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SUPPLEMENTAL APPROVAL SHEET

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PROJECT CGC-897 - TITLE I DESIGN FISSION PRODUCT STORAGE IN B-PLANT

To: H. P. Shaw From: J. B. Fecut

Document HW-69011, "Project CGC-897 - Title I Design, Fission Product Storage in B-Plant," by H. L. Caudill and L. L. Zahn, dated April 3, 1961, has been reviewed and is approved as the basis for Title II design by Chemical Processing Department design scope representatives noted below:

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# PROJECT CGC-897 - TITLE I DESIGN FISSION PRODUCT STORAGE IN B-PLANT

by

H. L. Caudill and L. L. Zahn, Jr.

April 3, 1961

Extraction Design and Development Facilities Engineering Operation CHEMICAL PROCESSING DEPARTMENT

This document has been reviewed and approved as a basis for project action.



By Authority of SEGYC RLOCG 4 6-18-9 Bv 21-93 Verified B

Submitted by:

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5/4/61

Date

Approved by:

Manager Facilities Engineering CHEMICAL PROCESSING DEPARTMENT

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Date



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## PROJECT CGC-897 - TITLE I DESIGN FISSION PRODUCT STORAGE IN B-PLANT



#### I. INTRODUCTION

A previous study, Reference 1, described proposed facilities at B-Plant which integrate future fission product and waste calcination activities. However, in the reactivation of B-Plant in accordance with this study, heavy expenditures, above budgeted funds, would be required at an early date for Phase I process changes coupled with general rehabilitation work and facilities for updating of radiological control. Since waste calcination activities in B-Plant are not scheduled until Fiscal Year 1966, the expense of B-Plant rehabilitation items would be borne solely by the Fission Product Program.

This report provides the Title I design of Phase I fission product facilities at B-Plant which can be provided with minimum capital expenditures. The facility described in this report accomplishes the overall processing objectives of the Reference 1 Facility, namely the recovery and storage of crude strontium-90 and rare-earth concentrates, although certain B-Plant improvements are deferred to later phases of the Fission Product and Waste Calcination Programs. This report is considered an addendum to the Reference 1 Engineering Study and the reader is referred to the former study for further information on the overall reactivation of B-Plant.

#### II. SUMMARY

Strontium-90 and rare-earth concentrates can be recovered from Purex IWW solution, processed and stored in B-Plant, using only 8 of the 40 cells in B-Plant to effect savings in Phase I construction costs.

The Title I design of the Phase I facilities presented in this report is based on an operating sequence which consists of: 1) performing initial recovery steps at Purex; 2) transferring the crude concentrate to B-Plant via the CR Vault, with use of both existing and new direct-buried lines; 3) further concentrating both strontium and rare-earths by precipitation and boildown; 4) storing the concentrates; 5) discharging liquid wastes to the B-Plant Waste Tank Farms; 6) discharging process off-gas through a new process vent system and through the existing sand filter to the stack.

Eight B-Plant cells would be reactivated. This work would comprise relocation and modification of existing vessels, construction of new jumpers, plus aqueous makeup and instrumentation revisions. Modifications would also be made to canyon ventilation and to the Hot Pipe Trench for Sections 3 through 8 inclusive. Other B-Plant improvements include new shielded decklevel sampling equipment, an SWP lobby, a locker room, maintenance shop, building telephone system and miscellaneous reactivation items.

1. HW-67666 - "Engineering Study - Strontium-90 and Rare-Earth Processing In B-Plant," by HL Caudill, JR LaRiviere, and LR Michels, dated January 17, 1961.



