

Rev 04-79)

## APPENDIX B

## PRETEST FUEL ROD CHARACTERIZATION

The pretest characterization of the four Test RIA 1-1 fuel rods and flow shrouds is presented in this appendix. The test rods were selected from 68 irradiated MAPI<sup>a</sup> rods and 35 unirradiated Saxton rods. The preirradiation characterization provides referential information for use in analytical models and for posttest comparison.

## 1. FUEL ROD DESIGN

The test fuel rods contained a 0.914-m-long fuel stack composed of 60 ceramic fuel pellets [nominally 94% theoretical density, 5.7 wt% (irradiated) and 5.8 wt% (unirradiated), enriched UO<sub>2</sub>], each nominally 8.5 mm in diameter and 15.2 mm long. The fuel pellets were contained in a zircaloy-4 cladding tube of nominally 9.9-mm outside diameter, and 0.62- (irradiated) and 0.53-mm (unirradiated) wall thicknesses. The nominal fuel-cladding diametral gap was 0.828 mm for the previously irradiated rods and 0.867 mm for the previously unirradiated rods.

Five test fuel rods were used in Test RIA 1-1. Two MAPI fuel rods, previously irradiated to a burnup of approximately 4600 MWd/tU (0.49 at.%) in the Saxton reactor were selected for use in the test.  $^{B-1}$  Three previously unirradiated fuel rods were also used in the test and were built by EG&G Idaho, Inc., using cladding provided to the U.S. Department of Energy by Westinghouse Electric Corporation. The two irradiated rods were designated Rods 801-1 and 801-2, and the three previously unirradiated rods were designated Rods 801-3, 801-4, and 801-5. Previously unirradiated Rod 801-4 was removed from the test assembly after the power calibration and preconditioning phases of the test were completed, and was replaced prior to the

a. The MAPI rods were built by Westinghouse Electric Co., and irradiated in the Saxton reactor for the Mitsubishi Atomic Power Industries, Inc., Tokyo, Japan.

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test power burst with Rod 801-5. Rod 801-5, which experienced only the test power burst, was used for the radiochemical determination of burnup to calculate the energy produced. The rod designations and prior irradiation history are summarized in Table B-1. The nominal design characteristics of these rods are listed in Table B-2.

	Wastinghouse Fleetric	
Teet DTA 1-1	Westinghouse Electric	
Rod Identification	Rod Identification	Average Burnup (MWd/tU)
801-1	M-42	4600
801-2	M-9	4650
801-3	958	Unirradiated
801-4	951	Unirradiated
801-5	950	Unirradiated

Table B-1. Test RIA 1-1 fuel rod designations and burnup

The cladding inside diameters were measured at angular orientations of 0 and 90 degrees, at increments of 25.4 mm on each of the three previously unirradiated fuel rods. The cladding outside diameters were measured at two angular orientations (0 and 90 degrees) at elevations of 0.025, 0.304, 0.609, and 0.914 m above the bottom of the cladding tube. The cladding inside diameter measurements were made with an air gage (accuracy  $\pm 2.5 \mu$ m), and the cladding outside diameter measurements were made using a micrometer (accuracy  $\pm 0.03$  mm). These measurements are presented for the three previously unirradiated rods (301-3, 801-4, and 801-5) in Tables B-3, B-4, and B-5, respectively.

The unirradiated Saxton rods used in the test were modified by exchanging the original unirradiated 9.5 wt% enriched fuel pellets with 5.8 wt% enriched pellets fabricated by Battelle Pacific Northwest Laboratory (PNL). The physical dimensions (diameter, length, weight, and density) of the 60  $UO_2$  fuel pellets comprising the fuel columns of the previously unirradiated fuel rods (801-3, and 801-5) are presented in Tables B-6 and B-7, respectively. The  $UO_2$  pellet data contained in these tables were obtained by measuring the length between the upper and lower

