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Comparison of upstream $T_e$ profiles with divertor heat flux and its implications on parallel and perpendicular transport in the SOL of DIII-D H-mode plasmas

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I. Introduction

On DIII-D we have performed a series of experiments designed to compare the upstream Thomson and midplane probe measurements of $T_e$ with the downstream divertor heat flux width. We find that there is only a weak dependence of the heat flux width on the temperature gradient scale length, in contrast to the strong dependence predicted by simple two-point models [1]. The UEDGE code [2] has also been used to model a number of the discharges and we find that the flows significantly affect the heat flux profiles.

Correlations between the heat flux width and a wide variety of plasma parameters have been made with the result that the only dependence found is on the plasma current, $I_p$. No dependence was found on, $P_{SOL}$, $P_{inj}$, $n_e$, $B_T$, the shear and normalized pressure gradient at the 95% flux surface ($s_{95}$, $\alpha_{95}$), nor collisionality. Comparing the DIII-D data with other multi-machine scaling relations, we find best agreement with the JET scaling [3].

II. Comparison of Upstream and Midplane $T_e$ Profiles

Upstream Thomson profiles of $T_e$ and $n_e$ measured at the upper outer region of the plasma at a major radius of 1.94 m were mapped to the midplane for comparison with the heat flux profiles. The profiles and heat flux data were “edge localized mode (ELM) synchronized” using a method of conditionally averaging the data falling between ELMs over many ELM cycles. Independent exponential fits to the core- and scrape-off layer (SOL)-side data were made to obtain measurements of the profile gradient scale lengths as shown in Fig. 1. The core-side fits were are clearly biased by the pedestal and not used in the subsequent analysis. SOL layer widths inferred from tanh-fits were also analyzed and showed the same trends as the SOL-side fits. To be consistent with other published data SOL-side gradient scale lengths were used in the following analysis.

A database was established by dividing the shot into 200 ms segments and averaging the various plasma parameters and profile data over this period. Additional parameters, such as
shear and normalized pressure gradient, were included in the database in order to perform regression analysis on them.

III. Heat Flux Widths

Divertor temperature profiles were measured with an IRTV camera mounted on the top of the DIII-D vacuum vessel. The heat flux to the target plate was then inferred using the THEODOR [4] code. To arrive at a heat flux width, offset exponentials, \( a_0 + a_1 e^{x/\lambda} \), were fit to both the left hand and right hand sides of the profiles. This was necessary as the baseline on the two sides of the profile were, in general, different. With this method an effective Loarte width (ratio of the integral of the heat flux profile to the peak heat flux) [5] could be calculated resulting in,

\[
\lambda_q = \left( \lambda_{\text{left}} + \lambda_{\text{right}} \right) \frac{R_{\text{div}} B^0_\theta}{R_{\text{mp}} B^0_\theta},
\]

where the factor, \( R_{\text{div}} B^0_\theta / R_{\text{mp}} B^0_\theta \), accounts for the flux expansion between the target plate and midplane. Figure 2 shows a typical outer divertor heat flux profile mapped to the midplane with offset exponentials fit to both sides of the peak.

The heat flux widths were fit to power scalings of \( I_p \), \( P_{\text{SOL}} \), \( P_{\text{inj}} \), \( n_e \), \( B_T \), shear and normalized pressure gradient at the 95% flux surface (\( s_{\text{95}} \), \( \alpha_{\text{95}} \)) and collisionality. The only trend found was an inverse dependence on \( I_p \) as shown in Fig. 3. A weak dependence on \( B_T \) cannot be ruled out, as there were few good points in the data set at low fields (\( B_T < 1.5 \text{ T} \)).
The measured heat flux width, \( \lambda_q \), shows a very weak dependence on the upstream \( T_e \) gradient scale length, \( \lambda_{Te} \), being essentially uncorrelated. This is shown in Fig. 4, which plots \( \lambda_q \) vs \( \lambda_{Te} \) and the fit to the data. This is in strong disagreement with the simple two-point model [1] that predicts \( \lambda_{Te} = (7/2) \lambda_q \). This line is plotted in Fig. 4. Even with the poor correlation, this dependence can be ruled out. The observed weak dependence is not unreasonable as radial transport, SOL radiation, and divertor recycling affect the heat flux within flux tubes; effects not taken into account in the simple model.

**IV. Scaling Relations**

We have compared our results with several other scaling relations developed for the heat flux width. The first is the JET conduction limited scaling relation [3] given by

\[
\lambda_{qJET} (\text{mm}) = 2.41 \times 10^{-5} B_T^{-1} (\text{T}) P_{SOL}^{-1/2} (\text{MW}) n_e^{1/4} (\text{m}^{-3}) q_{95} R^2 (\text{m}) .
\]

This is in quite good agreement with our data as shown in Fig. 5, which plots \( \lambda_{qDIII-D} \) versus \( \lambda_{qJET} \). The bulk of the dependence derives from the variation in \( q_{95}/B_T \sim 1/I_p \). Our data shows no dependence on \( n_e \), while the JET scaling has a very weak \( n_e^{1/4} \) dependence, so there is little variation arising from this term. Our data shows no dependence on \( P_{SOL} \) though the JET scaling relation has a \( P_{SOL}^{1/2} \) dependence. Since \( R \) is a constant for our data, the remaining scaling relation essentially reduces to \( \lambda_{qJET} \sim \lambda_{qH-1} \sim B_T^{-1} q_{95} \sim 1/I_p \). This is quite close to the \( I_p^{-1.24} \) scaling found above (Fig. 3).

We have also considered two other multi-machine scaling relations [5]:

\[
\lambda_{qH-1} (\text{mm}) = 5.2 P_{\text{div}}^{0.44} (\text{MW}) B_T^{0.45} (\text{T}) q_{95}^{0.57} ,
\]
\[
\lambda_{qH-2} (\text{mm}) = 5.4 P^{0.38} (\text{MW}) B_T^{0.71} (\text{T}) q_{95}^{0.30} .
\]

These are in extremely poor agreement with the DIII-D data. The H-1 scaling predicts a dependence on plasma current of approximately \( I_p^{1/2} \) and no
size dependence. The H-2 scaling predicts profile widths a factor of 10 lower than the measurements. In addition, there is no size dependence and there is no manifest dependence on $I_p$.

V. UEDGE Simulations

Efforts are underway to model four representative points on the $I_p$ scan of Fig. 3 with UEDGE [2] in order to determine what underlying physics might be changing with $I_p$ to affect $\lambda_q$. Inputs to UEDGE are the power flux through the SOL and the midplane profiles of $n_e$ and $T_e$. Transport coefficients are adjusted within UEDGE to obtain a match between the experimental and predicted profiles. Results are preliminary but still offer some insight.

Figure 6 shows a comparison of the measured heat flux profile and those obtained from a UEDGE simulation as a function of distance along the target plate for $I_p = 1.5$ MA. With impurities and drifts turned off and with impurities on and drifts off, the predicted profiles are narrow; approximately 50% of the width of the measured profile. Only with the flows (partially) turned on does the profile broaden by producing a shoulder on the SOL side. The profiles on the private flux side are very narrow and nearly constant for the varying conditions. The asymmetry between the private flux region and SOL is also reproduced by the simulation. The results appear to indicate that the plate physics and flows are important in determining the heat flux profile.

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