**A Re-Measurement of the Neutron-Induced
Gamma-Ray Production Cross Sections for
Iron in the Energy Range
 $850 \text{ keV} \leq E_n \leq 20.0 \text{ MeV}$**

G. T. Chapman
G. L. Morgan
F. G. Perey

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Neutron Physics Division

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CROSS SECTIONS FOR IRON IN THE ENERGY RANGE $850 \text{ keV} \leq E_n \leq 20.0 \text{ MeV}$

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ABSTRACT

Values of the gamma-ray production cross sections for neutron interactions with iron as reported by previous investigators have differed by as much as a factor of 1.5 or more at neutron energies greater than about 5 MeV. Because of this discrepancy, the measurements have been repeated at ORNL using the ORELA as a pulsed source of neutrons with energies between 850 keV and 20 MeV. The data were obtained using a NaI(Tl) gamma-ray spectrometer oriented at an angle of 125 degrees to the incident neutron beam. The sample was positioned in the beam at a distance of 47.35 meters from the neutron source. The resulting data, presented as differential cross sections ($d^2\sigma/d\Omega dE$) for gamma rays between 0.7 and 10.5 MeV, show good agreement with some previously published data, but are significantly different from previous ORNL measurements for neutron energies greater than 5 MeV.

INTRODUCTION

Gamma-ray production cross sections for neutron interactions in iron reported by Dickens, Morgan and Perey (1), hereinafter referred to as DMP, disagreed with the values of Orphan, Hoot, and John (2) by as much as a factor of 1.5 or more at neutron energies greater than 5 MeV. A re-examination by Rogers (3) of the data of Ref. 2 failed to remove this disagreement, although a few modifications were made. During the re-evaluation of gamma-ray production cross sections of iron for ENDF/B-IV, theoretical calculations (4), constrained by the available data above 5 MeV (5), predicted larger cross sections than those obtained by DMP and values closer to those obtained by Orphan et al. (3). Up to the threshold of the tertiary reactions it was felt that the theoretical calculations were generally reliable in view of the experience then gained using such models on other similar nuclides (4). In view of the data discrepancies for the ENDF/B-IV evaluation (6), the theoretical calculations were chosen to provide the magnitude of the cross sections up to 12 MeV, the threshold for (n,2n) reaction, and a compromise intermediate between the two data sets was used above 12 MeV since the calculations were not as reliable in this energy region. However, the gamma-ray energy distributions given by the DMP data were adopted because of their higher energy resolution and better overall agreement with the calculations below 12 MeV. In an effort to resolve this discrepancy, the measurements have been repeated, taking advantage of the experience and improvements acquired in two years of gamma-ray production measurements. The data are presented here in tabular and graphical form and are compared to the data given by DMP and

in Ref. 3 as well as the current evaluated data file for iron, DNA 4180 MOD 2 (6). Two methods of data analysis are employed; i.e., a pulse-height unfolding procedure (7) which gives detailed gamma-ray spectra for a series of coarse intervals of neutron energies, and a method of pulse-height weighting (8) in which integral properties of the cross section are derived as a function of the incident neutron energy. This second approach provides a more detailed description of the behavior of the cross section with neutron energy and greatly facilitates comparison of experimental and evaluated data.

EXPERIMENTAL PROCEDURE

A detailed description of the experimental procedure and apparatus is given elsewhere (1) and only a brief description will be given here. The iron sample, which consisted of a 30-cm by 30-cm by 0.625-cm (0.053 atoms/b) slab, was oriented 45° with respect to the neutron beam at a distance of 47.35 meters from the accelerator target. The neutron beam produced by the Oak Ridge Electron Accelerator (ORELA) consisted of pulses with a repetition rate of 800 pps and a pulse width of 20 ns. The electron energy was 150 MeV for these measurements. The beam was filtered through 1.5 inches of ^{238}U to reduce the intensity of the gamma flash at the sample location and through 0.75 inches of ^{10}B to remove essentially all neutrons with energies less than about 1 keV.

Gamma rays produced in the iron sample were detected by a 12.5-cm-diam by 12.5-cm-long NaI(Tl) crystal in a shield composed of lead and lithiated paraffin. A 25-cm-thick slab of lithium hydride was placed in front of the spectrometer to protect the crystal from neutrons which

were scattered in the sample. The crystal was located at a distance of 1.4 meters from the sample on a centerline making an angle of 125 degrees with the incident neutron beam. For background measurements, the collimator through the spectrometer was covered with a 5-cm-thick lead shield.

The neutron flux (Fig. 1) at the sample position was determined using a calibrated organic scintillator, and this measurement will be discussed later. The flux was constantly monitored during each run by the use of a small plastic scintillator at the edge of the neutron beam 30 meters from the source.

DATA REDUCTION AND RESULTS

Two methods of data reduction were used in the conversion of the measured pulse-height spectra to cross sections. With the first method, the pulse-height distributions resulting from integrating pulse-height spectra over specific intervals of neutron energy (time-of-flight) were unfolded using the program FERD (7) and measured gamma-ray response functions for this spectrometer. The neutron energy intervals were chosen to be 150 keV wide from 850 keV to 1 MeV, 500 keV wide from 1 MeV to 10 MeV, 1 MeV wide from 10 MeV to 15 MeV, and 2.5 MeV wide from 15 MeV to 20 MeV. This method gave gamma-ray spectra containing 124 points each over the energy range $700 \text{ keV} \leq E_\gamma \leq 10.6 \text{ MeV}$. These results are shown in Figs. 2 through 34. The first graph (Fig. 2) in this set is a three-dimensional representation showing the general characteristics of the cross section variations with both gamma-ray and neutron energy.

The following six graphs (Figs. 3 through 8) compare the current data and the data presented in DMP (see discussion to follow). Following this are 26 graphs (Figs. 9 through 34 and Table 1) showing the current data compared to the most recent evaluated file for iron, DNA 4180 MOD 2. The values shown for the evaluated file were obtained by averaging the data in the file over neutron energy intervals corresponding to those in the experimental data. The evaluated file gives some data as gamma-ray lines and these have been smeared to our experimental resolution to facilitate comparison.

The unfolding technique referred to above requires good statistical accuracy to produce acceptable results. Therefore it is necessary to use rather large neutron energy bins in performing the analysis. To reduce this dependency on statistical accuracy and to gain better resolution in the incident neutron energy, a second method of data analysis was used. This method, which is based on pulse-height weighting (8,9), provides only integral information about the secondary gamma-ray spectra. In this case the pulse-height weighting intervals varied from 0.020 MeV at 1 MeV to 0.5 MeV for incident energies of 20 MeV. These data are shown in Fig. 35. The minimum gamma-ray energy used in this analysis was 0.7 MeV. A pulse-height weighting analysis was also done using the original data from which the cross sections presented in DMP were derived. The results of this analysis are also shown on the graph along with the values given in Ref. 3 and in the evaluated file.

The numerical values of the cross sections obtained in the current experiment are given in Tables 1 and 2 at the end of this report. The values shown in the tables and in the graphs do not include an estimated

error of 10% due to uncertainties in determining the absolute neutron flux.

DISCUSSION OF THE RESULTS

As is evident from the curves shown in the graph depicting the pulse-height weighting results (Fig. 35), the current values agree well with the values given in Ref. 3 up through their maximum neutron energy of 14.5 MeV. There is also good agreement with the evaluated values up to about 12 MeV. Since the evaluated values are based on the Weisskopf Model (Complete Statistical Model), it is not surprising that they do not agree well above the $(n,2n)$ threshold at about 11.6 MeV.

The first ORNL data as given in DMP, however, do not agree with either set of data above about 4 MeV. The first six graphs in the set of graphs showing the pulse-height unfolding results show a comparison of the current data and the data from DMP. It is evident that the shape of the measured spectra is in good agreement for all neutron energies. However, the absolute magnitudes show a progressively greater deviation with increasing neutron energy.

The present results thus confirm the data of Ref. 3 as well as the calculations (4) used for the current evaluation of iron up to 12 MeV. However, the serious inconsistency in the original DMP measurement was very disturbing, especially in view of the large number of nuclides for which measurements have since been made using this system (10). We therefore undertook an extensive investigation in order to determine the problem with the earlier data and to check the results for other nuclides obtained at about the same time with this system.

Our earlier results covered nuclides with data acquired in the following order: carbon, tantalum, iron, and copper. As shown in Fig. 36, recent results from Rogers et al. (11) have confirmed the carbon results. We then undertook remeasurements of tantalum and copper to test these earlier data. Results of a pulse-height weighting analysis of these new measurements are compared to the earlier results in Figs. 37 and 38. The results for copper ($E_\gamma \geq 0.7$ MeV) agree very well over the entire energy range, thus confirming the correctness of the earlier data (12). This is very significant since the original data for Cu were the first data obtained using a new component in the system, a time digitizer, to measure time-of-flight. (The old system employed a time-to-amplitude converter in conjunction with an analogue to digital converter.)

The new data for tantalum ($E_\gamma \geq 1.0$ MeV) disagree somewhat with the older data (13), the discrepancy showing somewhat the same energy dependence as for iron. However, if the new Ta data are normalized using the same neutron flux measurement as was used for the original data, the results are in good agreement. These results are shown in Fig. 38 along with the results of calculations by Howerton et al. (14) based upon our earlier data.

In the three years since the time digitizer was installed there have been, beginning with the original experiment on copper, approximately ten measurements of the incident neutron flux. Re-examination of these measurements has shown that all are in agreement as to the magnitude and energy dependence of the incident neutron flux to within 5%. However, reference to Fig. 1 clearly shows that the original flux

measurement, used with the carbon, tantalum, and iron measurements, differs appreciably from the later measurement.

An NE-213 scintillation counter was employed for the flux measurements. In the first measurement, the efficiencies used with the detector were those described in the paper of Straker et al. (15) and were believed to be the results of work by Verbinski et al. (16). However, subsequent investigation, after data reduction for carbon, tantalum and iron, showed these efficiencies to be different from those of Verbinski et al. The efficiencies of Ref. 16 were used for all later work. The difference between the two curves shown in Fig. 1 is due completely to the difference in the two efficiencies. This also then explains the difference between the older and more recent tantalum measurements. The use of wrong neutron efficiencies in the case of carbon was almost exactly canceled by a change in the NaI detector efficiency which was subsequently made in the region of 4.43 MeV gamma-ray energy.

However, the error in the flux measurement explains only part ($\sim 40\%$) of the difference between the two measurements on iron. The major difference between the measurements on carbon, tantalum, and iron was that the iron experiment was carried out about six months after the carbon-tantalum experiment. Very extensive re-examination of the original iron measurement data tapes and log book gives no positive evidence for an error which can account for the remainder of the difference. Since the spectral shapes of the old and new measurements agree, it would appear as if for the old Fe measurements some unknown deadtime effect in the old time-of-flight system was not recognized and therefore not corrected for.

To summarize, the original neutron flux measurement was somewhat in error in the high energy region. This affected the measurements on carbon and tantalum in a minor fashion and explains part of the difference for the two iron measurements. No positive statement can be made about the remainder of the discrepancy. The remeasurement of the copper cross sections confirms the accuracy of the succeeding measurements (10).

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Fig. 1. Measured neutron flux for this work compared to the neutron flux reported by Dickens et al. (Ref. 1).

Fig. 2. Three-dimensional representation of the cross section variation as a function of neutron and gamma-ray energy.

Fig. 3. Comparison of this work with the results reported in Ref. 1 for $1.0 \text{ MeV} \leq E_n \leq 1.5 \text{ MeV}$.

ORNL-DWG 75-400

Fig. 4. Comparison of this work with the results reported in Ref. 1 for $2.0 \text{ MeV} \leq E_n \leq 2.5 \text{ MeV}$.

ORNL-DWG 75-401

Fig. 5. Comparison of this work with the results reported in Ref. 1 for $4.0 \text{ MeV} \leq E_n \leq 4.5 \text{ MeV}$.

Fig. 6. Comparison of this work with the results reported in Ref. 1 for $7.5 \text{ MeV} \leq E_n \leq 8.0 \text{ MeV}$.

ORNL-DWG 75-403

Fig. 7. Comparison of this work with the results reported in Ref. 1 for $9.5 \text{ MeV} \leq E_n \leq 10.0 \text{ MeV}$.

Fig. 8. Comparison of this work with the results reported in Ref. 1 for $14.0 \text{ MeV} \leq E_n \leq 15.1 \text{ MeV}$.

ORNL-DWG 75-405

Fig. 9. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-406

Fig. 10. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-407

Fig. 11. Comparison of this work with the DNA evaluation.

Fig. 12. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-409

Fig. 13. Comparison of this work with the DNA evaluation.

Fig. 14. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-411

Fig. 15. Comparison of this work with the DNA evaluation.

Fig. 16. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-413

Fig. 17. Comparison of this work with the DNA evaluation.

Fig. 18. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-415

Fig. 19. Comparison of this work with the DNA evaluation.

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Fig. 20. Comparison of this work with the DNA evaluation.

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Fig. 21. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-418

Fig. 22. Comparison of this work with the DNA evaluation.

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Fig. 23. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-420

Fig. 24. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-421

Fig. 25. Comparison of this work with the DNA evaluation.

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Fig. 26. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-423

Fig. 27. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-424

Fig. 28. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-425

Fig. 29. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-426

Fig. 30. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-427

Fig. 31. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-428

Fig. 32. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-429

Fig. 33. Comparison of this work with the DNA evaluation.

ORNL-DWG 75-430

Fig. 34. Comparison of this work with the DNA evaluation.

Fig. 35. Total gamma-ray production cross sections for iron at $\theta = 125^\circ$ and $E_\gamma \geq 0.8$ MeV.

Fig. 36. Comparison of the ORNL carbon cross section to recent measurements by Rogers et al. (from Ref. 11).

Fig. 37. Total gamma-ray production cross sections for copper at $\theta = 125^\circ$ and $E_\gamma \geq 0.7$ MeV (see text).

Fig. 38. Total gamma-ray production cross sections for tantalum at $\theta = 125^\circ$ and $E_\gamma \geq 0.7$ MeV (see text).

Table 1

TOTAL SECONDARY GAMMA RAY YIELD AND AVERAGE SECONDARY GAMMA RAY ENERGY FROM PE AS A FUNCTION OF THE INCIDENT NEUTRON ENERGY. THESE DATA RESULT FROM A PULSE HEIGHT WEIGHTING ANALYSIS FOR PULSE HEIGHTS GREATER THAN 0.700 MEV. UNCERTAINTIES ARE GIVEN IN PARENTHESES IN THE SAME UNITS AS THE DATA. THE UNCERTAINTIES IN TOTAL YIELD DO NOT INCLUDE A 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION. THE ANGLE IS 125 DEGREES.

INC. NT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY PHOTON YIELD (B/SR)	AVERAGE ENERGY (MEV)	INC. NT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY PHOTON YIELD (B/SR)	AVERAGE ENERGY (MEV)
0.811	0.010	0.810E-03 (0.496E-04)	0.386E 01 (0.305E 00)	4.838	0.062	0.223E 00 (0.188E-02)	0.149E 01 (0.174E-01)
0.832	0.011	0.736E-03 (0.506E-04)	0.393E 01 (0.341E 00)	4.952	0.052	0.233E 00 (0.210E-02)	0.150E 01 (0.188E-01)
0.850	0.007	0.603E-03 (0.612E-04)	0.423E 01 (0.526E 00)	5.098	0.095	0.246E 00 (0.189E-02)	0.152E 01 (0.161E-01)
0.868	0.011	0.991E-02 (0.131E-03)	0.103E 01 (0.225E-01)	5.293	0.100	0.249E 00 (0.189E-02)	0.153E 01 (0.160E-01)
0.891	0.012	0.178E-01 (0.173E-03)	0.935E 00 (0.144E-01)	5.499	0.106	0.258E 00 (0.196E-02)	0.156E 01 (0.162E-01)
0.911	0.008	0.125E-01 (0.171E-03)	0.986E 00 (0.221E-01)	5.700	0.096	0.265E 00 (0.209E-02)	0.157E 01 (0.171E-01)
0.928	0.008	0.153E-01 (0.192E-03)	0.947E 00 (0.190E-01)	5.897	0.101	0.274E 00 (0.219E-02)	0.159E 01 (0.173E-01)
0.949	0.013	0.266E-01 (0.214E-03)	0.888E 00 (0.107E-01)	6.104	0.106	0.283E 00 (0.227E-02)	0.162E 01 (0.176E-01)
0.971	0.009	0.166E-01 (0.204E-03)	0.917E 00 (0.181E-01)	6.303	0.093	0.285E 00 (0.241E-02)	0.163E 01 (0.188E-01)
0.989	0.009	0.297E-01 (0.281E-03)	0.889E 00 (0.129E-01)	6.493	0.097	0.290E 00 (0.237E-02)	0.165E 01 (0.185E-01)
1.023	0.024	0.336E-01 (0.203E-03)	0.872E 00 (0.796E-02)	6.692	0.102	0.303E 00 (0.251E-02)	0.168E 01 (0.189E-01)
1.073	0.026	0.365E-01 (0.201E-03)	0.875E 00 (0.719E-02)	6.900	0.106	0.314E 00 (0.270E-02)	0.171E 01 (0.197E-01)
1.127	0.028	0.418E-01 (0.219E-03)	0.875E 00 (0.676E-02)	7.096	0.089	0.325E 00 (0.276E-02)	0.171E 01 (0.199E-01)
1.179	0.024	0.410E-01 (0.233E-03)	0.878E 00 (0.739E-02)	7.300	0.116	0.329E 00 (0.285E-02)	0.174E 01 (0.204E-01)
1.225	0.022	0.352E-01 (0.228E-03)	0.880E 00 (0.843E-02)	7.513	0.097	0.328E 00 (0.290E-02)	0.177E 01 (0.212E-01)
1.274	0.027	0.305E-01 (0.199E-03)	0.885E 00 (0.870E-02)	7.711	0.101	0.331E 00 (0.305E-02)	0.181E 01 (0.225E-01)
1.326	0.025	0.322E-01 (0.228E-03)	0.880E 00 (0.933E-02)	7.916	0.105	0.330E 00 (0.294E-02)	0.186E 01 (0.222E-01)
1.377	0.026	0.463E-01 (0.270E-03)	0.869E 00 (0.743E-02)	8.102	0.081	0.328E 00 (0.321E-02)	0.188E 01 (0.248E-01)
1.428	0.024	0.477E-01 (0.302E-03)	0.868E 00 (0.801E-02)	8.296	0.112	0.344E 00 (0.326E-02)	0.191E 01 (0.239E-01)
1.477	0.025	0.542E-01 (0.314E-03)	0.876E 00 (0.738E-02)	8.496	0.087	0.355E 00 (0.358E-02)	0.194E 01 (0.259E-01)
1.524	0.022	0.650E-01 (0.387E-03)	0.873E 00 (0.745E-02)	8.704	0.121	0.354E 00 (0.354E-02)	0.198E 01 (0.259E-01)
1.574	0.028	0.637E-01 (0.362E-03)	0.870E 00 (0.710E-02)	8.919	0.094	0.357E 00 (0.378E-02)	0.201E 01 (0.283E-01)
1.626	0.024	0.644E-01 (0.398E-03)	0.869E 00 (0.776E-02)	9.110	0.097	0.345E 00 (0.353E-02)	0.203E 01 (0.282E-01)
1.675	0.025	0.675E-01 (0.410E-03)	0.872E 00 (0.763E-02)	9.307	0.100	0.344E 00 (0.349E-02)	0.203E 01 (0.278E-01)
1.727	0.027	0.536E-01 (0.383E-03)	0.875E 00 (0.908E-02)	9.511	0.104	0.347E 00 (0.368E-02)	0.208E 01 (0.294E-01)
1.776	0.022	0.589E-01 (0.450E-03)	0.878E 00 (0.971E-02)	9.722	0.107	0.357E 00 (0.393E-02)	0.209E 01 (0.308E-01)
1.821	0.023	0.485E-01 (0.399E-03)	0.893E 00 (0.107E-01)	9.902	0.073	0.363E 00 (0.429E-02)	0.211E 01 (0.340E-01)
1.874	0.030	0.676E-01 (0.447E-03)	0.876E 00 (0.835E-02)	10.127	0.152	0.358E 00 (0.419E-02)	0.218E 01 (0.327E-01)
1.929	0.025	0.685E-01 (0.492E-03)	0.881E 00 (0.919E-02)	10.398	0.118	0.373E 00 (0.442E-02)	0.219E 01 (0.341E-01)
1.974	0.019	0.741E-01 (0.602E-03)	0.882E 00 (0.102E-01)	10.639	0.123	0.361E 00 (0.430E-02)	0.222E 01 (0.350E-01)
2.048	0.055	0.839E-01 (0.438E-03)	0.880E 00 (0.653E-02)	10.889	0.127	0.368E 00 (0.437E-02)	0.223E 01 (0.354E-01)
2.155	0.052	0.865E-01 (0.491E-03)	0.878E 00 (0.706E-02)	11.147	0.132	0.378E 00 (0.458E-02)	0.228E 01 (0.364E-01)
2.254	0.048	0.881E-01 (0.544E-03)	0.893E 00 (0.781E-02)	11.369	0.090	0.392E 00 (0.544E-02)	0.227E 01 (0.420E-01)
2.353	0.051	0.945E-01 (0.555E-03)	0.894E 00 (0.746E-02)	11.599	0.140	0.394E 00 (0.528E-02)	0.233E 01 (0.406E-01)
2.448	0.045	0.938E-01 (0.615E-03)	0.905E 00 (0.838E-02)	11.884	0.145	0.404E 00 (0.552E-02)	0.237E 01 (0.418E-01)
2.550	0.057	0.103E 00 (0.643E-03)	0.905E 00 (0.798E-02)	12.129	0.100	0.384E 00 (0.607E-02)	0.238E 01 (0.494E-01)
2.658	0.051	0.949E-01 (0.627E-03)	0.917E 00 (0.869E-02)	12.383	0.154	0.384E 00 (0.532E-02)	0.241E 01 (0.436E-01)
2.751	0.043	0.103E 00 (0.759E-03)	0.944E 00 (0.989E-02)	12.643	0.106	0.385E 00 (0.674E-02)	0.245E 01 (0.556E-01)
2.851	0.056	0.104E 00 (0.747E-03)	0.990E 00 (0.100E-01)	12.858	0.109	0.386E 00 (0.659E-02)	0.241E 01 (0.543E-01)
2.955	0.048	0.122E 00 (0.898E-03)	0.100E 01 (0.104E-01)	13.135	0.169	0.382E 00 (0.586E-02)	0.246E 01 (0.485E-01)
3.052	0.050	0.120E 00 (0.891E-03)	0.102E 01 (0.107E-01)	13.420	0.116	0.358E 00 (0.689E-02)	0.243E 01 (0.622E-01)
3.154	0.052	0.138E 00 (0.103E-02)	0.108E 01 (0.112E-01)	13.656	0.119	0.352E 00 (0.711E-02)	0.242E 01 (0.641E-01)
3.248	0.041	0.141E 00 (0.119E-02)	0.111E 01 (0.131E-01)	13.897	0.122	0.363E 00 (0.712E-02)	0.242E 01 (0.625E-01)
3.347	0.057	0.141E 00 (0.114E-02)	0.115E 01 (0.129E-01)	14.145	0.126	0.353E 00 (0.693E-02)	0.237E 01 (0.629E-01)
3.449	0.045	0.151E 00 (0.127E-02)	0.115E 01 (0.137E-01)	14.400	0.129	0.325E 00 (0.690E-02)	0.241E 01 (0.683E-01)
3.541	0.047	0.155E 00 (0.127E-02)	0.120E 01 (0.139E-01)	14.662	0.133	0.306E 00 (0.745E-02)	0.236E 01 (0.751E-01)
3.637	0.049	0.159E 00 (0.142E-02)	0.125E 01 (0.152E-01)	14.931	0.136	0.306E 00 (0.714E-02)	0.236E 01 (0.724E-01)
3.736	0.051	0.175E 00 (0.145E-02)	0.127E 01 (0.146E-01)	15.280	0.212	0.296E 00 (0.657E-02)	0.233E 01 (0.673E-01)
3.840	0.053	0.175E 00 (0.142E-02)	0.134E 01 (0.151E-01)	15.712	0.221	0.263E 00 (0.636E-02)	0.223E 01 (0.697E-01)
3.948	0.055	0.177E 00 (0.147E-02)	0.134E 01 (0.155E-01)	16.243	0.310	0.235E 00 (0.564E-02)	0.216E 01 (0.684E-01)
4.060	0.058	0.187E 00 (0.153E-02)	0.137E 01 (0.156E-01)	16.798	0.245	0.227E 00 (0.606E-02)	0.210E 01 (0.739E-01)
4.158	0.040	0.198E 00 (0.188E-02)	0.138E 01 (0.182E-01)	17.298	0.256	0.215E 00 (0.615E-02)	0.203E 01 (0.788E-01)
4.238	0.041	0.194E 00 (0.180E-02)	0.141E 01 (0.182E-01)	17.821	0.267	0.222E 00 (0.642E-02)	0.196E 01 (0.766E-01)
4.343	0.064	0.198E 00 (0.171E-02)	0.142E 01 (0.170E-01)	18.274	0.185	0.223E 00 (0.850E-02)	0.197E 01 (0.967E-01)
4.451	0.044	0.193E 00 (0.180E-02)	0.144E 01 (0.191E-01)	18.748	0.289	0.239E 00 (0.640E-02)	0.182E 01 (0.705E-01)
4.540	0.045	0.199E 00 (0.193E-02)	0.146E 01 (0.195E-01)	19.237	0.200	0.238E 00 (0.897E-02)	0.184E 01 (0.934E-01)
4.632	0.047	0.208E 00 (0.203E-02)	0.146E 01 (0.195E-01)	19.749	0.312	0.247E 00 (0.851E-02)	0.192E 01 (0.858E-01)
4.727	0.048	0.210E 00 (0.204E-02)	0.147E 01 (0.193E-01)				