	MAPI <sup>a</sup>	1.	Unirradiated			
Characteristics	<u>801-1, -2</u>	801-3	801-4	801-5		
Fuelb Dual COLUMN CENTER						
Enrichment (wt% 235U of total U)	5.7	5.8	5.8	5.8		
Fabricated fuel density (% TD)	94	94.5	94.5	94.5		
Original pellet diameter (mm)	8.59	8.53	8.53	8.53		
Original pellet length (mm)	15.2	15.2	15.2	15.2		
Dish depth (mm)	0.33	0.33	0.33	0.33		
Fuel stack length (m)	0.914	0.914	0.914	0.914		
Burnup (MWd/tU)	4600					
Cladding <sup>c</sup>				DEN		
Outside diameter (mm)	9.995	9.93	9.93	9.93		
Wall thickness (mm)	0.622	0.533	0.533	0.533		
Inside diameter (mm)	8.75	8.864	8.864	8,864		
Yield strength (0.2% offset, MPa)	570	570	570	570		
Tensile (ultimate) strength (MPa)	700	700	700	700		
Elongation (%)	18	18	18	18		
Hardness (R <sub>B</sub> )	100	100	100	100		
End Cap Rod Stock <sup>d</sup>						
Yield strength (0.2% offset, MPa)	241	241	241	241		
Tensile strength (MPa)	413	413	413	413		
Elongation (%)	14	14	14	14		
Hardness (BHN)	180	180	180	180		
Compression Springs <sup>e</sup>						
Wire diameter (mm)	1.018	1.018	1.018	1.018		
Tensile strength (MPa)	162	162	162	162		
Spring outside diameter (mm)	8.382	8.382	8.382	8.382		
Free length (mm)	54.788	54.788	54.788	54.788		
Total number of coils	17	17	17	17		

Table B-2. Test RIA 1-1 fuel rod component nominal design data

Table B-2. (continued)

#### MAPI<sup>a</sup> Unirradiated Characteristics 801-3 801-1, -2 801-4 801-5 Fuel Rod Insulators None None None None Gas plenum length (mm) 45.7 45.7 45.7 45.7 Measured void volume $(10^{-2} \text{ m}^3)$ 6 8.2 5.7 5.6 Initial fill gas pressure (MPa) f -------Fill gas for PBF test h h h 2 Fill gas pressure for PBF test (MPa) 0.103 0.103 0.103 0.103 Fuel-cladding gap (diametral mm) 0.33 0.33 0.26 0.33 Overall rod length (m) 1.087 1.072 1.068 1.068 1.072

a. Data are preirradiation values.

b. Ceramic, dished and sintered, uranium dioxide pellets.

c. Zircaloy-4 alloy cladding, as-received (50% cold worked) and stress relieved.

d. Zircaloy-2 alloy manufactured to ASTM Standard B 351-67, Grade RA-1 (annealed): Rods 801-1, 801-3, 801-4, and 801-5 only.

e. Oil-tempered, chromium-vanadium alloy spring steel manufactured to ASTM Standard A 231-68; Rods 801-3, 801-4, and 801-5 only.

f. Air fill gas at 0.103 MPa.

g. 77.7% He/22.3% Ar fill gas.

h. Commercially pure helium gas (nominally 99.995% pure helium).

