOR 4	6.5000E-01	WEIGHT 1.00
14	6.0000E-01	5.0000E-01	AUTHOR 3	ENERGY 5.5000E-01	SIGCAPTURE 1.9000E-01
			AUTHOR 4	5.5000E-01	WEIGHT 1.00
15	5.0000E-01	4.0000E-01	AUTHOR 3	ENERGY 4.5000E-01	SIGCAPTURE 1.5000E-01
			AUTHOR 4	4.5000E-01	WEIGHT 1.00
16	4.0000E-01	3.0000E-01	AUTHOR 3	ENERGY 3.5000E-01	SIGCAPTURE 2.1000E-01
			AUTHOR 4	3.5000E-01	WEIGHT 1.00
17	3.0000E-01	2.0000E-01	AUTHOR 3	ENERGY 2.5000E-01	SIGCAPTURE 1.6000E-01
			AUTHOR 4	2.5000E-01	WEIGHT 1.00
18	2.0000E-01	1.0000E-01	AUTHOR 3	ENERGY 1.5000E-01	SIGCAPTURE 2.3000E-01
			AUTHOR 4	1.5000E-01	WEIGHT 1.00
19	1.0000E-01	9.0000E-02	AUTHOR 3	ENERGY 9.5000E-02	SIGCAPTURE 2.8000E-01
			AUTHOR 4	9.5000E-02	WEIGHT 9.00
20	9.0000E-02	8.0000E-02	AUTHOR 3	ENERGY 9.5000E-02	SIGCAPTURE 3.4600E-01
			AUTHOR 4	9.5000E-02	WEIGHT 9.00
21	8.0000E-02	7.0000E-02	AUTHOR 3	ENERGY 8.5000E-02	SIGCAPTURE 2.7000E-01
			AUTHOR 4	8.5000E-02	WEIGHT 1.00
22	7.0000E-02	6.0000E-02	AUTHOR 3	ENERGY 7.5000E-02	SIGCAPTURE 4.6000E-01
			AUTHOR 4	7.5000E-02	WEIGHT 1.00
23	6.0000E-02	5.0000E-02	AUTHOR 3	ENERGY 6.5000E-02	SIGCAPTURE 5.2000E-01
			AUTHOR 4	6.5000E-02	WEIGHT 1.00
24	5.0000E-02	4.0000E-02	AUTHOR 3	ENERGY 5.5000E-02	SIGCAPTURE 5.8000E-01
			AUTHOR 4	5.5000E-02	WEIGHT 9.00
25	4.0000E-02	3.0000E-02	AUTHOR 3	ENERGY 4.5000E-02	SIGCAPTURE 6.3000E-01
			AUTHOR 4	4.5000E-02	WEIGHT 1.00
26	3.0000E-02	2.0000E-02	AUTHOR 3	ENERGY 3.5000E-02	SIGCAPTURE 6.7000E-01
			AUTHOR 4	3.5000E-02	WEIGHT 1.00
			AUTHOR 3	2.5000E-02	WEIGHT 9.00
			AUTHOR 4	2.5000E-02	WEIGHT 9.00
			AUTHOR 3	2.5000E-02	WEIGHT 9.00
			AUTHOR 4	2.5000E-02	WEIGHT 1.00

			AUTHOR	ENERGY	SIGCAPTURE	WEIGHT
27	2.0000E-02	1.0000E-02	1	1.5000E-02	1.0000E-00	9.00
			2	1.5000E-02	1.1200E-00	9.00
			3	1.5000E-02	1.3000E-01	1.00
28	1.0000E-02	9.0004E-03	4	1.5000E-02	1.2500E-00	1.00
			1	9.5000E-03	1.2800E-00	9.00
			2	9.5000E-03	1.3600E-00	9.00
			3	9.5000E-03	1.2000E-01	1.00
29	9.0004E-03	8.0004E-03	4	1.5000E-02	1.5200E-00	1.00
			1	8.5000E-03	1.3200E-00	9.00
			2	8.5000E-03	1.3600E-00	9.00
			3	8.5000E-03	1.7000E-01	1.00
30	8.0004E-03	7.0004E-03	4	1.5000E-02	1.6000E-00	1.00
			1	7.5000E-03	1.3600E-00	9.00
			2	7.5000E-03	1.3900E-00	9.00
			3	7.5000E-03	1.0300E-00	1.00
31	7.0004E-03	6.0004E-03	4	1.5000E-02	1.7000E-00	1.00
			1	6.5000E-03	1.3800E-00	9.00
			2	6.5000E-03	1.3900E-00	9.00
			3	6.5000E-03	1.1200E-00	1.00
32	6.0004E-03	5.0004E-03	4	1.5000E-02	1.8000E-00	1.00
			1	5.5000E-03	1.5500E-00	9.00
			2	5.5000E-03	1.2100E-00	1.00
33	5.0004E-03	4.0004E-03	4	1.5000E-02	2.0000E-00	1.00
			1	4.5000E-03	1.6700E-00	9.00
			2	4.5000E-03	1.3500E-00	1.00
34	4.0004E-03	3.0004E-03	4	1.5000E-02	2.2000E-00	1.00
			1	3.5000E-03	1.8900E-00	9.00
			2	3.5000E-03	1.5200E-00	1.00
35	3.0004E-03	2.0004E-03	4	1.5000E-02	2.5000E-00	1.00
			1	2.5000E-03	2.4100E-00	9.00
			2	2.5000E-03	1.8500E-00	1.00
36	2.0004E-03	1.0004E-03	4	1.5000E-02	3.1000E-00	1.00
			1	1.5000E-03	3.2200E-00	9.00
			2	1.5000E-03	2.4000E-00	1.00
			3	1.5000E-03	4.4000E-00	1.00
37	1.0004E-03	9.0040E-04	4	1.5000E-02	5.3200E-00	9.00
38	9.0040E-04	8.0040E-04	1	9.5000E-04	5.4600E-00	9.00
39	8.0040E-04	7.0040E-04	1	8.5000E-04	5.6000E-00	9.00
40	7.0040E-04	6.0040E-04	1	7.5000E-04	5.7300E-00	9.00
41	6.0040E-04	5.0040E-04	1	6.5000E-04	5.8300E-00	9.00
42	5.0040E-04	4.0040E-04	1	5.5000E-04	5.9300E-00	9.00
43	4.0040E-04	3.0040E-04	1	4.5000E-04	6.0400E-00	9.00
44	3.0040E-04	2.0040E-04	1	3.5000E-04	6.8900E-00	9.00
45	2.0040E-04	1.0040E-04	1	2.5000E-04	7.2900E-00	9.00
			1	1.5000E-04	2.7300E-01	9.00