Table 2

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN PE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 0.85 TO 1.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	3.292E-03	8.300E-04	3.580E 00	7.245E-05	7.970E-05
7.100E-01	5.001E-03	8.678E-04	3.660E 00	6.565E-05	8.381E-05
7.300E-01	7.135E-03	9.319E-04	3.740E 00	6.949E-05	8.330E-05
7.500E-01	1.063E-02	9.780E-04	3.820E 00	6.648E-05	8.230E-05
7.700E-01	2.467E-02	1.073E-03	3.900E 00	5.388E-05	8.272E-05
7.900E-01	6.078E-02	1.174E-03	3.980E 00	4.637E-05	8.310E-05
8.100E-01	1.140E-01	1.244E-03	4.060E 00	6.328E-05	8.218E-05
8.300E-01	1.578E-01	1.279E-03	4.140E 00	9.483E-05	8.598E-05
8.500E-01	1.663E-01	1.150E-03	4.220E 00	1.179E-04	8.036E-05
8.700E-01	1.377E-01	1.049E-03	4.300E 00	8.821E-05	8.000E-05
8.900E-01	9.182E-02	7.752E-04	4.380E 00	2.060E-05	7.762E-05
9.100E-01	5.041E-02	5.514E-04	4.460E 00	-2.402E-05	-7.850E-05
9.300E-01	2.332E-02	3.626E-04	4.540E 00	-2.823E-05	-7.530E-05
9.500E-01	9.373E-03	2.956E-04	4.630E 00	-1.078E-05	-7.837E-05
9.700E-01	3.423E-03	2.073E-04	4.730E 00	2.365E-05	7.576E-05
1.000E 00	7.589E-04	2.086E-04	4.830E 00	4.886E-05	7.410E-05
1.040E 00	2.686E-04	1.633E-04	4.930E 00	5.347E-05	7.297E-05
1.080E 00	2.242E-04	1.561E-04	5.030E 00	3.134E-05	7.215E-05
1.120E 00	1.498E-04	1.503E-04	5.130E 00	1.948E-05	7.478E-05
1.160E 00	3.206E-05	1.426E-04	5.230E 00	3.838E-05	7.439E-05
1.200E 00	-7.180E-05	-1.332E-04	5.330E 00	6.761E-05	7.492E-05
1.240E 00	-3.562E-05	-1.427E-04	5.430E 00	5.640E-05	7.575E-05
1.280E 00	1.570E-04	1.383E-04	5.530E 00	2.407E-05	7.150E-05
1.320E 00	2.736E-04	1.406E-04	5.630E 00	9.159E-06	7.005E-05
1.360E 00	1.782E-04	1.413E-04	5.730E 00	1.639E-05	7.172E-05
1.400E 00	6.759E-05	1.412E-04	5.830E 00	4.415E-05	7.183E-05
1.440E 00	8.814E-05	1.346E-04	5.930E 00	8.517E-05	7.216E-05
1.480E 00	1.435E-04	1.286E-04	6.030E 00	9.399E-05	6.685E-05
1.520E 00	1.292E-04	1.353E-04	6.130E 00	4.005E-05	6.362E-05
1.560E 00	5.833E-05	1.341E-04	6.230E 00	-3.037E-05	-6.397E-05
1.600E 00	2.119E-05	1.253E-04	6.330E 00	-4.669E-05	-6.343E-05
1.640E 00	5.880E-05	1.268E-04	6.440E 00	-8.971E-06	-5.927E-05
1.680E 00	1.373E-04	1.266E-04	6.560E 00	3.418E-05	5.260E-05
1.720E 00	1.557E-04	1.294E-04	6.680E 00	6.830E-05	5.085E-05
1.760E 00	8.593E-05	1.236E-04	6.800E 00	8.973E-05	4.784E-05
1.800E 00	5.750E-07	1.183E-04	6.920E 00	7.247E-05	4.397E-05
1.850E 00	-4.594E-05	-1.141E-04	7.040E 00	3.829E-05	4.192E-05
1.910E 00	-1.889E-05	-1.139E-04	7.160E 00	2.813E-05	4.169E-05
1.970E 00	-2.855E-06	-1.115E-04	7.280E 00	3.519E-05	4.056E-05
2.030E 00	-2.184E-05	-1.135E-04	7.400E 00	5.317E-05	3.969E-05
2.090E 00	-4.445E-06	-1.194E-04	7.520E 00	7.848E-05	3.971E-05
2.150E 00	7.398E-05	1.174E-04	7.640E 00	7.904E-05	3.993E-05
2.210E 00	1.799E-04	1.103E-04	7.760E 00	4.990E-05	3.684E-05
2.270E 00	2.393E-04	9.684E-05	7.880E 00	3.474E-05	3.416E-05
2.330E 00	2.063E-04	9.285E-05	8.030E 00	5.823E-05	3.371E-05
2.390E 00	1.491E-04	9.680E-05	8.170E 00	7.918E-05	3.082E-05
2.450E 00	9.499E-05	9.070E-05	8.310E 00	9.833E-05	2.509E-05
2.510E 00	2.024E-05	9.167E-05	8.450E 00	1.233E-04	2.268E-05
2.570E 00	-1.959E-05	-9.563E-05	8.590E 00	1.123E-04	1.919E-05
2.630E 00	3.503E-05	9.478E-05	8.730E 00	6.675E-05	1.589E-05
2.690E 00	9.853E-05	8.579E-05	8.870E 00	3.275E-05	1.186E-05
2.750E 00	7.533E-05	8.234E-05	9.010E 00	2.375E-05	7.725E-06
2.810E 00	2.727E-05	9.003E-05	9.150E 00	1.928E-05	7.007E-06
2.870E 00	3.196E-05	8.934E-05	9.300E 00	9.744E-06	5.067E-06
2.930E 00	6.591E-05	9.195E-05	9.460E 00	2.093E-06	3.722E-06
2.990E 00	9.322E-05	8.772E-05	9.620E 00	1.558E-06	2.763E-06
3.050E 00	1.045E-04	8.571E-05	9.780E 00	3.087E-06	2.507E-06
3.110E 00	9.633E-05	8.602E-05	9.940E 00	3.382E-06	2.275E-06
3.180E 00	6.064E-05	8.744E-05	1.010E 01	2.463E-06	1.784E-06
3.260E 00	2.148E-05	9.015E-05	1.026E 01	1.466E-06	1.184E-06
3.340E 00	2.258E-05	8.981E-05	1.042E 01	6.088E-07	8.640E-07
3.420E 00	5.954E-05	8.522E-05	1.058E 01	2.485E-07	7.050E-07
3.500E 00	8.354E-05	8.333E-05			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	2.202E-03	1.005E-04
8.000E-01 - 1.000E 00	1.508E-02	1.422E-04
1.000E 00 - 1.200E 00	3.947E-05	2.721E-05
1.200E 00 - 1.400E 00	2.348E-05	1.169E-05
1.400E 00 - 1.600E 00	1.880E-05	2.663E-05
1.600E 00 - 1.800E 00	1.888E-05	2.515E-05
1.800E 00 - 2.000E 00	-4.023E-06	1.775E-05
2.000E 00 - 2.500E 00	5.573E-05	2.357E-05
2.500E 00 - 3.000E 00	2.291E-05	3.172E-05
3.000E 00 - 3.500E 00	3.045E-05	4.359E-05
3.500E 00 - 4.000E 00	3.230E-05	4.125E-05
4.000E 00 - 4.500E 00	2.995E-05	2.896E-05
4.500E 00 - 5.000E 00	1.003E-05	9.435E-06
5.000E 00 - 6.000E 00	4.018E-05	7.286E-05
6.000E 00 - 7.000E 00	3.512E-05	1.673E-05
7.000E 00 - 8.000E 00	4.962E-05	3.895E-05
8.000E 00 - 9.000E 00	7.924E-05	2.145E-05
9.000E 00 - 1.000E 01	7.707E-06	4.289E-06

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 1.00 TO 1.50 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	7.674E-03	1.362E-03	3.580E 00	3.413E-05	6.228E-05
7.100E-01	9.467E-03	1.411E-03	3.660E 00	3.003E-05	6.217E-05
7.300E-01	1.315E-02	1.570E-03	3.740E 00	2.559E-05	5.917E-05
7.500E-01	2.385E-02	1.613E-03	3.820E 00	4.886E-05	6.285E-05
7.700E-01	5.770E-02	1.738E-03	3.900E 00	7.307E-05	6.178E-05
7.900E-01	1.369E-01	1.723E-03	3.980E 00	7.209E-05	5.805E-05
8.100E-01	2.552E-01	1.665E-03	4.060E 00	7.222E-05	5.985E-05
8.300E-01	3.574E-01	1.772E-03	4.140E 00	7.459E-05	6.104E-05
8.500E-01	3.819E-01	1.447E-03	4.220E 00	5.712E-05	5.382E-05
8.700E-01	3.202E-01	1.437E-03	4.300E 00	4.016E-05	5.538E-05
8.900E-01	2.167E-01	9.013E-04	4.380E 00	4.591E-05	5.555E-05
9.100E-01	1.214E-01	6.945E-04	4.460E 00	4.703E-05	5.585E-05
9.300E-01	5.760E-02	4.035E-04	4.540E 00	2.499E-05	5.498E-05
9.500E-01	2.380E-02	3.692E-04	4.630E 00	8.646E-06	5.593E-05
9.700E-01	9.025E-03	2.375E-04	4.730E 00	1.233E-05	5.357E-05
1.000E 00	2.320E-03	3.032E-04	4.830E 00	-8.561E-07	-5.366E-05
1.040E 00	8.160E-04	1.915E-04	4.930E 00	-1.322E-05	-5.725E-05
1.080E 00	4.384E-04	1.568E-04	5.030E 00	1.159E-05	5.681E-05
1.120E 00	3.485E-04	1.509E-04	5.130E 00	5.778E-05	5.830E-05
1.160E 00	4.237E-04	1.386E-04	5.230E 00	8.973E-05	5.953E-05
1.200E 00	3.911E-04	1.339E-04	5.330E 00	9.819E-05	6.024E-05
1.240E 00	2.599E-04	1.356E-04	5.430E 00	8.858E-05	5.856E-05
1.280E 00	3.123E-04	1.419E-04	5.530E 00	5.665E-05	6.090E-05
1.320E 00	5.704E-04	1.463E-04	5.630E 00	2.487E-05	5.904E-05
1.360E 00	8.986E-04	1.539E-04	5.730E 00	9.829E-06	5.874E-05
1.400E 00	1.065E-03	1.550E-04	5.830E 00	8.891E-07	5.846E-05
1.440E 00	8.547E-04	1.342E-04	5.930E 00	-1.804E-05	-5.395E-05
1.480E 00	4.675E-04	1.152E-04	6.030E 00	-2.540E-05	-5.372E-05
1.520E 00	2.082E-04	1.186E-04	6.130E 00	-2.629E-06	-4.927E-05
1.560E 00	1.534E-04	1.085E-04	6.230E 00	3.814E-05	4.724E-05
1.600E 00	2.368E-04	1.083E-04	6.330E 00	6.191E-05	4.690E-05
1.640E 00	3.120E-04	1.039E-04	6.440E 00	5.249E-05	4.314E-05
1.680E 00	2.635E-04	1.013E-04	6.560E 00	2.938E-05	4.044E-05
1.720E 00	1.390E-04	1.075E-04	6.680E 00	1.762E-05	3.641E-05
1.760E 00	5.774E-05	1.044E-04	6.800E 00	2.232E-05	3.592E-05
1.800E 00	4.186E-05	9.880E-05	6.920E 00	2.618E-05	3.127E-05
1.850E 00	3.159E-05	8.722E-05	7.040E 00	2.864E-05	2.793E-05
1.910E 00	2.991E-05	8.880E-05	7.160E 00	4.073E-05	2.568E-05
1.970E 00	5.762E-05	8.439E-05	7.280E 00	5.348E-05	2.542E-05
2.030E 00	5.919E-05	8.476E-05	7.400E 00	4.775E-05	2.465E-05
2.090E 00	7.060E-05	8.729E-05	7.520E 00	3.425E-05	2.454E-05
2.150E 00	1.031E-04	9.080E-05	7.640E 00	3.101E-05	2.419E-05
2.210E 00	6.736E-05	9.200E-05	7.760E 00	3.168E-05	2.300E-05
2.270E 00	-2.089E-06	-8.151E-05	7.880E 00	2.222E-05	2.211E-05
2.330E 00	-4.693E-06	-7.773E-05	8.030E 00	4.781E-05	2.159E-05
2.390E 00	3.851E-05	7.123E-05	8.170E 00	6.149E-05	1.972E-05
2.450E 00	5.330E-05	7.010E-05	8.310E 00	4.769E-05	1.831E-05
2.510E 00	4.262E-05	6.422E-05	8.450E 00	3.855E-05	1.671E-05
2.570E 00	4.892E-05	6.718E-05	8.590E 00	5.931E-05	1.474E-05
2.630E 00	7.648E-05	6.636E-05	8.730E 00	7.968E-05	1.237E-05
2.690E 00	9.898E-05	6.291E-05	8.870E 00	7.129E-05	1.089E-05
2.750E 00	1.005E-04	5.974E-05	9.010E 00	4.818E-05	9.638E-06
2.810E 00	9.769E-05	6.464E-05	9.150E 00	2.800E-05	7.367E-06
2.870E 00	1.034E-04	6.634E-05	9.300E 00	1.311E-05	6.630E-06
2.930E 00	1.135E-04	6.626E-05	9.460E 00	5.145E-06	5.866E-06
2.990E 00	1.135E-04	6.695E-05	9.620E 00	7.626E-06	3.944E-06
3.050E 00	1.044E-04	6.596E-05	9.780E 00	1.236E-05	2.973E-06
3.110E 00	8.373E-05	6.489E-05	9.940E 00	1.123E-05	2.597E-06
3.180E 00	4.116E-05	6.758E-05	1.010E 01	6.059E-06	1.597E-06
3.260E 00	-9.216E-06	-6.809E-05	1.026E 01	2.077E-06	6.685E-07
3.340E 00	-3.888E-05	-6.598E-05	1.042E 01	4.161E-07	3.870E-07
3.420E 00	-3.312E-05	-6.366E-05	1.058E 01	9.101E-09	3.210E-07
3.500E 00	5.714E-06	6.312E-05			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	4.909E-03	1.613E-04
8.000E-01 - 1.000E 00	3.487E-02	1.841E-04
1.000E 00 - 1.200E 00	1.301E-04	3.384E-05
1.200E 00 - 1.400E 00	1.097E-04	2.890E-05
1.400E 00 - 1.600E 00	9.196E-05	2.421E-05
1.600E 00 - 1.800E 00	3.679E-05	2.082E-05
1.800E 00 - 2.000E 00	7.785E-06	1.766E-05
2.000E 00 - 2.500E 00	2.404E-05	2.162E-05
2.500E 00 - 3.000E 00	4.461E-05	3.264E-05
3.000E 00 - 3.500E 00	1.029E-05	7.890E-08
3.500E 00 - 4.000E 00	2.177E-05	3.070E-05
4.000E 00 - 4.500E 00	2.834E-05	2.851E-05
4.500E 00 - 5.000E 00	2.963E-06	4.752E-06
5.000E 00 - 6.000E 00	4.165E-05	4.582E-05
6.000E 00 - 7.000E 00	2.616E-05	2.290E-05
7.000E 00 - 8.000E 00	3.792E-05	2.447E-05
8.000E 00 - 9.000E 00	5.850E-05	1.571E-05
9.000E 00 - 1.000E 01	1.524E-05	5.230E-06

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 1.50 TO 1.99 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.446E-02	2.336E-03	3.580E 00	8.928E-05	7.645E-05
7.100E-01	1.726E-02	2.575E-03	3.660E 00	9.763E-05	7.609E-05
7.300E-01	2.608E-02	2.869E-03	3.740E 00	7.053E-05	7.588E-05
7.500E-01	5.690E-02	2.841E-03	3.820E 00	1.485E-05	7.200E-05
7.700E-01	1.435E-01	2.982E-03	3.900E 00	-5.876E-06	-7.185E-05
7.900E-01	3.037E-01	2.927E-03	3.980E 00	9.669E-06	7.065E-05
8.100E-01	4.864E-01	2.786E-03	4.060E 00	2.602E-05	7.527E-05
8.300E-01	5.907E-01	2.867E-03	4.140E 00	1.895E-05	7.629E-05
8.500E-01	5.572E-01	2.304E-03	4.220E 00	1.932E-05	7.368E-05
8.700E-01	4.190E-01	2.385E-03	4.300E 00	3.893E-05	6.841E-05
8.900E-01	2.576E-01	1.547E-03	4.380E 00	7.415E-05	6.481E-05
9.100E-01	1.330E-01	1.141E-03	4.460E 00	8.992E-05	6.800E-05
9.300E-01	5.966E-02	7.214E-04	4.540E 00	7.130E-05	7.229E-05
9.500E-01	2.418E-02	7.600E-04	4.630E 00	3.004E-05	7.329E-05
9.700E-01	9.357E-03	5.403E-04	4.730E 00	4.636E-06	7.218E-05
1.000E 00	2.730E-03	6.962E-04	4.830E 00	-5.174E-06	-6.818E-05
1.040E 00	1.353E-03	5.242E-04	4.930E 00	-3.265E-05	-6.952E-05
1.080E 00	7.894E-04	4.299E-04	5.030E 00	-2.416E-05	-6.914E-05
1.120E 00	7.381E-04	4.076E-04	5.130E 00	5.061E-05	7.149E-05
1.160E 00	9.076E-04	3.766E-04	5.230E 00	1.254E-04	7.170E-05
1.200E 00	8.589E-04	3.526E-04	5.330E 00	1.224E-04	7.207E-05
1.240E 00	1.069E-03	3.723E-04	5.430E 00	4.659E-05	6.866E-05
1.280E 00	2.499E-03	4.276E-04	5.530E 00	-4.378E-05	-6.643E-05
1.320E 00	7.707E-03	4.599E-04	5.630E 00	-9.835E-05	-6.785E-05
1.360E 00	1.508E-02	5.172E-04	5.730E 00	-9.414E-05	-7.057E-05
1.400E 00	1.755E-02	5.282E-04	5.830E 00	-3.049E-05	-7.060E-05
1.440E 00	1.302E-02	4.075E-04	5.930E 00	6.150E-05	6.930E-05
1.480E 00	6.459E-03	2.617E-04	6.030E 00	1.290E-04	6.647E-05
1.520E 00	2.435E-03	2.009E-04	6.130E 00	1.337E-04	6.379E-05
1.560E 00	1.147E-03	1.798E-04	6.230E 00	8.619E-05	6.079E-05
1.600E 00	9.339E-04	1.545E-04	6.330E 00	2.825E-05	5.849E-05
1.640E 00	8.974E-04	1.510E-04	6.440E 00	-6.610E-06	-5.941E-05
1.680E 00	8.583E-04	1.509E-04	6.560E 00	5.345E-06	5.534E-05
1.720E 00	6.867E-04	1.423E-04	6.680E 00	3.686E-05	4.862E-05
1.760E 00	3.896E-04	1.375E-04	6.800E 00	4.469E-05	4.337E-05
1.800E 00	1.451E-04	1.349E-04	6.920E 00	2.644E-05	3.989E-05
1.850E 00	4.976E-06	1.163E-04	7.040E 00	1.564E-05	3.739E-05
1.910E 00	-2.284E-05	-1.111E-04	7.160E 00	2.305E-05	3.508E-05
1.970E 00	8.599E-06	1.051E-04	7.280E 00	3.322E-05	3.386E-05
2.030E 00	3.971E-05	1.097E-04	7.400E 00	4.456E-05	3.355E-05
2.090E 00	1.922E-05	1.084E-04	7.520E 00	5.738E-05	3.351E-05
2.150E 00	2.175E-05	1.082E-04	7.640E 00	5.733E-05	3.297E-05
2.210E 00	7.335E-05	1.084E-04	7.760E 00	4.550E-05	3.266E-05
2.270E 00	1.030E-04	1.031E-04	7.890E 00	3.767E-05	3.239E-05
2.330E 00	9.336E-05	9.833E-05	8.030E 00	3.583E-05	3.109E-05
2.390E 00	9.496E-05	8.949E-05	8.170E 00	2.555E-05	3.138E-05
2.450E 00	1.271E-04	8.647E-05	8.310E 00	1.423E-05	2.982E-05
2.510E 00	1.319E-04	8.492E-05	8.450E 00	1.418E-05	2.686E-05
2.570E 00	9.287E-05	9.082E-05	8.590E 00	2.493E-05	2.576E-05
2.630E 00	6.090E-05	8.864E-05	8.730E 00	4.170E-05	2.428E-05
2.690E 00	6.738E-05	8.854E-05	8.870E 00	5.891E-05	2.155E-05
2.750E 00	9.333E-05	8.631E-05	9.010E 00	6.843E-05	1.872E-05
2.810E 00	1.102E-04	8.292E-05	9.150E 00	6.487E-05	1.627E-05
2.870E 00	9.787E-05	8.391E-05	9.300E 00	4.973E-05	1.315E-05
2.930E 00	6.710E-05	8.498E-05	9.460E 00	3.275E-05	9.992E-06
2.990E 00	2.061E-05	8.064E-05	9.620E 00	2.342E-05	7.445E-06
3.050E 00	-2.428E-05	-8.684E-05	9.780E 00	2.005E-05	4.892E-06
3.110E 00	-2.287E-05	-8.849E-05	9.940E 00	1.483E-05	3.922E-06
3.180E 00	4.552E-05	8.655E-05	1.010E 01	7.661E-06	2.431E-06
3.260E 00	9.495E-05	8.179E-05	1.026E 01	2.895E-06	1.204E-06
3.340E 00	7.419E-05	8.323E-05	1.042E 01	6.887E-07	7.468E-07
3.420E 00	6.512E-05	7.941E-05	1.058E 01	8.003E-08	6.285E-07
3.500E 00	8.196E-05	7.953E-05			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.104E-02	2.840E-04
8.000E-01 - 1.000E 00	5.070E-02	3.137E-04
1.000E 00 - 1.200E 00	2.184E-04	8.989E-05
1.200E 00 - 1.400E 00	1.417E-03	8.884E-05
1.400E 00 - 1.600E 00	1.272E-03	5.520E-05
1.600E 00 - 1.800E 00	1.357E-04	2.906E-05
1.800E 00 - 2.000E 00	2.247E-06	6.184E-06
2.000E 00 - 2.500E 00	3.661E-05	5.039E-05
2.500E 00 - 3.000E 00	4.151E-05	4.299E-05
3.000E 00 - 3.500E 00	2.255E-05	1.916E-05
3.500E 00 - 4.000E 00	2.515E-05	2.394E-05
4.000E 00 - 4.500E 00	2.148E-05	3.555E-05
4.500E 00 - 5.000E 00	5.029E-06	4.576E-06
5.000E 00 - 6.000E 00	1.467E-05	4.260E-06
6.000E 00 - 7.000E 00	4.858E-05	4.018E-05
7.000E 00 - 8.000E 00	3.968E-05	3.371E-05
8.000E 00 - 9.000E 00	3.261E-05	2.664E-05
9.000E 00 - 1.000E 01	3.677E-05	9.970E-06

