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Fission-Based Electric Propulsion for Interstellar Precursor Missions

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Abstract. This paper reviews the technology options for a fission-based electric propulsion system for interstellar precursor missions. To achieve a total ΔV of more than 100 km/s in less than a decade of thrusting with an electric propulsion system of 10,000 s Isp requires a specific mass for the power system of less than 35 kg/kWe. Three possible configurations are described: (1) a UZrH-fueled, NaK-cooled reactor with a steam Rankine conversion system, (2) a UN-fueled gas-cooled reactor with a recuperated Brayton conversion system, and (3) a UN-fueled heatpipe-cooled reactor with a recuperated Brayton conversion system. All three of these systems have the potential to meet the specific mass requirements for interstellar precursor missions in the near term. Advanced versions of a fission-based electric propulsion system might travel as much as several light years in 200 years.

INTRODUCTION

Interstellar precursor missions are those which stretch our technical capabilities in the directions needed for later interstellar travel. Nominally they involve missions to beyond Pluto with trip time of less than 20 years and velocities of over 50 km/s. Two potential near-term interstellar precursor missions are a Kuiper Belt Object rendezvous mission and a Heliopause Probe mission. Kuiper Belt Objects (KBOs) are a recently-discovered set of solar system bodies which lie at about the orbit of Pluto (40 AU) out to about 100 astronomical units (AU). There are estimated to be as many as 100,000 KBOs with a diameter greater than 100 km (Malhotra, 1999, Jewitt, 1999). KBOs are postulated to be composed of the pristine material which formed our solar system and may even have organic materials in them. A rendezvous mission including a lander would be needed to perform chemical analysis of the surface and sub-surface composition of KBOs. Although the distance to the KBOs is not exceedingly large, the need to accelerate, coast, and then decelerate for rendezvous results in a total mission delta-V of about 50 km/s. This places it in the precursor category. The heliopause occurs between 100-200 AU, so a mission to there in under 20 years also would require a velocity of around 50 km/s or more. More challenging missions would require a ΔV 's of well over 100 km/s.

To achieve a velocity of 100 km/s without a severe mass penalty for the propellant requires a specific impulse (Isp) of about 10,000 s. Such an Isp can be achieved with electric propulsion. To achieve this velocity in about a decade with this Isp requires a specific mass for the total spacecraft of less than about 45 kg/kWe (at 80% thruster efficiency). Subtracting off 10 kg/kWe for the thrusters, tankage, and remainder of the spacecraft leaves about 35 kg/kWe for the power system. Such a specific power can be provided by near-term fission-based power systems.

Several reactor and electrical conversion systems can be envisioned. Each has different advantages depending on the available launch mass, flight time, and development time and budget. One possible power system is a SNAP-10A derivative (using UZrH fuel) with a steam Rankine cycle conversion system. Another is a gas-cooled reactor (using SP-100 developed UN fuel) with a Brayton conversion system. A third is a heat-pipe-cooled system (with UN or UO₂-fuel) and a Brayton conversion system. None of these options require any new fuels or materials development.

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But they all require good engineering, testing, and integration. The various options and growth potential for the power system will be described and compared for the interstellar precursor missions.

SYSTEM ANALYSIS

To achieve the velocities needed for interstellar precursor missions (i.e., >100 km/s) in times acceptably short for such missions (i.e., < 20 years) requires both a high specific impulse and an adequately high acceleration (or thrust-to-mass ratio). The rocket equation (Sutton, 1986) dictates that the total mass of fuel needed increases exponentially with the ratio of the desired velocity increase (ΔV) over the specific impulse of the fuel (Isp) times the gravitational acceleration on earth (g):

$$m_{fuel} = m_{nonfuel} (\exp(\Delta V / (g I_{sp})) - 1), \quad (1)$$

where m_{fuel} is the mass of the propellant, $m_{nonfuel}$ is the mass of everything that is not propellant (scientific payload, communications, guidance, rocket engines, power supply, etc.). One of the best current chemical propellants is liquid oxygen/liquid hydrogen which has a specific impulse of 460 seconds. To achieve a total ΔV of 100 km/s with a payload, power system, engines, and other inert masses ("non-fuel mass") of 5000 kg would require 21 billion tonnes of fuel. Clearly, chemical propellants cannot do the job. But NASA has recently developed very reliable electric propulsion units with a demonstrated specific impulse of 3300 s (Polk, 1998). An even higher Isp can be achieved either by using a larger voltage on the grids or by using a lighter gas. NASA is also developing a VASIMR electric thruster which is projected to achieve up to 20,000 s Isp (Chang Diaz, 1998). To reach a total ΔV of 100 km/s with a specific impulse of 10,000 seconds and 5000 kg of payload and structure requires only 8.9 tonnes of propellant. This is much more reasonable.