The resulting Phase I facilities in the 221-B Building will have the capability to process the entire Purex IWW stream for isotope recovery. At a Purex 3.6 capacity factor, about 1 megacurie per month of strontium-90 and associated rare-earths containing about 4 megacuries promethium-147 per month can be recovered and stored in B-Plant with operations conducted on a 5-day, 3-shift basis. However, operability of the 221-B Building facilities as herein described is limited by the heat dissipation potential of the non-boiling tanks of the 241-B Waste Tank Farm complex, tentatively established at about 3 to 4 million BTU/hour. Since waste fission products discharged from the 221-B Building, including cerium-144 and other high heat producing isotopes, represent a heat input into the tank farm equivalent to about 620,000 BTU/hour during a one month operating period, the cerium rare-earth separation steps can only be performed in the 221-B Building for a total of about 3 months. During this 3-month period, however, about 10 megacuries of promethium-147 and 3 megacuries of strontium-90 can be recovered and stored separately. Beyond the above 3-month period, strontium-90 can be recovered at the Purex plant at a rate of approximately 0.6 megacuries per month and transferred to B-Plant for concentration and storage, with the cerium rare-earth fraction discharged to the boiling tanks of the Purex 241-A Tank Farm. On this basis, the 22,000 gallon storage capacity to be furnished in the 221-B Building will provide a total inventory of 12 megacuries of strontium-90 and a rare-earth fraction containing about 10 megacuries of prometium-147. Market requirements for promethium-147 beyond 11 megacuries would dictate the construction of new underground lines to route 221-B Building cerium wastes back to the Purex Tank Farm. These lines, described in Reference 1, are not presently incorporated into the Title I design.

The estimated project cost of Phase I facilities in accordance with this Title I design is \$1,400,000. Beneficial use can be obtained in about 14 months after authorization.

Flexibility has been provided in the facilities herein described for future conversion to the ultimate "Phase I" facility of Reference 1 in which equipment is more stragetically located in the canyon with respect to future Phase II and III fission product operations and waste calcination.

Facilities described in this report make use of existing direct-buried lines between the Purex Tank Farm and 244-CR Vault, between the 241-C-151 Diversion Box and B-Plant and between B-Plant and B-Tank Farms for transfer of high-level fission product streams. These lines will be pressure-tested at the start of the Title II design and rechecked for line integrity at regular intervals during operation. Further, strict solution inventory control will be exercised in operation of transfer tanks at Purex, the CR Vault and B-Plant for added assurance of line integrity.

Major alternates to the Phase I facility described in this report are discussed in Reference 1.



## III. PHASE I PROCESS DESCRIPTION AND DESIGN FLOW BASIS

The Phase I process is designed to recover strontium-90 and rare-earths from the Purex IWW stream and to provide segregated storage of both as crude fission product concentrates for aging. After an aging period to permit decay of the short half-life fission products, the concentrated solution will be processed in future Phase II and III facilities for the separation and purification of the desired products.

As mentioned previously, two separate processes are employed for operation of integrated Purex Head-End and 221-B Building facilities. Descriptions of these processes are as follows:

## A. Process I - Strontium-90 and Rare Earth Recovery at B-Plant

During the aforementioned 3 month period during which about 3 megacuries of strontium-90 and megacuries of promethium-147 are recovered and stored in the 221-B Building, concurrent with buildup of a 2 million BTU/hour heat inventory in the B-Tank Farms, the following process is employed:

As shown in Flow Diagram SK-2-18549 and the Reference 2 Chemical Flowsheet, processing is performed in the following sequence: 1) Purex Plant processing; 2) transfer of Purex Plant concentrates to B-Plant; 3) precipitation steps in B-Plant - lead removal, strontium-rare-earth separation, and cerium removal; 4) concentration by evaporation and 5) storage of concentrated solution.

At Purex, lead nitrate and sodium sulfate solutions are added to the IWW stream, following pH adjustment, to "carry" the desired fission products. The resulting sulfate precipitate containing the fission products is separated by centrifugation and held in the centrifuge bowl. Supernatant waste is transferred to the 241-A Waste Storage Tank Farm. A metathesis of the sulfate precipitate is then made in the centrifuge bowl by the addition of sodium hydroxide and sodium carbonate. The resulting precipitate is dissolved and transferred to the 244-CR Vault. The 244-CR Vault functions as an intermediate storage area and pumping station to B-Plant.