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Distance from Bottom	Cladding Inside Diameter (mm)		Cladding Outside Diameter (mm)	
of Cladding		900	00	900
0.0254	8.7427	8.7516	9,9151	9.913
0.0508	8.7462	8.7579	9.9172	9 919
0.0762	8.7488	8.7483	9.9294	9,929
0.1016	8.7490	8.7462	9,9291	9,929
0.1270	8.7488	8.7473	9.9281	9.929
0.1524	8.7475	8.7475	9,9296	9.928
0,1778	8.7465	8.7457	9,9286	9.928
0.2032	8,7450	8.7457	9,9286	9.920
0.2286	8.7450	8 7473	9 9281	9.920
0.2540	8.7432	8 7475	0 0284	0.025
0.2340	0.7452	0.7475	3.3204	9.925
0.2794	8.7432	8.7475	9.9291	9.925
0.3048	8.7427	8.7467	9.9291	9.927
0.3302	8.7422	8.7467	9.9286	9.925
0.3556	8.7412	8.7465	9.9299	9.925
0.3810	8.7499	8.7460	9.9291	9.925
0.4064	8.7406	8.7460	9.9296	9.926
0.4318	8.7399	8.7442	9.9286	9.927
0.4572	8.7434	8.7473	9.9278	9.926
0.4826	8.7389	8.7473	9.9268	9.926
0.5080	8.7391	8.7457	9.9268	9.925
0.5334	8.7404	8.7478	9.9268	9.925
0.5588	8.7406	8.7424	9.9266	9.926
0.5842	8.7419	8.7429	9.9271	9.925
0.6096	8.7386	8.7406	9.9271	9.925
0.6350	8.7381	8.7396	9.9278	9.925
0.6604	8.7373	8.7389	9.9284	9.925
0.6858	8.7366	8.7381	9.9306	9.926
0.7112	8,7371	8.7366	9.9306	9.927
0.7366	8,7371	8.7368	9,9301	9.926
0.7620	8.7386	8.7368	9.9284	9.926
0.7874	8,7389	8,7363	9,9291	9.927
0.8128	8.7394	8.7373	9.9314	9,930
0,8382	8.7366	8.7340	9.9294	9,930
0.8636	8.7394	8.7318	9.9235	9.926
0.8890	8,7473	8,7335	9,9253	9.927
0.9144	8.7429	8.7424	9,9289	9.932
0.0144	0.7425	0.7424	1.1203	0.002

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Table B-3. Cladding inside and outside diameter data for Rod 801-3

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Table B-4. Cladding inside and outside diameter data for Rod 801-4

4		Claddin Diam	ng Inside neter	Cladding Outside Diameter			
Distance from Bottom		AND INDENTE (mm) TERIAL AND		ana ang dan pananan ang ang ang ang ang ang ang ang a	(mm)		
7 8	of Cladding (m)	0°	<u>90</u> °	0°	<u>90</u> °		
0	0.0254	8.7490	8.7567	9.9055	9.907		
11	0.0508	8.7475	8.7541	9.9017	9.914		
2	0.0762	8.7536	8.7521	9.9304	9.914		
3	0.1016	8.7541	8.7457	9.9372	9.940		
4	0.1270	8.7533	8.7498	9.9327	9.936		
6	0.1524	8.7529	8.7462	9.9357	9.938		
7	0.1778	8.7533	8.7473	9.9388	9.939		
	0.2032	8.7528	8.7490	9.9365	9.934		
	0.2286	8.7495	8.7503	9.9388	9.939		
	0.2540	8.7495	8.7521	9.9383	9.936		
	0.2794	8.7531	8.7490	9.9360	9.934		
	0.3048	8.7506	8.7508	9.9327	9.935		
	0.3302	8.7495	8.7508	9.9337	9.934		
5	0.3556	8.7521	8.7465	9.9375	9.934		
6	0.3810	8.7485	8.7485	9.9322	9.940		
8	0.4064	8.7513	8.7437	9.9327	9.939		
	0.4318	8.7511	8.7485	9.9322	9.941		
	0.4572	8.7508	8.7498	9.9365	9.928		
	0.4826	8.7452	8.7561	9.9372	9.935		
	0.5080	8.7450	8.7556	9.9337	9.931		
4	0.5334	8.7470	8.7485	9.9352	9.937		
51	0.5588	8.7498	8.7490	9.9367	9.938		
6	0.5842	8.7498	8.7432	9.9365	9.940		
	0.6096	8.7508	8.7485	9.9403	9.950		
8	0.6350	8.7523	8.7450	9.9372	9.939		
5	0.6604	8.7536	8.7511	9.9383	9.935		
1	0.6858	8.7495	8.7645	9.9459	9.952		
	0.7112	8.7506	8.7500	9.9411	9.929		
3	0.7366	8.7539	8.7282	9.9190	9.931		
4	0.7620	8.7399	8.7394	9.9334	9.941		
16	0.7874	8.7452	8.7366	9.9339	9.943		
7	0.8128	8.7452	8.7371	9.9395	9.936		
8	0.8382	8.7455	8.7485	9.9428	9.932		
9	0.8636	8.7526	8.7417	9.9367	9.933		
Ĭ	0.8890	8.7455	8.7440	9.9370	9.939		
2	0.9144	8.7495	8.7437	9.9322	9.938		
3 1 - 10	0.9398	8.7485	8.7467	9.9344			