Electric propulsion requires very large amounts of electrical energy. Nuclear fission, in the form of a small research-sized reactor, is a near-term means for obtaining this energy. A fission reactor and power conversion system for space (excluding the radiator) would be about the size of an automobile and similar in power to research reactors found at many universities. The specific mass of the power system and other non-fuel portions of the spacecraft dictates how long it takes to accelerate to 100 km/s. This determines the time required to reach a given distance. For example, if the total non-fuel mass of the spacecraft is 5000 kg, and it delivers 250 kW of electrical power, the dry spacecraft has a total specific mass of 20 kg/kWe. If the electric thrusters are 75% efficient with an Isp of 10,000 s, the spacecraft would have a maximum acceleration of

$$a = \frac{2\eta P}{m_d g I_{sp}} = 7.6 \times 10^{-4} \text{ m/s}^2 = 78 \text{ microgees}, \quad (2)$$

where P is the electrical power, η is the thruster efficiency, m_d is the dry mass, g is the gravitational acceleration on earth, and Isp is the specific impulse. With this acceleration (or thrust-to-weight ratio), it would take 4.2 years to accelerate up to 100 km/s (disregarding the retarding force of the sun and the initial earth-orbital velocity of 29 km/s around the sun). This suggests that the power system needs to have a specific mass of around 20 kg/kWe or less to obtain acceptable acceleration times.

There are two general approaches to fission-based electric propulsion precursor missions. The first approach is to launch the spacecraft into earth orbit and use the electric propulsion system to propel the spacecraft out of earth orbit and then on to interstellar space. The second approach is to launch the spacecraft into an earth-escape trajectory (called a C3=0 trajectory) and then use the electric propulsion system. The advantage of an earth-orbit launch is that it does not require as large a launcher for a given spacecraft weight because the highly-efficient electric propulsion system will be used to get the spacecraft out of earth's gravity well. That will reduce launch costs. A disadvantage is that it requires a longer total burn time for the reactor and electric thruster. It also requires the spacecraft to survive a relatively slow passage through the Van Allen radiation belt. Overcoming these difficulties is one reason to use a C3=0 launch. In addition, the safety analysis might be easier since reactor operation would not begin until an earth-

escape trajectory is achieved. But the penalty for a C3=0 launch is that typically the mass that a given launcher can deliver to earth escape is about three times less mass than it can deliver to low-earth orbit.

A simple orbital mechanics code was developed and used to analyze the requirements for a KBO rendezvous mission, and the results are reported elsewhere in this symposium (Lipinski, 2000). That analysis showed that a KBO rendezvous could be achieved in 13.0 years with a 1000 kg science payload if the dry mass of the rest of the spacecraft (power, thrusters, navigation, communication, tankage, etc.) was 4000 kg and the electrical power was 100 kWe with a thruster total efficiency of 75%. The Isp for this system is 10,000 sec and the launch mass into a C3=0 trajectory is 7984 kg. If essentially the same spacecraft is launched into a 700-km low-earth orbit and an additional 634 kg of electric-thruster propellant is added, the spacecraft can spiral out of earth orbit and rendezvous in a total of 14.3 years. With a variable-Isp thruster (3,000 to 10,000 sec) such as VASIMR, the trip from LEO to KBO rendezvous can be made in 13.5 years with a LEO launch mass of 10,300 kg. The total ΔV for the trip are 45.9, 53.4, and 53.4 km/s, respectively, for the three options.

TUG DESIGN

As described previously, the specific mass of the electric power system for interstellar precursor missions should be about 35 kg/kW or less. This is higher in power and lower in specific mass (kg/kWe) than any space nuclear power system that has been fielded in the past, but quite reasonable for estimated masses of near-term space-reactor systems. The choice of technologies for the reactor system is fairly wide, although there is no off-the-shelf space-reactor system presently available. The SNAP program flew one space reactor in 1965 and built six other working reactors during the program. All of these used UZrH fuel with a NaK coolant (Anderson, 1983). The subsequent SP-100 program designed a reactor which used UN fuel with a Li coolant (Mondt, 1994; Mondt, 1995; El-Genk, 1994). The Russian Rorsat reactors (about 30 flown in space) used UMo fuel (Angelo, 1985). The Russian Topaz II reactor used UO₂ fuel and UZrH moderator. There are numerous other proposed designs in the literature (Angelo, 1985; El-Genk, 1994).