A batch-type process, compatible with existing bismuth phosphate equipment, is used in B-Plant to recover, concentrate, and store the desired fission products. The precipitation operations are performed in a standard precipitation section (Section 6). Lead removal is accomplished by precipitating the desired fission products as carbonates, leaving most of the lead in solution. After centrifugation the supernatant is transferred to the Waste Receiver Tank. The carbonate cake containing most of the fission products is dissolved. The resulting solution is treated with oxalic acid for separation of strontium and iron from the rare-earths.

<sup>2.</sup> HW-67728 - "B-Plant Fission Product Flowsheet - Part 1," by SJ Beard and BF Judson, dated January 12, 1961.





Rare-earths are precipitated as the oxalates. Most of the strontium and iron remain in solution. After centrifugation, the supernatant containing the crude strontium product is concentrated and stored. The rare-earths oxalate cake is dissolved and the resulting solution is treated with sodium acetate and hydrogen peroxide for separation of cerium, as a peroxyacetate precipitate from the rare-earths. After the removal of the peroxyacetate cake by centrifugation, the supernatant, containing the rareearths, is concentrated and stored. The cerium cake is dissolved and transferred to the Waste Receiver Tank.

The process waste, collected in B-Plant, is sampled, neutralized if necessary, and transferred to the 241-B Tank Farm complex for storage.

Under Process I, both Purex and 221-B Plant facilities have sufficient capacity to process the entire quantity of IWW solution generated at Purex at the 3.6 capacity factor rate. Process I recoveries for strontium-90 and promethium-147 isotopes are estimated at 80 and 75 per cent, respectively. However, processing capabilities and recoveries are highly dependent on the ability to clean out tanks and samplers between succeeding process operations. Development work now in progress on the Interim Strontium-90 Program will provide improved technology for B-Plant operations.

Since large amounts of heat are generated by the process solutions, coils and agitators are provided in each tank where cooling is needed. The maximum cooling potential of coils can only be obtained by complete submergence of the coils. Since the coil surfaces are not completely covered during certain steps under the present flowsheet, close inventory control or greater dilution must be employed in certain critical tanks for safe heat dissipation.

For the 3-month period of operations under Process I, a flowsheet volume of 120,000 gallons of waste will be added to the B-Tank Farms. Requirements for flushing of process equipment between steps might increase this volume to about 250,000 gallons. This volume can easily be accommodated in the B-Farms since, for example, about 2,750,000 gallons capacity remains in the 241-B Farm alone. However, during the 3-month period, 221-B Building wastes must be distributed into waste tanks in a judicious manner to prevent boiling in the tanks.

#### B. <u>Process II - Strontium-90 Recovery at Purex with Concentration and</u> Storage at B-Plant

On completion of operations under Process I, B-Plant and Purex head-end equipment are operated under Process II for a 15-month period for recovery and storage of additional strontium-90. Process II is in essence the Interim Strontium-90 Process performed currently in Purex head-end equipment. In current operations, all steps shown in the Flow Diagram and the Reference 2 Flowsheet are performed at Purex, with the exception of the peroxyacetate precipitation step which separates cerium from rare-earths. In current Purex operations, the supernate from the oxalate step, which contains dilute



strontium-90, is concentrated by chemical precipitation to permit storage in the CR Vault. Purex equipment limitations currently allow the processing of only 30 per cent of the total IWW stream for strontium-90 recovery with total rare-earths and cerium discharged to waste.

In Process II, the dilute strontium-90 supernate from the oxalate step is transferred to B-Plant for boildown concentration and storage. Under these conditions, Purex equipment can process about 60 per cent of the IWW stream, by virtue of elimination of the chemical precipitation step for strontium concentration.

During the 15-month period during which Process II operations are conducted, an estimated total volume of about 400,000 gallons of waste will be discharged to the 241-B Tank Farm complex. Since boildown concentration is the only step planned at B-Plant for Process II operations, the above 400,000 gallon waste batch will not add significantly to the Process I heat inventory at the B-Farms.

#### IV. REQUIRED PHASE I ADDITIONS AND MODIFICATIONS

The following additions and modifications are required for Phase I activities:

A. Underground Routings and Waste Disposal

Underground lines which serve for routing of high-level fission product streams between Purex and B-Plant facilities are described in the following drawings:

Drawing No.