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 1.99 TO 2.49 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.653E-02	3.621E-03	3.580E 00	6.221E-05	1.072E-04
7.100E-01	2.076E-02	3.822E-03	3.660E 00	1.543E-05	9.407E-05
7.300E-01	3.070E-02	4.078E-03	3.740E 00	1.597E-05	9.815E-05
7.500E-01	5.790E-02	4.085E-03	3.820E 00	2.125E-05	9.445E-05
7.700E-01	1.429E-01	4.389E-03	3.900E 00	3.069E-05	9.150E-05
7.900E-01	3.279E-01	4.464E-03	3.980E 00	5.815E-05	8.971E-05
8.100E-01	5.776E-01	4.409E-03	4.060E 00	1.028E-04	8.879E-05
8.300E-01	7.683E-01	4.730E-03	4.140E 00	1.379E-04	9.067E-05
8.500E-01	7.909E-01	3.942E-03	4.220E 00	1.013E-04	8.624E-05
8.700E-01	6.467E-01	3.992E-03	4.300E 00	9.177E-06	8.807E-05
8.900E-01	4.300E-01	2.643E-03	4.380E 00	-5.376E-05	-7.509E-05
9.100E-01	2.380E-01	2.039E-03	4.460E 00	-3.795E-05	-6.799E-05
9.300E-01	1.127E-01	1.314E-03	4.540E 00	1.660E-05	7.006E-05
9.500E-01	4.727E-02	1.356E-03	4.630E 00	7.155E-05	7.338E-05
9.700E-01	1.838E-02	1.008E-03	4.730E 00	8.456E-05	7.225E-05
1.000E 00	4.825E-03	1.257E-03	4.830E 00	4.394E-05	6.571E-05
1.040E 00	2.252E-03	9.369E-04	4.930E 00	-1.052E-05	-6.044E-05
1.080E 00	1.972E-03	7.900E-04	5.030E 00	-1.789E-05	-6.449E-05
1.120E 00	2.019E-03	7.092E-04	5.130E 00	2.263E-05	6.581E-05
1.160E 00	4.638E-03	7.219E-04	5.230E 00	5.676E-05	6.511E-05
1.200E 00	9.319E-03	7.115E-04	5.330E 00	3.510E-05	6.401E-05
1.240E 00	1.086E-02	7.563E-04	5.430E 00	-6.771E-06	-6.299E-05
1.280E 00	9.564E-03	7.754E-04	5.530E 00	1.738E-06	6.276E-05
1.320E 00	1.224E-02	7.984E-04	5.630E 00	5.516E-05	6.381E-05
1.360E 00	2.240E-02	8.481E-04	5.730E 00	7.529E-05	6.529E-05
1.400E 00	2.895E-02	8.572E-04	5.830E 00	3.146E-05	6.754E-05
1.440E 00	2.294E-02	6.934E-04	5.930E 00	-8.325E-06	-7.372E-05
1.480E 00	1.167E-02	4.682E-04	6.030E 00	5.412E-06	7.225E-05
1.520E 00	4.605E-03	3.492E-04	6.130E 00	5.987E-05	7.116E-05
1.560E 00	2.378E-03	3.269E-04	6.230E 00	9.133E-05	7.225E-05
1.600E 00	1.991E-03	2.823E-04	6.330E 00	7.276E-05	7.120E-05
1.640E 00	1.943E-03	2.832E-04	6.440E 00	3.122E-05	7.168E-05
1.680E 00	1.985E-03	2.703E-04	6.560E 00	2.237E-05	6.895E-05
1.720E 00	1.853E-03	2.590E-04	6.680E 00	3.544E-05	6.418E-05
1.760E 00	1.380E-03	2.255E-04	6.800E 00	3.679E-05	5.933E-05
1.800E 00	8.995E-04	2.252E-04	6.920E 00	2.496E-05	5.609E-05
1.850E 00	5.557E-04	1.895E-04	7.040E 00	1.300E-05	5.164E-05
1.910E 00	2.578E-04	1.865E-04	7.160E 00	6.644E-06	4.636E-05
1.970E 00	9.401E-05	1.851E-04	7.280E 00	2.139E-05	4.430E-05
2.030E 00	1.474E-04	1.591E-04	7.400E 00	5.784E-05	4.362E-05
2.090E 00	3.438E-04	1.596E-04	7.520E 00	8.482E-05	4.546E-05
2.150E 00	5.061E-04	1.556E-04	7.640E 00	7.788E-05	4.502E-05
2.210E 00	3.972E-04	1.510E-04	7.760E 00	3.877E-05	4.196E-05
2.270E 00	1.639E-04	1.353E-04	7.880E 00	-7.105E-06	-4.264E-05
2.330E 00	9.992E-05	1.296E-04	8.030E 00	-1.861E-05	-4.557E-05
2.390E 00	1.601E-04	1.141E-04	8.170E 00	1.184E-05	4.304E-05
2.450E 00	1.723E-04	1.189E-04	8.310E 00	4.832E-05	4.001E-05
2.510E 00	1.426E-04	1.109E-04	8.450E 00	5.580E-05	3.701E-05
2.570E 00	1.490E-04	1.204E-04	8.590E 00	3.398E-05	3.873E-05
2.630E 00	1.628E-04	1.187E-04	8.730E 00	1.902E-05	3.551E-05
2.690E 00	1.419E-04	1.151E-04	8.870E 00	2.336E-05	3.238E-05
2.750E 00	8.937E-05	1.147E-04	9.010E 00	4.436E-05	3.186E-05
2.810E 00	2.973E-05	1.175E-04	9.150E 00	5.123E-05	2.922E-05
2.870E 00	-7.116E-06	-1.138E-04	9.300E 00	3.928E-05	2.670E-05
2.930E 00	-1.086E-05	-1.125E-04	9.460E 00	3.561E-05	2.103E-05
2.990E 00	-9.410E-06	-1.080E-04	9.620E 00	5.033E-05	1.739E-05
3.050E 00	-2.548E-05	-1.076E-04	9.780E 00	6.104E-05	1.276E-05
3.110E 00	-4.020E-05	-1.085E-04	9.940E 00	4.700E-05	8.848E-06
3.180E 00	-8.467E-06	-1.079E-04	1.010E 01	2.386E-05	5.736E-06
3.260E 00	6.122E-05	9.584E-05	1.026E 01	8.109E-06	2.537E-06
3.340E 00	1.323E-04	1.035E-04	1.042E 01	1.636E-06	1.418E-06
3.420E 00	1.717E-04	9.450E-05	1.058E 01	4.259E-08	1.176E-06
3.500E 00	1.396E-04	9.835E-05			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.177E-02	4.169E-04
8.000E-01 - 1.000E 00	7.259E-02	5.318E-04
1.000E 00 - 1.200E 00	6.899E-04	1.646E-04
1.200E 00 - 1.400E 00	2.950E-03	1.587E-04
1.400E 00 - 1.600E 00	2.254E-03	9.576E-05
1.600E 00 - 1.800E 00	3.460E-04	5.161E-05
1.800E 00 - 2.000E 00	7.232E-05	3.807E-05
2.000E 00 - 2.500E 00	1.228E-04	6.986E-05
2.500E 00 - 3.000E 00	3.862E-05	2.203E-05
3.000E 00 - 3.500E 00	3.022E-05	3.763E-06
3.500E 00 - 4.000E 00	1.989E-05	4.815E-05
4.000E 00 - 4.500E 00	2.238E-05	1.900E-05
4.500E 00 - 5.000E 00	1.992E-05	1.921E-05
5.000E 00 - 6.000E 00	2.532E-05	3.001E-05
6.000E 00 - 7.000E 00	4.182E-05	6.629E-05
7.000E 00 - 8.000E 00	3.417E-05	3.012E-05
8.000E 00 - 9.000E 00	2.768E-05	3.055E-05
9.000E 00 - 1.000E 01	4.745E-05	2.032E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 2.49 TO 3.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.502E-02	4.695E-03	3.580E 00	-1.857E-04	-1.143E-04
7.100E-01	1.727E-02	4.638E-03	3.660E 00	-7.781E-05	-1.062E-04
7.300E-01	2.606E-02	4.983E-03	3.740E 00	-1.959E-05	-1.119E-04
7.500E-01	4.318E-02	5.203E-03	3.820E 00	-4.374E-05	-1.134E-04
7.700E-01	9.639E-02	5.730E-03	3.900E 00	-5.810E-05	-1.103E-04
7.900E-01	2.500E-01	5.611E-03	3.980E 00	4.707E-05	1.190E-04
8.100E-01	5.146E-01	5.979E-03	4.060E 00	1.791E-04	1.011E-04
8.300E-01	7.757E-01	6.504E-03	4.140E 00	2.505E-04	9.935E-05
8.500E-01	8.783E-01	5.998E-03	4.220E 00	2.085E-04	1.057E-04
8.700E-01	7.750E-01	5.906E-03	4.300E 00	1.471E-04	1.257E-04
8.900E-01	5.500E-01	4.351E-03	4.380E 00	1.034E-04	1.224E-04
9.100E-01	3.222E-01	3.358E-03	4.460E 00	6.927E-05	1.148E-04
9.300E-01	1.590E-01	2.328E-03	4.540E 00	3.945E-05	9.886E-05
9.500E-01	6.770E-02	2.351E-03	4.630E 00	1.554E-05	9.026E-05
9.700E-01	2.644E-02	1.904E-03	4.730E 00	1.729E-05	9.373E-05
1.000E 00	8.091E-03	2.134E-03	4.830E 00	1.638E-05	1.007E-04
1.040E 00	2.997E-03	1.766E-03	4.930E 00	-2.049E-05	-9.688E-05
1.080E 00	2.410E-03	1.639E-03	5.030E 00	-5.668E-05	-9.151E-05
1.120E 00	5.408E-03	1.557E-03	5.130E 00	-2.971E-05	-8.828E-05
1.160E 00	1.720E-02	1.618E-03	5.230E 00	3.192E-05	8.911E-05
1.200E 00	4.182E-02	1.744E-03	5.330E 00	6.859E-05	9.528E-05
1.240E 00	5.463E-02	1.726E-03	5.430E 00	8.912E-05	9.884E-05
1.280E 00	4.114E-02	1.693E-03	5.530E 00	1.305E-04	1.025E-04
1.320E 00	2.554E-02	1.531E-03	5.630E 00	1.522E-04	9.894E-05
1.360E 00	2.551E-02	1.454E-03	5.730E 00	1.037E-04	1.018E-04
1.400E 00	3.032E-02	1.522E-03	5.830E 00	6.089E-06	1.077E-04
1.440E 00	2.693E-02	1.345E-03	5.930E 00	-4.322E-05	-1.072E-04
1.480E 00	1.560E-02	9.464E-04	6.030E 00	-1.812E-05	-1.002E-04
1.520E 00	6.117E-03	8.089E-04	6.130E 00	4.472E-05	1.010E-04
1.560E 00	2.542E-03	7.917E-04	6.230E 00	1.116E-04	1.044E-04
1.600E 00	2.037E-03	7.074E-04	6.330E 00	1.376E-04	9.314E-05
1.640E 00	2.407E-03	7.173E-04	6.430E 00	1.092E-04	8.739E-05
1.680E 00	3.888E-03	7.664E-04	6.530E 00	1.451E-05	8.575E-05
1.720E 00	8.690E-03	8.813E-04	6.630E 00	-6.753E-05	-7.929E-05
1.760E 00	1.699E-02	8.646E-04	6.800E 00	-6.699E-05	-7.333E-05
1.800E 00	2.259E-02	9.790E-04	6.920E 00	1.274E-05	6.862E-05
1.850E 00	1.882E-02	7.322E-04	7.040E 00	1.105E-04	6.457E-05
1.910E 00	7.711E-03	4.977E-04	7.160E 00	1.544E-04	5.857E-05
1.970E 00	1.754E-03	3.723E-04	7.280E 00	1.210E-04	5.454E-05
2.030E 00	6.289E-04	3.387E-04	7.400E 00	5.614E-05	5.060E-05
2.090E 00	8.836E-04	2.958E-04	7.520E 00	2.495E-05	5.026E-05
2.150E 00	1.097E-03	3.122E-04	7.640E 00	3.011E-05	4.839E-05
2.210E 00	1.285E-03	2.921E-04	7.760E 00	4.360E-05	4.637E-05
2.270E 00	1.134E-03	2.619E-04	7.890E 00	4.142E-05	4.418E-05
2.330E 00	6.588E-04	2.278E-04	8.030E 00	2.053E-05	4.337E-05
2.390E 00	4.144E-04	2.043E-04	8.170E 00	-4.797E-06	-4.476E-05
2.450E 00	3.885E-04	1.814E-04	8.310E 00	7.780E-07	-4.437E-05
2.510E 00	3.723E-04	1.785E-04	8.450E 00	3.892E-05	4.241E-05
2.570E 00	4.847E-04	2.048E-04	8.590E 00	7.650E-05	4.063E-05
2.630E 00	6.796E-04	2.016E-04	8.730E 00	6.762E-05	4.016E-05
2.690E 00	6.502E-04	1.892E-04	8.870E 00	2.753E-05	3.941E-05
2.750E 00	4.366E-04	1.684E-04	9.010E 00	-4.087E-06	-3.558E-05
2.810E 00	2.865E-04	1.674E-04	9.150E 00	-1.040E-05	-3.260E-05
2.870E 00	2.039E-04	1.594E-04	9.300E 00	-2.223E-06	-3.129E-05
2.930E 00	1.351E-04	1.594E-04	9.460E 00	1.819E-05	2.847E-05
2.990E 00	5.755E 05	1.423E-04	9.620E 00	5.625E-05	2.110E-05
3.050E 00	3.781E-05	1.325E-04	9.780E 00	8.323E-05	1.607E-05
3.110E 00	4.219E-05	1.349E-04	9.940E 00	7.330E-05	1.510E-05
3.180E 00	7.035E-05	1.376E-04	1.010E 01	3.961E-05	9.536E-06
3.260E 00	6.540E-05	1.412E-04	1.026E 01	1.436E-05	4.306E-06
3.340E 00	-1.855E-05	-1.346E-04	1.042E 01	3.037E-06	2.483E-06
3.420E 00	-1.562E-04	-1.306E-04	1.058E 01	1.095E-07	2.036E-06
3.500E 00	-2.274E-04	-1.368E-04			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	8.884E-03	5.250E-04
8.000E-01 - 1.000E 00	8.146E-02	8.135E-04
1.000E 00 - 1.200E 00	1.993E-03	3.389E-04
1.200E 00 - 1.400E 00	7.338E-03	3.212E-04
1.400E 00 - 1.600E 00	2.677E-03	1.996E-04
1.600E 00 - 1.800E 00	1.753E-03	1.622E-04
1.800E 00 - 2.000E 00	2.116E-03	1.152E-04
2.000E 00 - 2.500E 00	4.024E-04	1.303E-04
2.500E 00 - 3.000E 00	1.905E-04	8.817E-05
3.000E 00 - 3.500E 00	-6.225E-06	1.455E-05
3.500E 00 - 4.000E 00	-3.742E-05	4.470E-05
4.000E 00 - 4.500E 00	7.831E-05	5.576E-05
4.500E 00 - 5.000E 00	5.283E-06	2.613E-05
5.000E 00 - 6.000E 00	4.594E-05	4.111E-05
6.000E 00 - 7.000E 00	3.009E-05	3.491E-05
7.000E 00 - 8.000E 00	6.988E-05	5.140E-05
8.000E 00 - 9.000E 00	3.086E-05	2.381E-05
9.000E 00 - 1.000E 01	3.329E-05	2.448E-07

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 3.00 TO 3.49 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.501E-02	6.686E-03	3.580E 00	5.933E-05	1.980E-04
7.100E-01	1.925E-02	6.756E-03	3.660E 00	2.822E-05	1.822E-04
7.300E-01	2.610E-02	7.308E-03	3.740E 00	-1.706E-05	-1.685E-04
7.500E-01	5.032E-02	7.370E-03	3.820E 00	5.491E-06	1.593E-04
7.700E-01	1.317E-01	7.728E-03	3.900E 00	9.591E-05	1.603E-04
7.900E-01	3.204E-01	7.831E-03	3.980E 00	1.891E-04	1.735E-04
8.100E-01	6.075E-01	8.311E-03	4.060E 00	1.965E-04	1.696E-04
8.300E-01	8.711E-01	9.092E-03	4.140E 00	1.158E-04	1.735E-04
8.500E-01	9.533E-01	8.276E-03	4.220E 00	4.520E-05	1.532E-04
8.700E-01	8.122E-01	8.034E-03	4.300E 00	7.233E-05	1.398E-04
8.900E-01	5.527E-01	6.284E-03	4.380E 00	1.274E-04	1.431E-04
9.100E-01	3.112E-01	4.978E-03	4.460E 00	1.210E-04	1.417E-04
9.300E-01	1.520E-01	3.885E-03	4.540E 00	2.862E-05	1.394E-04
9.500E-01	6.740E-02	4.046E-03	4.620E 00	-7.415E-05	-1.438E-04
9.700E-01	2.731E-02	3.653E-03	4.700E 00	-8.188E-05	-1.377E-04
1.000E 00	8.107E-03	4.106E-03	4.800E 00	-2.580E-06	-1.275E-04
1.040E 00	9.983E-03	3.604E-03	4.900E 00	7.350E-05	1.364E-04
1.080E 00	8.300E-03	3.192E-03	5.000E 00	1.108E-04	1.303E-04
1.120E 00	8.760E-03	3.031E-03	5.100E 00	7.856E-05	1.314E-04
1.160E 00	3.452E-02	3.290E-03	5.200E 00	-4.435E-06	-1.251E-04
1.200E 00	7.416E-02	3.381E-03	5.300E 00	-8.978E-05	-1.316E-04
1.240E 00	9.223E-02	3.449E-03	5.400E 00	-9.812E-05	-1.434E-04
1.280E 00	7.002E-02	3.268E-03	5.500E 00	-1.012E-05	-1.385E-04
1.320E 00	3.722E-02	3.126E-03	5.600E 00	1.329E-04	1.370E-04
1.360E 00	2.674E-02	3.220E-03	5.700E 00	2.393E-04	1.393E-04
1.400E 00	2.999E-02	3.152E-03	5.800E 00	2.264E-04	1.347E-04
1.440E 00	2.537E-02	2.862E-03	5.900E 00	1.145E-04	1.308E-04
1.480E 00	1.318E-02	2.619E-03	6.000E 00	7.056E-06	1.258E-04
1.520E 00	6.094E-03	2.394E-03	6.100E 00	-3.238E-05	-1.284E-04
1.560E 00	4.466E-03	2.319E-03	6.200E 00	-4.064E-06	-1.356E-04
1.600E 00	2.849E-03	2.247E-03	6.300E 00	2.173E-05	1.236E-04
1.640E 00	3.262E-03	2.394E-03	6.400E 00	2.571E-05	1.164E-04
1.680E 00	1.105E-02	2.570E-03	6.500E 00	2.229E-05	1.091E-04
1.720E 00	3.031E-02	3.286E-03	6.600E 00	5.106E-05	1.020E-04
1.760E 00	5.836E-02	3.241E-03	6.800E 00	1.008E-04	9.790E-05
1.800E 00	7.826E-02	3.857E-03	6.920E 00	1.129E-04	9.126E-05
1.850E 00	6.796E-02	2.488E-03	7.040E 00	6.593E-05	7.610E-05
1.910E 00	2.727E-02	1.659E-03	7.160E 00	3.453E-05	6.780E-05
1.970E 00	1.161E-02	1.509E-03	7.280E 00	5.056E-05	6.715E-05
2.030E 00	2.761E-02	1.602E-03	7.400E 00	7.493E-05	6.329E-05
2.090E 00	4.559E-02	1.735E-03	7.520E 00	6.721E-05	5.432E-05
2.150E 00	4.007E-02	1.378E-03	7.640E 00	2.732E-05	5.354E-05
2.210E 00	2.467E-02	1.014E-03	7.760E 00	-1.125E-05	-5.398E-05
2.270E 00	1.491E-02	8.016E-04	7.890E 00	-2.889E-05	-5.562E-05
2.330E 00	9.014E-03	5.892E-04	8.030E 00	8.530E-06	5.622E-05
2.390E 00	4.101E-03	4.902E-04	8.170E 00	8.547E-05	5.211E-05
2.450E 00	1.506E-03	4.885E-04	8.310E 00	1.320E-04	5.212E-05
2.510E 00	1.118E-03	4.788E-04	8.450E 00	1.131E-04	5.105E-05
2.570E 00	1.189E-03	4.940E-04	8.590E 00	5.666E-05	3.788E-05
2.630E 00	1.518E-03	4.793E-04	8.730E 00	1.133E-05	3.183E-05
2.690E 00	1.899E-03	4.288E-04	8.870E 00	-9.152E-06	-2.990E-05
2.750E 00	1.561E-03	3.947E-04	9.010E 00	-1.428E-05	-2.774E-05
2.810E 00	1.274E-03	3.909E-04	9.150E 00	-1.360E-05	-2.363E-05
2.870E 00	1.895E-03	3.873E-04	9.300E 00	-8.017E-06	-2.105E-05
2.930E 00	2.572E-03	3.714E-04	9.460E 00	8.415E-06	1.893E-05
2.990E 00	2.357E-03	3.171E-04	9.620E 00	3.502E-05	1.729E-05
3.050E 00	1.652E-03	3.073E-04	9.780E 00	5.608E-05	1.492E-05
3.110E 00	1.119E-03	2.790E-04	9.940E 00	5.121E-05	1.347E-05
3.180E 00	7.464E-04	2.280E-04	1.010E 01	3.086E-05	1.017E-05
3.260E 00	4.479E-04	2.390E-04	1.026E 01	1.264E-05	5.594E-06
3.340E 00	3.146E-04	2.154E-04	1.042E 01	3.064E-06	3.888E-06
3.420E 00	2.002E-04	2.021E-04	1.058E 01	2.943E-07	3.224E-06
3.500E 00	8.820E-05	2.031E-04			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.117E-02	7.416E-04
8.000E-01 - 1.000E 00	8.717E-02	1.209E-03
1.000E 00 - 1.200E 00	3.955E-03	6.723E-04
1.200E 00 - 1.400E 00	1.118E-02	6.539E-04
1.400E 00 - 1.600E 00	2.603E-03	5.150E-04
1.600E 00 - 1.800E 00	5.671E-03	5.781E-04
1.800E 00 - 2.000E 00	7.907E-03	4.141E-04
2.000E 00 - 2.500E 00	1.009E-02	4.964E-04
2.500E 00 - 3.000E 00	8.563E-04	2.091E-04
3.000E 00 - 3.500E 00	3.515E-04	1.205E-04
3.500E 00 - 4.000E 00	2.796E-05	6.201E-05
4.000E 00 - 4.500E 00	5.757E-05	7.722E-05
4.500E 00 - 5.000E 00	-3.984E-06	1.621E-05
5.000E 00 - 6.000E 00	6.866E-05	2.752E-05
6.000E 00 - 7.000E 00	3.789E-05	5.716E-05
7.000E 00 - 8.000E 00	3.142E-05	3.072E-05
8.000E 00 - 9.000E 00	5.533E-05	3.141E-05
9.000E 00 - 1.000E 01	1.882E-05	1.241E-06

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN PE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 3.49 TO 4.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.955E-02	8.449E-03	3.580E 00	7.836E-03	4.674E-04
7.100E-01	2.500E-02	8.531E-03	3.660E 00	4.493E-03	3.300E-04
7.300E-01	3.184E-02	9.024E-03	3.740E 00	1.726E-03	2.910E-04
7.500E-01	6.548E-02	9.553E-03	3.820E 00	3.685E-04	2.405E-04
7.700E-01	1.863E-01	1.003E-02	3.900E 00	1.264E-04	2.075E-04
7.900E-01	4.302E-01	9.804E-03	3.980E 00	1.974E-04	2.101E-04
8.100E-01	7.375E-01	1.037E-02	4.060E 00	1.300E-04	1.749E-04
8.300E-01	9.599E-01	1.080E-02	4.140E 00	2.020E-05	1.479E-04
8.500E-01	9.656E-01	9.441E-03	4.220E 00	-4.482E-05	-1.564E-04
8.700E-01	7.608E-01	9.564E-03	4.300E 00	-5.392E-05	-1.703E-04
8.900E-01	4.779E-01	7.611E-03	4.380E 00	-1.611E-05	-1.857E-04
9.100E-01	2.492E-01	6.671E-03	4.460E 00	-2.375E-06	-1.778E-04
9.300E-01	1.173E-01	5.521E-03	4.540E 00	2.450E-05	1.687E-04
9.500E-01	5.582E-02	5.866E-03	4.620E 00	9.950E-07	1.322E-04
9.700E-01	3.058E-02	5.136E-03	4.700E 00	-8.455E-05	-1.433E-04
1.000E 00	2.582E-02	5.932E-03	4.800E 00	-1.141E-04	-1.673E-04
1.040E 00	3.368E-02	5.210E-03	4.900E 00	-1.209E-05	-1.665E-04
1.080E 00	2.355E-02	4.837E-03	5.000E 00	1.371E-04	1.441E-04
1.120E 00	2.144E-02	4.397E-03	5.100E 00	2.071E-04	1.472E-04
1.160E 00	5.098E-02	4.799E-03	5.200E 00	2.064E-04	1.534E-04
1.200E 00	1.043E-01	4.859E-03	5.300E 00	2.449E-04	1.546E-04
1.240E 00	1.176E-01	4.710E-03	5.400E 00	3.087E-04	1.569E-04
1.280E 00	8.089E-02	4.643E-03	5.500E 00	2.712E-04	1.451E-04
1.320E 00	4.101E-02	4.618E-03	5.600E 00	1.532E-04	1.402E-04
1.360E 00	3.315E-02	4.785E-03	5.700E 00	-5.489E-06	-1.392E-04
1.400E 00	3.217E-02	4.982E-03	5.800E 00	-1.345E-04	-1.289E-04
1.440E 00	2.028E-02	4.542E-03	5.900E 00	-2.217E-04	-1.223E-04
1.480E 00	8.772E-03	4.296E-03	6.000E 00	-2.245E-04	-1.068E-04
1.520E 00	3.447E-03	3.805E-03	6.100E 00	-1.581E-04	-9.879E-05
1.560E 00	4.518E-03	3.808E-03	6.200E 00	-5.284E-05	-1.018E-04
1.600E 00	6.448E-03	3.348E-03	6.300E 00	4.176E-05	1.014E-04
1.640E 00	4.083E-03	3.719E-03	6.400E 00	1.061E-04	1.047E-04
1.680E 00	1.035E-02	4.057E-03	6.500E 00	1.411E-04	1.008E-04
1.720E 00	3.890E-02	4.834E-03	6.600E 00	1.502E-04	9.308E-05
1.760E 00	7.457E-02	4.450E-03	6.800E 00	1.555E-04	1.069E-04
1.800E 00	8.740E-02	5.029E-03	6.920E 00	1.262E-04	1.029E-04
1.850E 00	6.431E-02	3.522E-03	7.040E 00	7.812E-05	8.799E-05
1.910E 00	2.724E-02	2.900E-03	7.160E 00	1.744E-05	6.798E-05
1.970E 00	1.915E-02	2.995E-03	7.280E 00	3.945E-06	5.678E-05
2.030E 00	4.740E-02	3.241E-03	7.400E 00	3.212E-05	5.528E-05
2.090E 00	7.634E-02	3.317E-03	7.520E 00	6.451E-05	4.850E-05
2.150E 00	6.770E-02	2.640E-03	7.640E 00	5.421E-05	4.712E-05
2.210E 00	4.699E-02	2.331E-03	7.760E 00	2.408E-05	4.103E-05
2.270E 00	3.626E-02	1.943E-03	7.880E 00	1.752E-05	3.929E-05
2.330E 00	2.237E-02	1.658E-03	8.030E 00	4.347E-05	3.045E-05
2.390E 00	1.405E-02	1.577E-03	8.170E 00	5.449E-05	3.635E-05
2.450E 00	2.134E-02	1.469E-03	8.310E 00	2.407E-05	3.501E-05
2.510E 00	3.013E-02	1.517E-03	8.450E 00	-3.521E-06	-3.631E-05
2.570E 00	2.865E-02	1.445E-03	8.590E 00	6.349E-06	3.651E-05
2.630E 00	2.106E-02	1.216E-03	8.730E 00	3.454E-05	3.543E-05
2.690E 00	1.412E-02	1.065E-03	8.870E 00	4.926E-05	3.648E-05
2.750E 00	9.251E-03	9.465E-04	9.010E 00	3.821E-05	3.490E-05
2.810E 00	6.390E-03	9.333E-04	9.150E 00	1.018E-05	3.493E-05
2.870E 00	5.694E-03	9.259E-04	9.300E 00	-1.056E-05	-3.335E-05
2.930E 00	5.700E-03	8.761E-04	9.460E 00	-4.720E-06	-1.002E-05
2.990E 00	5.112E-03	8.340E-04	9.620E 00	2.774E-05	2.786E-05
3.050E 00	4.908E-03	7.999E-04	9.780E 00	5.530E-05	2.235E-05
3.110E 00	4.871E-03	7.533E-04	9.940E 00	5.603E-05	1.897E-05
3.180E 00	4.954E-03	7.148E-04	1.010E 01	3.626E-05	1.401E-05
3.260E 00	6.634E-03	7.484E-04	1.026E 01	1.726E-05	7.933E-06
3.340E 00	1.012E-02	7.299E-04	1.042E 01	5.584E-06	5.500E-06
3.420E 00	1.234E-02	7.519E-04	1.058E 01	1.396E-06	4.562E-06
3.500E 00	1.093E-02	6.035E-04			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.496E-02	9.408E-04
8.000E-01 - 1.000E 00	8.741E-02	1.530E-03
1.000E 00 - 1.200E 00	7.584E-03	9.837E-04
1.200E 00 - 1.400E 00	1.369E-02	9.458E-04
1.400E 00 - 1.600E 00	2.195E-03	8.259E-04
1.600E 00 - 1.800E 00	6.944E-03	8.469E-04
1.800E 00 - 2.000E 00	8.366E-03	6.637E-04
2.000E 00 - 2.500E 00	2.050E-02	1.122E-03
2.500E 00 - 3.000E 00	6.874E-03	5.387E-04
3.000E 00 - 3.500E 00	3.873E-03	3.711E-04
3.500E 00 - 4.000E 00	1.589E-03	1.582E-04
4.000E 00 - 4.500E 00	6.327E-06	2.220E-05
4.500E 00 - 5.000E 00	-1.738E-05	2.071E-05
5.000E 00 - 6.000E 00	1.099E-04	5.923E-05
6.000E 00 - 7.000E 00	4.865E-05	4.465E-05
7.000E 00 - 8.000E 00	3.473E-05	5.369E-05
8.000E 00 - 9.000E 00	3.027E-05	2.601E-05
9.000E 00 - 1.000E 01	2.288E-05	8.674E-06