Given the need for a small reactor and shield mass, and a high conversion efficiency, we propose consideration of the following three options for the KBO mission: (1) a UZrH-fueled, NaK-cooled reactor with a steam Rankine conversion system, (2) a UN-fueled gas-cooled reactor with a recuperated Brayton conversion system, and (3) a UN-fueled heatpipe-cooled reactor with a recuperated Brayton conversion system. Table 1 summarizes the key features for 100-kWe systems. All three systems use the same assumed low-mass deployable thermal radiator to maximize the conversion efficiency. Subsequent sections describe the options in more detail, and examples for the KBO mission are given in a companion paper in this conference (Lipinski, 2000). There has not been sufficient detailing of the designs to perform an accurate weight estimate for each system, but rough estimates indicate that all of these systems have the potential to have specific masses less than 35 kg/kWe.

TABLE 2. Comparison of three electric power systems for the KBO mission.

Component	UZrH/NaK-cooled/Rankine	UN/gas-cooled/Brayton	UN/Heatpipe-cooled/Brayton
Electric power (kW)	100	100	100
Thermal power (kW)	345	220	333
Thermal efficiency (%)	29	46	30
Nuclear fuel	UZrH	UN	UN
Primary coolant	NaK	He/Xe	Na
Fuel clad material	Hastelloy	Nb1%Zr/Re	Nb1%Zr/Re
Vessel material	316 SS	Super Alloy	Mo
Conversion cycle	Rankine	Brayton	Brayton
Energy conversion working fluid	water	He/Xe	He/Xe
Thermal radiator type	heatpipe/fin	heatpipe/fin	heatpipe/fin
Radiator working fluid	ammonia	ammonia	ammonia
Reactor coolant exit temp (K)	723	1200	1200

ZrH-Fueled NaK-Cooled Reactor with Rankine Conversion System

The SNAP series of reactors used UZrH fuel and a "thermal" neutron spectrum. That is, the neutrons released by fission were slowed down by collisions with the hydrogen in the fuel so that they could interact with the uranium more easily. This results in a minimum amount of fuel needed to achieve a self-sustaining reaction, which allows the reactor, and also the radiation shield, to be near minimum mass. The SNAP program produced six complete operating reactor systems at various power levels in the 1960s (Angelo, 1985). One system, SNAP-10A, was flown in space (see Figure 1).

The fuel type used in the SNAP series (UZrH) is the same as is used in numerous research reactors throughout the U.S. It was specifically designed for this class of research reactor because of its inherently safe response to temperature changes, automatically reducing the number of fissions if the reactor temperature increases. This feature also allows the system to adjust for load fluctuations without having to move any control elements.