Title

SK-2-18636	General Layout - Outside Lines
SK-2-18637	Plot Plan - Outside Lines
SK-2-18646	Hydraulic Flow Diagram, 202-A to 221-B

Briefly, fission product concentrates are transferred from Purex to B-Plant via the 244-CR Vault. The 244-CR Vault, which is the low-point in the system, serves as an intermediate pump station and gravity drain for about a mile of 3-inch line leading to B-Plant. In general, existing lines are used wherever possible to minimize capital expenditures.

Fission product concentrates are jetted from Purex head-end vessels through a 3-inch encased line to the 241-A-152 Diversion Box at the Purex Tank Farm. From the Purex Tank Farm, solution is routed to either the 002 or 003 Tanks in the 244-CR Vault, through an existing 3-inch direct-buried line and existing jumpers, as described in Drawing SK-2-18646. The 002 and 003 Tanks serve not only as pump tanks but as intermediate Phase I storage tanks for material processed through the carbonate metathesis step. These tanks, which have existing pumps, agitators, cooling coils, weight-factor instrumentation and sampling facilities are currently being used for concentrate storage under the Interim Strontium-90 Program. The two tanks have a combined storage capacity of about 25,000 gallons.



From TK-002 or 003, solution is pumped through about 500 feet of new 3-inch direct buried line to the 241-C-151 Diversion Box. This new line is tied into the C-151 Box and the 003 Vault through existing stubs. From the 241-C-151 Box, solution is routed through about 4,000 feet of 3-inch direct-buried line to the 241-B-154 Box and then through 1,700 feet of 3-1/2-inch encased line to the 241-BX-154 Box. Fission product concentrates then enter the 221-B Building at Section 8.

Liquid wastes from B-Plant, which leave the canyon at Section 5, are jetted through direct-buried lines to the 241-B Tank Farm, via the 241-B-154, B-152 and B-153 Diversion Boxes. Wastes can also be routed to the BX and BY Tank Farms for added flexibility.

Spare nozzles have been retained in the OO2 and OO3 Tank Vaults for future tie-in of new encased lines between the Purex 241-A-152 Diversion Box and B-Plant, through use of the 241-ER-151 Diversion Box. These future routings are described in Drawings SK-2-3409 through SK-2-3413 of Reference 1.

The routings adopted in the present Title I design (SK-2-18637, 18636, and 18646) have several shortcomings:

- The existing direct-buried line between the 241-A-152 Diversion Box and TK-003 in the CR Vault has several trapped sections. Following the jetting of concentrates from Purex, the jet should be allowed to gas (as in present operation) to remove concentrates from the line,
- 2. The 4,000 foot section of direct-buried line between the 241-C-151 Box and the 241-B-154 Box is 16 years old and was in the ground for about 2 years before cathodic protection was provided. This line and others involved in solution transfers will be hydrostatically tested before the start of Title II design and at regular intervals during operation. Further, during normal operation, solution inventory control will be invoked for Purex, CR Vault and B-Plant tanks involved in transfers, for early detection of possible line leakage.
- 3. Process wastes from B-Plant will normally discharge to the 241-B Tank Farm. Although about 3,250,000 gallons of various wastes are presently stored in B-Tank Farm, about 2,750,000 gallons net storage capacity is available for neutralized wastes from B-Plant fission product operations. As discussed earlier, prudence must be exercised in the distribution of fission product wastes in the tank farms for safe tank operation.

The storage of B-Plant wastes in the B-Tank Farms is considered to be of a temporary nature and the Reference 1 underground routings should be provided when construction funds become available.



## B. Receiving and Precipitation Equipment

The following drawings define B-Plant receiving and precipitation equipment and the required modifications:

Drawing No.