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN PE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 4.00 TO 4.50 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.627E-02	1.032E-02	3.580E 00	2.418E-02	1.077E-03
7.100E-01	2.895E-02	1.009E-02	3.660E 00	1.633E-02	7.782E-04
7.300E-01	4.216E-02	1.054E-02	3.740E 00	8.026E-03	5.280E-04
7.500E-01	6.953E-02	1.079E-02	3.820E 00	3.682E-03	4.178E-04
7.700E-01	1.632E-01	1.115E-02	3.900E 00	2.056E-03	3.317E-04
7.900E-01	3.780E-01	1.122E-02	3.980E 00	1.081E-03	3.117E-04
8.100E-01	6.876E-01	1.178E-02	4.060E 00	4.880E-04	2.898E-04
8.300E-01	9.513E-01	1.266E-02	4.140E 00	3.158E-04	2.515E-04
8.500E-01	1.007E 00	1.176E-02	4.220E 00	2.464E-04	2.699E-04
8.700E-01	8.283E-01	1.179E-02	4.300E 00	1.316E-04	2.604E-04
8.900E-01	5.842E-01	9.466E-03	4.380E 00	8.139E-06	2.385E-04
9.100E-01	2.984E-01	8.421E-03	4.460E 00	-8.600E-05	-2.211E-04
9.300E-01	1.448E-01	6.943E-03	4.540E 00	-1.587E-04	-2.009E-04
9.500E-01	6.697E-02	7.766E-03	4.620E 00	-2.307E-04	-2.090E-04
9.700E-01	3.663E-02	6.979E-03	4.700E 00	-4.763E-05	-2.561E-04
1.000E 00	3.675E-02	7.688E-03	4.800E 00	2.896E-04	2.335E-04
1.040E 00	4.525E-02	7.095E-03	4.900E 00	5.189E-04	2.072E-04
1.080E 00	3.419E-02	6.444E-03	5.000E 00	4.971E-04	1.753E-04
1.120E 00	3.067E-02	5.837E-03	5.100E 00	3.585E-04	1.976E-04
1.160E 00	6.713E-02	6.330E-03	5.200E 00	1.984E-04	1.941E-04
1.200E 00	1.312E-01	6.421E-03	5.300E 00	1.405E-04	1.755E-04
1.240E 00	1.503E-01	6.495E-03	5.400E 00	1.411E-04	1.792E-04
1.280E 00	1.181E-01	6.280E-03	5.500E 00	1.419E-04	1.435E-04
1.320E 00	6.426E-02	5.554E-03	5.600E 00	1.066E-04	1.535E-04
1.360E 00	4.345E-02	5.674E-03	5.700E 00	1.085E-05	1.642E-04
1.400E 00	4.054E-02	5.921E-03	5.800E 00	-1.049E-04	-1.646E-04
1.440E 00	2.981E-02	5.308E-03	5.900E 00	-1.614E-04	-1.333E-04
1.480E 00	1.555E-02	4.782E-03	6.000E 00	-1.233E-04	-1.324E-04
1.520E 00	8.861E-03	4.647E-03	6.100E 00	-4.363E-05	-1.465E-04
1.560E 00	6.217E-03	4.787E-03	6.200E 00	5.970E-05	1.604E-04
1.600E 00	3.456E-03	4.655E-03	6.300E 00	1.146E-04	1.509E-04
1.640E 00	5.024E-03	4.852E-03	6.400E 00	9.634E-05	1.295E-04
1.680E 00	1.673E-02	4.941E-03	6.500E 00	3.561E-06	1.065E-04
1.720E 00	4.205E-02	5.772E-03	6.600E 00	-5.935E-05	-1.070E-04
1.760E 00	7.146E-02	5.663E-03	6.800E 00	-5.915E-05	-1.011E-04
1.800E 00	8.433E-02	6.311E-03	6.900E 00	-8.472E-06	-9.351E-05
1.850E 00	6.711E-02	4.515E-03	7.040E 00	2.638E-05	8.447E-05
1.910E 00	3.021E-02	3.854E-03	7.160E 00	3.907E-05	8.066E-05
1.970E 00	1.892E-02	4.130E-03	7.280E 00	4.232E-05	7.589E-05
2.030E 00	4.443E-02	4.624E-03	7.400E 00	4.876E-05	6.838E-05
2.090E 00	7.072E-02	4.386E-03	7.520E 00	5.110E-05	7.165E-05
2.150E 00	6.248E-02	3.850E-03	7.640E 00	3.180E-05	6.513E-05
2.210E 00	4.655E-02	3.176E-03	7.760E 00	1.019E-05	6.647E-05
2.270E 00	3.542E-02	2.844E-03	7.890E 00	-2.971E-06	-6.223E-05
2.330E 00	2.140E-02	2.600E-03	8.030E 00	8.439E-06	5.706E-05
2.390E 00	1.735E-02	2.392E-03	8.170E 00	4.255E-05	5.703E-05
2.450E 00	2.844E-02	2.650E-03	8.310E 00	7.347E-05	5.428E-05
2.510E 00	4.360E-02	2.809E-03	8.450E 00	6.773E-05	4.878E-05
2.570E 00	4.652E-02	2.420E-03	8.590E 00	3.347E-05	4.910E-05
2.630E 00	3.794E-02	2.187E-03	8.730E 00	7.766E-06	4.612E-05
2.690E 00	3.054E-02	1.981E-03	8.870E 00	1.798E-05	3.940E-05
2.750E 00	2.559E-02	1.772E-03	9.010E 00	3.788E-05	3.629E-05
2.810E 00	2.181E-02	1.723E-03	9.150E 00	4.498E-05	3.275E-05
2.870E 00	1.890E-02	1.588E-03	9.300E 00	2.747E-05	2.906E-05
2.930E 00	1.615E-02	1.593E-03	9.460E 00	6.090E-06	2.614E-05
2.990E 00	1.288E-02	1.654E-03	9.620E 00	4.920E-06	1.857E-05
3.050E 00	1.037E-02	1.574E-03	9.780E 00	1.826E-05	1.473E-05
3.110E 00	9.970E-03	1.464E-03	9.940E 00	2.647E-05	1.355E-05
3.180E 00	9.658E-03	1.350E-03	1.010E 01	2.320E-05	1.149E-05
3.260E 00	1.003E-02	1.289E-03	1.026E 01	1.538E-05	8.103E-06
3.340E 00	1.655E-02	1.340E-03	1.042E 01	7.541E-06	7.055E-06
3.420E 00	2.405E-02	1.357E-03	1.058E 01	3.369E-06	6.441E-06
3.500E 00	2.676E-02	1.230E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.385E-02	1.078E-03
8.000E-01 - 1.000E 00	9.181E-02	1.898E-03
1.000E 00 - 1.200E 00	1.021E-02	1.309E-03
1.200E 00 - 1.400E 00	1.839E-02	1.207E-03
1.400E 00 - 1.600E 00	3.258E-03	9.907E-04
1.600E 00 - 1.800E 00	7.134E-03	1.064E-03
1.800E 00 - 2.000E 00	8.641E-03	8.743E-04
2.000E 00 - 2.500E 00	2.040E-02	1.644E-03
2.500E 00 - 3.000E 00	1.416E-02	9.717E-04
3.000E 00 - 3.500E 00	7.361E-03	6.926E-04
3.500E 00 - 4.000E 00	5.464E-03	3.172E-04
4.000E 00 - 4.500E 00	1.069E-04	9.618E-05
4.500E 00 - 5.000E 00	5.009E-05	1.279E-05
5.000E 00 - 6.000E 00	1.195E-04	1.011E-04
6.000E 00 - 7.000E 00	4.282E-07	1.843E-06
7.000E 00 - 8.000E 00	2.923E-05	5.295E-05
8.000E 00 - 9.000E 00	3.705E-05	4.900E-05
9.000E 00 - 1.000E 01	2.231E-05	2.349E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 4.50 TO 5.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.679E-02	1.175E-02	3.580E 00	3.564E-02	1.416E-03
7.100E-01	2.362E-02	1.181E-02	3.660E 00	2.483E-02	1.065E-03
7.300E-01	3.659E-02	1.228E-02	3.740E 00	1.328E-02	7.547E-04
7.500E-01	5.753E-02	1.204E-02	3.820E 00	6.162E-03	5.536E-04
7.700E-01	1.219E-01	1.215E-02	3.900E 00	2.842E-03	4.608E-04
7.900E-01	3.047E-01	1.223E-02	3.980E 00	1.458E-03	4.384E-04
8.100E-01	6.156E-01	1.306E-02	4.060E 00	1.070E-03	3.913E-04
8.300E-01	9.232E-01	1.399E-02	4.140E 00	9.583E-04	3.218E-04
8.500E-01	1.044E 00	1.355E-02	4.220E 00	7.756E-04	3.402E-04
8.700E-01	9.166E-01	1.341E-02	4.300E 00	7.285E-04	3.151E-04
8.900E-01	6.479E-01	1.099E-02	4.380E 00	8.389E-04	3.059E-04
9.100E-01	3.833E-01	9.734E-03	4.460E 00	7.482E-04	3.061E-04
9.300E-01	1.983E-01	8.175E-03	4.540E 00	4.390E-04	2.957E-04
9.500E-01	9.534E-02	8.998E-03	4.630E 00	1.228E-04	2.858E-04
9.700E-01	5.277E-02	8.220E-03	4.730E 00	-2.167E-05	-2.660E-04
1.000E 00	5.070E-02	8.935E-03	4.830E 00	1.870E-05	2.143E-04
1.040E 00	6.049E-02	8.230E-03	4.930E 00	1.032E-04	2.238E-04
1.080E 00	4.555E-02	7.292E-03	5.030E 00	2.363E-04	2.219E-04
1.120E 00	3.905E-02	6.759E-03	5.130E 00	3.799E-04	2.092E-04
1.160E 00	6.905E-02	7.368E-03	5.230E 00	4.287E-04	2.318E-04
1.200E 00	1.454E-01	8.217E-03	5.330E 00	3.086E-04	2.283E-04
1.240E 00	1.913E-01	7.444E-03	5.430E 00	1.230E-04	1.931E-04
1.280E 00	1.474E-01	7.551E-03	5.530E 00	-5.667E-06	-1.790E-04
1.320E 00	8.058E-02	6.271E-03	5.630E 00	-6.715E-05	-1.729E-04
1.360E 00	5.334E-02	6.025E-03	5.730E 00	-8.979E-05	-1.783E-04
1.400E 00	4.781E-02	6.277E-03	5.830E 00	-1.024E-04	-1.682E-04
1.440E 00	3.586E-02	5.744E-03	5.930E 00	-1.318E-04	-1.499E-04
1.480E 00	1.908E-02	5.665E-03	6.030E 00	-1.578E-04	-1.388E-04
1.520E 00	1.119E-02	5.581E-03	6.130E 00	-1.213E-04	-1.337E-04
1.560E 00	7.382E-03	5.679E-03	6.230E 00	2.613E-05	1.384E-04
1.600E 00	1.794E-03	5.223E-03	6.330E 00	2.379E-04	1.588E-04
1.640E 00	3.257E-03	5.739E-03	6.430E 00	3.459E-04	1.531E-04
1.680E 00	1.583E-02	5.531E-03	6.530E 00	2.770E-04	1.283E-04
1.720E 00	4.091E-02	6.771E-03	6.630E 00	1.017E-04	1.184E-04
1.760E 00	7.199E-02	6.559E-03	6.800E 00	-1.400E-05	-1.244E-04
1.800E 00	8.822E-02	7.240E-03	6.920E 00	-8.314E-06	-1.091E-04
1.850E 00	7.319E-02	5.988E-03	7.040E 00	5.271E-05	9.667E-05
1.910E 00	3.837E-02	4.866E-03	7.160E 00	7.237E-05	9.160E-05
1.970E 00	3.024E-02	4.639E-03	7.280E 00	4.162E-05	8.517E-05
2.030E 00	5.552E-02	4.883E-03	7.400E 00	1.583E-06	7.276E-05
2.090E 00	7.390E-02	5.059E-03	7.520E 00	9.410E-07	7.222E-05
2.150E 00	6.187E-02	4.552E-03	7.640E 00	4.406E-05	7.510E-05
2.210E 00	4.676E-02	3.531E-03	7.760E 00	8.929E-05	6.749E-05
2.270E 00	3.651E-02	3.556E-03	7.890E 00	1.243E-04	6.481E-05
2.330E 00	2.420E-02	3.315E-03	8.030E 00	1.099E-04	6.059E-05
2.390E 00	1.766E-02	3.090E-03	8.170E 00	4.827E-05	5.603E-05
2.450E 00	3.005E-02	3.071E-03	8.310E 00	-9.127E-06	-5.118E-05
2.510E 00	4.719E-02	3.167E-03	8.450E 00	-1.958E-05	-3.976E-05
2.570E 00	4.795E-02	3.158E-03	8.590E 00	6.458E-06	3.922E-05
2.630E 00	4.023E-02	2.769E-03	8.730E 00	3.049E-05	3.807E-05
2.690E 00	3.683E-02	2.481E-03	8.870E 00	4.058E-05	3.201E-05
2.750E 00	3.093E-02	2.279E-03	9.010E 00	3.794E-05	2.990E-05
2.810E 00	2.050E-02	2.273E-03	9.150E 00	3.842E-05	2.721E-05
2.870E 00	1.583E-02	2.209E-03	9.300E 00	3.279E-05	2.652E-05
2.930E 00	1.659E-02	2.098E-03	9.460E 00	2.083E-05	2.420E-05
2.990E 00	1.642E-02	2.030E-03	9.620E 00	1.512E-05	1.816E-05
3.050E 00	1.714E-02	1.990E-03	9.780E 00	2.113E-05	1.589E-05
3.110E 00	1.993E-02	1.966E-03	9.940E 00	2.402E-05	1.475E-05
3.180E 00	2.139E-02	1.928E-03	1.010E 01	1.888E-05	1.226E-05
3.260E 00	2.203E-02	1.694E-03	1.026E 01	1.173E-05	8.162E-06
3.340E 00	2.718E-02	1.512E-03	1.042E 01	5.516E-06	7.115E-06
3.420E 00	3.361E-02	1.588E-03	1.058E 01	2.254E-06	6.898E-06
3.500E 00	3.805E-02	1.511E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.115E-02	1.212E-03
8.000E-01 - 1.000E 00	9.832E-02	2.173E-03
1.000E 00 - 1.200E 00	1.217E-02	1.523E-03
1.200E 00 - 1.400E 00	2.294E-02	1.380E-03
1.400E 00 - 1.600E 00	3.900E-03	1.137E-03
1.600E 00 - 1.800E 00	7.034E-03	1.229E-03
1.800E 00 - 2.000E 00	1.024E-02	1.071E-03
2.000E 00 - 2.500E 00	2.161E-02	1.927E-03
2.500E 00 - 3.000E 00	1.510E-02	1.245E-03
3.000E 00 - 3.500E 00	1.239E-02	8.771E-04
3.500E 00 - 4.000E 00	8.217E-03	4.259E-04
4.000E 00 - 4.500E 00	4.366E-04	1.666E-04
4.500E 00 - 5.000E 00	6.270E-05	7.721E-05
5.000E 00 - 6.000E 00	1.012E-04	1.606E-05
6.000E 00 - 7.000E 00	8.594E-05	2.457E-05
7.000E 00 - 8.000E 00	5.730E-05	7.602E-05
8.000E 00 - 9.000E 00	2.634E-05	1.888E-05
9.000E 00 - 1.000E 01	2.616E-05	2.177E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 5.00 TO 5.51 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	2.987E-02	1.270E-02	3.580E 00	4.873E-02	1.766E-03
7.100E-01	3.083E-02	1.226E-02	3.660E 00	3.720E-02	1.512E-03
7.300E-01	4.066E-02	1.308E-02	3.740E 00	2.229E-02	1.156E-03
7.500E-01	6.061E-02	1.354E-02	3.820E 00	1.312E-02	9.675E-04
7.700E-01	1.272E-01	1.387E-02	3.900E 00	1.013E-02	8.935E-04
7.900E-01	3.125E-01	1.373E-02	3.980E 00	7.736E-03	7.403E-04
8.100E-01	6.257E-01	1.465E-02	4.060E 00	4.478E-03	6.447E-04
8.300E-01	9.463E-01	1.617E-02	4.140E 00	2.378E-03	5.310E-04
8.500E-01	1.101E 00	1.576E-02	4.220E 00	1.600E-03	4.704E-04
8.700E-01	1.012E 00	1.517E-02	4.300E 00	1.292E-03	4.536E-04
8.900E-01	7.510E-01	1.271E-02	4.380E 00	1.367E-03	4.114E-04
9.100E-01	4.565E-01	1.149E-02	4.460E 00	1.797E-03	4.339E-04
9.300E-01	2.357E-01	9.756E-03	4.540E 00	1.998E-03	4.106E-04
9.500E-01	1.144E-01	1.081E-02	4.630E 00	1.555E-03	3.773E-04
9.700E-01	6.546E-02	9.702E-03	4.730E 00	9.917E-04	3.374E-04
1.000E 00	5.464E-02	1.058E-02	4.830E 00	6.734E-04	2.997E-04
1.040E 00	7.083E-02	9.392E-03	4.930E 00	3.347E-04	2.803E-04
1.080E 00	6.436E-02	8.589E-03	5.030E 00	1.055E-04	2.692E-04
1.120E 00	4.361E-02	8.076E-03	5.130E 00	7.972E-05	2.574E-04
1.160E 00	8.108E-02	8.357E-03	5.230E 00	2.232E-04	2.491E-04
1.200E 00	1.757E-01	9.238E-03	5.330E 00	3.002E-04	2.521E-04
1.240E 00	2.341E-01	9.212E-03	5.430E 00	2.170E-04	2.394E-04
1.280E 00	1.943E-01	9.203E-03	5.530E 00	-1.674E-05	-2.055E-04
1.320E 00	1.135E-01	8.249E-03	5.630E 00	-2.488E-04	-1.963E-04
1.360E 00	6.330E-02	7.824E-03	5.730E 00	-3.556E-04	-1.658E-04
1.400E 00	5.225E-02	8.085E-03	5.830E 00	-2.501E-04	-1.635E-04
1.440E 00	4.695E-02	7.085E-03	5.930E 00	3.409E-05	1.569E-04
1.480E 00	2.901E-02	6.609E-03	6.030E 00	2.918E-04	1.463E-04
1.520E 00	1.527E-02	6.580E-03	6.130E 00	3.781E-04	1.542E-04
1.560E 00	1.382E-02	6.680E-03	6.230E 00	3.041E-04	1.456E-04
1.600E 00	1.375E-02	6.528E-03	6.330E 00	1.921E-04	1.407E-04
1.640E 00	1.266E-02	6.359E-03	6.440E 00	9.204E-05	1.273E-04
1.680E 00	2.064E-02	6.645E-03	6.560E 00	4.613E-05	1.232E-04
1.720E 00	4.806E-02	7.483E-03	6.680E 00	1.085E-05	1.234E-04
1.760E 00	8.649E-02	7.316E-03	6.800E 00	-2.840E-05	-1.180E-04
1.800E 00	1.081E-01	8.212E-03	6.920E 00	-3.976E-05	-1.069E-04
1.850E 00	9.440E-02	7.152E-03	7.040E 00	-2.424E-05	-1.011E-04
1.910E 00	5.668E-02	5.553E-03	7.160E 00	-1.256E-06	-9.348E-05
1.970E 00	4.372E-02	5.873E-03	7.280E 00	1.258E-05	8.216E-05
2.030E 00	6.236E-02	5.772E-03	7.400E 00	1.188E-05	7.228E-05
2.090E 00	7.759E-02	5.826E-03	7.520E 00	1.909E-05	7.394E-05
2.150E 00	6.459E-02	4.983E-03	7.640E 00	3.218E-05	6.873E-05
2.210E 00	4.378E-02	4.147E-03	7.760E 00	4.416E-05	6.155E-05
2.270E 00	3.448E-02	3.986E-03	7.890E 00	5.228E-05	5.262E-05
2.330E 00	2.611E-02	3.837E-03	8.030E 00	3.617E-05	5.501E-05
2.390E 00	2.390E-02	3.643E-03	8.170E 00	2.187E-05	5.693E-05
2.450E 00	3.478E-02	3.609E-03	8.310E 00	1.054E-05	4.938E-05
2.510E 00	5.162E-02	3.741E-03	8.450E 00	5.913E-06	4.688E-05
2.570E 00	6.045E-02	3.616E-03	8.590E 00	1.864E-06	4.746E-05
2.630E 00	5.305E-02	3.359E-03	8.730E 00	1.001E-05	4.764E-05
2.690E 00	4.057E-02	3.182E-03	8.870E 00	3.652E-05	4.305E-05
2.750E 00	3.429E-02	2.854E-03	9.010E 00	5.671E-05	4.171E-05
2.810E 00	2.911E-02	2.719E-03	9.150E 00	5.553E-05	3.864E-05
2.870E 00	2.269E-02	2.545E-03	9.300E 00	2.326E-05	3.789E-05
2.930E 00	1.879E-02	2.524E-03	9.460E 00	-4.380E-07	-3.487E-05
2.990E 00	1.753E-02	2.691E-03	9.620E 00	3.245E-06	2.833E-05
3.050E 00	1.863E-02	2.601E-03	9.780E 00	2.403E-05	2.314E-05
3.110E 00	2.164E-02	2.519E-03	9.940E 00	3.537E-05	1.913E-05
3.180E 00	2.689E-02	2.384E-03	1.010E 01	3.003E-05	1.610E-05
3.260E 00	2.960E-02	2.153E-03	1.026E 01	1.981E-05	1.116E-05
3.340E 00	2.772E-02	2.240E-03	1.042E 01	9.313E-06	9.206E-06
3.420E 00	3.410E-02	1.983E-03	1.058E 01	4.524E-06	7.699E-06
3.500E 00	4.636E-02	2.132E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.170E-02	1.334E-03
8.000E-01 - 1.000E 00	1.071E-01	2.526E-03
1.000E 00 - 1.200E 00	1.467E-02	1.765E-03
1.200E 00 - 1.400E 00	2.900E-02	1.726E-03
1.400E 00 - 1.600E 00	5.496E-03	1.367E-03
1.600E 00 - 1.800E 00	9.072E-03	1.401E-03
1.800E 00 - 2.000E 00	1.380E-02	1.266E-03
2.000E 00 - 2.500E 00	2.300E-02	2.225E-03
2.500E 00 - 3.000E 00	1.838E-02	1.504E-03
3.000E 00 - 3.500E 00	1.397E-02	1.145E-03
3.500E 00 - 4.000E 00	1.287E-02	6.322E-04
4.000E 00 - 4.500E 00	1.170E-03	2.494E-04
4.500E 00 - 5.000E 00	5.168E-04	1.678E-04
5.000E 00 - 6.000E 00	1.055E-05	6.490E-05
6.000E 00 - 7.000E 00	1.206E-04	7.050E-05
7.000E 00 - 8.000E 00	2.086E-05	3.049E-05
8.000E 00 - 9.000E 00	1.869E-05	4.891E-05
9.000E 00 - 1.000E 01	2.544E-05	2.005E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 5.51 TO 6.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	2.430E-02	1.486E-02	3.580E 00	4.780E-02	2.103E-03
7.100E-01	2.212E-02	1.428E-02	3.660E 00	3.868E-02	1.733E-03
7.300E-01	3.761E-02	1.451E-02	3.740E 00	2.696E-02	1.542E-03
7.500E-01	6.309E-02	1.479E-02	3.820E 00	1.828E-02	1.443E-03
7.700E-01	1.297E-01	1.535E-02	3.900E 00	1.339E-02	1.206E-03
7.900E-01	3.112E-01	1.539E-02	3.980E 00	1.181E-02	1.105E-03
8.100E-01	6.227E-01	1.644E-02	4.060E 00	9.314E-03	9.597E-04
8.300E-01	9.534E-01	1.819E-02	4.140E 00	5.194E-03	8.517E-04
8.500E-01	1.121E 00	1.789E-02	4.220E 00	3.737E-03	8.309E-04
8.700E-01	1.027E 00	1.760E-02	4.300E 00	5.067E-03	8.521E-04
8.900E-01	7.507E-01	1.436E-02	4.380E 00	5.616E-03	7.824E-04
9.100E-01	4.540E-01	1.298E-02	4.460E 00	4.406E-03	7.603E-04
9.300E-01	2.418E-01	1.129E-02	4.540E 00	3.248E-03	7.250E-04
9.500E-01	1.226E-01	1.193E-02	4.620E 00	2.988E-03	6.801E-04
9.700E-01	6.828E-02	1.101E-02	4.700E 00	2.524E-03	6.631E-04
1.000E 00	5.975E-02	1.231E-02	4.800E 00	1.472E-03	5.902E-04
1.040E 00	7.690E-02	1.139E-02	4.900E 00	8.833E-04	5.041E-04
1.080E 00	5.977E-02	1.035E-02	5.000E 00	1.524E-03	4.636E-04
1.120E 00	5.324E-02	9.187E-03	5.100E 00	2.288E-03	4.303E-04
1.160E 00	8.823E-02	1.007E-02	5.200E 00	2.170E-03	4.907E-04
1.200E 00	1.961E-01	9.905E-03	5.300E 00	1.580E-03	4.771E-04
1.240E 00	2.750E-01	1.039E-02	5.400E 00	1.030E-03	4.153E-04
1.280E 00	2.285E-01	1.058E-02	5.500E 00	4.705E-04	3.110E-04
1.320E 00	1.235E-01	9.130E-03	5.600E 00	9.620E-05	2.580E-04
1.360E 00	7.420E-02	8.778E-03	5.700E 00	-6.000E-06	-2.603E-04
1.400E 00	6.675E-02	8.472E-03	5.800E 00	1.324E-04	2.727E-04
1.440E 00	4.891E-02	7.790E-03	5.900E 00	3.071E-04	2.544E-04
1.480E 00	2.889E-02	7.357E-03	6.000E 00	3.456E-04	2.262E-04
1.520E 00	2.404E-02	7.215E-03	6.100E 00	2.470E-04	2.005E-04
1.560E 00	2.279E-02	7.435E-03	6.200E 00	5.773E-05	1.965E-04
1.600E 00	1.983E-02	7.319E-03	6.300E 00	-8.766E-05	-1.925E-04
1.640E 00	2.091E-02	7.410E-03	6.400E 00	-1.549E-04	-1.860E-04
1.680E 00	3.035E-02	7.273E-03	6.500E 00	-1.209E-04	-1.670E-04
1.720E 00	5.766E-02	8.260E-03	6.600E 00	-2.624E-05	-1.372E-04
1.760E 00	9.472E-02	7.910E-03	6.800E 00	4.760E-05	1.146E-04
1.800E 00	1.140E-01	8.693E-03	6.920E 00	7.292E-05	1.174E-04
1.850E 00	1.016E-01	7.849E-03	7.040E 00	7.115E-05	1.079E-04
1.910E 00	6.586E-02	6.286E-03	7.160E 00	6.414E-05	9.060E-05
1.970E 00	4.558E-02	6.613E-03	7.280E 00	7.367E-05	8.664E-05
2.030E 00	6.616E-02	6.449E-03	7.400E 00	1.116E-04	7.591E-05
2.090E 00	8.276E-02	5.957E-03	7.520E 00	1.328E-04	6.936E-05
2.150E 00	6.449E-02	5.672E-03	7.640E 00	1.158E-04	6.110E-05
2.210E 00	4.246E-02	4.922E-03	7.760E 00	6.004E-05	5.350E-05
2.270E 00	3.296E-02	4.355E-03	7.890E 00	-1.875E-06	-5.761E-05
2.330E 00	2.637E-02	4.194E-03	8.030E 00	-2.570E-05	-4.914E-05
2.390E 00	2.705E-02	4.156E-03	8.170E 00	-1.017E-05	-4.489E-05
2.450E 00	4.088E-02	4.269E-03	8.310E 00	1.616E-05	4.867E-05
2.510E 00	5.773E-02	4.292E-03	8.450E 00	3.417E-05	4.323E-05
2.570E 00	6.342E-02	4.060E-03	8.590E 00	5.553E-05	4.098E-05
2.630E 00	5.325E-02	3.715E-03	8.730E 00	6.933E-05	4.151E-05
2.690E 00	4.129E-02	3.395E-03	8.870E 00	5.510E-05	4.415E-05
2.750E 00	3.411E-02	3.085E-03	9.010E 00	2.194E-05	4.350E-05
2.810E 00	2.618E-02	3.132E-03	9.150E 00	-1.861E-06	-3.915E-05
2.870E 00	2.063E-02	3.127E-03	9.300E 00	2.088E-07	3.499E-05
2.930E 00	2.066E-02	2.920E-03	9.460E 00	1.529E-05	2.979E-05
2.990E 00	2.049E-02	3.018E-03	9.620E 00	2.675E-05	2.586E-05
3.050E 00	2.056E-02	2.951E-03	9.780E 00	3.579E-05	2.535E-05
3.110E 00	2.493E-02	2.891E-03	9.940E 00	3.872E-05	2.496E-05
3.180E 00	3.089E-02	2.770E-03	1.010E 01	2.918E-05	1.863E-05
3.260E 00	3.012E-02	2.698E-03	1.026E 01	1.696E-05	1.178E-05
3.340E 00	2.838E-02	2.475E-03	1.042E 01	5.790E-06	1.007E-05
3.420E 00	3.388E-02	2.392E-03	1.058E 01	1.577E-06	8.674E-06
3.500E 00	4.469E-02	2.202E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.156E-02	1.490E-03
8.000E-01 - 1.000E 00	1.082E-01	2.868E-03
1.000E 00 - 1.200E 00	1.580E-02	2.084E-03
1.200E 00 - 1.400E 00	3.362E-02	1.928E-03
1.400E 00 - 1.600E 00	6.645E-03	1.506E-03
1.600E 00 - 1.800E 00	1.076E-02	1.548E-03
1.800E 00 - 2.000E 00	1.506E-02	1.406E-03
2.000E 00 - 2.500E 00	2.407E-02	2.480E-03
2.500E 00 - 3.000E 00	1.877E-02	1.696E-03
3.000E 00 - 3.500E 00	1.469E-02	1.328E-03
3.500E 00 - 4.000E 00	1.413E-02	7.952E-04
4.000E 00 - 4.500E 00	2.884E-03	4.239E-04
4.500E 00 - 5.000E 00	1.079E-03	3.114E-04
5.000E 00 - 6.000E 00	9.420E-04	3.044E-04
6.000E 00 - 7.000E 00	2.893E-05	1.041E-05
7.000E 00 - 8.000E 00	7.302E-05	5.396E-05
8.000E 00 - 9.000E 00	3.024E-05	2.151E-05
9.000E 00 - 1.000E 01	1.892E-05	1.788E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 6.00 TO 6.51 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.582E-02	1.564E-02	3.580E 00	4.393E-02	2.261E-03
7.100E-01	2.708E-02	1.531E-02	3.660E 00	3.430E-02	1.975E-03
7.300E-01	4.276E-02	1.533E-02	3.740E 00	2.553E-02	1.740E-03
7.500E-01	6.652E-02	1.587E-02	3.820E 00	1.968E-02	1.591E-03
7.700E-01	1.311E-01	1.648E-02	3.900E 00	1.557E-02	1.479E-03
7.900E-01	3.113E-01	1.582E-02	3.980E 00	1.176E-02	1.504E-03
8.100E-01	6.284E-01	1.689E-02	4.060E 00	8.214E-03	1.250E-03
8.300E-01	9.636E-01	1.872E-02	4.140E 00	6.044E-03	1.161E-03
8.500E-01	1.128E 00	1.839E-02	4.220E 00	6.296E-03	1.114E-03
8.700E-01	1.032E 00	1.828E-02	4.300E 00	7.432E-03	1.103E-03
8.900E-01	7.601E-01	1.508E-02	4.380E 00	7.298E-03	1.111E-03
9.100E-01	4.652E-01	1.374E-02	4.460E 00	6.136E-03	1.095E-03
9.300E-01	2.494E-01	1.190E-02	4.540E 00	5.152E-03	1.058E-03
9.500E-01	1.276E-01	1.295E-02	4.620E 00	4.407E-03	9.347E-04
9.700E-01	7.417E-02	1.236E-02	4.700E 00	4.287E-03	9.163E-04
1.000E 00	6.367E-02	1.391E-02	4.830E 00	4.350E-03	8.433E-04
1.040E 00	8.073E-02	1.244E-02	4.930E 00	3.986E-03	7.821E-04
1.080E 00	7.185E-02	1.167E-02	5.030E 00	4.321E-03	6.978E-04
1.120E 00	6.039E-02	1.068E-02	5.130E 00	5.008E-03	6.011E-04
1.160E 00	1.078E-01	1.025E-02	5.230E 00	4.710E-03	5.733E-04
1.200E 00	2.255E-01	1.095E-02	5.330E 00	3.451E-03	5.650E-04
1.240E 00	2.856E-01	1.142E-02	5.430E 00	2.149E-03	4.611E-04
1.280E 00	2.313E-01	1.142E-02	5.530E 00	1.331E-03	4.046E-04
1.320E 00	1.438E-01	9.414E-03	5.630E 00	9.327E-04	3.267E-04
1.360E 00	8.298E-02	9.588E-03	5.730E 00	6.202E-04	2.935E-04
1.400E 00	6.121E-02	1.000E-02	5.830E 00	4.339E-04	2.803E-04
1.440E 00	5.116E-02	9.331E-03	5.930E 00	3.815E-04	2.575E-04
1.480E 00	3.313E-02	8.325E-03	6.030E 00	3.534E-04	2.478E-04
1.520E 00	2.128E-02	8.295E-03	6.130E 00	1.955E-04	2.105E-04
1.560E 00	2.161E-02	7.926E-03	6.230E 00	6.956E-05	2.039E-04
1.600E 00	2.340E-02	7.686E-03	6.330E 00	2.506E-05	2.012E-04
1.640E 00	2.476E-02	8.117E-03	6.430E 00	1.287E-05	1.719E-04
1.680E 00	3.413E-02	8.296E-03	6.560E 00	-2.084E-05	-1.412E-04
1.720E 00	5.756E-02	9.358E-03	6.680E 00	-5.548E-05	-1.172E-04
1.760E 00	9.002E-02	9.605E-03	6.800E 00	-7.753E-06	-1.135E-04
1.800E 00	1.103E-01	9.988E-03	6.920E 00	8.503E-05	1.081E-04
1.850E 00	1.014E-01	8.105E-03	7.040E 00	1.342E-04	9.442E-05
1.910E 00	6.832E-02	6.821E-03	7.160E 00	1.330E-04	8.197E-05
1.970E 00	5.174E-02	6.489E-03	7.280E 00	1.043E-04	7.891E-05
2.030E 00	7.380E-02	7.166E-03	7.400E 00	8.265E-05	8.178E-05
2.090E 00	9.497E-02	6.974E-03	7.520E 00	4.805E-05	7.114E-05
2.150E 00	7.721E-02	6.166E-03	7.640E 00	4.123E-05	7.130E-05
2.210E 00	5.099E-02	5.693E-03	7.760E 00	3.167E-05	6.509E-05
2.270E 00	4.268E-02	4.932E-03	7.890E 00	2.576E-05	7.373E-05
2.330E 00	3.425E-02	4.352E-03	8.030E 00	6.397E-06	6.001E-05
2.390E 00	3.378E-02	4.418E-03	8.170E 00	1.179E-05	4.751E-05
2.450E 00	4.712E-02	4.542E-03	8.310E 00	2.975E-05	5.623E-05
2.510E 00	6.052E-02	4.627E-03	8.450E 00	4.321E-05	4.990E-05
2.570E 00	6.610E-02	4.461E-03	8.590E 00	2.846E-05	4.246E-05
2.630E 00	5.892E-02	4.159E-03	8.730E 00	1.360E-05	3.624E-05
2.690E 00	4.544E-02	3.742E-03	8.870E 00	1.526E-05	3.694E-05
2.750E 00	3.630E-02	3.576E-03	9.010E 00	2.966E-05	3.215E-05
2.810E 00	3.246E-02	3.412E-03	9.150E 00	3.611E-05	2.596E-05
2.870E 00	3.017E-02	3.391E-03	9.300E 00	2.763E-05	2.428E-05
2.930E 00	2.872E-02	3.228E-03	9.460E 00	1.208E-05	2.270E-05
2.990E 00	2.739E-02	3.375E-03	9.620E 00	5.102E-06	2.211E-05
3.050E 00	2.699E-02	3.235E-03	9.780E 00	6.062E-06	1.708E-05
3.110E 00	3.035E-02	3.135E-03	9.940E 00	1.071E-05	1.418E-05
3.180E 00	3.380E-02	3.027E-03	1.010E 01	1.071E-05	1.206E-05
3.260E 00	3.186E-02	2.827E-03	1.026E 01	9.352E-06	8.409E-06
3.340E 00	3.141E-02	2.552E-03	1.042E 01	6.152E-06	8.526E-06
3.420E 00	3.673E-02	2.598E-03	1.058E 01	4.593E-06	8.822E-06
3.500E 00	4.506E-02	2.477E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.184E-02	1.580E-03
8.000E-01 - 1.000E 00	1.096E-01	3.031E-03
1.000E 00 - 1.200E 00	1.817E-02	2.290E-03
1.200E 00 - 1.400E 00	3.575E-02	2.096E-03
1.400E 00 - 1.600E 00	6.727E-03	1.706E-03
1.600E 00 - 1.800E 00	1.087E-02	1.767E-03
1.800E 00 - 2.000E 00	1.551E-02	1.484E-03
2.000E 00 - 2.500E 00	2.845E-02	2.744E-03
2.500E 00 - 3.000E 00	2.146E-02	1.876E-03
3.000E 00 - 3.500E 00	1.642E-02	1.430E-03
3.500E 00 - 4.000E 00	1.363E-02	9.104E-04
4.000E 00 - 4.500E 00	3.528E-03	5.756E-04
4.500E 00 - 5.000E 00	2.207E-03	4.462E-04
5.000E 00 - 6.000E 00	2.262E-03	4.365E-04
6.000E 00 - 7.000E 00	6.002E-05	7.012E-05
7.000E 00 - 8.000E 00	7.026E-05	7.608E-05
8.000E 00 - 9.000E 00	2.207E-05	4.542E-05
9.000E 00 - 1.000E 01	1.718E-05	2.188E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 6.51 TO 7.01 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	3.426E-02	1.641E-02	3.580E 00	3.928E-02	2.706E-03
7.100E-01	3.406E-02	1.638E-02	3.660E 00	3.531E-02	2.161E-03
7.300E-01	3.312E-02	1.744E-02	3.740E 00	2.975E-02	2.021E-03
7.500E-01	6.428E-02	1.732E-02	3.820E 00	2.219E-02	1.906E-03
7.700E-01	1.564E-01	1.720E-02	3.900E 00	1.782E-02	1.732E-03
7.900E-01	3.439E-01	1.743E-02	3.980E 00	1.475E-02	1.721E-03
8.100E-01	6.465E-01	1.888E-02	4.060E 00	1.004E-02	1.598E-03
8.300E-01	9.755E-01	2.008E-02	4.140E 00	6.051E-03	1.554E-03
8.500E-01	1.141E 00	1.956E-02	4.220E 00	5.705E-03	1.590E-03
8.700E-01	1.041E 00	1.961E-02	4.300E 00	7.820E-03	1.613E-03
8.900E-01	7.641E-01	1.693E-02	4.380E 00	8.956E-03	1.555E-03
9.100E-01	4.701E-01	1.556E-02	4.460E 00	8.345E-03	1.516E-03
9.300E-01	2.538E-01	1.278E-02	4.540E 00	7.598E-03	1.453E-03
9.500E-01	1.307E-01	1.422E-02	4.630E 00	6.960E-03	1.338E-03
9.700E-01	8.159E-02	1.306E-02	4.730E 00	6.779E-03	1.263E-03
1.000E 00	8.086E-02	1.407E-02	4.830E 00	6.743E-03	1.189E-03
1.040E 00	9.713E-02	1.316E-02	4.930E 00	5.839E-03	1.143E-03
1.080E 00	8.366E-02	1.249E-02	5.030E 00	5.738E-03	1.087E-03
1.120E 00	5.853E-02	1.187E-02	5.130E 00	6.879E-03	9.902E-04
1.160E 00	1.107E-01	1.235E-02	5.230E 00	7.833E-03	8.660E-04
1.200E 00	2.388E-01	1.227E-02	5.330E 00	7.221E-03	7.791E-04
1.240E 00	3.084E-01	1.329E-02	5.430E 00	5.198E-03	7.242E-04
1.280E 00	2.585E-01	1.342E-02	5.530E 00	3.272E-03	6.383E-04
1.320E 00	1.523E-01	1.101E-02	5.630E 00	2.144E-03	5.800E-04
1.360E 00	9.388E-02	1.074E-02	5.730E 00	1.698E-03	4.941E-04
1.400E 00	8.235E-02	1.050E-02	5.830E 00	1.513E-03	4.378E-04
1.440E 00	6.108E-02	9.944E-03	5.930E 00	1.311E-03	4.002E-04
1.480E 00	3.718E-02	9.534E-03	6.030E 00	1.067E-03	3.422E-04
1.520E 00	2.755E-02	9.137E-03	6.130E 00	9.116E-04	3.321E-04
1.560E 00	2.525E-02	9.524E-03	6.230E 00	7.826E-04	2.936E-04
1.600E 00	2.536E-02	9.097E-03	6.330E 00	5.512E-04	2.450E-04
1.640E 00	3.058E-02	9.352E-03	6.430E 00	2.638E-04	2.451E-04
1.680E 00	4.275E-02	9.415E-03	6.530E 00	4.892E-05	2.156E-04
1.720E 00	6.696E-02	1.054E-02	6.630E 00	1.844E-05	1.653E-04
1.760E 00	9.840E-02	1.030E-02	6.800E 00	1.188E-04	1.358E-04
1.800E 00	1.166E-01	1.113E-02	6.920E 00	2.456E-04	1.332E-04
1.850E 00	1.037E-01	9.525E-03	7.040E 00	3.022E-04	1.152E-04
1.910E 00	6.878E-02	8.014E-03	7.160E 00	2.581E-04	1.012E-04
1.970E 00	5.908E-02	7.388E-03	7.280E 00	1.978E-04	8.918E-05
2.030E 00	8.058E-02	7.566E-03	7.400E 00	1.452E-04	7.663E-05
2.090E 00	1.001E-01	6.931E-03	7.520E 00	7.729E-05	7.905E-05
2.150E 00	8.004E-02	6.751E-03	7.640E 00	-3.794E-06	-7.805E-05
2.210E 00	5.703E-02	5.922E-03	7.760E 00	-5.810E-05	-6.907E-05
2.270E 00	4.932E-02	5.525E-03	7.890E 00	-4.404E-05	-5.971E-05
2.330E 00	3.958E-02	5.331E-03	8.030E 00	1.870E-05	5.432E-05
2.390E 00	3.732E-02	5.254E-03	8.170E 00	6.066E-05	5.114E-05
2.450E 00	5.248E-02	5.453E-03	8.310E 00	7.178E-05	4.562E-05
2.510E 00	7.503E-02	5.511E-03	8.450E 00	7.865E-05	4.390E-05
2.570E 00	8.032E-02	5.567E-03	8.590E 00	8.046E-05	4.599E-05
2.630E 00	6.462E-02	4.929E-03	8.730E 00	5.374E-05	4.627E-05
2.690E 00	4.861E-02	4.441E-03	8.870E 00	5.976E-06	4.670E-05
2.750E 00	3.922E-02	3.858E-03	9.010E 00	-1.713E-05	-3.784E-05
2.810E 00	3.348E-02	3.733E-03	9.150E 00	-1.773E-05	-3.708E-05
2.870E 00	2.986E-02	3.568E-03	9.300E 00	1.807E-06	4.007E-05
2.930E 00	2.877E-02	3.699E-03	9.460E 00	1.804E-05	3.068E-05
2.990E 00	2.975E-02	3.565E-03	9.620E 00	3.541E-05	2.779E-05
3.050E 00	3.208E-02	3.358E-03	9.780E 00	5.279E-05	3.154E-05
3.110E 00	3.527E-02	3.232E-03	9.940E 00	5.656E-05	2.650E-05
3.180E 00	3.759E-02	3.205E-03	1.010E 01	3.896E-05	2.055E-05
3.260E 00	3.684E-02	2.939E-03	1.026E 01	1.816E-05	1.387E-05
3.340E 00	3.805E-02	2.851E-03	1.042E 01	5.321E-06	1.233E-05
3.420E 00	4.381E-02	2.770E-03	1.058E 01	-9.841E-08	-1.079E-05
3.500E 00	4.467E-02	2.594E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.288E-02	1.719E-03
8.000E-01 - 1.000E 00	1.115E-01	3.283E-03
1.000E 00 - 1.200E 00	1.986E-02	2.520E-03
1.200E 00 - 1.400E 00	3.900E-02	2.402E-03
1.400E 00 - 1.600E 00	8.098E-03	1.918E-03
1.600E 00 - 1.800E 00	1.235E-02	1.983E-03
1.800E 00 - 2.000E 00	1.618E-02	1.715E-03
2.000E 00 - 2.500E 00	3.120E-02	3.030E-03
2.500E 00 - 3.000E 00	2.374E-02	2.154E-03
3.000E 00 - 3.500E 00	1.894E-02	1.512E-03
3.500E 00 - 4.000E 00	1.420E-02	1.048E-03
4.000E 00 - 4.500E 00	4.021E-03	7.875E-04
4.500E 00 - 5.000E 00	3.363E-03	6.316E-04
5.000E 00 - 6.000E 00	4.197E-03	6.845E-04
6.000E 00 - 7.000E 00	3.991E-04	2.242E-04
7.000E 00 - 8.000E 00	9.734E-05	2.830E-05
8.000E 00 - 9.000E 00	5.090E-05	4.302E-05
9.000E 00 - 1.000E 01	2.112E-05	1.572E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 7.01 TO 7.51 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	2.753E-02	1.787E-02	3.580E 00	4.542E-02	2.877E-03
7.100E-01	3.804E-02	1.802E-02	3.660E 00	3.832E-02	2.540E-03
7.300E-01	5.344E-02	1.841E-02	3.740E 00	3.110E-02	2.339E-03
7.500E-01	8.945E-02	1.826E-02	3.820E 00	2.613E-02	2.189E-03
7.700E-01	1.832E-01	1.833E-02	3.900E 00	2.004E-02	2.085E-03
7.900E-01	3.806E-01	1.864E-02	3.980E 00	1.457E-02	2.000E-03
8.100E-01	6.856E-01	2.060E-02	4.060E 00	1.218E-02	1.902E-03
8.300E-01	1.002E 00	2.223E-02	4.140E 00	1.044E-02	1.873E-03
8.500E-01	1.156E 00	2.130E-02	4.220E 00	8.931E-03	1.801E-03
8.700E-01	1.050E 00	2.105E-02	4.300E 00	9.186E-03	1.873E-03
8.900E-01	7.713E-01	1.823E-02	4.380E 00	9.473E-03	1.861E-03
9.100E-01	4.794E-01	1.660E-02	4.460E 00	7.091E-03	1.782E-03
9.300E-01	2.678E-01	1.423E-02	4.540E 00	4.555E-03	1.715E-03
9.500E-01	1.400E-01	1.588E-02	4.630E 00	6.610E-03	1.732E-03
9.700E-01	7.700E-02	1.461E-02	4.730E 00	1.054E-02	1.716E-03
1.000E 00	6.765E-02	1.605E-02	4.830E 00	9.878E-03	1.683E-03
1.040E 00	1.010E-01	1.442E-02	4.930E 00	9.314E-03	1.518E-03
1.080E 00	8.521E-02	1.358E-02	5.030E 00	9.689E-03	1.483E-03
1.120E 00	6.338E-02	1.196E-02	5.130E 00	8.079E-03	1.401E-03
1.160E 00	1.351E-01	1.294E-02	5.230E 00	6.580E-03	1.392E-03
1.200E 00	2.706E-01	1.322E-02	5.330E 00	6.173E-03	1.220E-03
1.240E 00	3.454E-01	1.398E-02	5.430E 00	5.919E-03	1.095E-03
1.280E 00	2.838E-01	1.394E-02	5.530E 00	5.827E-03	1.013E-03
1.320E 00	1.671E-01	1.251E-02	5.630E 00	5.551E-03	8.918E-04
1.360E 00	1.064E-01	1.178E-02	5.730E 00	5.027E-03	8.194E-04
1.400E 00	7.904E-02	1.172E-02	5.830E 00	4.382E-03	7.826E-04
1.440E 00	5.451E-02	1.085E-02	5.930E 00	3.627E-03	7.258E-04
1.480E 00	4.031E-02	1.024E-02	6.030E 00	2.715E-03	6.762E-04
1.520E 00	3.411E-02	9.327E-03	6.130E 00	1.960E-03	5.793E-04
1.560E 00	3.371E-02	9.711E-03	6.230E 00	1.591E-03	5.151E-04
1.600E 00	3.791E-02	9.961E-03	6.330E 00	1.721E-03	4.924E-04
1.640E 00	4.112E-02	9.591E-03	6.440E 00	1.966E-03	4.134E-04
1.680E 00	5.014E-02	1.020E-02	6.560E 00	1.707E-03	3.582E-04
1.720E 00	7.532E-02	1.041E-02	6.680E 00	9.689E-04	3.009E-04
1.760E 00	1.086E-01	1.102E-02	6.800E 00	3.854E-04	2.666E-04
1.800E 00	1.246E-01	1.202E-02	6.920E 00	1.884E-04	2.491E-04
1.850E 00	1.055E-01	1.074E-02	7.040E 00	1.651E-04	2.089E-04
1.910E 00	6.675E-02	9.512E-03	7.160E 00	1.644E-04	1.643E-04
1.970E 00	6.372E-02	8.290E-03	7.280E 00	1.740E-04	1.404E-04
2.030E 00	8.591E-02	8.243E-03	7.400E 00	1.463E-04	1.329E-04
2.090E 00	9.294E-02	8.453E-03	7.520E 00	8.494E-05	1.170E-04
2.150E 00	8.014E-02	7.576E-03	7.640E 00	5.386E-05	1.002E-04
2.210E 00	6.026E-02	6.581E-03	7.760E 00	5.225E-05	8.159E-05
2.270E 00	4.791E-02	6.787E-03	7.890E 00	4.600E-05	7.843E-05
2.330E 00	4.288E-02	6.081E-03	8.030E 00	1.824E-05	7.661E-05
2.390E 00	3.862E-02	6.210E-03	8.170E 00	8.342E-06	6.315E-05
2.450E 00	4.845E-02	6.297E-03	8.310E 00	3.116E-05	6.211E-05
2.510E 00	7.374E-02	6.109E-03	8.450E 00	5.602E-05	5.988E-05
2.570E 00	8.428E-02	5.933E-03	8.590E 00	4.761E-05	5.367E-05
2.630E 00	6.947E-02	5.368E-03	8.730E 00	3.120E-05	5.228E-05
2.690E 00	5.240E-02	4.806E-03	8.870E 00	1.980E-05	4.875E-05
2.750E 00	4.118E-02	4.578E-03	9.010E 00	1.044E-05	4.749E-05
2.810E 00	3.352E-02	4.757E-03	9.150E 00	-4.549E-06	-4.578E-05
2.870E 00	3.136E-02	4.576E-03	9.300E 00	-8.211E-06	-4.431E-05
2.930E 00	3.318E-02	4.290E-03	9.460E 00	1.342E-05	4.254E-05
2.990E 00	3.566E-02	4.268E-03	9.620E 00	4.449E-05	4.081E-05
3.050E 00	3.815E-02	3.943E-03	9.780E 00	6.652E-05	3.651E-05
3.110E 00	4.180E-02	3.844E-03	9.940E 00	6.219E-05	3.357E-05
3.180E 00	4.692E-02	3.610E-03	1.010E 01	4.228E-05	2.502E-05
3.260E 00	4.596E-02	3.470E-03	1.026E 01	2.556E-05	1.679E-05
3.340E 00	4.044E-02	3.685E-03	1.042E 01	9.911E-06	1.422E-05
3.420E 00	4.140E-02	3.262E-03	1.058E 01	3.216E-06	1.213E-05
3.500E 00	4.550E-02	3.193E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.513E-02	1.837E-03
8.000E-01 - 1.000E 00	1.137E-01	3.600E-03
1.000E 00 - 1.200E 00	2.172E-02	2.698E-03
1.200E 00 - 1.400E 00	4.348E-02	2.591E-03
1.400E 00 - 1.600E 00	8.728E-03	2.036E-03
1.600E 00 - 1.800E 00	1.421E-02	2.085E-03
1.800E 00 - 2.000E 00	1.654E-02	1.955E-03
2.000E 00 - 2.500E 00	3.112E-02	3.500E-03
2.500E 00 - 3.000E 00	2.519E-02	2.473E-03
3.000E 00 - 3.500E 00	2.127E-02	1.801E-03
3.500E 00 - 4.000E 00	1.560E-02	1.208E-03
4.000E 00 - 4.500E 00	4.866E-03	9.271E-04
4.500E 00 - 5.000E 00	4.208E-03	8.333E-04
5.000E 00 - 6.000E 00	5.938E-03	1.066E-03
6.000E 00 - 7.000E 00	1.367E-03	4.071E-04
7.000E 00 - 8.000E 00	1.051E-04	1.230E-04
8.000E 00 - 9.000E 00	2.946E-05	5.791E-05
9.000E 00 - 1.000E 01	2.736E-05	1.251E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 7.51 TO 8.02 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	2.202E-02	1.956E-02	3.580E 00	4.671E-02	3.423E-03
7.100E-01	2.708E-02	1.868E-02	3.660E 00	4.344E-02	2.952E-03
7.300E-01	4.947E-02	1.877E-02	3.740E 00	3.284E-02	2.808E-03
7.500E-01	8.187E-02	1.881E-02	3.820E 00	2.265E-02	2.775E-03
7.700E-01	1.496E-01	1.954E-02	3.900E 00	1.878E-02	2.576E-03
7.900E-01	3.448E-01	2.004E-02	3.980E 00	1.712E-02	2.405E-03
8.100E-01	6.848E-01	2.112E-02	4.060E 00	1.557E-02	2.315E-03
8.300E-01	1.013E 00	2.273E-02	4.140E 00	1.309E-02	2.241E-03
8.500E-01	1.130E 00	2.260E-02	4.220E 00	1.048E-02	2.198E-03
8.700E-01	9.885E-01	2.215E-02	4.300E 00	1.088E-02	2.166E-03
8.900E-01	7.117E-01	1.925E-02	4.380E 00	1.392E-02	2.209E-03
9.100E-01	4.401E-01	1.725E-02	4.460E 00	1.367E-02	2.147E-03
9.300E-01	2.425E-01	1.467E-02	4.540E 00	9.655E-03	2.174E-03
9.500E-01	1.258E-01	1.672E-02	4.620E 00	8.079E-03	2.020E-03
9.700E-01	7.650E-02	1.576E-02	4.700E 00	9.451E-03	2.009E-03
1.000E 00	7.392E-02	1.719E-02	4.830E 00	7.885E-03	1.917E-03
1.040E 00	9.221E-02	1.580E-02	4.930E 00	7.234E-03	1.863E-03
1.080E 00	8.337E-02	1.411E-02	5.030E 00	9.450E-03	1.804E-03
1.120E 00	6.396E-02	1.304E-02	5.130E 00	1.005E-02	1.738E-03
1.160E 00	1.247E-01	1.367E-02	5.230E 00	1.039E-02	1.733E-03
1.200E 00	2.637E-01	1.399E-02	5.330E 00	1.005E-02	1.638E-03
1.240E 00	3.418E-01	1.388E-02	5.430E 00	7.879E-03	1.507E-03
1.280E 00	2.863E-01	1.403E-02	5.530E 00	6.074E-03	1.496E-03
1.320E 00	1.757E-01	1.207E-02	5.630E 00	4.616E-03	1.376E-03
1.360E 00	1.081E-01	1.215E-02	5.730E 00	3.041E-03	1.286E-03
1.400E 00	7.846E-02	1.222E-02	5.830E 00	2.990E-03	1.251E-03
1.440E 00	5.003E-02	1.193E-02	5.930E 00	4.448E-03	1.122E-03
1.480E 00	3.533E-02	1.073E-02	6.030E 00	6.082E-03	9.943E-04
1.520E 00	3.306E-02	1.035E-02	6.130E 00	7.155E-03	9.312E-04
1.560E 00	3.428E-02	1.067E-02	6.230E 00	7.100E-03	8.263E-04
1.600E 00	4.157E-02	9.700E-03	6.330E 00	5.946E-03	7.806E-04
1.640E 00	5.412E-02	1.015E-02	6.440E 00	4.097E-03	7.417E-04
1.680E 00	6.577E-02	9.546E-03	6.560E 00	2.145E-03	6.324E-04
1.720E 00	7.973E-02	1.122E-02	6.680E 00	9.085E-04	5.686E-04
1.760E 00	1.015E-01	1.110E-02	6.800E 00	6.534E-04	4.493E-04
1.800E 00	1.178E-01	1.212E-02	6.920E 00	1.060E-03	4.406E-04
1.850E 00	1.043E-01	1.069E-02	7.040E 00	1.455E-03	3.344E-04
1.910E 00	6.987E-02	8.905E-03	7.160E 00	1.385E-03	2.771E-04
1.970E 00	6.796E-02	8.685E-03	7.280E 00	1.016E-03	2.359E-04
2.030E 00	7.571E-02	9.302E-03	7.400E 00	6.582E-04	1.959E-04
2.090E 00	8.245E-02	9.009E-03	7.520E 00	3.703E-04	1.529E-04
2.150E 00	8.003E-02	7.915E-03	7.640E 00	1.781E-04	1.342E-04
2.210E 00	5.948E-02	6.895E-03	7.760E 00	7.075E-05	1.196E-04
2.270E 00	4.375E-02	6.853E-03	7.890E 00	2.315E-05	1.069E-04
2.330E 00	4.637E-02	6.402E-03	8.030E 00	-3.374E-06	-4.118E-05
2.390E 00	4.424E-02	6.622E-03	8.170E 00	-2.294E-05	-7.296E-05
2.450E 00	4.554E-02	6.823E-03	8.310E 00	-1.479E-05	-6.326E-05
2.510E 00	7.003E-02	6.594E-03	8.450E 00	9.982E-06	6.087E-05
2.570E 00	8.746E-02	6.141E-03	8.590E 00	2.527E-05	5.662E-05
2.630E 00	7.375E-02	5.812E-03	8.730E 00	4.710E-05	4.922E-05
2.690E 00	5.383E-02	5.450E-03	8.870E 00	7.616E-05	4.850E-05
2.750E 00	4.322E-02	5.038E-03	9.010E 00	9.265E-05	4.667E-05
2.810E 00	3.693E-02	4.815E-03	9.150E 00	7.451E-05	4.041E-05
2.870E 00	3.636E-02	4.806E-03	9.300E 00	4.195E-05	3.127E-05
2.930E 00	3.889E-02	4.635E-03	9.460E 00	2.160E-05	2.569E-05
2.990E 00	3.991E-02	4.684E-03	9.620E 00	9.810E-06	2.344E-05
3.050E 00	4.191E-02	4.578E-03	9.780E 00	7.761E-06	1.956E-05
3.110E 00	4.630E-02	4.572E-03	9.940E 00	8.080E-06	1.815E-05
3.180E 00	4.615E-02	4.299E-03	1.010E 01	1.051E-05	1.607E-05
3.260E 00	4.347E-02	4.107E-03	1.026E 01	9.876E-06	1.267E-05
3.340E 00	4.882E-02	3.866E-03	1.042E 01	5.430E-06	1.239E-05
3.420E 00	4.884E-02	3.681E-03	1.058E 01	4.223E-06	1.100E-05
3.500E 00	4.723E-02	3.687E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.336E-02	1.920E-03
8.000E-01 - 1.000E 00	1.095E-01	3.776E-03
1.000E 00 - 1.200E 00	2.081E-02	2.882E-03
1.200E 00 - 1.400E 00	4.371E-02	2.612E-03
1.400E 00 - 1.600E 00	8.359E-03	2.189E-03
1.600E 00 - 1.800E 00	1.523E-02	2.109E-03
1.800E 00 - 2.000E 00	1.672E-02	1.933E-03
2.000E 00 - 2.500E 00	2.995E-02	3.720E-03
2.500E 00 - 3.000E 00	2.677E-02	2.650E-03
3.000E 00 - 3.500E 00	2.300E-02	2.067E-03
3.500E 00 - 4.000E 00	1.609E-02	1.452E-03
4.000E 00 - 4.500E 00	6.529E-03	1.110E-03
4.500E 00 - 5.000E 00	4.193E-03	9.901E-04
5.000E 00 - 6.000E 00	6.818E-03	1.479E-03
6.000E 00 - 7.000E 00	3.606E-03	6.796E-04
7.000E 00 - 8.000E 00	5.892E-04	1.801E-04
8.000E 00 - 9.000E 00	2.159E-05	5.323E-06
9.000E 00 - 1.000E 01	3.189E-05	2.787E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 8.02 TO 8.52 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	2.206E-02	1.985E-02	3.580E 00	4.587E-02	3.901E-03
7.100E-01	2.452E-02	2.056E-02	3.660E 00	4.068E-02	3.508E-03
7.300E-01	3.735E-02	2.117E-02	3.740E 00	3.391E-02	3.541E-03
7.500E-01	7.070E-02	2.048E-02	3.820E 00	3.155E-02	3.038E-03
7.700E-01	1.590E-01	2.036E-02	3.900E 00	2.776E-02	3.007E-03
7.900E-01	3.671E-01	2.038E-02	3.980E 00	2.222E-02	2.809E-03
8.100E-01	6.938E-01	2.206E-02	4.060E 00	1.827E-02	2.888E-03
8.300E-01	1.003E 00	2.342E-02	4.140E 00	1.556E-02	2.743E-03
8.500E-01	1.112E 00	2.353E-02	4.220E 00	1.451E-02	2.669E-03
8.700E-01	9.699E-01	2.372E-02	4.300E 00	1.444E-02	2.698E-03
8.900E-01	6.896E-01	2.114E-02	4.380E 00	1.231E-02	2.591E-03
9.100E-01	4.160E-01	1.962E-02	4.460E 00	9.453E-03	2.558E-03
9.300E-01	2.248E-01	1.668E-02	4.540E 00	9.461E-03	2.660E-03
9.500E-01	1.177E-01	1.769E-02	4.630E 00	1.028E-02	2.428E-03
9.700E-01	7.268E-02	1.645E-02	4.730E 00	9.566E-03	2.423E-03
1.000E 00	7.206E-02	1.811E-02	4.830E 00	7.777E-03	2.252E-03
1.040E 00	9.867E-02	1.715E-02	4.930E 00	6.706E-03	2.188E-03
1.080E 00	7.930E-02	1.520E-02	5.030E 00	7.782E-03	2.299E-03
1.120E 00	6.136E-02	1.372E-02	5.130E 00	8.113E-03	2.160E-03
1.160E 00	1.329E-01	1.508E-02	5.230E 00	8.843E-03	2.068E-03
1.200E 00	2.759E-01	1.525E-02	5.330E 00	1.037E-02	2.037E-03
1.240E 00	3.637E-01	1.577E-02	5.430E 00	9.780E-03	2.023E-03
1.280E 00	2.948E-01	1.569E-02	5.530E 00	8.868E-03	2.038E-03
1.320E 00	1.741E-01	1.383E-02	5.630E 00	8.198E-03	1.790E-03
1.360E 00	1.022E-01	1.332E-02	5.730E 00	6.470E-03	1.719E-03
1.400E 00	7.519E-02	1.296E-02	5.830E 00	5.288E-03	1.685E-03
1.440E 00	6.132E-02	1.210E-02	5.930E 00	5.594E-03	1.640E-03
1.480E 00	4.614E-02	1.099E-02	6.030E 00	6.478E-03	1.598E-03
1.520E 00	3.900E-02	1.068E-02	6.130E 00	7.261E-03	1.378E-03
1.560E 00	4.023E-02	1.042E-02	6.230E 00	7.859E-03	1.308E-03
1.600E 00	4.012E-02	1.052E-02	6.330E 00	7.615E-03	1.206E-03
1.640E 00	4.449E-02	1.184E-02	6.430E 00	6.543E-03	1.177E-03
1.680E 00	5.921E-02	1.117E-02	6.560E 00	5.664E-03	1.058E-03
1.720E 00	8.208E-02	1.188E-02	6.680E 00	4.990E-03	8.991E-04
1.760E 00	1.043E-01	1.197E-02	6.800E 00	4.497E-03	7.796E-04
1.800E 00	1.131E-01	1.240E-02	6.920E 00	4.089E-03	7.158E-04
1.850E 00	9.965E-02	1.099E-02	7.040E 00	3.166E-03	6.294E-04
1.910E 00	7.331E-02	9.646E-03	7.160E 00	2.132E-03	4.838E-04
1.970E 00	6.325E-02	9.188E-03	7.280E 00	1.674E-03	4.011E-04
2.030E 00	7.309E-02	9.256E-03	7.400E 00	1.428E-03	3.573E-04
2.090E 00	8.583E-02	8.849E-03	7.520E 00	9.535E-04	2.909E-04
2.150E 00	7.057E-02	8.480E-03	7.640E 00	4.408E-04	2.151E-04
2.210E 00	5.081E-02	7.750E-03	7.760E 00	1.848E-04	1.899E-04
2.270E 00	3.450E-02	7.548E-03	7.880E 00	1.122E-04	1.648E-04
2.330E 00	5.500E-02	6.850E-03	8.000E 00	5.849E-05	1.162E-04
2.390E 00	4.674E-02	7.243E-03	8.170E 00	7.314E-05	1.105E-04
2.450E 00	5.585E-02	7.467E-03	8.310E 00	1.110E-04	1.065E-04
2.510E 00	7.655E-02	7.053E-03	8.450E 00	1.019E-04	9.630E-05
2.570E 00	8.625E-02	6.781E-03	8.590E 00	5.867E-05	8.388E-05
2.630E 00	7.714E-02	6.435E-03	8.730E 00	2.178E-05	8.599E-05
2.690E 00	5.693E-02	5.958E-03	8.870E 00	1.531E-07	8.569E-05
2.750E 00	4.181E-02	5.624E-03	9.010E 00	1.328E-06	7.141E-05
2.810E 00	3.810E-02	5.546E-03	9.150E 00	2.034E-05	6.480E-05
2.870E 00	3.724E-02	5.373E-03	9.300E 00	4.560E-05	6.679E-05
2.930E 00	3.970E-02	5.229E-03	9.440E 00	6.182E-05	5.837E-05
2.990E 00	4.494E-02	5.100E-03	9.620E 00	7.121E-05	4.855E-05
3.050E 00	4.649E-02	4.991E-03	9.780E 00	7.412E-05	4.657E-05
3.110E 00	4.719E-02	5.103E-03	9.940E 00	5.772E-05	3.865E-05
3.180E 00	5.057E-02	4.816E-03	1.010E 01	3.300E-05	2.743E-05
3.260E 00	5.148E-02	4.618E-03	1.026E 01	1.588E-05	1.842E-05
3.340E 00	5.206E-02	4.372E-03	1.042E 01	4.607E-06	1.584E-05
3.420E 00	5.004E-02	4.108E-03	1.058E 01	8.821E-07	1.539E-05
3.500E 00	4.628E-02	3.968E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.303E-02	2.062E-03
8.000E-01 - 1.000E 00	1.072E-01	4.029E-03
1.000E 00 - 1.200E 00	2.134E-02	3.111E-03
1.200E 00 - 1.400E 00	4.481E-02	2.914E-03
1.400E 00 - 1.600E 00	9.717E-03	2.232E-03
1.600E 00 - 1.800E 00	4.666E-02	2.333E-03
1.800E 00 - 2.000E 00	1.640E-02	2.034E-03
2.000E 00 - 2.500E 00	3.099E-02	3.949E-03
2.500E 00 - 3.000E 00	2.754E-02	2.942E-03
3.000E 00 - 3.500E 00	2.473E-02	2.301E-03
3.500E 00 - 4.000E 00	1.760E-02	1.687E-03
4.000E 00 - 4.500E 00	1.199E-03	1.348E-03
4.500E 00 - 5.000E 00	4.347E-03	1.186E-03
5.000E 00 - 6.000E 00	7.894E-03	1.931E-03
6.000E 00 - 7.000E 00	5.957E-03	1.076E-03
7.000E 00 - 8.000E 00	1.145E-03	3.226E-04
8.000E 00 - 9.000E 00	5.724E-05	9.574E-05
9.000E 00 - 1.000E 01	5.172E-05	5.533E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 8.52 TO 9.01 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	3.295E-02	2.302E-02	3.580E 00	5.775E-02	4.456E-03
7.100E-01	2.662E-02	2.324E-02	3.660E 00	4.514E-02	4.298E-03
7.300E-01	3.405E-02	2.413E-02	3.740E 00	3.294E-02	4.123E-03
7.500E-01	6.766E-02	2.371E-02	3.820E 00	2.882E-02	3.786E-03
7.700E-01	1.667E-01	2.433E-02	3.900E 00	2.606E-02	3.580E-03
7.900E-01	3.993E-01	2.358E-02	3.980E 00	2.249E-02	3.597E-03
8.100E-01	7.431E-01	2.520E-02	4.060E 00	2.031E-02	3.547E-03
8.300E-01	1.041E 00	2.721E-02	4.140E 00	1.957E-02	3.284E-03
8.500E-01	1.125E 00	2.539E-02	4.220E 00	2.010E-02	3.218E-03
8.700E-01	9.655E-01	2.522E-02	4.300E 00	2.231E-02	3.278E-03
8.900E-01	6.765E-01	2.195E-02	4.380E 00	2.227E-02	3.156E-03
9.100E-01	3.972E-01	2.170E-02	4.460E 00	1.539E-02	3.058E-03
9.300E-01	2.034E-01	1.942E-02	4.540E 00	9.164E-03	3.084E-03
9.500E-01	9.891E-02	2.202E-02	4.630E 00	9.391E-03	3.011E-03
9.700E-01	6.395E-02	1.949E-02	4.730E 00	9.683E-03	3.001E-03
1.000E 00	8.503E-02	2.067E-02	4.830E 00	8.896E-03	2.771E-03
1.040E 00	1.142E-01	1.778E-02	4.930E 00	1.167E-02	2.875E-03
1.080E 00	8.468E-02	1.723E-02	5.030E 00	1.359E-02	2.773E-03
1.120E 00	8.069E-02	1.572E-02	5.130E 00	1.338E-02	2.741E-03
1.160E 00	1.458E-01	1.560E-02	5.230E 00	1.280E-02	2.668E-03
1.200E 00	2.950E-01	1.842E-02	5.330E 00	1.150E-02	2.564E-03
1.240E 00	3.858E-01	1.726E-02	5.430E 00	8.411E-03	2.569E-03
1.280E 00	3.103E-01	1.703E-02	5.530E 00	4.798E-03	2.497E-03
1.320E 00	1.830E-01	1.509E-02	5.630E 00	5.017E-03	2.476E-03
1.360E 00	1.047E-01	1.530E-02	5.730E 00	8.231E-03	2.235E-03
1.400E 00	7.221E-02	1.516E-02	5.830E 00	1.133E-02	2.181E-03
1.440E 00	5.722E-02	1.414E-02	5.930E 00	1.231E-02	2.378E-03
1.480E 00	5.014E-02	1.240E-02	6.030E 00	1.072E-02	2.119E-03
1.520E 00	4.309E-02	1.190E-02	6.130E 00	8.199E-03	1.932E-03
1.560E 00	3.603E-02	1.163E-02	6.230E 00	6.250E-03	1.792E-03
1.600E 00	3.985E-02	1.145E-02	6.330E 00	4.873E-03	1.711E-03
1.640E 00	5.057E-02	1.189E-02	6.440E 00	5.296E-03	1.790E-03
1.680E 00	6.732E-02	1.208E-02	6.560E 00	7.565E-03	1.507E-03
1.720E 00	8.838E-02	1.337E-02	6.680E 00	8.954E-03	1.391E-03
1.760E 00	1.017E-01	1.335E-02	6.800E 00	8.777E-03	1.269E-03
1.800E 00	1.104E-01	1.420E-02	6.920E 00	7.977E-03	1.158E-03
1.850E 00	1.125E-01	1.112E-02	7.040E 00	6.487E-03	1.042E-03
1.910E 00	7.071E-02	1.103E-02	7.160E 00	4.629E-03	8.907E-04
1.970E 00	4.598E-02	1.162E-02	7.280E 00	3.170E-03	7.933E-04
2.030E 00	7.550E-02	1.117E-02	7.400E 00	2.451E-03	6.890E-04
2.090E 00	8.841E-02	9.720E-03	7.520E 00	2.176E-03	6.013E-04
2.150E 00	7.472E-02	9.714E-03	7.640E 00	1.889E-03	5.132E-04
2.210E 00	6.329E-02	9.458E-03	7.760E 00	1.547E-03	4.308E-04
2.270E 00	4.938E-02	8.937E-03	7.890E 00	1.197E-03	3.910E-04
2.330E 00	5.248E-02	8.010E-03	8.030E 00	8.252E-04	3.201E-04
2.390E 00	5.410E-02	7.673E-03	8.170E 00	5.506E-04	2.521E-04
2.450E 00	5.404E-02	8.418E-03	8.310E 00	3.854E-04	1.924E-04
2.510E 00	6.544E-02	8.911E-03	8.450E 00	2.771E-04	1.927E-04
2.570E 00	7.343E-02	7.511E-03	8.590E 00	2.182E-04	1.474E-04
2.630E 00	6.508E-02	7.472E-03	8.730E 00	1.665E-04	9.327E-05
2.690E 00	5.716E-02	6.836E-03	8.870E 00	1.121E-04	8.501E-05
2.750E 00	5.117E-02	6.258E-03	9.010E 00	3.913E-05	6.796E-05
2.810E 00	4.435E-02	6.114E-03	9.150E 00	-1.535E-05	-4.752E-05
2.870E 00	4.208E-02	5.973E-03	9.300E 00	-2.747E-05	-1.880E-05
2.930E 00	4.617E-02	5.890E-03	9.460E 00	-1.110E-05	-3.356E-05
2.990E 00	5.079E-02	6.099E-03	9.620E 00	1.559E-05	2.902E-05
3.050E 00	4.907E-02	6.306E-03	9.780E 00	4.134E-05	2.874E-05
3.110E 00	4.824E-02	6.702E-03	9.940E 00	5.717E-05	3.062E-05
3.180E 00	5.130E-02	5.527E-03	1.010E 01	5.659E-05	2.977E-05
3.260E 00	4.838E-02	5.167E-03	1.026E 01	4.468E-05	2.333E-05
3.340E 00	5.051E-02	4.704E-03	1.042E 01	2.345E-05	2.058E-05
3.420E 00	5.720E-02	4.874E-03	1.058E 01	7.787E-06	2.032E-05
3.500E 00	6.076E-02	4.625E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.415E-02	2.386E-03
8.000E-01 - 1.000E 00	1.076E-01	4.549E-03
1.000E 00 - 1.200E 00	2.403E-02	3.413E-03
1.200E 00 - 1.400E 00	4.712E-02	3.259E-03
1.400E 00 - 1.600E 00	9.620E-03	2.531E-03
1.600E 00 - 1.800E 00	1.533E-02	2.536E-03
1.800E 00 - 2.000E 00	1.600E-02	2.307E-03
2.000E 00 - 2.500E 00	3.217E-02	4.554E-03
2.500E 00 - 3.000E 00	2.747E-02	3.352E-03
3.000E 00 - 3.500E 00	2.584E-02	2.721E-03
3.500E 00 - 4.000E 00	1.904E-02	2.018E-03
4.000E 00 - 4.500E 00	1.004E-02	1.634E-03
4.500E 00 - 5.000E 00	4.988E-03	1.466E-03
5.000E 00 - 6.000E 00	1.009E-02	2.495E-03
6.000E 00 - 7.000E 00	7.539E-03	1.587E-03
7.000E 00 - 8.000E 00	2.747E-03	6.414E-04
8.000E 00 - 9.000E 00	3.223E-04	1.702E-04
9.000E 00 - 1.000E 01	1.099E-05	1.453E-06