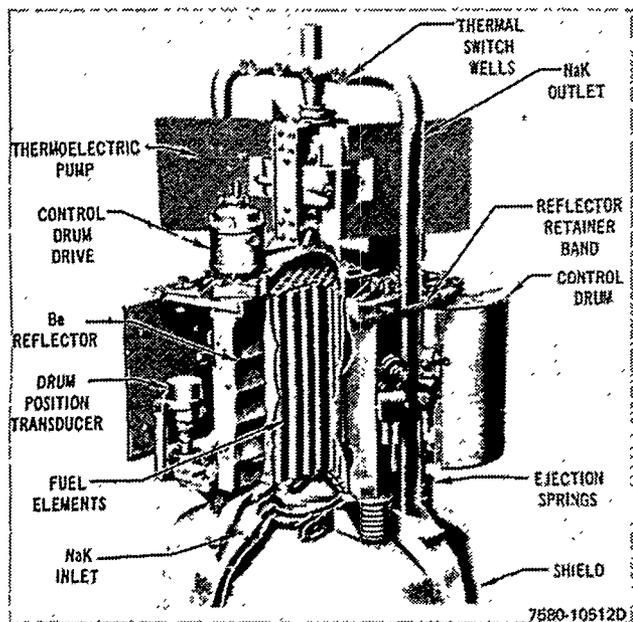


FIGURE 1. SNAP-10A Reactor

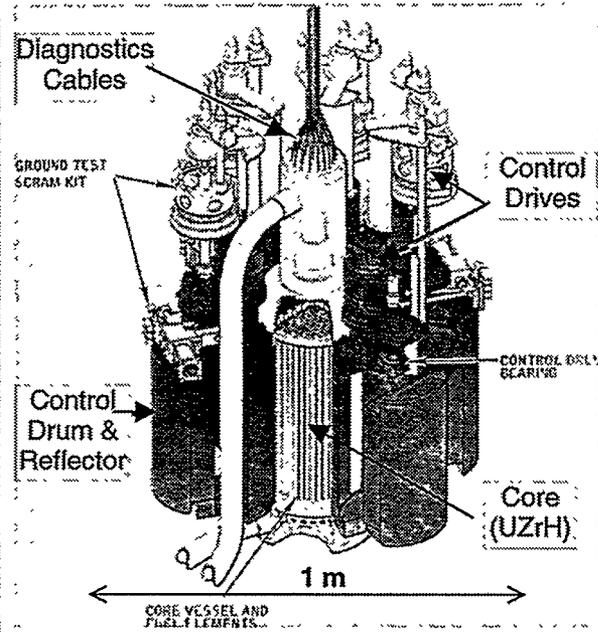


FIGURE 2. SNAP-8DR Reactor.

There were two SNAP reactors which produced 600 kW of thermal power (kW_{th}): SNAP-8ER and SNAP-8DR. The presently envisioned design would be similar to SNAP-8DR (see Figure 2). We would operate the reactor with a peak fuel temperature of only 800 K to extend the lifetime of the fuel. An enrichment of 93% is preferred, but a lower enrichment is also possible. The SNAP-10A spaceflight reactor and shield weighed about 268 and 98 kg respectively, summing to nearly 366 kg. To allow for a larger power and total burnup capability, we estimate the reactor mass would be 500 kg. To allow for shielding a large radiator, we estimate the shield would be 400 kg.

UZrH fuel cannot be operated at as high a temperature as UN fuel. This necessitates the use of a low-temperature conversion system such as a steam Rankine conversion. Steam Rankine systems are a highly mature technology on earth with an extremely large industrial and extensive experience with reactor systems. However, they have never been tested or used in space, and this represents a major technical risk for this option. Figure 3 shows an overview of the conversion cycle. Heat is extracted from the reactor coolant (NaK) and converted to steam. The steam drives a turbine as it expands, and then condenses at a heat exchanger connected to the thermal radiator. A pump recirculates the condensed water back to the boiler. The turbine, pump, and alternator are all on a single shaft floating on a liquid bearing. The system has one moving element. The radiator consists of many parallel and separate capillary pumped loops with ammonia as the coolant. The temperature increase in the steam generator

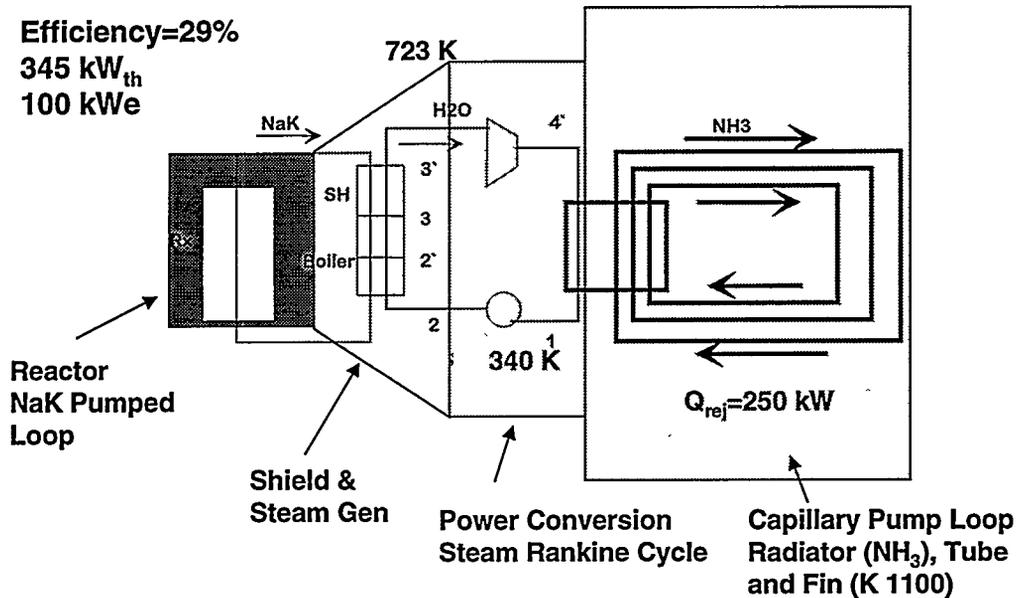


FIGURE 3. Overview of Rankine conversion system.

and superheater is about 383 K, but the temperature increase in the reactor coolant can be considerably less by designing the steam generator appropriately.