 $\mathtt{Title}$ 

SK-2-18633	Cell Arrangement, Section 4 and 5, Cells 8 and 9
<b>SK-2-1863</b> 2	Cell Arrangement, Section 6, Cells 11 and 12
<b>SK-2-1861</b> 4	Hot Pipe Trench Schematic
<b>SK-2-1863</b> 4	Typical Coil and Agitator Arrangement

The feed solution is received in an existing 5000 gallon tank, 5-TK-9, as shown on SK-2-18633. This tank, located in Cell 9, is used for receiving, sampling, feed adjustment, and storage. Approximately 4000 gallon feed batches are transferred from the 244-CR Vault, to eliminate the need for more frequent transfers and feed adjustment. Processing within B-Plant is confined to batch operations with feed batches of approximately 370 gallons per run.

To permit the use of the existing tank, 5-TK-9, for receiving and storage service, an agitator and cooling coil as shown on SK-2-18634, are provided. New jumpers and lines needed are shown on Drawings SK-2-18633 and 18614.

The equipment used for performing the three precipitation and separation steps is shown on SK-2-18633. One "standard section," Section 6, is used for the lead removal, the strontium rare-earth separation, and the cerium removal steps. A standard section is composed of two cells, one containing a precipitator and a solution tank and the other a centrifuge and a catch tank. The solution tank in Cell 11 and the centrifuge in Cell 12 are to be relocated from Cell 27 and Cell 28, Section 14.

As shown on the Process Flow Diagram SK-2-18549, a 370 gallon batch is routed to the precipitator for the lead removal step. A carbonate precipitate is formed by the addition of sodium hydroxide and sodium carbonate to the feed solution. The resulting carbonate precipitate contains the fission products and leaves most of the lead in solution. The mixture is transferred to the centrifuge at the rate of 7 gallon per minute. During centrifugation, the supernatant containing the lead flows by gravity to the catch tank and is subsequently jetted to the process waste collection tank. After separation, the cake is washed with sodium carbonate and water, dissolved with dilute nitric acid and jetted to the solution tank. The dissolved solution collected from three batch-runs is transferred back to the precipitator tank for a rare-earth precipitation. By the addition of oxalic acid to the accumulated solution from the lead removal step, the rare-earths are precipitated from solution as the oxalates. Strontium and iron remain in solution. After centrifugation and washing of the oxalate precipitate, the supernatant containing the strontium and iron is transferred to the concentrator in Cell 5. The oxalate precipitate containing the rare-earths is dissolved





with HNO<sub>3</sub> in the centrifuge and transferred back to the precipitator tank. The foregoing oxalate precipitation step is performed primarily for the removal of iron which appears to interfere with the cerium removal step.

The dissolved oxalate cake solution is treated with sodium acetate and hydrogen peroxide in the precipitator to separate cerium as a peroxyacetate precipitate. Rare-earths are not precipitated under these conditions and remain in solution. After centrifugation the supernatant containing the rare-earths is transferred to the Cell 5 Concentrator and the cerium cake is dissolved and transferred to the Waste Receiver.

Since the process on which the flowsheet is based is currently under development, considerable flexibility has been provided in terms of alternate routings for recycling solutions between tanks. Such alternate routings must be employed with caution to avoid exceeding the heat transfer potential of vessel cooling coils.

## C. Concentration and Storage of Fission Product Concentrates

Concentration of the dilute solution from the preceeding precipitation steps is necessary to provide a maximum inventory of strontium-90 and rare-earths in minimum storage volume. Concentration and storage equipment are depicted in the following drawings:

Drawing No.	Title
SK-2-18631	Cell Arrangement, Section 3, Cell 5
SK-2-18623	Cell Arrangement, Section 3, Cell 6
SK-2-18617	Cell Arrangement, Section 4, Cell 7
SK-2-18633	Cell Arrangement, Section 4, Cell 8

As shown on SK-2-18631, equipment consists of a concentrator, two condensers, and two condensate receiver tanks. An existing dissolver is modified for service as a concentrator. The coil in this dissolver has sufficient surface to permit the transfer of approximately 1 million BTU per hour with steam condensing at 29 psia.

Two existing 750 gallon tanks will be modified and relocated in Cell 5 one for use as a process condensate receiver, the other as a steam condensate receiver. Existing dissolver columns will be modified for use as condensers by provision of new shells for the existing coil sections of the columns.