GROUP	EVALUATED	MEASURES AND INTERPOLATED VALUES REDUCED TO THE MEAN ENERGY			
		SIGC	AUTHOR		WEIGHT
1	5.1000E-03	4.7000E-03 4.7000E-03	3 4	MEASURED MEASURED	1.00 1.00
2	5.8000E-03	SIGC	AUTHOR		WEIGHT
3	6.9000E-03	5.7000E-03 5.3000E-03	3 4	MEASURED MEASURED	1.00 1.00
4	8.5000E-03	SIGC	AUTHOR		WEIGHT
5	1.1000E-02	8.3000E-03 7.1000E-03	3 4	MEASURED MEASURED	1.00 1.00
6	1.5000E-02	1.1000E-02 8.5000E-03	3 4	MEASURED MEASURED	1.00 1.00
7	2.4000E-02	SIGC	AUTHOR		WEIGHT
8	4.0000E-02	2.4000E-02 2.7000E-02	3 4	MEASURED MEASURED	1.00 1.00
9	7.5000E-02	SIGC	AUTHOR		WEIGHT
10	1.1200E-01	7.3000E-02 6.4000E-02	3 4	MEASURED MEASURED	1.00 1.00
11	1.2300E-01	SIGC	AUTHOR		WEIGHT
12	1.3300E-01	1.2000E-01 1.2000E-01	3 4	MEASURED MEASURED	1.00 1.00
13	1.4400E-01	SIGC	AUTHOR		WEIGHT
14	1.5300E-01	1.3000E-01 1.4000E-01	3 4	MEASURED MEASURED	1.00 1.00
15	1.6100E-01	SIGC	AUTHOR		WEIGHT
16	1.6500E-01	1.6000E-01 1.3000E-01	3 4	MEASURED MEASURED	1.00 1.00
17	1.8200E-01	1.6000E-01 1.5000E-01	3 4	MEASURED MEASURED	1.00 1.00
18	2.2200E-01	SIGC	AUTHOR		WEIGHT
19	2.7200E-01	2.0200E-01 2.9400E-01	1 3	MEASURED MEASURED	9.00 1.00
20	2.8800E-01	2.2000E-01 2.7000E-01	1 3	MEASURED MEASURED	9.00 1.00
21	3.1000E-01	SIGC	AUTHOR		WEIGHT
22	3.4100E-01	2.8000E-01 4.9000E-01	1 3 4	MEASURED MEASURED MEASURED	9.00 1.00 1.00
		4.6800E-01 3.3000E-01	1 3	MEASURED MEASURED	9.00 1.00
		5.7000E-01	4	MEASURED	1.00

		SIGC	AUTHOR		WEIGHT
23	3.9000E-01	5.8300E-01	1	MEASURED	9.00
		3.7000E-01	3	MEASURED	1.00
		6.3000E-01	4	MEASURED	1.00
24	4.6100E-01	SIGC	AUTHOR		WEIGHT
		5.9200E-01	1	MEASURED	9.00
		4.2000E-01	3	MEASURED	1.00
		6.7000E-01	4	MEASURED	1.00
25	5.8300E-01	SIGC	AUTHOR		WEIGHT
		6.9099E-01	1	MEASURED	9.00
		4.7000E-01	3	MEASURED	1.00
		7.9999E-01	4	MEASURED	1.00
26	7.9500E-01	SIGC	AUTHOR		WEIGHT
		7.8999E-01	1	MEASURED	9.00
		8.9599E-01	2	MEASURED	9.00
		5.4899E-01	3	MEASURED	1.00
		9.4999E-01	4	MEASURED	1.00
27	1.0500E 00	SIGC	AUTHOR		WEIGHT
		9.9999E-01	1	MEASURED	9.00
		1.1200E 00	2	MEASURED	9.00
		7.2999E-01	3	MEASURED	1.00
		1.2500E 00	4	MEASURED	1.00
28	1.2400E 00	SIGC	AUTHOR		WEIGHT
		1.2800E 00	1	MEASURED	9.00
		1.3600E 00	2	MEASURED	9.00
		9.1999E-01	3	MEASURED	1.00
		1.5000E 00	4	MEASURED	1.00
29	1.2900E 00	SIGC	AUTHOR		WEIGHT
		1.3200E 00	1	MEASURED	9.00
		1.3600E 00	2	MEASURED	9.00
		6.6998E-01	3	MEASURED	1.00
		1.6000E 00	4	MEASURED	1.00
30	1.3600E 00	SIGC	AUTHOR		WEIGHT
		1.3600E 00	1	MEASURED	9.00
		1.3900E 00	2	MEASURED	9.00
		1.0300E 00	3	MEASURED	1.00
		1.7000E 00	4	MEASURED	1.00
31	1.4400E 00	SIGC	AUTHOR		WEIGHT
		1.3800E 00	1	MEASURED	9.00
		1.3900E 00	2	MEASURED	9.00
		1.1200E 00	3	MEASURED	1.00
		1.8000E 00	4	MEASURED	1.00
32	1.5500E 00	SIGC	AUTHOR		WEIGHT
		1.5499E 00	1	MEASURED	9.00
		1.2100E 00	3	MEASURED	1.00
		1.9999E 00	4	MEASURED	1.00
33	1.7000E 00	SIGC	AUTHOR		WEIGHT
		1.6700E 00	1	MEASURED	9.00
		1.3500E 00	3	MEASURED	1.00
		2.2000E 00	4	MEASURED	1.00
34	1.6400E 00	SIGC	AUTHOR		WEIGHT
		1.8898E 00	1	MEASURED	9.00
		1.5198E 00	3	MEASURED	1.00
		2.4998E 00	4	MEASURED	1.00
35	2.5400E 00	SIGC	AUTHOR		WEIGHT
		2.4097E 00	1	MEASURED	9.00
		1.8498E 00	3	MEASURED	1.00
		3.0996E 00	4	MEASURED	1.00
36	3.2300E 00	SIGC	AUTHOR		WEIGHT
		3.2184E 00	1	MEASURED	9.00
		2.3988E 00	3	MEASURED	1.00
		4.3978E 00	4	MEASURED	1.00
37	6.4100E 00	SIGC	AUTHOR		WEIGHT
38	6.2100E 00	SIGC	AUTHOR		WEIGHT
39	3.1400E 00	SIGC	AUTHOR		WEIGHT
40	5.1000E 00	SIGC	AUTHOR		WEIGHT
41	6.5900E 00	SIGC	AUTHOR		WEIGHT
42	6.6000E 00	SIGC	AUTHOR		WEIGHT
43	8.1600E 00	SIGC	AUTHOR		WEIGHT
44	7.8600E 00	SIGC	AUTHOR		WEIGHT
45	2.5040E 01	SIGC	AUTHOR		WEIGHT
		2.7231E 01	1	MEASURED	9.00