The Department of Energy and NASA's Jet Propulsion Laboratory developed operational hardware for a 25-kW toluene Rankine system driven by solar thermal energy (Nesmith 1985). The program also generated several conceptual designs for 100 kWe systems. We estimate the weight of a TAP with a multi-stage turbine for the KBO mission would be about 90 kg. The additional components for conversion system components (excluding the radiator) are estimated to be about 160 kg, summing to a total of 250 kg.

A major key to success for this system is development of a large lightweight (deployable) radiator. This allows heat rejection to space at a low temperature, which can result in a high conversion efficiency overall. NASA and commercial firms have been working on such radiators (some of the designs and specific diagrams are proprietary). The most advanced designs use an array of parallel capillary pumped loops with thermal fins between them. Ammonia is typically used as the coolant. The use of many separate loops minimizes the need for shielding from micrometeor impact. The total mass of these radiators is about 2 kg/m^2 (which would be 1 kg/m^2 of radiating surface if used in a flat configuration with two-sided radiation). Capillary pumped loops, loop heat pipes, and heat pipes have all been demonstrated in space, but these new lightweight radiator configurations have not.

The total radiating area needed for the baseline design is about 360 m^2 at 330 K. Traditionally space reactor radiators have been designed as conical with only the outside surface radiating. This was done to maximize the radiator area behind the radiation shield. However, to allow easier deployment of the radiator, to minimize the radiator mass, and to help reduce the shield mass, we presently envision the radiator as being flat with both sides radiating. Thus the physical radiator would be 180 m^2 with a total radiating surface of 360 m^2 . The radiator would deploy from a manifold extending along the boom. With this potential design and the mass numbers quoted for small advanced systems as background, the radiator system is estimated to weigh 360 kg.

The sum of all these estimated weights (reactor, shield, conversion, radiator) is 1510 kg. Structure, additional controls, other components, and contingency will add to this total estimate, but staying below the 2350-kg limit seems achievable.

UN-Fueled Gas-Cooled Reactor with Brayton Conversion System

The most recent U.S. space reactor power program, SP-100, developed detailed designs, advanced reactor fuel, a "zero-power" reactor critical assembly, radiation-hardened control drives, and various other hardware components in the 1980s. The baseline SP-100 was designed to produce 2400 kW thermal and 100 kW electric with a lifetime of 7 to 10 years (Mondt, 1994, Mondt 1995, El-Genk 1994). The projected specific power at program termination was about 42 kg/kWe. It used a high-temperature advanced fuel (UN) which was developed and proven with nuclear burn-up tests during the program. The fuel was not designed to slow down the fission neutrons, so the neutron spectrum was "fast" and the resulting core size and U-235 mass for the reactor to achieve criticality was thus larger than for the SNAP series.

The proposed design consists of a gas-cooled fission reactor with a closed Brayton cycle for power conversion at 100 kWe. The main difference is replacement of the 4.2% efficient thermoelectric conversion system with a closed Brayton cycle and generator to obtain about 46% conversion efficiency. Such a high efficiency is achieved by using a large thermal radiator, which allows a lower thermal sink temperature. There is a very extensive industrial data base and fabrication experience for open-cycle Brayton units: they form the basis for commercial and military jet engines as well as helicopter engines. Brayton conversion systems have one moving part: a single shaft connected to the turbine, the electrical generator rotor, and the compressor. In closed systems, this single shaft floats on a gas bearing bled off from the main gas flow and returned to it. A 52,000-hr ground test of a 10.7-kWe closed Brayton unit was conducted at NASA/LeRC in 1965.

The reactor is gas cooled (30/70 mole-% He/Xe) to couple better with the Brayton system. The fuel is uranium nitride (UN), which is the same fuel extensively tested for longevity in the SP-100 program. The active core is 0.40 m in diameter and 0.5 m long. The radial reflector is 0.15-m thick beryllium, and the axial reflector is 0.10-m thick BeO. The fuel rods are held in a lattice of BeO, which provides a small amount of moderation. There is a strong negative thermal feedback which allows the reactor power to naturally follow variations in load without needing adjustment of the control elements.