Supernatant is received from Section 6 on a batch basis. Approximately 1700 gallons of dilute solution is processed per batch-run for the desired eight-fold volume reduction. Since the volume of the concentrator is 1500 gallons, concentrator feed is added incrementally under controlled conditions from the precipitator section. Concentration equipment will not be timecontrolling in the overall cycle and considerable flexibility is incorporated into Cell 5 facilities.





Non-condensable gas from Condenser (3-E-10-1) is discharged through the condensate receiver into the Vessel Vent Header. Since the pressure drop through the vapor line and condenser is negligible at the flow rates anticipated, vessel vent vacuum maintains the desired pressure differential between the concentrator and the condensate receiver tank. The process condensate collected in 3-TK-4-1 is transferred to the Waste Receiver Tank.

The steam condensate from the concentrator is routed to condenser 3-E-10-2 and collected in 3-TK-4-2 for sampling. The collection and sampling of steam condensate allows detection of any leak in the concentrator coil and provides containment in case of coil failure. The probability of coil failure in the concentrator is greater than for the other vessels. After sampling, the condensate is routed to waste via 3-TK-4-1.

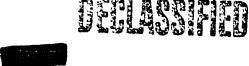
Concentrates containing strontium-90 and the rare-earths are jetted from the concentrator to storage tanks.

Storage of concentrate is required for decay of the undesirable short-lived fission products. Approximately 18 months aging of concentrates is required from reactor discharge before final product specifications can be met. Aging requirements are a function of the final product designs, however, and may be subject to change with developments in this field.

Concentrate storage tanks are located in Cells 6, 7, and 8 of Sections 3 and 4. The storage tanks are located as follows: Two 4500 gallon tanks in Cell 6, as shown on SK-2-18623, one 4500 gallon tank in Cell 7 along with the vessel vent equipment, as shown on SK-2-18617, two 4500 gallon tanks in Cell 8 as shown on SK-2-18633. Three of these tanks are to be transferred from U-Plant. The two existing tanks in Cell 8 will remain in position. Coils and agitators shown on SK-2-18634 are to be installed in the tanks in Cell 7 and 8. Two of the tanks to be moved from U-Plant and relocated in Cell 6, have cooling coils of high capacity.

The fission product concentrates will be stored at a specific volume of 2 gallons per ton of uranium processed through the Purex Plant. With the Purex Plant operating at a 3.6 capacity factor, approximately 3 months will be required to fill one tank. Careful scheduling of concentrates through the storage tankage is necessary because of changing heat release with decay time and because of varying coil capacities. When a tank is full, its contents are transferred as a batch to the next tank, which has either equal or reduced cooling capacity. The series operation for filling the tanks permits the use of the two tanks having the highest cooling potential for the first 6 months storage and eliminates possibilities of cross contamination between various aged solutions. For an 80 per cent overall recovery of Purex available strontium-90, each full storage tank will contain approximately 3.2 megacuries of strontium-90. After the initial 18 month period for filling and aging the initial material, 1 megacurie of strontium-90 per month would be available for purification.





Solutions are to be stored in the acid state at a bulk temperature of approximately  $160^{\circ}$  F. Cooling coils are instrumented to permit calculation of heat inventory in each tank, for purposes of scheduling batches through the system.

As shown on Drawings SK-2-18654, 18614, 18631, 18623, 18617, and 18633 alternate routings are provided for flexibility within the storage system and will allow segregated storage of the strontium-90 and rare-earth fractions; if so desired. Alternate routings are also provided to allow the contents of the various tanks to be moved in case of a coil failure.

# D. Waste Collection and Disposal

Phase I facilities are required in B-Plant for collection and disposal of the following type of waste: 1) process waste; 2) cell drainage; 3) steam condensate from process vessels; and 4) cooling water from process vessels. The process design is shown in the following drawings:

Drawing No.	Title
sk-2-18633	Cell Arrangement, Section 5, Cells 8 and 9
sk-2-18631	Cell Arrangement, Section 3, Cell 5
sk-2-18637	Plot Plan, Outside Lines

The process waste is collected in Cell 9, Section 5. As shown on Drawing SK-2-18633, the tank used for waste collection is an existing tank; 5-TK-6, retained in position. Although this tank was used for waste service under former B-Plant operations, it will require a new coil and agitator, as shown on SK-2-18634. The process waste is routed as follows: from Section 3 to Section 5, and from Section 6 to Section 5.