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	Diam	leter	Cladding Outside Diameter		
Distance from Bottom	INDENTE (	m) TERIAL management	(1997)		
of Cladding	0	0	-	-	
(m)		90		900	
0.0254	8.7465	8.7574	9.9022	9.916	
0.0508	8.7442	8.7513	9,9022	9.937	
0.0762	8.7511	8.7432	9,9378	9.948	
0.1016	8.7533	8.7437	9.9375	9.943	
0.1270	8,7541	8,7391	9,9375	9,943	
		••••••	,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	5.545	
0.1524	8,7539	8,7399	9,9385	9.946	
0,1778	8,7511	8.7409	9,9390	9,941	
0.2032	8.7478	8.7445	9,9357	9.940	
0.2286	8.7470	8.7401	9,9403	9,941	
0.2540	8 7488	8 7304	0 0370	0 041	
0.2340	0.7400	0.7334	3.3570	3.341	
0.2794	8 7473	8 7414	0 0300	0 0 20	
0 3048	8 7414	8 7445	0 0300	9.939	
0.3302	9 7/22	9 7450	0.0270	9.942	
0.3502	9 7442	9 7401	9.9370	9.944	
0.3330	0.7442	0.7401	9.93/0	9.942	
0.3810	0.7473	0./442	3.3320	9.943	
0.4064	8.7457	8.7442	9.9337	9.939	
0.4318	8.7440	8.7450	9.9352	9.941	
0.4572	8.7462	8.7498	9.9365	9.940	
0.4826	8.7432	8.7462	9.9370	9.941	
0.5080	8.7432	8.7409	9.9383	9.941	
0.5334	8.7452	8.7406	9.9370	9.941	
0.5588	8.7473	8.7386	9.9383	9.937	
0,5842	8.7473	8.7406	9,9400	9.937	
0,6096	8.7460	8,7381	9,9413	9,939	
0.6350	8.7437	8.7396	9.9385	9.937	
0 6604	8 7427	8 7409	0 9367	9.941	
0.6059	9 7/10	0.7409	0.0220	0.042	
0.0050	9 7400	0.7423	9.9339	9.942	
0.7112	0.7409	0.7440	9.9330	9.943	
0.7500	0.7404	0.7434	9.9344	9.941	
0.7820	0./3/0	0.7390	9.9400	3.3420	
0.7874	8.7414	8.7389	9.9380	9.944	
0.8128	8.7386	8.7394	9.9423	9.942	
0.8382	8.7424	8.7366	9.9390	9.942	
0.8636	8.7409	8.7399	9.9372	9.944	
0.8890	8.7424	8.7396	9,9375	9.940	
0.9144	8.7404	8.7412	9.9367	9.938	
0.9398			9.9355		

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Table B-6. Fuel pellet physical characterization data for Rod 801-3