Figure 4 shows a schematic of the Brayton cycle and the associated state points. The reference design produces 100 kWe with 46% total thermal efficiency and has a specific mass of about 26 kg/kWe. A key feature is the heat

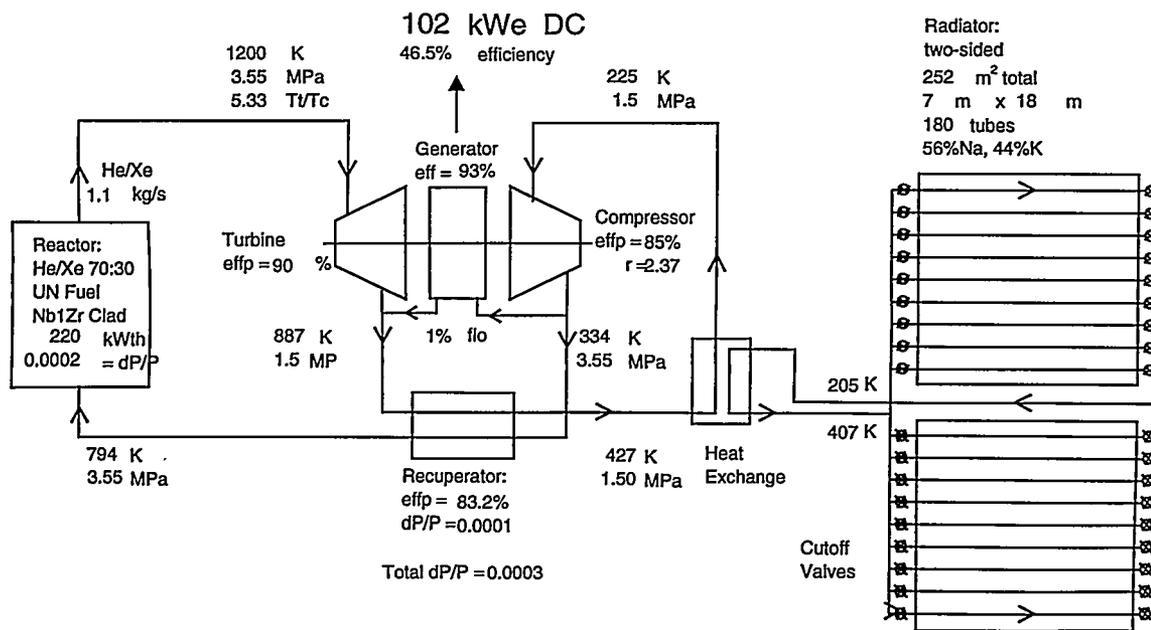


FIGURE 4. Brayton cycle schematic and associated state points.

exchanger which recuperates some heat from the turbine exit and uses it to preheat the gas returning from the heat sink before it re-enters the reactor. This recuperation step gives the cycle a greater conversion efficiency than a

“non-recuperated” Brayton cycle. The radiator is opened out flat to radiate from two sides and is 126 m² per side. This allows a low heat-sink temperature and enables the high electrical conversion efficiency. Detailed weight estimates are not available for this configuration, but the larger reactor size will likely make it heavier than the UZrH Rankine system.

UN-Fueled Heatpipe-Cooled Reactor with Brayton Conversion System

Houts, Poston, and Emrich have reported on various designs for heatpipe cooled reactors (Houts, 1998; Poston, 1996). In the heatpipe power system (HPS) heatpipes are inserted into the reactor core at regular locations to remove heat by boiling and wicking of the coolant. This heat is then transferred via a heat exchanger to the working fluid of a conversion system. For our application, we would use UN fuel in the reactor, sodium in the heat pipe, and He/Xe in the Brayton conversion system. Thus the system state points would be very similar to the gas-cooled Brayton system. Figure 5 shows a fuel/heatpipe module and a cross section of the reactor.

There are numerous advantages and features of the HPS that make it an attractive near-term system:

1. **Safety.** The HPS is designed to be subcritical for all credible launch accidents.
2. **Reliability.** The HPS has no single-point failures.
3. **Long life.** The design lifetime is in excess of 10 years.
4. **Modularity.** The HPS consists of independent fuel/heatpipe modules which can be tested individually.
5. **Testability.** The HPS system launch hardware can be tested at full power using electrical heaters in place of fuel rods. Unirradiated fuel rods inserted before launch. Full-power nuclear tests might not be required.
6. **Versatility.** The HPS can use a variety of fuel forms, structural materials, coolants, and conversion systems.
7. **Scalability.** The HPS design scales well to beyond 1000 kW thermal power.
8. **Simplicity.** There are few system integration issues since there are no in-core shutdown rods, no hermetically sealed refractory metal vessel or flowing loops, no electromagnetic pumps, no coolant thaw systems, no gas separators, and no auxiliary coolant loop for decay heat removal.
9. **Fabricability.** Most of the fabricated parts are small modules with similar metals; there is no pressure vessel.
10. **Near Term.** The system needs no development of advanced materials or components. It can be developed quickly and inexpensively with few nuclear tests.
11. **Low Mass.** The HPS system has a high fuel fraction in the core since it uses no in-core shutdown rods. The potential for in-space fueling (because of no pressure vessel) allows a more compact form while still meeting launch safety requirements.