The cell drainage from all of the cells collects in Section 5, Cell 10, and is routed to 5-TK-6, Cell 9.

The steam condensate from the concentrator is routed to Condenser 3-E-10-2, collected in 3-TK-4-2 Section 3, and routed to 5-TK-6 Section 5, via 3-TK-4-1, Section 3.

The process waste is accumulated in Cell 9, Section 5, at an approximate flowsheet rate of 60 gallons per ton of uranium processed through the Purex Plant. Requirements for flushing precipitator section vessels between processing steps and for general decontamination might boost this value to 120 gallons per ton. The total accumulated waste is sampled, neutralized if necessary and routed to the 241-B Tank Farm complex. An alternate route is provided for rework of the waste if needed. Caution must be employed to prevent boiling of the waste solution within the B-Tank Farm storage tanks.

The water used for cooling the process vessels is discharged to the B-Plant Swamp via an existing 24-inch sewer line and retention basins. Discharged cooling water is monitored continuously for any activity which could be released through leaks in the process cooling coils.





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#### E. Vessel Vent System

Installation of a process vent system in B-Plant will provide positive control of off-gas from all processing equipment. Process vent systems are provided both at Purex and Redox to prevent pressurization of processing equipment and to decontaminate process off-gases and vapors prior to disposal to the ventilation filter. Previous operations in B-Plant permitted direct-venting of hot vessels directly to the cells. The resulting off-gases were then swept into the air tunnel and to the stack via the sand filter. The proposed Cell 7 Vessel Vent System shown in the following drawings will eliminate any direct venting of hot process equipment to the cells, and will provide separate filtration of all off-gases and vapors prior to discharge to the Canyon Ventilation System.

Drawing No.

SK-2-18529	Tank Modifications for Sealing Agitator Shafts
SK-2-18618	Vessel Vent System - Schematic Phase I
<b>SK-</b> 2-18620	Vessel Vent Off-Gas Heater
SK-2-18619	Vessel Vent Off-Gas Filter
<b>SK-2-18537</b>	Modifications of Centrifuge Discharge and
	Supernatant Tank Inlet Piping
SK-2-18617	Cell Arrangement, Section 4, Cell 7
SK-2-18530	Vessel Vent Header - Schematic Phase I, II,
	and III

The new vessel vent system is shown schematically in Drawing SK-2-18618. A 6-inch vessel vent header is installed along the bottom of the Hot Pipe Trench to service all the process cells from Cell 5 through Cell 12. Existing lines from the cells to the Hot Pipe Trench are utilized for all cells except Cell 7 which has two new 4-inch lines cored through from the Hot Pipe Trench.

The new vessel vent equipment located in Cell 7 is shown on SK-2-18617 and consists of a heater to vaporize any liquid present; a fiber-glass filter for removal of particulate material; an air-operated jet to provide 20 inches water suction on the header while handling approximately 1200 lb/hr of off-gas and vapors at the temperature of 250° F. The motive fluid for the jet is air at 100 psia, with an emergency steam back-up.

As shown on Drawing SK-2-18620 a new off-gas heater (4-E-21) is required. The heater has six heating elements, each rated at 12 KW, as per Drawing H-2-3422, and is capable of heating the off-gas stream sufficiently to protect the fiber-glass filter medium from moisture. Heating capacity will be available to heat off-gas to  $390^{\circ}$  F for silver-reactor service, if needed in the future.

A new filter vessel (4-F-22) with approximately 20 square feet cross-sectional area, is shown on Drawing SK-2-18619. Packing for the vessel consists of three layers of 115K Fiber-glass, packed to densities of 1.5, 3.0, and 9.0 lbs/ft<sup>3</sup> and depths of 18, 14, and 12-inches, respectively.





A new vessel vent condensate tank (4-TK-23) of 300 gallons capacity collects drainage from the vent header. Any condensate collected is routed to waste.