4	Diameter	Length	Weight	Immersion Density	Centerline Hole Diameter	51
Pellet	(mm)	(mm)	<u>(g)</u>	(g/cm <sup>3</sup> )	(mm)	
51.	5 5210	15 2420	9 4106	10 3921	•	
9 L	8.5319	15.2430	0.4100	10.3021		
2	8.5367	15.0/31	8.34/1	10.3975	0	
3	8.5446	15.1966	8.4704	10.4096	0	
4	8.5410	15.2215	8.4933	10.4429	0	
5	8.5402	15.2425	8.4857	10.3837	0	
6	8,5352	15.4031	8.5689	10.3989	1.8034	
7	8.5405	15.3744	8.5574	10.3913	1.8034	
8	8.5222	15.2016	8.4056	10.3606	1.8034	
ğ	8.5408	15,1460	8.4217	10.3958	1.8034	
10	8.5344	15.4242	8.5943	10.4310	1.8034	
11	8.5408	15.2509	8.4574	10.3834	1.8034	
12	8.5420	15.1648	8.4493	10.4196	1.8034	
13	8.5344	15.1638	8.8300		0	
14	8.5281	15.1618	8.7741		0	
15	8.5420	15.0561	8.7639	10.4315	0	
16	8.5441	15.4562	8,9714		0	
17	8 5413	15.2639	8.8796		0	
19	8 5364	15.2164	8.8539		õ	
10	0.5504	15 6000	8 5550		Ő	
19	0.5415	15.0990	8.0530		ő	
20	0.3392	15.5778	0.0330		U	
21	8.5410	15.1409	8.8344		0	
22	8.5443	15.1374	8.8208		0	
23	8.5479	15.9913	8.7393		0	
24	8.5443	15.7325	8.1745		0	
25	8.5364	15.5214	8.0365	10.4124	0	
~	0 5406	15 4053	8 0300		0	
26	8.5490	15.4955	0.0200		0	
21	8.5458	15.1834	0.0400	02	0	
28	8.5357	15.4155	8.9570		0	
29	8.5509	15.2900	8.9030	2	0	
30	8.5354	15.1508	8.7874		0	
31	8.5392	15.3848	8.9768	40	0	
32	8.5451	15,2151	8.8712	21	0	
33	8.5453	15.8745	8.6554		0	
34	8.5420	15,0353	8.7539		0	
35	8.5225	15,9319	8.6675	10.4450	0	
				And A		
36	8.5372	15.0940	8.7564		0	
37	8.5400	15.3063	8.9399		0	
38	8.5476	15.3035	8.9392		. 0	
39	8.5471	15.4046	8.9571		0	4
40	8 5491	15.4275	8-0014		0	

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Table B-6. (continued)

	Diameter	Length	Weight	Immersion Density	Centerline Hole Diameter
Pellet	<u>(mm)</u>	<u>(mm)</u>	<u>(g)</u>	<u>(g/cm<sup>-</sup>)</u>	(mm)
41	8.5397	15.9098	8.7019		0
42	8.5474	15.4109	8.0227		0
43	8.5357	15.1559	8.8494		Ō
44	8.5468	15.6588	8.1387		Ö
45	8.5458	15.1049	8.7627	10.4140	0
46	8.5474	15.5009	8.0304		0
47	8.5476	15.4468	8.0154		0
48	8.5428	15.5118	8.0555		Ö
49	8.5420	15.0586	8.7864		0
50	8.5395	15.4102	8.9863		Ŏ
51	8.5425	15.3190	8.9443		0
52	8.5425	15.9804	8.7204		Ő
53	8.5420	15.3012	8.9094		Ő
54	8.5420	15.1511	8.8177	1	ő
55	8.5463	15.2502	8.9035	10.4434	0
56	8.5519	15.2103	8.8337		0
57	8.5453	15.3391	8.9714		0
58	8.5418	15.2118	8.8020		0
59	8.5476	15.4056	8.9522		0
60	8.5390	15.4752	6.0920		õ

dish shoulder of each pellet. Both diametral and length measurements were performed using a Bausch and Lomb optical gage, Model BR-25. The measurements were made with an accuracy of  $\pm 0.005$  mm. An analytical balance, accurate to  $\pm 1$  mg was used to obtain pellet mass and density data. Fuel pellet data for previously irradiated Rods 801-1 and 801-2 were unavailable, and the fuel pellet data used for unirradiated Rod 801-4 was unqualified, since the rod was irradiated in-pile only during the power calibration and preconditioning phase of the test.