COV(I,J) I = 1 J FROM 1 TO 45

6.1515E-03	4.0567E-03	4.1543E-03	7.128E-03	8.9128E-03	8.9128E-03	7.9411E-03	7.9411E-03	6.7974E-03
-1.6130E-03	-4.0280E-03	-4.1582E-03	-7.128E-03	-8.9128E-03	-8.9128E-03	-7.9411E-03	-7.9411E-03	-6.7974E-03
-4.6000E-04	-1.5582E-04	-4.0280E-04	-6.0289E-04	-6.0289E-04	-6.0289E-04	-4.0440E-04	-4.0440E-04	-3.0552E-04
-1.0871E-04	-5.0551E-04	-5.0551E-04	-5.0551E-04	-5.0551E-04	-5.0551E-04	-1.0438E-04	-1.0438E-04	-1.0438E-04
COV(I,J)	I = 2	J FROM 1 TO 45						
4.0567E-03	8.642E-03	5.497E-03	3.582E-03	9.796E-03	1.0919E-02	1.2572E-02	6.5808E-03	6.5516E-03
-1.2558E-04	-3.7149E-04	-1.3578E-04	-1.4743E-04	-1.4743E-04	-1.4743E-04	-1.2562E-04	-1.2562E-04	-1.2562E-04
-8.2558E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04
COV(I,J)	I = 3	J FROM 1 TO 45						
4.5467E-03	5.497E-03	8.642E-03	1.0919E-02	1.2572E-02	1.0919E-02	1.2572E-02	6.5808E-03	6.5516E-03
-1.4136E-04	-8.0545E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04
-8.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04	-1.0791E-04
COV(I,J)	I = 4	J FROM 1 TO 45						
7.3818E-03	7.302E-03	1.0598E-02	1.3841E-02	1.8717E-02	2.177E-02	2.4020E-02	1.3235E-02	1.2392E-02
-1.0074E-03	-1.0074E-03	-1.0074E-03	-1.0074E-03	-1.0074E-03	-1.0074E-03	-1.0074E-03	-1.0074E-03	-1.0074E-03
-1.7971E-03	-3.8772E-03	-3.8772E-03	-3.8772E-03	-3.8772E-03	-3.8772E-03	-3.8772E-03	-3.8772E-03	-3.8772E-03
COV(I,J)	I = 5	J FROM 1 TO 45						
8.7128E-03	9.796E-03	1.4525E-02	1.8717E-02	2.177E-02	2.4020E-02	2.6235E-02	1.9887E-02	1.6667E-02
-1.3996E-03	-1.0791E-03	-1.0791E-03	-1.0791E-03	-1.0791E-03	-1.0791E-03	-1.0791E-03	-1.0791E-03	-1.0791E-03
-2.8449E-03	-5.9975E-03	-6.0766E-03	-6.0766E-03	-6.0766E-03	-6.0766E-03	-6.0766E-03	-6.0766E-03	-6.0766E-03
COV(I,J)	I = 6	J FROM 1 TO 45						
7.8432E-03	1.0791E-02	1.0791E-02	1.2572E-02	1.2572E-02	1.3235E-02	1.3235E-02	2.04667E-02	1.86667E-02
-2.6618E-03	-1.6476E-03	-1.6476E-03	-1.6476E-03	-1.6476E-03	-1.6476E-03	-1.6476E-03	-1.6476E-03	-1.6476E-03
-4.0737E-03	-8.0917E-03	-8.0917E-03	-8.0917E-03	-8.0917E-03	-8.0917E-03	-8.0917E-03	-8.0917E-03	-8.0917E-03
COV(I,J)	I = 7	J FROM 1 TO 45						
3.1438E-02	1.2572E-02	1.8648E-02	2.177E-02	2.4020E-02	2.6235E-02	2.8889E-02	4.2535E-02	2.5521E-02
-3.5570E-03	-1.7886E-03	-1.7886E-03	-1.7886E-03	-1.7886E-03	-1.7886E-03	-1.7886E-03	-1.7886E-03	-1.7886E-03
-3.0745E-03	-7.6968E-03	-7.6968E-03	-7.6968E-03	-7.6968E-03	-7.6968E-03	-7.6968E-03	-7.6968E-03	-7.6968E-03
COV(I,J)	I = 8	J FROM 1 TO 45						
2.9411E-03	6.6808E-03	1.2317E-02	1.375E-02	1.778E-02	2.0667E-02	2.5521E-02	3.8889E-02	4.1500E-02
-9.1463E-04	-1.0231E-03	-1.0231E-03	-1.0231E-03	-1.0231E-03	-1.0231E-03	-1.0231E-03	-1.0231E-03	-1.0231E-03
-3.2984E-03	-6.6125E-03	-6.6125E-03	-6.6125E-03	-6.6125E-03	-6.6125E-03	-6.6125E-03	-6.6125E-03	-6.6125E-03
RELATIVE ERROR FROM GROUP 1 TO 45								
0.07843	0.06216	0.09280	0.14765	0.19437	0.20624	0.24252	0.27496	0.31786
0.02439	0.09148	0.22673	0.26380	0.30308	0.34959	0.41011	0.42248	0.46325
0.038056	0.39346	0.48480	0.29235	0.20996	0.149736	0.18025	0.19742	0.20729
0.08318	0.10971	0.10942	0.21085	0.15253	0.13385	0.17000	0.17142	0.15056
0.06628	0.02470	0.21163	0.05016	0.08750				