LAUNCH APPROVAL

There is a precedent for operating small research-sized reactors in space. There are presently over 30 shut-down nuclear reactors orbiting earth at about 600 km altitude. All but one of these are Russian reactors from Rorsat high-power radar satellites. The one U.S. reactor is SNAP-10A, launched in 1965. Every U.S. launch of a payload involving nuclear material must be reviewed by an Interagency Nuclear Safety Review Panel (INSRP) (Sholtis, 1994). The INSRP reviews the sponsors assessment of the risk and reports to the Office of Science and Technology Policy under the Executive Branch. The President or his designee (usually the Science Advisor) then decides whether to grant launch approval. This process has been followed for 25 launches of nuclear materials over the past 40 years and approval has always been granted. All but one of these launches have involved radioisotope power sources, but a space reactor would follow the same process.

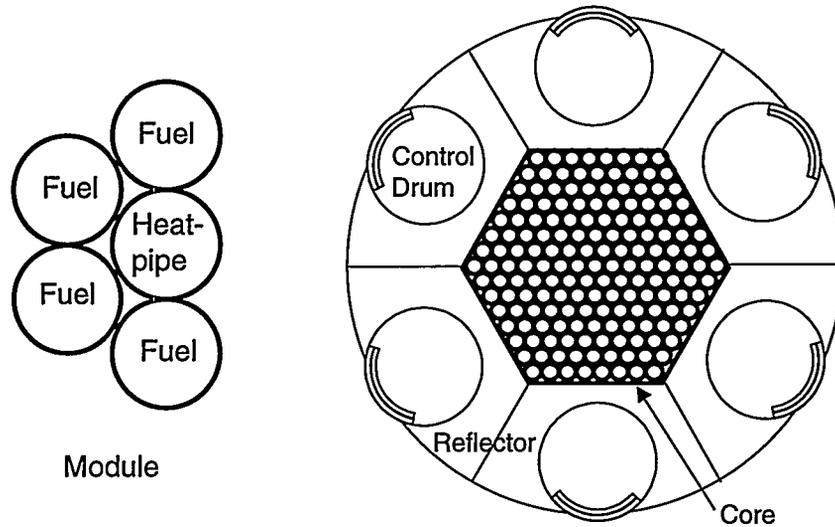


FIGURE 5. Module and reactor for heatpipe power system.

One key safety feature of a fission reactor is that it is barely radioactive before it is used. The radiological inventory in the fresh fuel is about the same as a truckload of uranium ore, with which uranium miners work safely in nations around the world. The space reactor can be tested prior to launch at essentially zero power to prevent any inventory buildup, so the primary concern for launch safety will be to assure that the reactor will not turn on for any conceivable launch accident scenario. This requires solid engineering, but is not difficult. In addition, once the reactor begins operation in space, it should be assured that it cannot reenter the earth's atmosphere. With high circular orbits and low-thrust ion-propulsion, this should be fairly straightforward. Once it is on its way away from earth, the safety issues vanish, and waste disposal is automatic.

POTENTIAL FOR INTERSTELLAR FLIGHT

To have ready access to nearby stars with travel times less than a few years will require speeds faster than the speed of light and "breakthrough" physics. We may all hope for that future day, but in the meantime, we should consider what is feasible using known laws of physics and extrapolations of present-day technologies.

For journeys far outside the solar system, the effect of the sun's gravity and the orbital velocity of the earth may be neglected. Then, for a spacecraft with electric thrusters running at a steady constant power, derivation of the velocity and distance traveled in a given time of thrusting is simply a matter of integrating equation (2) with m_d replaced with the total mass (including propellant), which is time dependent. The final velocity (v_b) after a total thrusting time (or "burn" time) of t_b and the distance the spacecraft has traveled (d) after thrusting for that duration and then coasting for an additional duration of t_c are:

$$v_b = gI_{sp} \ln \left(\frac{t_b}{\tau} + 1 \right), \quad (3)$$

$$d = gI_{sp} \left(t_b - \tau \ln \left(\frac{t_b}{\tau} + 1 \right) \right) + v_b t_c. \quad (4)$$

Here τ is a characteristic time which is equal to the time that it takes for the thrusters to consume an amount of propellant equal to the total dry mass of the spacecraft:

$$\tau = \frac{m_d (g I_{sp})^2}{2\eta P} \quad (5)$$

Note that equation (3) is the classical rocket equation in a different form. There is an optimum I_{sp} which results in a maximum d for a given set of t_b , t_c , η , P , and m_d .

