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831,017 DETERMINATION OF ACTINIDES IN URINE AND FECAL SAMPLES Terry Trent Mc Kibbin DE-AC07-84ID12435

DETERMINATION OF ACTINIDES IN URINE AND FECAL SAMPLES

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DETERMINATION OF ACTINIDES IN URINE AND FECAL SAMPLES

Contractual Origin Of The Invention

The United States Government has rights in this invention pursuant to Contract No. DE-AC07-84ID12435 between the United States Department of Energy and Westinghouse Idaho Nuclear Company, Inc.

FIELD OF THE INVENTION

This invention relates to a method of separating and then analyzing urine and fecal samples for trace amounts of uranium (U), plutonium (Pu), and americium (Am) within the same sample.

BACKGROUND OF THE INVENTION

In the event of ingestion of certain radioactive actinides, it is desirable to monitor the discharge or release of these actinides by counting on an alpha spectrometer the body discharges within both urine and fecal matter.

In the case of urine, the sample is boiled to dryness in an acid. The dried urine sample is then ashed in a muffle furnace and then dissolved in another acid. The actinides are then gathered on a ferric oxide precipitate.

In the case of a fecal sample, there is a similar sample preparation process consisting of drying, ashing in the muf-

file oven, treating the ash with a mixture of acids and then precipitating with a ferrous phosphate.

In a prior art, fecal sample procedure developed for the Department of Energy, identified as USDOE, RESL/ID, A-16, 1981, the precipitate carrying the actinides is reprecipitated on calcium fluoride further separated the actinides from aluminum, iron, titanium, and protactinium. The calcium fluoride is dissolved in acidic aluminum nitrate, and the actinide series elements thorium through plutonium are extracted into a quaternary amine. Selective back extractions further separate the actinides from each other. The uranium accompanies the plutonium through the extraction but is separated from the plutonium in the cerium fluoride precipitation step; the uranium staying in the filtrate. The uranium can be recovered by the addition of titanous chloride and more cerium carrier to the fluoride filtrate. All fractions are electrodeposited, or carried on cerium fluoride, on filter paper, and analyzed by alpha spectrometry for isotopic identification and quantification.

The DOE process creates some hazardous wastes, including perchlorated organic liquid, i.e., aliphatic quaternary amine.

It is, therefore, a purpose of this invention to provide a simpler, cost effective procedure that generates less hazardous wastes that can be easily disposed.

SUMMARY OF THE INVENTION

The invention generally stated is a process for analyzing urine and feces for trace amounts of actinides, i.e. uranium (U), plutonium (Pu), and americium (Am) by use of a chloride form anion exchange resin consisting of:

dissolving a precipitate containing the actinides in a solution of hydrochloric acid (HCl) and hydrofluoric acid (HF);

reducing the dissolved precipitate;

adding a neodymium (Nd) carrier, thereby forming a neodymium fluoride (NdF) precipitate;

dissolving the NdF precipitate, adding ammonium hydroxide and reprecipitating as neodymium hydroxide ($\text{Nd}(\text{OH})_3$);

dissolving the $\text{Nd}(\text{OH})_3$ precipitate in a dilute nitric acid (HNO_3);

treating the dissolved $\text{Nd}(\text{OH})_3$ with sodium nitrite, thereby adjusting the oxidation state of the plutonium (Pu) to a valence of four (IV);

reprecipitating the dissolved $\text{Nd}(\text{OH})_3$ by the addition of ammonium hydroxide (NH_4OH);

dissolving the reprecipitated $\text{Nd}(\text{OH})_3$ by the addition of hydrochloric acid;

passing the dissolved $\text{Nd}(\text{OH})_3$ through a chloride form anion exchange resin;

washing the resin in hydrochloric acid, thereby removing americium (Am) and salts from the resin;

eluting the uranium (U) and plutonium (Pu) with a 1 M hydrochloric acid and adding an Nd carrier;

5 adding hydrofluoric acid (HF) to the eluant, thereby forming a neodymium fluoride (NdF) precipitate carrying tetravalent plutonium (Pu);

filtering the NdF precipitate, thereby forming a filtered precipitate and a filtrate;

10 counting a plutonium activity of the filtered precipitate on an alpha spectrometer;

treating the filtrate with titanium trichloride (TiCl_3), thereby reducing the uranium;

15 adding a neodymium carrier to the filtrate thereby forming a NdF precipitate carrying uranium;

filtering the NdF precipitate;

counting a uranium activity of the filtered precipitate on the alpha spectrometer;

20 adding ammonium hydroxide to a remaining eluant portion, thereby forming a precipitate carrying americium;

dissolving the precipitate;

complexing the americium with EDTA forming a solution;

25 neutralizing the solution with ammonium hydroxide and discarding the precipitate;

coprecipitating the americium on a neodymium hydroxide precipitate using potassium hydroxide;
oxidizing the americium to a valence of six (VI);
selectively reducing any cerium present;
5 precipitating the neodymium fluoride (NdF);
centrifuging and discarding the NdF precipitate;
reducing the Am (VI) to Am (III);
adding an Nd carrier;
letting the solution stand for 15 minutes;
10 filtering on a 0.1 μm filter; and then
counting an americium activity of the filtered precipitate on the alpha spectrometer.