CARDS OUTPUT, WARNING THE ORDER IN THE GROUPS ARE NOW INVERTED
 COV(I,J) CARDS IN THE ENDF/B FORMA

0.1000E-04 0.0	0.1004E 0 0	0.1000E 01	0.2004E 0 0	0.2004E 0 0	45126533102
0.1000E-04 0.0	0.1000E-04 0	0.1000E 0	0.1000E 0	0.1000E 0	1266533102
0.1000E-04 0.0	0.3004E 0 0	0.3004E 03	0.4004E 0 0	0.4004E 0 0	1266533102
0.1000E-04 0.0	0.6004E 0 0	0.6004E 03	0.7004E 0 0	0.7004E 0 0	1266533102
0.1000E-04 0.0	0.9004E 0 0	0.9004E 03	0.4000E 0 0	0.4000E 0 0	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 04	0.1091E-01	0.1091E-01	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 04	0.1642E-01	0.1642E-01	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 04	0.1270E-02	0.1270E-02	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 05	0.1326E-01	0.1326E-01	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 05	0.2666E-01	0.2666E-01	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 05	0.1948E-01	0.1948E-01	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 06	0.4000E 0 0	0.4000E 0 0	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 06	0.7000E 0 0	0.7000E 0 0	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 06	0.1000E 0 0	0.1000E 0 0	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 07	0.4000E 0 0	0.4000E 0 0	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 07	0.7000E 0 0	0.7000E 0 0	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 07	0.1000E 0 0	0.1000E 0 0	1266533102
0.2000E 0 0					
0.1000E-04 0.0	0.2004E 0 0	0.2004E 03	0.1000E 01	0.3004E 0 0	104 52
0.1000E-04 0.0	0.1000E 0 0	0.1000E 04	0.1062E-01	0.1062E-01	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 03	0.7554E-03	0.7554E-03	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 03	0.8527E-03	0.8527E-03	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 04	0.6252E-02	0.6252E-02	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 04	0.9411E-02	0.9411E-02	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 04	0.7277E-02	0.7277E-02	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 04	0.1422E-03	0.1422E-03	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 05	0.7602E-03	0.7602E-03	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 05	0.1528E-01	0.1528E-01	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 05	0.1117E-01	0.1117E-01	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 06	0.4000E 0 0	0.4000E 0 0	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 06	0.7000E 0 0	0.7000E 0 0	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 06	0.1000E 0 0	0.1000E 0 0	1266533102
0.2000E 0 0					
0.1000E-04 0.0	0.3004E 0 0	0.3004E 03	0.1000E 01	0.4004E 0 0	104 52
0.1000E-04 0.0	0.1000E 0 0	0.1000E 04	0.4479E-01	0.4479E-01	1266533102
0.1000E-04 0.0	0.3000E 0 0	0.3000E 03	0.3187E-02	0.3187E-02	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 03	0.3598E-01	0.3598E-01	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 04	0.2638E-01	0.2638E-01	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 04	0.3971E-02	0.3971E-02	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 04	0.3070E-02	0.3070E-02	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 05	0.3207E-01	0.3207E-01	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 05	0.6449E-01	0.6449E-01	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 05	0.4711E-01	0.4711E-01	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 06	0.4000E 0 0	0.4000E 0 0	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 06	0.7000E 0 0	0.7000E 0 0	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 07	0.4000E 0 0	0.4000E 0 0	1266533102
0.1000E-04 0.0	0.6000E 0 0	0.6000E 07	0.7000E 0 0	0.7000E 0 0	1266533102
0.1000E-04 0.0	0.9000E 0 0	0.9000E 07	0.1000E 0 0	0.1000E 0 0	1266533102
0.2000E 0 0					
0.1000E-04 0.0	0.4004E 0 0	0.4004E 03	0.1000E 01	0.5004E 0 0	104 52
0.1000E-04 0.0	0.4004E 0 0	0.4004E 03	0.1000E 01	0.5004E 0 0	1266533102