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 9.01 TO 9.48 MEV. ANGLE = 125. DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.717E-03	2.320E-02	3.580E 00	4.477E-02	4.910E-03
7.100E-01	2.424E-02	2.170E-02	3.660E 00	3.763E-02	4.657E-03
7.300E-01	4.841E-02	2.126E-02	3.740E 00	3.638E-02	4.492E-03
7.500E-01	1.040E-01	2.285E-02	3.820E 00	3.438E-02	4.107E-03
7.700E-01	2.170E-01	2.392E-02	3.900E 00	3.026E-02	3.786E-03
7.900E-01	4.230E-01	2.388E-02	3.980E 00	2.755E-02	3.721E-03
8.100E-01	7.256E-01	2.530E-02	4.060E 00	2.276E-02	3.589E-03
8.300E-01	1.002E 00	2.637E-02	4.140E 00	1.896E-02	3.640E-03
8.500E-01	1.062E 00	2.625E-02	4.220E 00	2.044E-02	3.574E-03
8.700E-01	8.688E-01	2.623E-02	4.300E 00	2.201E-02	3.624E-03
8.900E-01	5.721E-01	2.310E-02	4.380E 00	1.743E-02	3.584E-03
9.100E-01	3.262E-01	2.202E-02	4.460E 00	1.037E-02	3.518E-03
9.300E-01	1.773E-01	1.936E-02	4.540E 00	9.530E-03	3.460E-03
9.500E-01	1.005E-01	1.950E-02	4.630E 00	1.164E-02	3.427E-03
9.700E-01	6.960E-02	1.880E-02	4.730E 00	9.245E-03	3.406E-03
1.000E 00	7.323E-02	2.067E-02	4.830E 00	7.876E-03	3.321E-03
1.040E 00	9.859E-02	1.946E-02	4.930E 00	9.658E-03	3.173E-03
1.080E 00	8.493E-02	1.743E-02	5.030E 00	1.121E-02	3.105E-03
1.120E 00	6.756E-02	1.616E-02	5.130E 00	1.325E-02	3.146E-03
1.160E 00	1.504E-01	1.742E-02	5.230E 00	1.596E-02	3.062E-03
1.200E 00	3.021E-01	1.725E-02	5.330E 00	1.621E-02	2.969E-03
1.240E 00	3.683E-01	1.743E-02	5.430E 00	1.400E-02	2.886E-03
1.280E 00	3.006E-01	1.629E-02	5.530E 00	1.134E-02	2.778E-03
1.320E 00	1.738E-01	1.454E-02	5.630E 00	7.111E-03	2.758E-03
1.360E 00	1.038E-01	1.412E-02	5.730E 00	2.300E-03	2.706E-03
1.400E 00	8.261E-02	1.481E-02	5.830E 00	1.378E-03	2.652E-03
1.440E 00	6.079E-02	1.388E-02	5.930E 00	4.662E-03	2.555E-03
1.480E 00	3.473E-02	1.391E-02	6.030E 00	8.851E-03	2.374E-03
1.520E 00	2.214E-02	1.330E-02	6.130E 00	1.266E-02	2.387E-03
1.560E 00	2.547E-02	1.264E-02	6.230E 00	1.463E-02	2.222E-03
1.600E 00	3.706E-02	1.237E-02	6.330E 00	1.293E-02	2.133E-03
1.640E 00	5.290E-02	1.187E-02	6.440E 00	8.840E-03	2.097E-03
1.680E 00	6.480E-02	1.307E-02	6.560E 00	6.192E-03	1.926E-03
1.720E 00	7.561E-02	1.421E-02	6.680E 00	5.480E-03	1.862E-03
1.760E 00	9.905E-02	1.407E-02	6.800E 00	5.140E-03	1.702E-03
1.800E 00	1.152E-01	1.391E-02	6.920E 00	6.009E-03	1.574E-03
1.850E 00	9.991E-02	1.290E-02	7.040E 00	7.352E-03	1.424E-03
1.910E 00	7.159E-02	1.044E-02	7.160E 00	7.236E-03	1.302E-03
1.970E 00	5.957E-02	1.106E-02	7.280E 00	5.975E-03	1.210E-03
2.030E 00	6.799E-02	1.222E-02	7.400E 00	4.699E-03	1.052E-03
2.090E 00	7.254E-02	1.101E-02	7.520E 00	3.544E-03	9.075E-04
2.150E 00	6.458E-02	9.636E-03	7.640E 00	2.418E-03	8.376E-04
2.210E 00	5.053E-02	9.025E-03	7.760E 00	1.788E-03	7.458E-04
2.270E 00	4.468E-02	8.829E-03	7.890E 00	1.863E-03	6.748E-04
2.330E 00	4.537E-02	8.324E-03	8.030E 00	2.078E-03	6.690E-04
2.390E 00	4.558E-02	7.868E-03	8.170E 00	1.747E-03	4.565E-04
2.450E 00	5.858E-02	8.194E-03	8.310E 00	1.234E-03	3.704E-04
2.510E 00	7.663E-02	8.466E-03	8.450E 00	8.366E-04	3.210E-04
2.570E 00	7.695E-02	8.446E-03	8.590E 00	4.370E-04	2.599E-04
2.630E 00	6.562E-02	7.526E-03	8.730E 00	1.034E-04	1.642E-04
2.690E 00	5.406E-02	6.891E-03	8.870E 00	3.240E-05	1.185E-04
2.750E 00	4.432E-02	6.771E-03	9.010E 00	8.960E-05	1.146E-04
2.810E 00	3.974E-02	6.989E-03	9.150E 00	1.562E-04	9.128E-05
2.870E 00	3.780E-02	6.898E-03	9.300E 00	1.356E-04	7.174E-05
2.930E 00	3.740E-02	6.428E-03	9.460E 00	9.412E-05	5.921E-05
2.990E 00	4.014E-02	6.022E-03	9.620E 00	6.045E-05	4.550E-05
3.050E 00	4.696E-02	6.039E-03	9.780E 00	4.005E-05	3.759E-05
3.110E 00	5.236E-02	6.259E-03	9.940E 00	2.789E-05	3.137E-05
3.180E 00	5.059E-02	6.192E-03	1.010E 01	1.698E-05	2.779E-05
3.260E 00	4.897E-02	5.677E-03	1.026E 01	1.265E-05	1.916E-05
3.340E 00	5.025E-02	5.392E-03	1.042E 01	7.975E-06	1.735E-05
3.420E 00	4.997E-02	5.072E-03	1.058E 01	6.474E-06	1.975E-05
3.500E 00	5.031E-02	4.956E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.655E-02	2.276E-03
8.000E-01 - 1.000E 00	9.932E-02	4.533E-03
1.000E 00 - 1.200E 00	2.304E-02	3.575E-03
1.200E 00 - 1.400E 00	4.586E-02	3.134E-03
1.400E 00 - 1.600E 00	7.971E-03	2.691E-03
1.600E 00 - 1.800E 00	1.475E-02	2.653E-03
1.800E 00 - 2.000E 00	1.613E-02	2.328E-03
2.000E 00 - 2.500E 00	2.842E-02	4.664E-03
2.500E 00 - 3.000E 00	2.603E-02	3.580E-03
3.000E 00 - 3.500E 00	2.481E-02	2.844E-03
3.500E 00 - 4.000E 00	1.829E-02	2.176E-03
4.000E 00 - 4.500E 00	9.499E-03	1.797E-03
4.500E 00 - 5.000E 00	4.830E-03	1.673E-03
5.000E 00 - 6.000E 00	9.670E-03	2.849E-03
6.000E 00 - 7.000E 00	8.698E-03	1.993E-03
7.000E 00 - 8.000E 00	4.151E-03	9.867E-04
8.000E 00 - 9.000E 00	8.230E-04	2.991E-04
9.000E 00 - 1.000E 01	8.708E-05	6.034E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 9.48 TO 9.98 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.758E-02	2.511E-02	3.580E 00	5.073E-02	5.521E-03
7.100E-01	4.815E-02	2.321E-02	3.660E 00	4.461E-02	5.300E-03
7.300E-01	6.180E-02	2.483E-02	3.740E 00	4.055E-02	4.800E-03
7.500E-01	8.240E-02	2.597E-02	3.820E 00	3.773E-02	4.685E-03
7.700E-01	2.004E-01	2.706E-02	3.900E 00	2.979E-02	4.626E-03
7.900E-01	4.593E-01	2.663E-02	3.980E 00	2.500E-02	4.774E-03
8.100E-01	7.823E-01	2.803E-02	4.060E 00	2.478E-02	4.516E-03
8.300E-01	1.023E 00	2.955E-02	4.140E 00	2.096E-02	4.560E-03
8.500E-01	1.054E 00	2.832E-02	4.220E 00	1.722E-02	4.318E-03
8.700E-01	8.660E-01	2.862E-02	4.300E 00	1.820E-02	4.646E-03
8.900E-01	5.799E-01	2.371E-02	4.380E 00	1.683E-02	4.421E-03
9.100E-01	3.319E-01	2.212E-02	4.460E 00	1.353E-02	4.177E-03
9.300E-01	1.800E-01	1.960E-02	4.540E 00	1.455E-02	4.279E-03
9.500E-01	1.069E-01	2.111E-02	4.620E 00	1.636E-02	4.129E-03
9.700E-01	8.280E-02	1.921E-02	4.700E 00	1.396E-02	3.757E-03
1.000E 00	8.670E-02	2.088E-02	4.800E 00	1.251E-02	3.713E-03
1.040E 00	9.681E-02	1.976E-02	4.900E 00	1.620E-02	3.657E-03
1.080E 00	8.582E-02	1.935E-02	5.000E 00	1.707E-02	3.635E-03
1.120E 00	8.324E-02	1.800E-02	5.100E 00	1.268E-02	3.638E-03
1.160E 00	1.603E-01	1.869E-02	5.200E 00	1.173E-02	3.254E-03
1.200E 00	2.988E-01	1.973E-02	5.300E 00	1.282E-02	3.330E-03
1.240E 00	3.783E-01	1.911E-02	5.400E 00	1.160E-02	3.301E-03
1.280E 00	3.125E-01	1.762E-02	5.500E 00	1.005E-02	3.314E-03
1.320E 00	1.794E-01	1.626E-02	5.600E 00	1.028E-02	2.934E-03
1.360E 00	1.059E-01	1.534E-02	5.700E 00	1.070E-02	2.964E-03
1.400E 00	8.883E-02	1.595E-02	5.800E 00	1.030E-02	2.951E-03
1.440E 00	6.702E-02	1.397E-02	5.900E 00	1.049E-02	2.977E-03
1.480E 00	3.812E-02	1.510E-02	6.000E 00	1.117E-02	3.049E-03
1.520E 00	2.670E-02	1.532E-02	6.100E 00	1.086E-02	2.728E-03
1.560E 00	2.917E-02	1.551E-02	6.200E 00	1.037E-02	2.593E-03
1.600E 00	3.025E-02	1.410E-02	6.300E 00	1.068E-02	2.593E-03
1.640E 00	3.924E-02	1.408E-02	6.400E 00	1.031E-02	2.621E-03
1.680E 00	5.867E-02	1.356E-02	6.500E 00	7.773E-03	2.494E-03
1.720E 00	7.768E-02	1.397E-02	6.600E 00	5.169E-03	2.309E-03
1.760E 00	9.696E-02	1.376E-02	6.800E 00	4.781E-03	2.244E-03
1.800E 00	1.118E-01	1.345E-02	6.920E 00	7.072E-03	2.023E-03
1.850E 00	9.928E-02	1.270E-02	7.040E 00	9.648E-03	1.912E-03
1.910E 00	6.187E-02	1.217E-02	7.160E 00	9.659E-03	1.734E-03
1.970E 00	5.822E-02	1.237E-02	7.280E 00	7.710E-03	1.620E-03
2.030E 00	6.830E-02	1.116E-02	7.400E 00	5.754E-03	1.498E-03
2.090E 00	8.204E-02	1.054E-02	7.520E 00	3.793E-03	1.398E-03
2.150E 00	7.215E-02	9.686E-03	7.640E 00	1.993E-03	1.270E-03
2.210E 00	4.987E-02	1.008E-02	7.760E 00	1.799E-03	1.087E-03
2.270E 00	4.143E-02	9.635E-03	7.890E 00	3.195E-03	9.840E-04
2.330E 00	4.074E-02	8.589E-03	8.030E 00	4.153E-03	9.159E-04
2.390E 00	4.804E-02	8.644E-03	8.170E 00	3.647E-03	7.662E-04
2.450E 00	6.390E-02	8.805E-03	8.310E 00	2.603E-03	5.976E-04
2.510E 00	7.851E-02	8.595E-03	8.450E 00	1.754E-03	4.656E-04
2.570E 00	7.632E-02	8.932E-03	8.590E 00	1.022E-03	3.108E-04
2.630E 00	6.226E-02	8.306E-03	8.730E 00	5.759E-04	2.150E-04
2.690E 00	5.599E-02	7.891E-03	8.870E 00	3.829E-04	1.627E-04
2.750E 00	5.627E-02	7.633E-03	9.010E 00	2.772E-04	1.320E-04
2.810E 00	5.142E-02	7.339E-03	9.150E 00	1.615E-04	1.015E-04
2.870E 00	4.471E-02	7.077E-03	9.300E 00	4.849E-05	7.292E-05
2.930E 00	4.406E-02	6.657E-03	9.460E 00	-2.804E-06	-5.103E-05
2.990E 00	4.598E-02	7.414E-03	9.620E 00	5.161E-06	4.266E-05
3.050E 00	4.922E-02	7.193E-03	9.780E 00	2.407E-05	3.867E-05
3.110E 00	5.444E-02	6.697E-03	9.940E 00	3.817E-05	3.492E-05
3.180E 00	5.694E-02	6.586E-03	1.010E 01	4.384E-05	3.249E-05
3.260E 00	5.216E-02	6.176E-03	1.026E 01	4.324E-05	2.669E-05
3.340E 00	4.735E-02	6.105E-03	1.042E 01	3.248E-05	2.848E-05
3.420E 00	4.595E-02	5.809E-03	1.058E 01	2.042E-05	2.757E-05
3.500E 00	4.797E-02	5.736E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.720E-02	2.563E-03
8.000E-01 - 1.000E 00	1.016E-01	4.803E-03
1.000E 00 - 1.200E 00	2.421E-02	3.838E-03
1.200E 00 - 1.400E 00	4.714E-02	3.440E-03
1.400E 00 - 1.600E 00	8.719E-03	3.000E-03
1.600E 00 - 1.800E 00	1.373E-02	2.766E-03
1.800E 00 - 2.000E 00	1.504E-02	2.493E-03
2.000E 00 - 2.500E 00	2.950E-02	4.803E-03
2.500E 00 - 3.000E 00	2.844E-02	3.868E-03
3.000E 00 - 3.500E 00	2.524E-02	3.183E-03
3.500E 00 - 4.000E 00	1.972E-02	2.508E-03
4.000E 00 - 4.500E 00	9.441E-03	2.228E-03
4.500E 00 - 5.000E 00	7.386E-03	1.941E-03
5.000E 00 - 6.000E 00	1.160E-02	3.219E-03
6.000E 00 - 7.000E 00	8.466E-03	2.475E-03
7.000E 00 - 8.000E 00	5.241E-03	1.398E-03
8.000E 00 - 9.000E 00	1.826E-03	4.508E-04
9.000E 00 - 1.000E 01	6.092E-05	4.579E-05