We may use these relations to estimate what performance fission-based systems might be able to deliver. Since travel to the nearby stars without new physics is extremely challenging, we will allow a leisurely 200 years for the total mission. Figure 6 shows the total distance traveled after 200 years for various burn durations up to 50 years; the final velocity after 50 years of burn is also shown. The various curves represent different total specific masses for the spacecraft without propellant (m_d/P). Fission power systems at the 1-MW level using present technology should be able to reach 10 kg/kWe, and higher-powered systems with more advanced reactors and conversion systems should be able to reach 1 or 0.1 kg/kWe. A very large system with advanced technology might reach 0.01 kg/kWe. Each curve has a fixed specific impulse which was chosen to maximize the distance traveled for most of the span of the curve.

The figure shows that this technology can bring us a long distance toward interstellar flight. The number of AUs indicated on the right side of the figure is very large and shows just how far this system can penetrate very deep space. That we can even envision travel to a light year without tapping a large fraction of earth's resources and without postulating nation-sized propulsion systems is also impressive. The burn time is not unreasonable; terrestrial reactors are expected to last about 50 years. Refueling the reactor can be accommodated by inclusion of replacement nuclear fuel in the determination of the dry mass. Or, it may be considered as part of the overall "propellant", thrown overboard at zero velocity, and averaged in with the electric-propulsion propellant velocity to yield an effective specific impulse. The same can be done for the electric thrusters which would need replacement. (Most of the curves resulted in about five dry masses worth of propellant being consumed.) The specific impulses are challenging but not unreasonable for large systems.

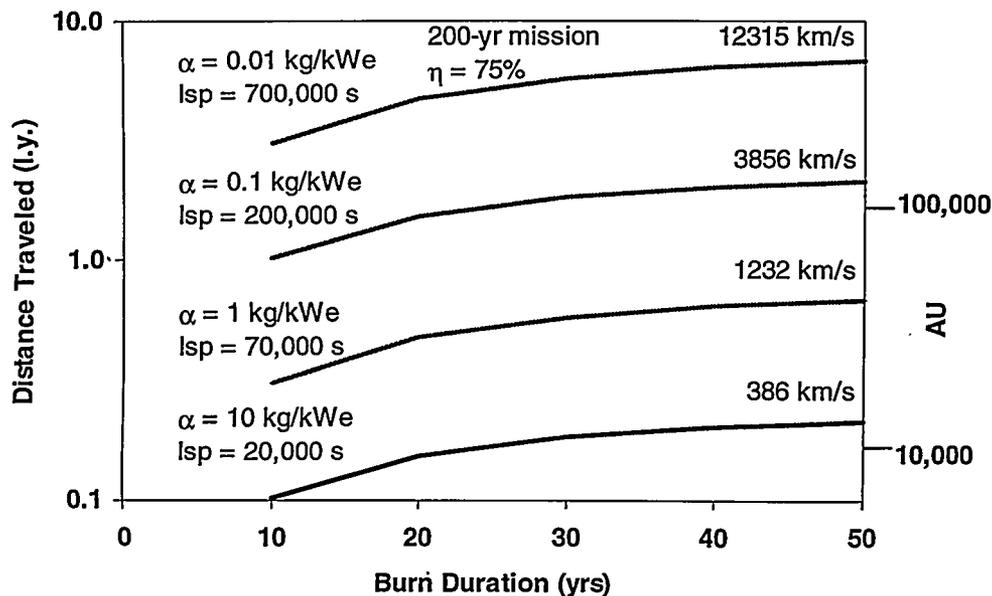


FIGURE 6. Distance traveled in 200 years by a fission-powered electric-propulsion spacecraft.

SUMMARY

Three possible power sources for a fission-based electric propulsion system are described: (1) a UZrH-fueled, NaK-cooled reactor with a steam Rankine conversion system, (2) a UN-fueled gas-cooled reactor with a recuperated Brayton conversion system, and (3) a UN-fueled heatpipe-cooled reactor with a recuperated Brayton conversion system. All three of these systems have the potential to meet the specific mass requirements for interstellar precursor missions capable of 100 km/s in the near term. Advanced versions of a fission-based electric propulsion system might travel as much as several light years in 200 years.

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