The pretest void volumes for the previously irradiated rods were not recorded, but the design void volume of the rods (801-1 and 801-2) was  $6 \times 10^{-2} \text{ m}^3$ . The pretest void volumes from previously unirradiated Rods 801-3, 801-4, and 801-5 are 8.2  $\pm 0.3$ , 5.7  $\pm 0.3$ , and 5.6  $\pm 0.3 \times 10^{-2} \text{ m}^3$ , respectively. The somewhat larger void volume of Rod 801-3 was

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Immersion Centerline Hole Density Weight Diameter Length Diameter  $(g/cm^3)$ (mm) Pellet (mm) (mm) (g) 15.1351 8.7802 0 8.5362 1 0 2 8.5314 15.0807 8.7992 3 8.8345 0 8.5402 15.1051 0 4 8.5441 15.4043 9.0332 0 5 9.0327 10.4514 8.5405 15.4671 6 8.5369 15.3721 8.9950 0 ---0 7 8.8239 8.5405 15.1031 8.8100 0 8 8.5372 15.1437 0 ---15.2974 8.9638 9 8.5441 10 15.2679 8.9459 0 8.5471 8.5453 15.1021 8.8255 0 11 15.3647 8.9951 0 12 8.5339 0 13 8.5392 15.2039 8.8443 0 8.9004 14 8.5443 15.1658 10.4281 0 15 8.5349 15.1105 8.8101 0 8.8934 15.1897 16 8.5364 ---8.9355 0 17 8.5415 15.2651 ---0 18 8.5380 15.3373 8.9796 15.2984 8.9480 ---0 19 8.5357 ---0 15.4615 9.0480 20 8.5402 15.3198 8.9565 0 21 8.5395 0 22 8.5362 15.3286 8.9608 23 15.2408 8.9134 0 8.5385 ---0 24 8.5369 15.2847 8.9331 0 25 15.5019 9.0593 10.4411 8.5397 0 26 8.5395 15.2075 8.9008 C 27 15.3960 9.0010 8.5377 0 28 8.5374 15.0734 8.7665 0 29 8.5430 15.1719 8.8775 \_\_\_ 0 30 8.5420 15.1854 9.0395 31 8.5397 15.0924 8.8250 0 0 32 15.2530 8.9169 8.5298 0 33 8.5395 15.2786 8.9160 34 8.5319 15.2458 8.8625 0 8.8685 10.4589 35 15.1628 0 8.5413 0 15.1191 8.8122 36 8.5410 37 8.5410 15.4818 9.0339 0 0 38 8.5354 15.2672 8.9224 0 39 8.5397 15.9591 8.7338 ---40 0 8.5410 15.2580 8.8762

## Table B-7. Fuel pellet physical characterization data for Rod 801-5

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	Diameter	Length	Weight	Immersion Density 3	Centerline Hole Diameter
Pellet	(mm)	<u>(mm)</u>	<u>(g)</u>	<u>(g/cm<sup>3</sup>)</u>	(mm)
41	8.5425	15.2728	8.9022		0
42	8.5402	15.4026	8.9811		Ö
43	8.5344	15.4300	8.9796		0
44	8.5316	15.1498	8.8560		Ō
45	8.5377	15.1712	8.8353	10.4197	0
46	8.5369	15.2730	8.9332		0
47	8.5341	15.2037	8.8230		0
48	8.5514	15.3916	9.0179	\	0
49	8.5339	15.1638	8.8362	/	0
50	8.5471	15.0851	8.8280		0
51	8.5466	15.1717	8.8829		0
52	8.5329	15.2687	8.9024	· · · · · · · · · · · · · · · · · · ·	0
53	8.5395	15.3114	8.9260		0
54	8.5344	15.1089	8.8276		0
55	8.5413	15.4610	9.0632	10.4673	0
56	8.5392	15.5138	9.0772		0
57	8.5418	15.0805	8.8083		0
58	8.5496	15.2519	8.8770		0
59	8.5364	15.2883	8.9157		0
60	8.5499	10.6655	6.2049		0