CORRELATION MATRIX CARDS FOR THE DRAWING

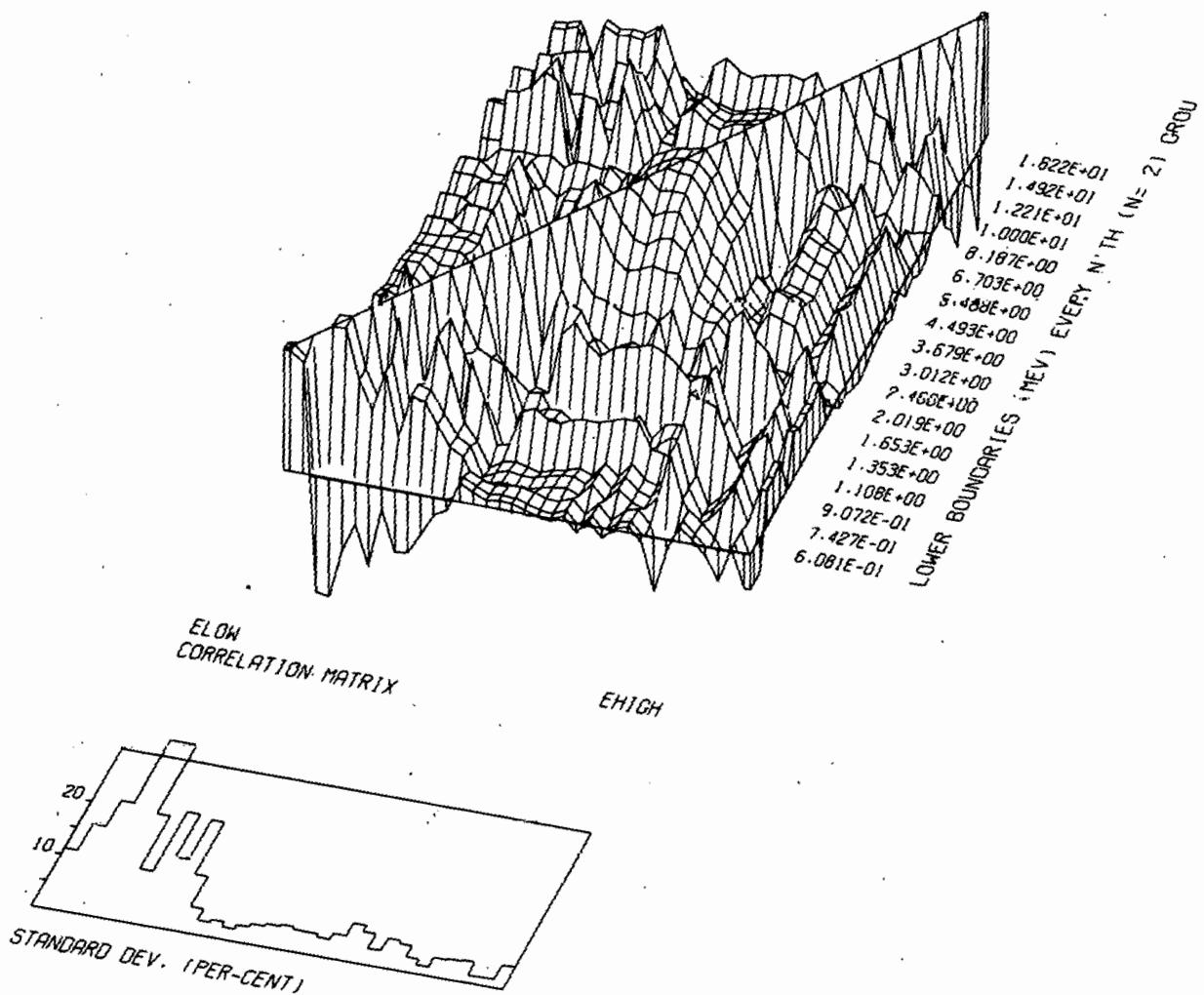
0.1004E-03 0.8750E 01
0.2004E-03 0.5016E 01
0.3004E-03 0.2116E 02
0.4004E-03 0.2470E 01
0.5004E-03 0.6628E 01
0.6004E-03 0.1506E 01
0.7004E-03 0.1714E 02
0.8004E-03 0.4135E 01
0.9004E-03 0.1700E 03
0.10000E-00NNN 0.1338E 02
0.20000E-00NNN 0.1152E 02
0.30000E-00NNN 0.2108E 02
0.50000E-00NNN 0.1094E 02
0.60000E-00NNN 0.1097E 02
0.70000E-00NNN 0.8318E 01
0.80000E-00NNN 0.7929E 01
0.90000E-00NNN 0.8678E 01
0.10000E-01 0.1010E 02
0.20000E-01 0.9736E 01
0.30000E-01 0.2180E 02
0.40000E-01 0.2923E 02
0.50000E-01 0.4848E 02
0.60000E-01 0.3932E 02
0.70000E-01 0.3806E 02
0.80000E-01 0.3633E 02
0.90000E-01 0.3225E 02
0.10000E 00 0.3410E 02
0.20000E 00 0.1904E 02
0.30000E 00 0.3649E 02
0.40000E 00 0.3031E 02
0.50000E 00 0.2638E 02
0.60000E 00 0.2267E 02
0.70000E 00 0.9178E 01
0.80000E 00 0.2439E 01
0.90000E 00 0.1786E 01
0.10000E 01 0.1054E 02
0.20000E 01 0.1435E 02
0.30000E 01 0.2062E 02
0.40000E 01 0.1944E 02
0.50000E 01 0.1607E 02
0.60000E 01 0.1176E 02
0.70000E 01 0.6216E 01
0.80000E 01 0.7843E 01
0.1000E 010.1000E 010.1000E 01-1000E 010.1000E 01-1000E 01-1000E 01
0.2000E 01-1000E 01-1000E 01-2192E -01-3642E 000.5911E 00-1320E 00-2524E -03
0.300E 00-1313E -030.1205E 000.1439E 00-2292E 00-2403E -010.7218E 00
0.4722E 000.8352E 000.7750E 000.7629E 000.7351E 000.6903E 000.7781E 00
0.000.000.000.000.000.000.000.000.000.000.00
0.000.000.000.000.000.000.000.000.000.000.00
0.1000E 010.1000E 010.1000E 01-1000E 01-1000E 010.1000E 01-1000E 01-1000E 01
0.2000E 01-1000E 01-1000E 01-2192E -01-3642E 000.5911E 00-1320E 00-2524E -03
0.300E 00-1313E -030.1205E 000.1439E 00-2292E 00-2403E -010.7218E 00
0.4722E 000.8352E 000.7750E 000.7629E 000.7351E 000.6903E 000.7781E 00
0.000.000.000.000.000.000.000.000.000.000.00
0.1000E 010.1000E 010.1000E 01-1000E 01-1000E 010.1000E 01-1000E 01-1000E 01
0.1000E 01-1000E 01-2192E -01-3642E 000.5911E 00-1320E 00-2524E -03

VI. THE THREE-DIMENSIONAL CORRELATION MATRIX PLOT

As noted, part of the card output of the program can be used to generate a three-dimensional plot of the correlation matrix. This can be seen in Fig. 1, where this plot is shown for the ^{238}U fission cross section.