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 10.00 TO 11.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	2.599E-02	2.350E-02	3.580E 00	4.811E-02	5.327E-03
7.100E-01	2.732E-02	2.427E-02	3.660E 00	4.296E-02	4.658E-03
7.300E-01	5.108E-02	2.515E-02	3.740E 00	3.689E-02	4.691E-03
7.500E-01	9.455E-02	2.415E-02	3.820E 00	3.314E-02	4.593E-03
7.700E-01	2.072E-01	2.460E-02	3.900E 00	2.797E-02	4.359E-03
7.900E-01	4.614E-01	2.400E-02	3.980E 00	2.427E-02	4.096E-03
8.100E-01	7.905E-01	2.502E-02	4.060E 00	2.239E-02	4.056E-03
8.300E-01	1.014E 00	2.666E-02	4.140E 00	2.107E-02	3.819E-03
8.500E-01	1.012E 00	2.538E-02	4.220E 00	2.154E-02	3.988E-03
8.700E-01	8.140E-01	2.518E-02	4.300E 00	2.222E-02	4.130E-03
8.900E-01	5.389E-01	2.202E-02	4.380E 00	2.239E-02	3.980E-03
9.100E-01	3.043E-01	2.080E-02	4.460E 00	2.146E-02	3.882E-03
9.300E-01	1.668E-01	1.868E-02	4.540E 00	1.916E-02	3.813E-03
9.500E-01	1.083E-01	2.007E-02	4.620E 00	1.752E-02	3.694E-03
9.700E-01	8.649E-02	1.852E-02	4.700E 00	1.702E-02	3.701E-03
1.000E 00	8.118E-02	2.031E-02	4.800E 00	1.495E-02	3.634E-03
1.040E 00	1.011E-01	1.862E-02	4.900E 00	1.326E-02	3.522E-03
1.080E 00	8.423E-02	1.684E-02	5.000E 00	1.425E-02	3.430E-03
1.120E 00	8.581E-02	1.620E-02	5.100E 00	1.409E-02	3.412E-03
1.160E 00	1.800E-01	1.783E-02	5.200E 00	1.202E-02	3.346E-03
1.200E 00	3.412E-01	1.870E-02	5.300E 00	1.314E-02	3.354E-03
1.240E 00	4.105E-01	1.805E-02	5.400E 00	1.660E-02	3.068E-03
1.280E 00	3.133E-01	1.695E-02	5.500E 00	1.480E-02	2.901E-03
1.320E 00	1.823E-01	1.630E-02	5.600E 00	1.139E-02	2.908E-03
1.360E 00	1.082E-01	1.524E-02	5.700E 00	1.109E-02	2.768E-03
1.400E 00	7.577E-02	1.482E-02	5.800E 00	1.108E-02	2.914E-03
1.440E 00	5.908E-02	1.285E-02	5.900E 00	1.188E-02	2.812E-03
1.480E 00	4.808E-02	1.210E-02	6.000E 00	1.479E-02	2.703E-03
1.520E 00	3.773E-02	1.165E-02	6.100E 00	1.564E-02	2.582E-03
1.560E 00	3.701E-02	1.303E-02	6.200E 00	1.319E-02	2.539E-03
1.600E 00	4.585E-02	1.320E-02	6.300E 00	1.108E-02	2.528E-03
1.640E 00	5.520E-02	1.302E-02	6.400E 00	9.724E-03	2.438E-03
1.680E 00	6.787E-02	1.320E-02	6.500E 00	7.743E-03	2.295E-03
1.720E 00	8.848E-02	1.405E-02	6.600E 00	7.186E-03	2.281E-03
1.760E 00	1.044E-01	1.352E-02	6.800E 00	8.103E-03	2.118E-03
1.800E 00	1.008E-01	1.368E-02	6.920E 00	8.723E-03	2.064E-03
1.850E 00	7.759E-02	1.323E-02	7.040E 00	9.273E-03	1.952E-03
1.910E 00	6.112E-02	1.186E-02	7.160E 00	9.415E-03	1.789E-03
1.970E 00	5.936E-02	1.095E-02	7.280E 00	8.954E-03	1.696E-03
2.030E 00	6.741E-02	1.085E-02	7.400E 00	8.283E-03	1.587E-03
2.090E 00	8.144E-02	9.590E-03	7.520E 00	6.883E-03	1.548E-03
2.150E 00	7.021E-02	9.156E-03	7.640E 00	5.155E-03	1.411E-03
2.210E 00	5.014E-02	8.478E-03	7.760E 00	4.554E-03	1.318E-03
2.270E 00	4.725E-02	8.633E-03	7.890E 00	5.268E-03	1.184E-03
2.330E 00	4.311E 00	7.979E-03	8.030E 00	5.669E-03	1.108E-03
2.390E 00	3.905E-02	9.040E-03	8.170E 00	5.024E-03	9.746E-04
2.450E 00	5.319E-02	8.668E-03	8.310E 00	4.288E-03	9.094E-04
2.510E 00	8.090E-02	8.369E-03	8.450E 00	3.558E-03	7.318E-04
2.570E 00	9.284E-02	7.405E-03	8.590E 00	2.702E-03	6.337E-04
2.630E 00	7.744E-02	7.241E-03	8.730E 00	2.054E-03	5.710E-04
2.690E 00	5.773E-02	6.615E-03	8.870E 00	1.681E-03	4.519E-04
2.750E 00	5.120E-02	6.353E-03	9.010E 00	1.417E-03	3.709E-04
2.810E 00	4.872E-02	6.519E-03	9.150E 00	1.217E-03	3.414E-04
2.870E 00	3.838E-02	6.639E-03	9.300E 00	9.766E-04	2.425E-04
2.930E 00	3.564E-02	6.968E-03	9.460E 00	6.566E-04	1.813E-04
2.990E 00	4.553E-02	6.783E-03	9.620E 00	3.597E-04	1.462E-04
3.050E 00	5.062E-02	6.417E-03	9.780E 00	1.924E-04	8.753E-05
3.110E 00	4.801E-02	6.445E-03	9.940E 00	1.286E-04	6.514E-05
3.180E 00	4.921E-02	6.079E-03	1.010E 01	8.560E-05	5.233E-05
3.260E 00	5.432E-02	5.709E-03	1.026E 01	4.603E-05	3.004E-05
3.340E 00	5.320E-02	5.659E-03	1.042E 01	1.679E-05	2.203E-05
3.420E 00	4.937E-02	6.264E-03	1.058E 01	3.266E-06	2.050E-05
3.500E 00	4.792E-02	5.730E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.705E-02	2.447E-03
8.000E-01 - 1.000E 00	9.812E-02	4.434E-03
1.000E 00 - 1.200E 00	2.588E-02	3.551E-03
1.200E 00 - 1.400E 00	4.924E-02	3.330E-03
1.400E 00 - 1.600E 00	5.607E-03	2.540E-03
1.600E 00 - 1.800E 00	1.563E-02	2.686E-03
1.800E 00 - 2.000E 00	1.387E-02	2.434E-03
2.000E 00 - 2.500E 00	2.863E-02	4.510E-03
2.500E 00 - 3.000E 00	2.921E-02	3.466E-03
3.000E 00 - 3.500E 00	2.524E-02	3.041E-03
3.500E 00 - 4.000E 00	1.852E-02	2.362E-03
4.000E 00 - 4.500E 00	1.095E-02	1.988E-03
4.500E 00 - 5.000E 00	8.103E-03	1.829E-03
5.000E 00 - 6.000E 00	1.304E-02	3.079E-03
6.000E 00 - 7.000E 00	1.035E-02	2.363E-03
7.000E 00 - 8.000E 00	7.069E-03	1.527E-03
8.000E 00 - 9.000E 00	3.349E-03	7.308E-04
9.000E 00 - 1.000E 01	6.492E-04	1.920E-04