accounted for by the presence of 12 fuel pellets in the fuel column containing centerline holes to contain the fuel thermocouple. The top end cap of Rod 801-1 was removed and replaced with an end cap containing a pressure transducer. The rod was prepressurized with a mixture of 77.7% helium and 22.3% argon gases at ambient temperature to 0.103 MPa. This gas mixture approximates the thermal conductivity of the fill gas in an irradiated fuel rod. Rod 801-2 was not opened prior to testing and contained its initial inventory of air plus fission gases at about 0.103 MPa. Previously irradiated Rods 801-3, 801-4, and 801-5 were backfilled at ambient temperatures with a commercially pure helium gas to pressures of 0.103 MPa each. The upper end cap of Rod 801-3 was replaced with an end cap containing an internal pressure transducer, plenum thermometer, and centerline fuel thermocouple. The top 12 fuel pellets in the fuel column (between 0.740

and 0.914 m from the bottom of the fuel stack) were replaced with fuel pellets containing 1.8-mm-diameter drill holes to accommodate insertion of the fuel centerline thermocouple.

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### 2. CLADDING TEXTURE CHARACTERIZATION

The thermal-mechanical processing of zircaloy into tubing produces a strong crystallographic texture. Typically, the commonly observed texture is a bimodal distribution of basal poles concentrated in the radial-totangential plane of the tube. The strong crystallographic texture produces a pronounced anisotropy in the mechanical properties of the tubing. A wide range of textures may be found, depending on the manufacturing process. In the following sections, the nominal crystallographic texture of the Saxton cladding used in the Test RIA 1-1 fuel rods is characterized, and the plastic behavior of the cladding is analyzed.

The crystallographic texture of the irradiated Saxton tubing was determined by inverse pole figure techniques from X-ray diffraction data. Measurement of the integrated intensities, {I}, of diffraction peaks from a sample allowed construction of a contour map (inverse pole figure) using the ratio of intensities, {I:I<sub>0</sub>}, assigned to appropriate orientations in the standard projection, as shown in Figure B-1, for the (0001) standard projection of  $\alpha$  zircaloy.<sup>B-2,B-3</sup> The angle  $\phi$  shows the extent of tilt of the diffracting plane's pole from the basal pole (0001). The angle  $\alpha$  shows the plane's rotation about the basal pole. Crystallographic planes lying parallel to the plane of the sample surface will contribute to the measured diffracted intensity for a scan at the Bragg angle,  $\phi$ . The preferred orientation in basal poles is then obtained from the orientation distribution function of intensities in the diffractometer pattern.

The inverse pole figures for the tangential (t), radial (r), and axial directions typical to the Saxton cladding are shown in Figure B-2 (insets a, b, and c, respectively), for the unirradiated cladding (inset d), and for the axial direction in the previously irradiated cladding.<sup>B-1</sup> The (0001) basal poles lie in both the radial and tangential directions, with about 45% of the poles lying in the tangential direction. The nominal





normalized weighted average of the basal poles in these samples is  $F \ge 0$ ( $\phi$  is slightly > 45-degrees) in the r-to-t plane of the tube.<sup>a</sup> A large number of the (1010) and (1120) poles lie in the axial direction. Since the (1010) and (1120) texture coefficients in the axial direction are about double the value of the coefficient of the (0001) pole in either the radial or tangential directions, each of the basal orientations could contribute to (1010) or (1120) orientation in the axial direction.<sup>B-4</sup> Possible orientations of the basal poles are illustrated in Figure B-3.

The orientation parameter, f, for a given reference direction relates the materials properties to the degree of physical isotropy. The f parameter for the Saxton cladding was evaluated for f, denoting the radial

a. The normalized weighted average of basal poles in the r-to-t plane of the tube, texture factor F, range from F = 1 when all basal poles are oriented radially, to F = -1 when all the basal poles are oriented tangentially. When all the basal poles are equally distributed about the 45-degree direction, F = 0.