Other objects, advantages, and capabilities of the present invention will become more apparent as the description
15 proceeds.

DETAILED DESCRIPTION OF THE INVENTION

Urine is initially treated by boiling to dryness with nitric acid. The dry organic material is destroyed in a
20 muffle furnace, and the resulting residue is dissolved in dilute hydrochloric acid. The actinides are then gathered on a ferric hydroxide precipitate.

The initial treatment of the fecal sample is generally described as follows. The fecal sample is dried on a hot
25 plate, ignited, and muffled in a furnace at 550°C for sixteen hours. Up to five grams of ash is treated with hydrochloric,

nitric, perchloric, and hydrofluoric acids to remove organic and siliceous material. The slurry of salts remaining are taken through a pyrophosphate fusion on a hot plate to facilitate the complete dissolution in 2 Molar (M) HCl. Iron(III) is added to act as a carrier. A reduction is performed using ascorbic acid and titanium trichloride. The sample pH is carefully adjusted to 3.0-3.5 with ammonium hydroxide which precipitates iron(II) phosphate. The americium, reduced uranium, and plutonium are carried on the iron phosphate precipitate.

The precipitate containing the actinides from either of the above initial treatments is dissolved in a 2 M HCl and HF solution. A reduction is performed using $TiCl_3$, a neodymium carrier is added, and the actinides are gathered on the precipitated neodymium fluoride. The fluoride precipitate is dissolved in a nitric/boric acid mixture and reprecipitated as $Nd(OH)_3$. The hydroxide precipitate is dissolved in dilute nitric acid, and the solution is treated with sodium nitrite to adjust the oxidation state of the plutonium to a valence of four, IV.

Neodymium hydroxide is reprecipitated using ammonium hydroxide and then dissolved with HCl to obtain an 8-10 M HCl solution. The solution is passed through a 100-200 mesh 1-X8 chloride form anion exchange resin manufactured by Bio-Rad Co. using a 4 cm x 1 cm diameter column. The 1-X8 resin refers to a cross-linking within the resin structure. The

resin is washed with up to 25 mL of 10 M HCl to remove americium and salts from the resin. The uranium and plutonium are elutriated together with 10-12 mL of 1 M HCl.

5 The plutonium and uranium are separated using sequential neodymium fluoride precipitations. Fifty micrograms of neodymium and 1-2 mL of hydrofluoric acid is added to the eluant. Neodymium fluoride precipitates carrying the tetravalent plutonium. The precipitate is filtered onto a 0.1 micron membrane filter and subsequently counted on an alpha spectrometer. The filtrate containing the uranium is treated
10 with $TiCl_3$ to reduce the uranium, and another 50 micrograms of neodymium carrier is added. The neodymium fluoride containing the uranium is collected on a 0.1 micron membrane filter and counted as above.

15 The americium is coprecipitated on a neodymium hydroxide precipitate. The precipitate is dissolved, the americium complexed with EDTA, and the solution is barely neutralized to precipitate interfering cations. The americium is then gathered on a neodymium hydroxide precipitate using strong
20 base, such as potassium hydroxide. The final clean-up from the rare earths and thorium employs an oxidation of the americium to the VI valence and precipitation of the thorium and rare earths as fluorides. A selective reduction and a neodymium fluoride precipitation are used to gather the americium
25 um for counting using alpha spectrometry.

The process above creates wastes as follows: aqueous acids and bases and solid anion exchange resin, which are more easily disposed of.

5 While a preferred embodiment of the invention has been disclosed, various modes of carrying out the principles disclosed herein are contemplated as being within the scope of the following claims. Therefore, it is understood that the scope of the invention is not to be limited except as otherwise set forth in the claims.

ABSTRACT OF THE DISCLOSURE

A method of determining the radioactivity of specific actinides that are carried in urine or fecal sample material is disclosed. The samples are ashed in a muffle furnace, dissolved in an acid, and then treated in a series of steps of reduction, oxidation, dissolution, and precipitation, including a unique step of passing a solution through a chloride form anion exchange resin for separation of uranium and plutonium from americium.

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