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 11.00 TO 12.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.908E-02	2.955E-02	3.580E 00	5.404E-02	5.947E-03
7.100E-01	3.434E-02	2.909E-02	3.660E 00	4.624E-02	5.518E-03
7.300E-01	6.117E-02	2.895E-02	3.740E 00	4.031E-02	5.862E-03
7.500E-01	1.099E-01	2.820E-02	3.820E 00	4.247E-02	5.402E-03
7.700E-01	2.437E-01	2.858E-02	3.900E 00	3.860E-02	5.255E-03
7.900E-01	5.067E-01	2.855E-02	3.980E 00	3.120E-02	5.205E-03
8.100E-01	8.223E-01	2.928E-02	4.060E 00	2.489E-02	4.981E-03
8.300E-01	1.032E 00	3.104E-02	4.140E 00	1.895E-02	5.378E-03
8.500E-01	1.022E 00	2.991E-02	4.220E 00	1.815E-02	5.378E-03
8.700E-01	8.133E-01	3.006E-02	4.300E 00	1.842E-02	5.343E-03
8.900E-01	5.344E-01	2.655E-02	4.380E 00	1.714E-02	5.120E-03
9.100E-01	3.058E-01	2.519E-02	4.460E 00	2.002E-02	4.973E-03
9.300E-01	1.701E-01	2.304E-02	4.540E 00	2.497E-02	4.900E-03
9.500E-01	1.050E-01	2.475E-02	4.630E 00	2.457E-02	5.037E-03
9.700E-01	8.072E-02	2.324E-02	4.730E 00	1.829E-02	4.830E-03
1.000E 00	8.683E-02	2.508E-02	4.830E 00	1.490E-02	4.767E-03
1.040E 00	1.132E-01	2.333E-02	4.930E 00	1.565E-02	4.537E-03
1.080E 00	8.943E-02	2.140E-02	5.030E 00	1.480E-02	4.560E-03
1.120E 00	7.752E-02	2.142E-02	5.130E 00	1.545E-02	4.292E-03
1.160E 00	1.823E-01	2.058E-02	5.230E 00	1.997E-02	4.288E-03
1.200E 00	3.628E-01	2.150E-02	5.330E 00	2.150E-02	4.493E-03
1.240E 00	4.277E-01	2.036E-02	5.430E 00	1.802E-02	4.187E-03
1.280E 00	3.284E-01	1.866E-02	5.530E 00	1.666E-02	4.362E-03
1.320E 00	1.952E-01	1.644E-02	5.630E 00	1.781E-02	3.934E-03
1.360E 00	1.068E-01	1.638E-02	5.730E 00	1.574E-02	4.060E-03
1.400E 00	7.215E-02	1.636E-02	5.830E 00	1.319E-02	4.297E-03
1.440E 00	5.237E-02	1.507E-02	5.930E 00	1.467E-02	3.939E-03
1.480E 00	3.541E-02	1.489E-02	6.030E 00	1.653E-02	3.824E-03
1.520E 00	3.571E-02	1.505E-02	6.130E 00	1.614E-02	3.668E-03
1.560E 00	4.378E-02	1.502E-02	6.230E 00	1.473E-02	3.542E-03
1.600E 00	5.880E-02	1.438E-02	6.330E 00	1.309E-02	3.552E-03
1.640E 00	7.464E-02	1.458E-02	6.440E 00	1.233E-02	3.493E-03
1.680E 00	7.398E-02	1.485E-02	6.560E 00	1.378E-02	3.300E-03
1.720E 00	7.764E-02	1.623E-02	6.680E 00	1.369E-02	3.131E-03
1.760E 00	1.019E-01	1.531E-02	6.800E 00	1.194E-02	3.066E-03
1.800E 00	1.151E-01	1.510E-02	6.920E 00	1.126E-02	2.997E-03
1.850E 00	8.758E-02	1.335E-02	7.040E 00	1.011E-02	2.734E-03
1.910E 00	6.102E-02	1.311E-02	7.160E 00	8.318E-03	2.648E-03
1.970E 00	7.248E-02	1.206E-02	7.280E 00	7.794E-03	2.593E-03
2.030E 00	8.223E-02	1.258E-02	7.400E 00	8.331E-03	2.454E-03
2.090E 00	8.253E-02	1.202E-02	7.520E 00	9.258E-03	2.362E-03
2.150E 00	6.009E-02	1.098E-02	7.640E 00	1.000E-02	2.295E-03
2.210E 00	4.995E-02	1.107E-02	7.760E 00	9.775E-03	2.233E-03
2.270E 00	5.124E-02	1.074E-02	7.890E 00	8.182E-03	2.090E-03
2.330E 00	5.231E-02	9.825E-03	8.030E 00	5.298E-03	1.940E-03
2.390E 00	6.107E-02	9.254E-03	8.170E 00	2.867E-03	1.799E-03
2.450E 00	6.546E-02	9.375E-03	8.310E 00	3.189E-03	1.673E-03
2.510E 00	7.352E-02	9.551E-03	8.450E 00	5.068E-03	1.545E-03
2.570E 00	8.420E-02	9.356E-03	8.590E 00	5.845E-03	1.341E-03
2.630E 00	7.328E-02	8.996E-03	8.730E 00	5.401E-03	1.176E-03
2.690E 00	4.530E-02	8.907E-03	8.870E 00	4.503E-03	1.099E-03
2.750E 00	3.076E-02	8.736E-03	9.010E 00	3.183E-03	9.474E-04
2.810E 00	3.881E-02	8.058E-03	9.150E 00	1.865E-03	8.250E-04
2.870E 00	5.046E-02	7.921E-03	9.300E 00	1.191E-03	7.025E-04
2.930E 00	4.972E-02	7.632E-03	9.460E 00	1.490E-03	5.664E-04
2.990E 00	4.323E-02	6.303E-03	9.620E 00	1.955E-03	4.586E-04
3.050E 00	4.717E-02	8.078E-03	9.780E 00	1.978E-03	3.120E-04
3.110E 00	5.887E-02	7.633E-03	9.940E 00	1.338E-03	2.154E-04
3.180E 00	6.056E-02	7.245E-03	1.010E 01	6.264E-04	1.398E-04
3.260E 00	6.637E-02	7.359E-03	1.026E 01	1.988E-04	6.098E-05
3.340E 00	3.979E-02	7.504E-03	1.042E 01	3.770E-05	3.473E-05
3.420E 00	4.407E-02	6.506E-03	1.058E 01	9.080E-08	2.935E-05
3.500E 00	5.134E-02	6.738E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.930E-02	2.870E-03
8.000E-01 - 1.000E 00	9.916E-02	5.345E-03
1.000E 00 - 1.200E 00	2.683E-02	4.386E-03
1.200E 00 - 1.400E 00	5.132E-02	3.628E-03
1.400E 00 - 1.600E 00	9.153E-03	3.014E-03
1.600E 00 - 1.800E 00	1.662E-02	3.027E-03
1.800E 00 - 2.000E 00	1.538E-02	2.625E-03
2.000E 00 - 2.500E 00	3.175E-02	5.336E-03
2.500E 00 - 3.000E 00	2.717E-02	4.298E-03
3.000E 00 - 3.500E 00	2.452E-02	3.661E-03
3.500E 00 - 4.000E 00	2.171E-02	2.812E-03
4.000E 00 - 4.500E 00	9.997E-03	2.599E-03
4.500E 00 - 5.000E 00	9.597E-03	2.403E-03
5.000E 00 - 6.000E 00	1.678E-02	4.227E-03
6.000E 00 - 7.000E 00	1.348E-02	3.355E-03
7.000E 00 - 8.000E 00	8.806E-03	2.396E-03
8.000E 00 - 9.000E 00	4.465E-03	1.459E-03
9.000E 00 - 1.000E 01	1.743E-03	5.466E-04

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 12.00 TO 13.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.269E-02	3.451E-02	3.580E 00	5.117E-02	7.667E-03
7.100E-01	5.071E-02	3.293E-02	3.660E 00	4.308E-02	7.880E-03
7.300E-01	7.616E-02	3.262E-02	3.740E 00	3.393E-02	7.538E-03
7.500E-01	1.147E-01	3.302E-02	3.820E 00	2.604E-02	6.802E-03
7.700E-01	2.316E-01	3.561E-02	3.900E 00	2.614E-02	6.926E-03
7.900E-01	4.697E-01	3.503E-02	3.980E 00	3.281E-02	6.550E-03
8.100E-01	7.726E-01	3.539E-02	4.060E 00	3.275E-02	6.617E-03
8.300E-01	9.886E-01	3.786E-02	4.140E 00	2.304E-02	6.380E-03
8.500E-01	9.792E-01	3.593E-02	4.220E 00	1.645E-02	6.072E-03
8.700E-01	7.621E-01	3.528E-02	4.300E 00	1.741E-02	6.243E-03
8.900E-01	4.906E-01	3.068E-02	4.380E 00	2.086E-02	6.183E-03
9.100E-01	2.862E-01	2.937E-02	4.460E 00	1.944E-02	5.993E-03
9.300E-01	1.651E-01	2.689E-02	4.540E 00	1.423E-02	6.340E-03
9.500E-01	9.516E-02	2.799E-02	4.630E 00	1.543E-02	6.089E-03
9.700E-01	6.246E-02	2.544E-02	4.730E 00	2.382E-02	6.068E-03
1.000E 00	7.550E-02	2.713E-02	4.830E 00	2.695E-02	6.091E-03
1.040E 00	1.106E-01	2.487E-02	4.930E 00	2.191E-02	5.639E-03
1.080E 00	7.635E-02	2.306E-02	5.030E 00	1.850E-02	6.403E-03
1.120E 00	7.835E-02	2.394E-02	5.130E 00	2.035E-02	5.736E-03
1.160E 00	1.918E-01	2.475E-02	5.230E 00	1.999E-02	6.038E-03
1.200E 00	3.625E-01	2.636E-02	5.330E 00	1.745E-02	5.785E-03
1.240E 00	4.506E-01	2.595E-02	5.430E 00	1.568E-02	5.184E-03
1.280E 00	3.713E-01	2.496E-02	5.530E 00	1.738E-02	5.526E-03
1.320E 00	2.181E-01	2.007E-02	5.630E 00	1.937E-02	5.179E-03
1.360E 00	1.195E-01	1.975E-02	5.730E 00	1.834E-02	4.984E-03
1.400E 00	8.325E-02	2.025E-02	5.830E 00	1.892E-02	5.156E-03
1.440E 00	4.991E-02	1.786E-02	5.930E 00	2.101E-02	4.803E-03
1.480E 00	2.163E-02	1.853E-02	6.030E 00	1.826E-02	4.566E-03
1.520E 00	3.019E-02	1.841E-02	6.130E 00	1.225E-02	4.409E-03
1.560E 00	4.861E-02	1.628E-02	6.230E 00	1.122E-02	4.318E-03
1.600E 00	5.310E-02	1.632E-02	6.330E 00	1.449E-02	4.466E-03
1.640E 00	5.725E-02	1.747E-02	6.440E 00	1.686E-02	4.469E-03
1.680E 00	6.677E-02	1.790E-02	6.560E 00	1.963E-02	4.258E-03
1.720E 00	8.114E-02	1.863E-02	6.680E 00	2.079E-02	4.228E-03
1.760E 00	9.859E-02	1.727E-02	6.800E 00	1.577E-02	4.202E-03
1.800E 00	1.077E-01	1.681E-02	6.920E 00	1.043E-02	4.023E-03
1.850E 00	9.014E-02	1.509E-02	7.040E 00	9.249E-03	3.753E-03
1.910E 00	5.113E-02	1.539E-02	7.160E 00	9.594E-03	3.597E-03
1.970E 00	4.782E-02	1.552E-02	7.280E 00	1.058E-02	3.728E-03
2.030E 00	5.980E-02	1.595E-02	7.400E 00	1.185E-02	3.522E-03
2.090E 00	7.249E-02	1.508E-02	7.520E 00	1.070E-02	3.252E-03
2.150E 00	7.047E-02	1.287E-02	7.640E 00	7.383E-03	3.112E-03
2.210E 00	4.570E-02	1.307E-02	7.760E 00	5.193E-03	3.047E-03
2.270E 00	3.624E-02	1.139E-02	7.880E 00	5.656E-03	2.970E-03
2.330E 00	3.992E-02	1.176E-02	8.030E 00	9.340E-03	2.725E-03
2.390E 00	3.836E-02	1.150E-02	8.170E 00	7.997E-03	2.562E-03
2.450E 00	4.674E-02	1.219E-02	8.310E 00	7.431E-03	2.390E-03
2.510E 00	6.571E-02	1.188E-02	8.450E 00	6.565E-03	2.253E-03
2.570E 00	6.882E-02	1.096E-02	8.590E 00	5.916E-03	2.009E-03
2.630E 00	5.956E-02	1.059E-02	8.730E 00	5.644E-03	1.847E-03
2.690E 00	5.400E-02	9.673E-03	8.870E 00	5.272E-03	1.771E-03
2.750E 00	4.994E-02	9.396E-03	9.010E 00	3.816E-03	1.593E-03
2.810E 00	5.129E-02	9.618E-03	9.150E 00	1.641E-03	1.365E-03
2.870E 00	5.274E-02	9.237E-03	9.300E 00	2.015E-04	1.231E-03
2.930E 00	4.578E-02	9.503E-03	9.460E 00	1.166E-03	1.054E-03
2.990E 00	3.797E-02	9.615E-03	9.620E 00	3.221E-03	7.667E-04
3.050E 00	3.868E-02	9.936E-03	9.780E 00	4.068E-03	5.767E-04
3.110E 00	4.958E-02	9.975E-03	9.940E 00	2.946E-03	4.279E-04
3.180E 00	6.064E-02	9.504E-03	1.010E 01	1.402E-03	2.593E-04
3.260E 00	4.822E-02	8.563E-03	1.026E 01	4.357E-04	1.041E-04
3.340E 00	4.389E-02	9.433E-03	1.042E 01	7.897E-05	5.412E-05
3.420E 00	5.303E-02	8.150E-03	1.058E 01	-2.002E-06	-4.448E-05
3.500E 00	5.459E-02	7.854E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.901E-02	3.390E-03
8.000E-01 - 1.000E 00	9.319E-02	6.221E-03
1.000E 00 - 1.200E 00	2.644E-02	4.916E-03
1.200E 00 - 1.400E 00	5.573E-02	4.564E-03
1.400E 00 - 1.600E 00	8.577E-03	3.564E-03
1.600E 00 - 1.800E 00	1.536E-02	3.517E-03
1.800E 00 - 2.000E 00	1.334E-02	3.101E-03
2.000E 00 - 2.500E 00	2.581E-02	6.458E-03
2.500E 00 - 3.000E 00	2.715E-02	4.980E-03
3.000E 00 - 3.500E 00	2.469E-02	4.555E-03
3.500E 00 - 4.000E 00	1.850E-02	3.657E-03
4.000E 00 - 4.500E 00	1.102E-02	3.134E-03
4.500E 00 - 5.000E 00	1.039E-02	3.022E-03
5.000E 00 - 6.000E 00	1.872E-02	5.435E-03
6.000E 00 - 7.000E 00	1.542E-02	4.302E-03
7.000E 00 - 8.000E 00	8.619E-03	3.332E-03
8.000E 00 - 9.000E 00	6.426E-03	2.165E-03
9.000E 00 - 1.000E 01	2.319E-03	9.558E-04