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direction of the tubing. The radial orientation of basal poles over the stereogram nominally ranged to  $f_r \gtrsim 1/2$ ,<sup>a</sup> suggesting a mixture of orientations present. Although not measured, the expected contractile strain ratio, R, for the Saxton tubing deformed axially in tension, is R  $\sim$  1:2.

The texture coefficients in the axial direction indicated that some grains underwent a 30-degree rotation of the (1120) poles about the c-axis so that some of the (1010) poles lie in the axial direction as a result of the fabrication. Evidence of residual cold work in the axial direction in the Saxton tubing is seen from the ratio of texture coefficients in the (1010) and (1120) directions, which is > 1 [the (1010) texture coefficient is 3.95, compared with the (1120) coefficient of 2.32, with the ratio (1010) being 1.70].<sup>B-5</sup>

The coefficients obtained in the axial direction for irradiated cladding exhibited only slightly larger values of the texture coefficients for  $(10\overline{10})$  and  $(11\overline{20})$  poles, compared with unirradiated cladding values. The comparison suggests that no significant changes occurred in the tubing texture as a result of the Saxton irradiation.

From the available data, an approximate pole figure representative of the Saxton cladding was constructed, showing the basal pole texture, and is presented in Figure B-4. The corresponding von Mises yield locus representing the yield strength anisotropy is sketched below the pole figure. The basal pole intensity maxima are near, but slightly greater than, 45 degrees in the r-to-t plane.

The zircaloy deforms primarily by prismatic plane slip for temperatures up to about 680 to 780 K.<sup>B-6</sup> Basal pole deformation occurs by twinning. Although a variety of twinning modes exist, the resolved shear stress required for twinning is greater than the resolved shear stress for slip. Tube textures indicating a direction with a large concentration of

a. Alignment of the basal poles perpendicular or parallel to the radial direction varies from  $0 \le f_r \le 1$ , respectively. Basal poles equally distributed about 45 degrees from the radial direction give a value  $f_r = 1/2$ , and physically isotropic,  $f_r = 1/3$ .



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> basal poles will tend to resist deformation along that direction. The Saxton rod deformation texture has a tendency for more basal poles to be aligned in the radial direction (F slightly > 0); therefore, deformation in the tangential direction would be slightly more favorable than in the radial direction, but nearly equal amounts of tangential and radial deformation may occur.

> Since deformation by basal pole twinning is energetically less favorable than prismatic slip, it is assumed that all of the plastic deformation occurring in the Saxton cladding would be by either prismatic slip, in a directon perpendicular to the axial direction (c axis of the hexagonal grains), or dislocation slip, representative of twinning modes. Preferred radial positioning of the basal poles leads to a condition of high hoop resistance. Axial loading with active, radially aligned basal poles produces elongation by diameter reduction, with little change in wall thickness, whereas active tangentially aligned basal poles produce elongation by decreasing wall thickness, with little change in diameter.<sup>B-5</sup>

## 3. REFERENCES

- B-1. G. W. Gibson et al., <u>Characteristics of UO<sub>2</sub>-Zircaloy Fuel Rod</u> <u>Materials from the Saxton Reactor for Use in the Power Burst</u> <u>Facility</u>, <u>ANCR-NUREG-1321</u>, September 1976.
- B-2. J. J. Kearns, <u>Thermal Expansion and Preferred Orientation in</u> <u>Zircaloy</u>, WAPD-TM-472, November 1965.
- B-3. L. F. P. Van Swam et al., "Relationship Between Contractile Strain Ratio R, and Texture in Zirconium Alloy Tubing," <u>Metallurgical Trans</u>actions, A 10A, 1979, pp. 483-487.
- B-4. C. R. Woods et al., <u>Properties of Zircaloy-4 Tubing</u>, WAPD-TM-585, 1966.
  - B-5. E. Tenckhoff and P. L. Rittenhouse, "Annealing Textures in Zircaloy Tubing," Journal of Nuclear Materials, 35, 1970, pp. 14-23.

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