Several conclusions can be deduced from this plot:

- (1) A lot of structure can be seen. This corresponds to the fact that the cross section was analyzed with very much detail.
- (2) The standard deviation shows large uncertainties in the low-energy region. This reflects the lack of data in this region.
- (3) Positive correlation corresponds to differences in magnitude but not in shape; negative correlation corresponds to differences in shape. Then, differences in shape in the low-energy range can be detected. This reflects the fact that in this zone the cross section rises so fast that differences in cross-section values correspond to a large difference in shape.
- (4) The recent measurement of Behrens et al. correlates all the analyzed energy intervals.⁹



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1. Greebler, P., et al., "Implications of Nuclear Data Uncertainties to Reactor Design," *Nuclear Data for Reactor*, Proc. Conf., Helsinki, 1970, IAEA, vol. I, 17.
2. Barré, Y., et al., "Roles Respectifs des Evaluations et des Experiences Integrales pour la Physique des Reacteurs Rapides," *Applications of Nuclear Data in Science and Technology*, Symp., Paris, 1973.
3. Usachev, L. N., et al., "Nuclear Data Sources and Their Implications on the Design of Fast Reactors," *Applications of Nuclear Data in Science and Technology*, Symp., Paris, 1973.
4. Hummel, H., et al., FRA-TM-54, ANL (1973).
5. Votinen, V. O., et al., "The Light-Water Industry. Nuclear Data Needs," Conf. on Nuclear Cross Sections and Technology, Washington, D.C., March 1975.
6. Minutes of the spring and fall 1973 meeting of the cross-section evaluation group and F. G. Perey. "Format and procedures for ENDF/B data covariance files."

7. Perey, F. G., et al., "Estimated Data Covariance Files of Evaluated Cross Section - Examples for U-235 and U-238," paper presented at ANS meeting, Atlanta, September 1974.
8. Perey, F. G., "Estimated Uncertainties in Nuclear Data. An Approach," Conf. on Nuclear Cross Sections and Technology, Washington, D.C., March 1975.
9. Behrens, J. W., et al., *Conf. of Nuclear Cross Sections and Technology*, paper GB-14, Washington, D.C., March 1975.

APPENDIX A. LIST OF REFERENCES USED IN THE CALCULATION
OF THE COVARIANCE MATRICES

^{238}U Fission: In this case the covariance [COV(I,J)] was calculated for 36 groups from 20 MeV to 550.24 keV in a 0.1 lethergy unit interval. The following set of references was used:

1. Nyer, W., Report LA-719 (1948).
One measurement at 14 MeV. Original data.
2. Uttley, C. A., et al., Report AERE NP/R 1996 (1956).
Two measurements at 14.1 and 15 MeV. Original data.
3. Smith, R. K., et al., *Bull. Am. Phys. Soc.* 2: 196 (1957).
Thirty seven measurements at 1-20 MeV. Results from Sowerby, M. G., et al., Report AERE-R7273 (1973).
4. Billand, P., et al., *Geneva Conf.* 16: 106 (1958).
One measurement at 13.6 MeV. Original data.
5. Flerov, N. N., et al., *J. Nucl. Energy* 11: 173 (1959).
One measurement at 14.6 MeV. Original data.
6. Mangialajo, M., et al., *Nucl. Phys.* 43: 124 (1963).
Eight measurements at 13.74-14.75 MeV. Original data.
7. Barral, R. C., et al., Report AFWL-TR-68-134 (1969).
One measurement at 14.6 MeV. Original data.
8. Henkel, R. L., et al., Report LA-1495 (1952) and Report LA-2122 (1957).
Thirty eight measurements at 1.3-6.92 MeV. Data renormalized to 0.5176b at 4.50 MeV.

9. Sailor, V. L., Report WASH-745, p. 31 (1958).
Four measurements at 8.8-13.86 MeV. Renormalized to 1.109 b at 13.86 MeV.
10. Adams, B., et al., *J. Nucl. Energy* 14: 85 (1961).
Fourteen measurements at 12.7-19.4 MeV. Renormalized to 1.125 b at 14 MeV.
11. Pankratov, V. M., et al., *J. Nucl. Energy* 16: 494 (1962).
Fifteen measurements at 10.6-20.4 MeV. Renormalized to 1.125 b at 14 MeV.
12. Pankratov, V. M., *At. Energ.* 14: 167 (1963).
Thirty six measurements at 5.1-22.5 MeV. Renormalized to 1.125 b at 14 MeV.
13. Emma, V., et al., *Nucl. Phys.* 63: 641 (1965).

The following are ratio measurements relative to the ^{235}U fission cross section. They were reduced to ^{238}U fission data utilizing the ^{235}U fission cross-section ENDF/B-IV evaluation.
14. Nyer, W., Report LA-938 (1950).
One measurement at 14 MeV.
15. Jarvis, G. A., et al., Report LA-1571 (1953).
One measurement at 2.5 MeV.
16. Lamphere, R. W., *Phys. Rev.* 104: 1654 (1956).
Eighty eight measurements at 0.573-2.995 MeV, measurement multiplied by 0.95.