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 13.00 TO 14.00 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	1.114E-02	3.591E-02	3.580E 00	3.687E-02	8.895E-03
7.100E-01	1.851E-02	3.627E-02	3.660E 00	3.466E-02	7.994E-03
7.300E-01	4.607E-02	3.629E-02	3.740E 00	2.904E-02	7.523E-03
7.500E-01	8.717E-02	3.565E-02	3.820E 00	2.771E-02	8.019E-03
7.700E-01	1.803E-01	3.593E-02	3.900E 00	2.933E-02	7.520E-03
7.900E-01	3.930E-01	3.605E-02	3.980E 00	2.341E-02	7.735E-03
8.100E-01	6.845E-01	3.918E-02	4.060E 00	2.151E-02	7.338E-03
8.300E-01	9.086E-01	4.053E-02	4.140E 00	2.607E-02	6.979E-03
8.500E-01	9.400E-01	3.755E-02	4.220E 00	2.555E-02	7.425E-03
8.700E-01	7.718E-01	3.722E-02	4.300E 00	2.059E-02	7.502E-03
8.900E-01	5.091E-01	3.465E-02	4.380E 00	1.950E-02	7.313E-03
9.100E-01	2.772E-01	3.380E-02	4.460E 00	2.060E-02	7.475E-03
9.300E-01	1.449E-01	2.944E-02	4.540E 00	1.942E-02	7.072E-03
9.500E-01	1.003E-01	3.078E-02	4.630E 00	1.845E-02	7.110E-03
9.700E-01	9.780E-02	2.807E-02	4.730E 00	2.356E-02	7.198E-03
1.000E 00	1.092E-01	2.882E-02	4.830E 00	2.449E-02	7.047E-03
1.040E 00	1.076E-01	2.821E-02	4.930E 00	1.875E-02	6.843E-03
1.080E 00	5.798E-02	2.637E-02	5.030E 00	1.797E-02	6.483E-03
1.120E 00	5.706E-02	2.678E-02	5.130E 00	1.945E-02	6.688E-03
1.160E 00	1.801E-01	2.728E-02	5.230E 00	1.476E-02	6.589E-03
1.200E 00	3.442E-01	2.928E-02	5.330E 00	1.050E-02	6.404E-03
1.240E 00	4.148E-01	3.041E-02	5.430E 00	1.336E-02	6.105E-03
1.280E 00	3.477E-01	2.755E-02	5.530E 00	1.961E-02	5.857E-03
1.320E 00	2.299E-01	2.333E-02	5.630E 00	2.247E-02	6.044E-03
1.360E 00	1.412E-01	2.165E-02	5.730E 00	2.177E-02	5.981E-03
1.400E 00	9.148E-02	2.208E-02	5.830E 00	1.890E-02	5.695E-03
1.440E 00	6.122E-02	1.981E-02	5.930E 00	1.385E-02	5.640E-03
1.480E 00	4.601E-02	1.945E-02	6.030E 00	9.448E-03	5.389E-03
1.520E 00	4.261E-02	1.806E-02	6.130E 00	8.735E-03	5.425E-03
1.560E 00	3.649E-02	1.853E-02	6.230E 00	1.202E-02	5.536E-03
1.600E 00	2.930E-02	1.978E-02	6.330E 00	1.772E-02	5.255E-03
1.640E 00	4.105E-02	2.125E-02	6.430E 00	2.219E-02	5.113E-03
1.680E 00	6.442E-02	1.928E-02	6.530E 00	1.994E-02	4.940E-03
1.720E 00	8.207E-02	1.962E-02	6.630E 00	1.486E-02	4.885E-03
1.760E 00	8.926E-02	1.830E-02	6.730E 00	1.279E-02	4.726E-03
1.800E 00	9.044E-02	1.834E-02	6.830E 00	1.206E-02	4.417E-03
1.850E 00	8.627E-02	1.732E-02	6.930E 00	1.018E-02	4.295E-03
1.910E 00	6.762E-02	1.490E-02	7.030E 00	9.435E-03	4.249E-03
1.970E 00	4.843E-02	1.525E-02	7.130E 00	1.071E-02	4.091E-03
2.030E 00	6.577E-02	1.463E-02	7.230E 00	1.122E-02	3.809E-03
2.090E 00	7.080E-02	1.555E-02	7.330E 00	1.099E-02	3.764E-03
2.150E 00	5.142E-02	1.592E-02	7.430E 00	1.156E-02	3.751E-03
2.210E 00	3.522E-02	1.433E-02	7.530E 00	1.099E-02	3.599E-03
2.270E 00	2.799E-02	1.359E-02	7.630E 00	8.341E-03	3.270E-03
2.330E 00	3.951E-02	1.201E-02	7.730E 00	6.480E-03	3.191E-03
2.390E 00	5.281E-02	1.115E-02	7.830E 00	5.165E-03	2.947E-03
2.450E 00	5.975E-02	1.223E-02	7.930E 00	3.300E-03	2.750E-03
2.510E 00	7.071E-02	1.244E-02	8.030E 00	4.235E-03	2.546E-03
2.570E 00	7.520E-02	1.240E-02	8.130E 00	7.466E-03	2.383E-03
2.630E 00	6.954E-02	1.166E-02	8.230E 00	7.984E-03	2.168E-03
2.690E 00	6.168E-02	1.077E-02	8.330E 00	5.106E-03	1.941E-03
2.750E 00	4.865E-02	1.105E-02	8.430E 00	2.717E-03	1.738E-03
2.810E 00	3.734E-02	1.206E-02	8.530E 00	2.279E-03	1.582E-03
2.870E 00	3.730E-02	1.164E-02	8.630E 00	2.435E-03	1.400E-03
2.930E 00	4.022E-02	1.041E-02	8.730E 00	2.741E-03	1.125E-03
2.990E 00	3.725E-02	1.049E-02	8.830E 00	3.383E-03	9.164E-04
3.050E 00	3.687E-02	1.059E-02	8.930E 00	3.716E-03	6.565E-04
3.110E 00	4.553E-02	1.069E-02	9.030E 00	2.742E-03	4.494E-04
3.170E 00	5.374E-02	1.009E-02	9.130E 00	1.367E-03	2.900E-04
3.230E 00	5.270E-02	9.434E-03	9.230E 00	4.562E-04	1.269E-04
3.290E 00	5.063E-02	9.120E-03	9.330E 00	8.979E-05	6.963E-05
3.350E 00	5.079E-02	9.439E-03	9.430E 00	8.249E-07	5.755E-05
3.410E 00	4.522E-02	8.807E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.471E-02	2.609E-03
8.000E-01 - 1.000E 00	9.062E-02	6.785E-03
1.000E 00 - 1.200E 00	2.448E-02	5.492E-03
1.200E 00 - 1.400E 00	5.448E-02	5.146E-03
1.400E 00 - 1.600E 00	9.774E-03	3.860E-03
1.600E 00 - 1.800E 00	1.352E-02	3.899E-03
1.800E 00 - 2.000E 00	1.406E-02	3.194E-03
2.000E 00 - 2.500E 00	2.557E-02	6.814E-03
2.500E 00 - 3.000E 00	2.658E-02	5.714E-03
3.000E 00 - 3.500E 00	2.417E-02	4.896E-03
3.500E 00 - 4.000E 00	1.583E-02	4.008E-03
4.000E 00 - 4.500E 00	1.117E-02	3.669E-03
4.500E 00 - 5.000E 00	1.047E-02	3.519E-03
5.000E 00 - 6.000E 00	1.716E-02	6.130E-03
6.000E 00 - 7.000E 00	1.460E-02	5.031E-03
7.000E 00 - 8.000E 00	1.026E-02	3.810E-03
8.000E 00 - 9.000E 00	5.507E-03	2.486E-03
9.000E 00 - 1.000E 01	2.884E-03	1.078E-03

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN PE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 14.00 TO 15.10 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	-2.802E-02	-4.042E-02	3.580E 00	3.915E-02	9.415E-03
7.100E-01	-1.913E-02	-3.743E-02	3.660E 00	3.000E-02	9.363E-03
7.300E-01	1.394E-02	3.788E-02	3.740E 00	2.371E-02	8.971E-03
7.500E-01	6.230E-02	3.926E-02	3.820E 00	2.080E-02	8.295E-03
7.700E-01	1.462E-01	3.981E-02	3.900E 00	2.075E-02	7.606E-03
7.900E-01	3.097E-01	4.121E-02	3.980E 00	1.865E-02	7.872E-03
8.100E-01	5.283E-01	4.276E-02	4.060E 00	1.670E-02	8.798E-03
8.300E-01	7.066E-01	4.266E-02	4.140E 00	2.062E-02	8.243E-03
8.500E-01	7.549E-01	4.121E-02	4.220E 00	2.754E-02	8.171E-03
8.700E-01	6.677E-01	4.036E-02	4.300E 00	2.989E-02	8.417E-03
8.900E-01	5.180E-01	3.536E-02	4.380E 00	2.476E-02	8.021E-03
9.100E-01	3.732E-01	3.238E-02	4.460E 00	1.888E-02	7.970E-03
9.300E-01	2.570E-01	3.045E-02	4.540E 00	1.784E-02	8.025E-03
9.500E-01	1.681E-01	3.173E-02	4.630E 00	1.592E-02	7.850E-03
9.700E-01	1.104E-01	3.166E-02	4.730E 00	9.990E-03	7.919E-03
1.000E 00	8.177E-02	3.349E-02	4.830E 00	1.073E-02	7.311E-03
1.040E 00	9.488E-02	3.031E-02	4.930E 00	1.728E-02	7.283E-03
1.080E 00	8.020E-02	2.651E-02	5.030E 00	2.013E-02	7.026E-03
1.120E 00	8.236E-02	2.593E-02	5.130E 00	1.673E-02	6.670E-03
1.160E 00	1.611E-01	3.080E-02	5.230E 00	1.360E-02	7.583E-03
1.200E 00	2.780E-01	3.067E-02	5.330E 00	1.188E-02	7.048E-03
1.240E 00	3.610E-01	3.142E-02	5.430E 00	9.626E-03	7.128E-03
1.280E 00	3.383E-01	2.793E-02	5.530E 00	1.015E-02	6.570E-03
1.320E 00	2.463E-01	2.374E-02	5.630E 00	1.133E-02	6.307E-03
1.360E 00	1.635E-01	2.296E-02	5.730E 00	1.202E-02	6.380E-03
1.400E 00	1.092E-01	2.275E-02	5.830E 00	1.494E-02	6.395E-03
1.440E 00	6.900E-02	2.085E-02	5.930E 00	2.059E-02	6.427E-03
1.480E 00	3.568E-02	2.024E-02	6.030E 00	2.483E-02	6.441E-03
1.520E 00	1.595E-02	2.124E-02	6.130E 00	2.401E-02	5.804E-03
1.560E 00	1.747E-02	2.227E-02	6.230E 00	1.608E-02	5.582E-03
1.600E 00	4.047E-02	1.857E-02	6.330E 00	7.372E-03	5.807E-03
1.640E 00	6.425E-02	2.007E-02	6.440E 00	3.184E-03	5.455E-03
1.680E 00	6.578E-02	1.859E-02	6.560E 00	2.697E-03	5.214E-03
1.720E 00	6.165E-02	1.911E-02	6.680E 00	4.143E-03	5.303E-03
1.760E 00	6.978E-02	1.862E-02	6.800E 00	1.041E-02	5.316E-03
1.800E 00	7.382E-02	1.845E-02	6.920E 00	1.829E-02	4.841E-03
1.850E 00	5.092E-02	2.083E-02	7.040E 00	2.078E-02	4.653E-03
1.910E 00	2.066E-02	1.816E-02	7.160E 00	1.723E-02	4.523E-03
1.970E 00	2.307E-02	1.852E-02	7.280E 00	1.027E-02	4.233E-03
2.030E 00	4.254E-02	1.568E-02	7.400E 00	1.933E-03	4.198E-03
2.090E 00	4.144E-02	1.483E-02	7.520E 00	-4.134E-03	-4.162E-03
2.150E 00	4.221E-02	1.336E-02	7.640E 00	-3.416E-03	-3.784E-03
2.210E 00	4.528E-02	1.266E-02	7.760E 00	3.536E-03	3.592E-03
2.270E 00	4.466E-02	1.278E-02	7.890E 00	1.157E-02	3.527E-03
2.330E 00	5.236E-02	1.209E-02	8.030E 00	1.500E-02	1.171E-03
2.390E 00	5.967E-02	1.280E-02	8.170E 00	1.255E-02	3.114E-03
2.450E 00	5.250E-02	1.279E-02	8.310E 00	7.105E-03	2.747E-03
2.510E 00	5.003E-02	1.362E-02	8.450E 00	2.282E-03	2.634E-03
2.570E 00	5.880E-02	1.246E-02	8.590E 00	-2.819E-04	-2.533E-03
2.630E 00	5.271E-02	1.185E-02	8.730E 00	-1.825E-04	-2.236E-03
2.690E 00	3.822E-02	1.133E-02	8.870E 00	2.393E-03	2.030E-03
2.750E 00	3.656E-02	1.086E-02	9.010E 00	5.037E-03	1.971E-03
2.810E 00	3.614E-02	1.137E-02	9.150E 00	4.830E-03	1.826E-03
2.870E 00	2.778E-02	1.120E-02	9.300E 00	2.471E-03	1.528E-03
2.930E 00	2.387E-02	1.059E-02	9.460E 00	1.439E-03	1.320E-03
2.990E 00	2.760E-02	1.058E-02	9.620E 00	2.721E-03	1.033E-03
3.050E 00	3.202E-02	1.218E-02	9.780E 00	3.779E-03	7.034E-04
3.110E 00	3.565E-02	1.227E-02	9.940E 00	3.026E-03	5.417E-04
3.180E 00	4.143E-02	1.071E-02	1.010E 01	1.544E-03	3.741E-04
3.260E 00	4.451E-02	9.701E-03	1.026E 01	5.410E-04	1.718E-04
3.340E 00	4.324E-02	9.864E-03	1.042E 01	1.173E-04	9.823E-05
3.420E 00	4.350E-02	1.004E-02	1.058E 01	8.903E-06	8.106E-05
3.500E 00	4.520E-02	9.782E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.042E-02	2.535E-03
8.000E-01 - 1.000E 00	8.326E-02	7.224E-03
1.000E 00 - 1.200E 00	2.350E-02	5.818E-03
1.200E 00 - 1.400E 00	5.263E-02	5.314E-03
1.400E 00 - 1.600E 00	8.248E-03	4.222E-03
1.600E 00 - 1.800E 00	1.283E-02	3.798E-03
1.800E 00 - 2.000E 00	7.055E-03	3.781E-03
2.000E 00 - 2.500E 00	2.378E-02	6.705E-03
2.500E 00 - 3.000E 00	1.959E-02	5.741E-03
3.000E 00 - 3.500E 00	2.022E-02	5.313E-03
3.500E 00 - 4.000E 00	1.365E-02	4.351E-03
4.000E 00 - 4.500E 00	1.145E-02	4.127E-03
4.500E 00 - 5.000E 00	7.194E-03	3.816E-03
5.000E 00 - 6.000E 00	1.413E-02	6.735E-03
6.000E 00 - 7.000E 00	1.178E-02	5.462E-03
7.000E 00 - 8.000E 00	7.278E-03	2.180E-03
8.000E 00 - 9.000E 00	5.084E-03	1.236E-03
9.000E 00 - 1.000E 01	3.192E-03	1.222E-03

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 15.10 TO 17.60 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	I-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	5.785E-02	3.785E-02	3.580E 00	3.810E-02	6.246E-03
7.100E-01	7.438E-02	3.447E-02	3.660E 00	3.018E-02	5.810E-03
7.300E-01	5.443E-02	3.573E-02	3.740E 00	2.486E-02	6.502E-03
7.500E-01	4.183E-02	3.600E-02	3.820E 00	2.513E-02	5.681E-03
7.700E-01	9.564E-02	3.553E-02	3.900E 00	2.022E-02	5.653E-03
7.900E-01	2.403E-01	3.308E-02	3.980E 00	1.109E-02	5.740E-03
8.100E-01	4.221E-01	3.303E-02	4.060E 00	3.589E-03	6.050E-03
8.300E-01	5.504E-01	3.618E-02	4.140E 00	-1.525E-04	-6.250E-03
8.500E-01	5.792E-01	3.635E-02	4.220E 00	2.827E-03	5.757E-03
8.700E-01	5.249E-01	3.581E-02	4.300E 00	9.683E-03	5.638E-03
8.900E-01	4.340E-01	3.190E-02	4.380E 00	1.393E-02	5.804E-03
9.100E-01	3.396E-01	2.912E-02	4.460E 00	1.417E-02	5.714E-03
9.300E-01	2.559E-01	2.805E-02	4.540E 00	1.298E-02	5.224E-03
9.500E-01	1.851E-01	3.087E-02	4.630E 00	1.243E-02	5.298E-03
9.700E-01	1.269E-01	2.984E-02	4.730E 00	1.489E-02	5.121E-03
1.000E 00	7.349E-02	2.886E-02	4.830E 00	1.180E-02	5.226E-03
1.040E 00	6.667E-02	2.731E-02	4.930E 00	5.322E-03	5.273E-03
1.080E 00	5.465E-02	2.237E-02	5.030E 00	4.772E-03	5.193E-03
1.120E 00	3.510E-02	2.473E-02	5.130E 00	9.739E-03	5.062E-03
1.160E 00	8.139E-02	2.368E-02	5.230E 00	1.457E-02	4.886E-03
1.200E 00	2.012E-01	2.719E-02	5.330E 00	1.735E-02	4.836E-03
1.240E 00	2.910E-01	2.556E-02	5.430E 00	1.470E-02	4.541E-03
1.280E 00	2.937E-01	2.850E-02	5.530E 00	1.120E-02	5.176E-03
1.320E 00	2.457E-01	2.551E-02	5.630E 00	7.848E-03	4.616E-03
1.360E 00	1.698E-01	2.333E-02	5.730E 00	5.461E-03	4.581E-03
1.400E 00	1.089E-01	2.016E-02	5.830E 00	5.562E-03	4.247E-03
1.440E 00	7.579E-02	1.638E-02	5.930E 00	8.179E-03	4.177E-03
1.480E 00	5.731E-02	1.509E-02	6.030E 00	1.041E-02	4.210E-03
1.520E 00	4.212E-02	1.421E-02	6.130E 00	1.126E-02	4.092E-03
1.560E 00	3.669E-02	1.484E-02	6.230E 00	1.079E-02	3.744E-03
1.600E 00	4.733E-02	1.415E-02	6.330E 00	8.532E-03	3.687E-03
1.640E 00	5.239E-02	1.438E-02	6.440E 00	5.792E-03	3.730E-03
1.680E 00	4.164E-02	1.446E-02	6.560E 00	5.691E-03	3.779E-03
1.720E 00	3.247E-02	1.639E-02	6.680E 00	7.612E-03	3.528E-03
1.760E 00	3.741E-02	1.613E-02	6.800E 00	9.286E-03	3.376E-03
1.800E 00	5.319E-02	1.507E-02	6.920E 00	9.912E-03	3.234E-03
1.850E 00	6.188E-02	1.347E-02	7.040E 00	8.985E-03	3.007E-03
1.910E 00	4.174E-02	1.356E-02	7.160E 00	6.746E-03	2.768E-03
1.970E 00	3.656E-02	1.178E-02	7.280E 00	5.209E-03	2.717E-03
2.030E 00	4.344E-02	1.225E-02	7.400E 00	5.258E-03	2.644E-03
2.090E 00	3.972E-02	1.304E-02	7.520E 00	5.209E-03	2.497E-03
2.150E 00	3.566E-02	1.269E-02	7.640E 00	4.106E-03	2.296E-03
2.210E 00	2.740E-02	1.076E-02	7.760E 00	2.749E-03	2.213E-03
2.270E 00	1.800E-02	1.126E-02	7.890E 00	1.946E-03	2.181E-03
2.330E 00	1.779E-02	1.077E-02	8.030E 00	2.361E-03	2.072E-03
2.390E 00	2.729E-02	1.026E-02	8.170E 00	4.102E-03	1.785E-03
2.450E 00	3.622E-02	9.992E-03	8.310E 00	5.426E-03	1.767E-03
2.510E 00	3.775E-02	1.111E-02	8.450E 00	4.498E-03	1.711E-03
2.570E 00	3.381E-02	1.071E-02	8.590E 00	2.369E-03	1.470E-03
2.630E 00	3.185E-02	1.127E-02	8.730E 00	1.459E-03	1.360E-03
2.690E 00	2.990E-02	9.342E-03	8.870E 00	1.896E-03	1.291E-03
2.750E 00	2.722E-02	8.439E-03	9.010E 00	2.052E-03	1.184E-03
2.810E 00	2.714E-02	7.944E-03	9.150E 00	1.204E-03	1.054E-03
2.870E 00	2.619E-02	8.229E-03	9.300E 00	-9.461E-08	-9.689E-04
2.930E 00	2.467E-02	8.461E-03	9.460E 00	2.010E-05	8.662E-04
2.990E 00	2.659E-02	7.732E-03	9.620E 00	1.361E-03	6.678E-04
3.050E 00	2.858E-02	7.868E-03	9.780E 00	2.347E-03	4.599E-04
3.110E 00	2.479E-02	7.680E-03	9.940E 00	2.034E-03	3.870E-04
3.180E 00	2.054E-02	8.148E-03	1.010E 01	1.051E-03	2.490E-04
3.260E 00	2.203E-02	8.433E-03	1.026E 01	3.470E-04	1.037E-04
3.340E 00	2.201E-02	7.465E-03	1.042E 01	6.445E-05	6.052E-05
3.420E 00	2.208E-02	6.735E-03	1.058E 01	-1.376E-06	-5.099E-05
3.500E 00	3.197E-02	6.279E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	I-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.019E-02	3.507E-03
8.000E-01 - 1.000E 00	6.997E-02	6.410E-03
1.000E 00 - 1.200E 00	1.447E-02	5.008E-03
1.200E 00 - 1.400E 00	4.677E-02	5.080E-03
1.400E 00 - 1.600E 00	1.142E-02	3.098E-03
1.600E 00 - 1.800E 00	8.510E-03	3.042E-03
1.800E 00 - 2.000E 00	9.400E-03	2.648E-03
2.000E 00 - 2.500E 00	1.544E-02	5.681E-03
2.500E 00 - 3.000E 00	1.458E-02	4.618E-03
3.000E 00 - 3.500E 00	1.186E-02	3.801E-03
3.500E 00 - 4.000E 00	1.311E-02	2.982E-03
4.000E 00 - 4.500E 00	3.700E-03	1.837E-03
4.500E 00 - 5.000E 00	5.606E-03	2.618E-03
5.000E 00 - 6.000E 00	1.007E-02	4.710E-03
6.000E 00 - 7.000E 00	8.682E-03	3.668E-03
7.000E 00 - 8.000E 00	4.757E-03	2.504E-03
8.000E 00 - 9.000E 00	3.134E-03	1.589E-03
9.000E 00 - 1.000E 01	1.214E-03	4.757E-04

Table 2 (continued)

DIFFERENTIAL CROSS SECTIONS FOR GAMMA RAY PRODUCTION IN FE. THE FIRST SET OF NUMBERS IS THE DOUBLY DIFFERENTIAL CROSS SECTION, WHILE THE SECOND SET IS THE GAMMA RAY PRODUCTION CROSS SECTION FOR THE DESIGNATED GAMMA RAY ENERGY INTERVALS. THIS SECOND SET RESULTS FROM INTEGRATION OF THE DOUBLY DIFFERENTIAL DATA. THE UNCERTAINTIES ARE GIVEN IN THE SAME UNITS AS THE DATA AND DO NOT INCLUDE AN ESTIMATED 10 PERCENT ERROR IN ABSOLUTE NORMALIZATION.

INCIDENT NEUTRON ENERGY = 17.60 TO 20.10 MEV. ANGLE = 125 DEGREES.

PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)	PHOTON ENERGY (MEV)	X-SECTION (B/SR/MEV)	ERROR (B/SR/MEV)
6.900E-01	3.883E-02	5.018E-02	3.580E 00	2.423E-02	7.368E-03
7.100E-01	5.452E-02	5.087E-02	3.660E 00	1.562E-02	6.685E-03
7.300E-01	5.485E-02	4.792E-02	3.740E 00	1.643E-02	7.233E-03
7.500E-01	7.431E-02	4.381E-02	3.820E 00	1.607E-02	6.755E-03
7.700E-01	1.271E-01	4.010E-02	3.900E 00	1.304E-02	6.088E-03
7.900E-01	2.185E-01	3.745E-02	3.980E 00	1.284E-02	6.532E-03
8.100E-01	3.301E-01	4.022E-02	4.060E 00	1.109E-02	5.923E-03
8.300E-01	4.263E-01	4.084E-02	4.140E 00	4.420E-03	6.146E-03
8.500E-01	4.707E-01	4.074E-02	4.220E 00	4.298E-05	5.509E-03
8.700E-01	4.525E-01	4.277E-02	4.300E 00	2.203E-03	6.526E-03
8.900E-01	3.928E-01	4.294E-02	4.380E 00	4.400E-03	5.779E-03
9.100E-01	3.223E-01	4.017E-02	4.460E 00	3.868E-03	5.966E-03
9.300E-01	2.633E-01	3.621E-02	4.540E 00	4.947E-03	5.697E-03
9.500E-01	2.184E-01	3.816E-02	4.630E 00	1.086E-02	5.710E-03
9.700E-01	1.785E-01	3.520E-02	4.730E 00	1.792E-02	5.508E-03
1.000E 00	1.239E-01	3.443E-02	4.830E 00	1.712E-02	5.329E-03
1.040E 00	7.992E-02	3.016E-02	4.930E 00	9.184E-03	4.943E-03
1.080E 00	6.807E-02	2.987E-02	5.030E 00	2.497E-03	4.866E-03
1.120E 00	6.115E-02	2.774E-02	5.130E 00	3.217E-04	5.467E-03
1.160E 00	7.971E-02	2.782E-02	5.230E 00	8.142E-04	5.379E-03
1.200E 00	1.830E-01	3.188E-02	5.330E 00	3.470E-03	4.846E-03
1.240E 00	2.651E-01	3.071E-02	5.430E 00	5.804E-03	4.159E-03
1.280E 00	2.605E-01	3.611E-02	5.530E 00	6.622E-03	4.166E-03
1.320E 00	2.604E-01	2.887E-02	5.630E 00	7.420E-03	4.406E-03
1.360E 00	2.301E-01	2.665E-02	5.730E 00	6.645E-03	4.235E-03
1.400E 00	1.548E-01	2.679E-02	5.830E 00	4.961E-03	4.139E-03
1.440E 00	1.069E-01	2.311E-02	5.930E 00	3.787E-03	4.164E-03
1.480E 00	9.074E-02	2.139E-02	6.030E 00	4.241E-03	4.415E-03
1.520E 00	6.714E-02	1.822E-02	6.130E 00	4.014E-03	4.185E-03
1.560E 00	4.119E-02	2.004E-02	6.230E 00	2.330E-03	3.942E-03
1.600E 00	3.563E-02	2.137E-02	6.330E 00	1.725E-03	3.943E-03
1.640E 00	4.682E-02	2.014E-02	6.440E 00	4.557E-03	4.115E-03
1.680E 00	5.572E-02	1.924E-02	6.560E 00	7.946E-03	3.721E-03
1.720E 00	5.526E-02	1.860E-02	6.680E 00	8.154E-03	3.308E-03
1.760E 00	5.040E-02	1.722E-02	6.800E 00	7.265E-03	3.143E-03
1.800E 00	4.843E-02	1.821E-02	6.920E 00	6.269E-03	2.898E-03
1.850E 00	5.360E-02	1.761E-02	7.040E 00	3.978E-03	2.812E-03
1.910E 00	5.357E-02	1.334E-02	7.160E 00	2.035E-03	2.567E-03
1.970E 00	3.238E-02	1.498E-02	7.280E 00	2.509E-03	2.467E-03
2.030E 00	2.780E-02	1.431E-02	7.400E 00	3.348E-03	2.358E-03
2.090E 00	3.331E-02	1.498E-02	7.520E 00	2.405E-03	2.135E-03
2.150E 00	2.793E-02	1.513E-02	7.640E 00	7.413E-04	1.947E-03
2.210E 00	2.010E-02	1.413E-02	7.760E 00	1.963E-04	1.969E-03
2.270E 00	2.159E-02	1.159E-02	7.890E 00	1.167E-03	1.953E-03
2.330E 00	3.810E-02	1.280E-02	8.010E 00	2.501E-03	1.722E-03
2.390E 00	4.960E-02	1.065E-02	8.170E 00	3.374E-03	1.576E-03
2.450E 00	4.375E-02	1.281E-02	8.310E 00	3.298E-03	1.402E-03
2.510E 00	3.215E-02	1.263E-02	8.450E 00	2.013E-03	1.410E-03
2.570E 00	3.069E-02	1.134E-02	8.590E 00	7.741E-04	1.252E-03
2.630E 00	3.724E-02	1.095E-02	8.730E 00	5.558E-04	9.356E-04
2.690E 00	3.677E-02	1.034E-02	8.870E 00	9.901E-04	8.493E-04
2.750E 00	2.545E-02	1.091E-02	9.010E 00	9.630E-04	9.054E-04
2.810E 00	1.574E-02	1.098E-02	9.150E 00	5.198E-04	7.503E-04
2.870E 00	1.557E-02	1.002E-02	9.300E 00	2.000E-04	6.085E-04
2.930E 00	1.884E-02	9.367E-03	9.460E 00	3.876E-04	6.179E-04
2.990E 00	1.664E-02	9.594E-03	9.620E 00	7.268E-04	5.222E-04
3.050E 00	1.162E-02	1.027E-02	9.780E 00	8.506E-04	3.584E-04
3.110E 00	1.384E-02	9.465E-03	9.940E 00	6.735E-04	3.050E-04
3.180E 00	2.328E-02	8.436E-03	1.010E 01	4.027E-04	2.530E-04
3.260E 00	2.156E-02	7.741E-03	1.026E 01	1.905E-04	1.428E-04
3.340E 00	1.329E-02	8.021E-03	1.042E 01	6.247E-05	9.759E-05
3.420E 00	2.117E-02	7.367E-03	1.058E 01	1.728E-05	7.941E-05
3.500E 00	3.041E-02	6.849E-03			

INTEGRATED DATA

PHOTON ENERGY INTERVAL (MEV)	X-SECTION (B/SR)	ERROR (B/SR)
7.000E-01 - 8.000E-01	1.062E-02	4.406E-03
8.000E-01 - 1.000E 00	6.383E-02	7.827E-03
1.000E 00 - 1.200E 00	1.705E-02	5.919E-03
1.200E 00 - 1.400E 00	4.799E-02	6.080E-03
1.400E 00 - 1.600E 00	1.589E-02	4.261E-03
1.600E 00 - 1.800E 00	1.006E-02	3.787E-03
1.800E 00 - 2.000E 00	9.471E-03	3.076E-03
2.000E 00 - 2.500E 00	1.653E-02	6.633E-03
2.500E 00 - 3.000E 00	1.277E-02	5.313E-03
3.000E 00 - 3.500E 00	9.293E-03	4.183E-03
3.500E 00 - 4.000E 00	8.736E-03	3.388E-03
4.000E 00 - 4.500E 00	2.337E-03	2.994E-03
4.500E 00 - 5.000E 00	6.011E-03	2.706E-03
5.000E 00 - 6.000E 00	4.261E-03	4.580E-03
6.000E 00 - 7.000E 00	5.330E-03	3.677E-03
7.000E 00 - 8.000E 00	1.970E-03	2.237E-03
8.000E 00 - 9.000E 00	1.857E-03	1.263E-03
9.000E 00 - 1.000E 01	5.871E-04	5.551E-04

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