17. Uttley, C. A., et al., Report AERE NP/R 1996 (1956).
One measurement at 14.1 MeV.
18. Kalinin, S. P., et al., *Geneva Conf.* 16: 136 (1958).
Twenty measurements at 2.95-8.30 MeV.
19. Berezin, A. A., et al., *J. Nucl. Energy* 11: 175 (1960).
One measurement at 14.6 MeV.
20. Smirenkin, G. N., et al., *At. Energ.* 13: 974 (1962).
One measurement at 2.5 MeV.
21. Stein, W. E., et al., p. 627 in *Proc. Conf. Neutron Cross Sections and Technology*, vol. I, Washington, D.C., 1968.
Fourteen measurements at 1.5-5 MeV.
22. White, P. H., et al., *J. Nucl. Energy* 21: 671 (1967).
Three measurements at 2.25-14.1 MeV.
23. Grundl, J. A., *Nucl. Sci. Eng.* 30: 39 (1967).
Sixteen measurements at 1.68-8.07 MeV.
24. Poenitz, W. P., et al., *J. Nucl. Energy* 26: 483 (1972).
Three measurements at 2-3 MeV.
25. Meadows, J. W., *Nucl. Sci. Eng.* 49: 310 (1972).
Twenty one measurements at 0.898-5.33 MeV.
26. Behrens, J. W., et al., *Conf. of Nuclear Cross Sections and Technology*, paper GB-14, Washington, D.C., March 1975.
White source, 30.23-0.1077 MeV.

^{238}U Capture. In this case the covariance [COV(I,J)] was calculated for 91 groups for energies ranging from 10 MeV to 100 eV. From 10 MeV down to 4.095 keV, 0.1 lethargy unit intervals were used. The energy range

between 4.095 keV and 100 eV was divided in decimal intervals. The following set of references was used:

1. Rose, B., Report AERE NP/R 1743 (1955). See also *J. Nucl. Energy* 8: 197 (1959).
Eleven measurements at 29-840 keV. Original data.
2. Broda, E., et al., Report BR-754. Reported by Rose in ref. 1.
Seven measurements at 400 keV-4.05 MeV. Data revised by Davey in ref. 19.
3. English and Gueron, National Research Council of Canada, Montreal Laboratory Report MC-69. Reported by Rose in ref. 1.
Two measurements at 220-850 keV. Original data.
4. Macklin, R. L., et al., *Phys. Rev.* 107: 504 (1957).
One measurement at 25 keV. Original data.
5. Lyon, W. S., et al., *Phys. Rev.* 114: 1619 (1959).
One measurement at 195 keV. Original data.
6. Bilpuch, E. G., *Ann. Phys.* 10: 455 (1960).
Thirteen measurements at 2.55-217 keV. Data revised by Davey in ref. 19.
7. Given, B. G., et al., *Phys. Rev.* 120: 556 (1960).
Seven measurements at 0.175-1 MeV. Data revised by Davey in ref. 19.
8. Gibbons, J. H., et al., *Phys. Rev.* 122: 182 (1961).
Two measurements at 30-65 keV. Original data.

9. Bergginst, I., *Ark. Fys.* 23: 425 (1963).
Eight measurements at 18-300 keV. Data revised by Davey in ref. 19.
10. de Saussure, G., et al., Report ORNL-3360 (1963).
Two measurements at 30-64 keV. Data revised by Davey in ref. 19.
11. Macklin, R. L., et al., Report WASH-1046, p. 88 (1963).
Sixteen measurements at 8.8-54.7 keV. Data according to a recent evaluation of σ (Ta) in ref. 19.
12. Tolstikov, V. A., et al., *J. Nucl. Energy* 18: 599 (1964).
Eleven measurements at 15-180 keV. Corrected by non-1/v ^{10}B behavior and normalized to the Belanova (ref. 13) value. Data from Sowerby (ref. 20).
13. Belanova, T. A., et al., *J. Nucl. Energy* 20: 411 (1966).
One measurement at 22.8 keV as corrected by Miller et al. in *Nucl. Sci. Eng.* 35: 295 (1969).
14. Menlove, H. O., et al., *Nucl. Sci. Eng.* 33: 24 (1968).
Nine measurements at 24-503 keV. Original data.
15. Moxon, M. C., Report AERE-R6074 (1969).
White source, time of flight; 0.50-100 keV. Results average in decimal intervals. Date revised in 1970 (ref. 20).
16. Fricke, M. P., et al., IAEA Conf., Helsinki (1970).
White source, time of flight; 1-100 keV. Results average in decimal intervals, 100-752 keV point results. Original data.
17. de Saussure, G., et al., *Nucl. Sci. Eng.* 51: 385 (1973).
White source, time of flight; 0.100-100 keV. Results average over decimal intervals. Original data.

18. Langner, I., et al., Report KFK-880 (1968).
Evaluation.
19. Davey, W. G., *Nucl. Sci. Eng.* 39: 337 (1970).
Evaluation.
20. Sowerby, M. G., et al., Report AERE-R7273 (1973).
Evaluation.
21. Ponitkin, Y. G., et al., Report IAEA-CN-26, Helsinki (1970).
Ten measurements at 24-145 keV. Original data.
22. Ponitkin, Y. G., et al., *At. Energ.* 33: 782 (1972).
Two measurements at 132-160 keV. Original data.
23. Block, R. C., et al., Conf-720901, vol. II, p. 1107 (1972).
One measurement at 24 keV. Original data.
24. Chelnokov, V. B., et al., "Neutron cross section for radioactive capture," preprint USSR Institute of Physics and Power Engineering (1973).
Ten measurements at 3.3-35 keV. Original data.
25. Ryves, T. B., et al., *J. Nucl. Energy* 27: 519 (1973).
Four measurements at 157-624 keV. Original data.
26. Pearlstein, S., et al., UKAEA Progress Report (1973).
Eleven measurements at 200-1750 keV. Original data.
27. Spencer, R. R., et al., private communication, 1975.

The following are ratio measurements relative to the ^{235}U fission cross section. They were reduced to ^{238}U capture data utilizing the ^{235}U fission cross-section ENDF/B-IV evaluation.

28. Linemberg, G. A., et al., Report LA-467 (1946).
Thirteen measurements at 0.005-5.9 MeV. Original data.
29. Barry, J. F., et al., *J. Nucl. Energy* 18: 481 (1964).
Thirteen measurements at 0.127-7.6 MeV. Original data.
30. Weston, L. W., et al., Report EANDC 330, p. 64 (1963).
See also ref. 10.
Two measurements at 30-64 keV. Original data.
31. Poenitz, W. P., *Nucl. Sci. Eng.* 49: 383 (1970) and *Trans. Am. Nucl. Soc.* 12: 279 (1968).
Thirteen measurements at 30 keV-1.4 MeV. Original data.
32. Sowerby, M. G., et al., Report AERE-R7273 (1973).
Evaluation.

^{239}Pu Capture: In this case the covariance [COV(I,J)] was calculated for 36 groups for energies ranging from 1 MeV to 100 eV. The energy range was divided in decimal intervals. The following set of references was used:

1. Sowerby, M. G., et al., *At. Energy Rev.* 10(4): 453 (1972).
IAEA.
2. Gwin, R., et al., *Nucl. Sci. Eng.* 45: 25 (1971).
3. Schomberg, M. G., et al., p. 315 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.
4. Czirr, J. R., et al., p. 331 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.

5. Belyaer, F. N., et al., p. 336 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.
6. Farrell, J. A., et al., p. 543 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.
7. Kononov, V. N., et al., p. 345 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.
8. Ryabov, Yu. V., et al., p. 345 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.
9. Weston, L. W., et al., memorandum to R. E. Chrien, 1972.
10. Gwin, R., et al., to be published in *Nuclear Science Engineering*.

^{239}Pu Fission: In this case the covariance [COV(I,J)] was calculated for 45 groups for energies ranging from 10 MeV to 100 eV. The energy range was divided in decimal intervals. The following set of references was used:

1. Sowerby, M. G., et al., Report AERE-R-7273, UKAEA (1973).
2. Hunter, R. E., et al., Report LA-5172 (1973).
3. Gwin, R., to be published in *Nuclear Science Engineering*.
4. Weston, L. W., memorandum to R. E. Chrien, 1972.
5. James, G. D., p. 267 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.
6. Gwin, R., et al., *Nucl. Sci. Eng.* 45: 25 (1971).
7. Shunk, E. R., et al., p. 979 in *Proc. Conf. Neutron Cross Section Technology*, vol. II, CONF-660303, 1966.
8. Blons, J., *Nucl. Sci. Eng.* 51: 95 (1973).

9. Schomberg, M. G., et al., p. 315 in *Proc. Conf. Nuclear Data for Reactors*, vol. I, IAEA, Helsinki, 1970.

^{240}Pu and ^{241}Pu Capture. ^{241}Pu Fission: In this case the covariance was calculated for 45 groups for energies ranging from 10 MeV to 100 eV. The following sets of references were used:

^{240}Pu Capture Cross Section

1. Weston, L. W., and Todd, J. H., "Measurement of the Neutron Capture Cross Sections of the Actinides," Proceedings of the Conference on Nuclear Cross Sections and Technology, Washington, D.C., 1975, to be published.
100 eV to 300 keV with weight of 9 on the variance.
2. Hockenbury, R. W., Moyer, W. R., and Block, R. C., *Nucl. Sci. Eng.* 49: 153-161 (1972).
6 to 30 keV with weight of 9 on the variance.
3. Prince, A., *Conf. Nucl. Data for Reactors*, CN-26/91, Helsinki, 1970.
1 keV to 10 MeV with weight of 1 on the variance (model calculation).
4. Yiftah, S., Schmidt, J. J., Caner, M., and Segev, M., p. 123 in *Fast Reactor Physics*, vol. I, IAEA, Vienna, 1968.
1 keV to 10 MeV with weight of 1 on the variance (model calculation).

^{241}Pu Capture Cross Section

1. Weston, L. W., and Todd, J. H., "Neutron Fission and Absorption Cross Sections for ^{239}Pu and ^{241}Pu ," *Trans. Am. Nucl. Soc.* 15(1): 480 (1972).
100 eV to 200 keV with weight of 9 on the variance.
2. Caner, M., and Yiftah, S., p. 735 in *Conf. Nucl. Data for Reactors*, vol. II, IAEA, Vienna, 1970.
100 eV to 10 MeV with weight of 1 on the variance (model calculation).
3. Prince, A., *Conf. Nucl. Data for Reactors*, CN-26/91, Helsinki, 1970.
80 keV to 10 MeV with weight of 1 on the variance (model calculation).

 ^{241}Pu Fission Cross Section (all references with weight of 1 on the variance)

1. Weston, L. W., and Todd, J. H., same reference as with ^{241}Pu capture.
100 eV to 200 keV.
2. Simpson, O. D., et al., p. 910 in *Proc. Conf. on Neutron Cross Section and Technology*, vol. II, Washington, D.C., 1966.
100 eV to 10 keV.
3. Blons, J., et al., p. 469 in *Conf. Nucl. Data for Reactors*, vol. I, Vienna, 1970.
4. Migneco, E., Theohald, J. P., and Wartena, J. A., p. 437 in *Conf. Nuclear Data for Reactors*, vol. I, Vienna, 1970.
100 eV to 2 keV.

5. Smith, H. L., Smith, R. K., and Henkel, R. L., *Phys. Rev.* 125: 1329 (1962).
200 keV to 10 MeV.
6. Zabo, I., et al., *Neutron Standards and Flux Normalization*, p. 256, Argonne National Laboratory (1970).
20 keV to 1 MeV.
7. White, P. H., and Warner, G. P., *J. Nucl. Energy* 21: 671 (1967).
40 keV to 6 MeV.
8. Butler, D. K., and Sjoblom, R. L., *Phys. Rev.* 124: 1129 (1961).
300 keV to 2 MeV.

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APPENDIX B. STANDARD DEVIATION AND CORRELATION MATRICES
CORRESPONDING TO DIFFERENT CROSS SECTIONS

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