Allowance will be made for future installation of a vessel vent silverreactor, should iodine removal become necessary. The vessel vent equipment will be designed to allow remote removal and relocation to another cell if needed in the future.

To minimize air inleakage into the vessel vent system, unused tank nozzles are plugged and the agitator shafts are sealed in accordance with Drawing SK-2-18529. The centrifuge is vented via the supernatant tank by the addition of new nozzle connections and a new discharge line as shown on Drawing SK-2-18537.

## F. Hot Pipe Trench Modifications

Phase I revisions to the Hot Pipe Trench are defined in the following drawings:

Drawing No.

SK-2-18654 SK-2-18614 Sheets 1 and 2

Cell Allocation and Primary Routings Hot Fipe Trench Schematic

In general, modifications to the Hot Pipe Trench are confined to Sections 3 through 6, inclusive, except that the feed line for concentrates enters the trench at Section 8. A shielding barrier is provided across the trench between Cells 12 and 13, to permit future trench construction activities beyond Phase I. Piping stubs extending through the barrier are intended to provide desired Phase II routings plus the eventual Phase I routings of Reference 1.

#### G. Aqueous Make-up

As shown on Drawing SK-2-18576, existing tanks and routings are utilized where possible for chemical make-up and addition to process vessels. All chemical additions are made through existing scale tanks located in the Operating Gallery.

The following chemicals are needed for Phase I processing: 1) nitric acid; 2) sodium hydroxide; 3) sodium carbonate; 4) oxalic acid; 5) sodium acetate; 6) hydrogen peroxide; and 7) chemicals for demineralizer - undefined.

A new water demineralizer system is required to limit the addition of ions detriped al to future Phase II purification processes. The tank formerly used a. B-Plant for supplying sodium nitrate will be used for storage of demineralized water while the demineralizer is being regenerated.





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A new system for handling hydrogen peroxide is needed. The system adopted consists of a new combined head and storage tank, water jet, and related piping. As shown on Drawing SK-2-18576, evacuation of the system transfers peroxide from the drum to the storage tank. After transfer to the evacuated tank, the peroxide flows by gravity to the chemical make-up tank. The maximum suction lift required for emptying the drum is approximately 10 feet.

Sufficient floor space is available for storage of bulk chemicals. However, material handling equipment, such as a fork-lift truck, etc., will be required for AMU service. As Phase 1 process development work continues, chemical requirements may be subject to change. Sufficient flexibility is incorporated into the existing tankage and header systems so that no major problems are anticipated for future aqueous make-up.

# H. Instrumentation and Cold-Side Facilities

The primary instrument functions required for Phase I processing are defined on Drawings SK-2-18549, 18576, and 18618. Primary measuring elements and recording or indicating instruments presently installed will be tested and reconditioned where feasible. New services are needed for measurement of water flow to the cooling coils, steam flow to the concentrator and temperature of water discharged from the cooling coils. Provision will be made in Gallery Piping for alternate raw water, steam or air supply to process tank coils.

Jets to be used for transfer of process solutions in the Canyon Building are of standard 10, 30, and 75 gallon per minute capacity. The jets are controlled by means of existing gang-valves.

New jumpers required for instruments and jets are shown on Drawings SK-2-18617, 18631, 18623, 18633, and 18632. Cold-side piping and other requirements are also indicated on these drawings.

#### I. Sampling

Special equipment for the withdrawal of process samples and determination of solution pH at canyon deck-level is shown schematically in Drawing SK-2-18527. A description of this equipment is provided in Reference 1. Tentatively, it is planned to provide two portable Gilmont sampler units and one portable pH probe unit of the referenced design along with an electric fork-lift truck for Phase I operations. A special station will be required on the canyon deck for performance of maintenance and washdown functions on the above portable equipment.

## J. Ventilation

Reference 1 provides information on the status of ventilation equipment in the 221-B and 291-B Facilities. As a result of recent tests performed on this equipment, and described in Reference 1, extensive modification of ventilation facilities will be required. Drawing SK-2-18655 shows the





required modifications both for Phase I and Phase II. Phase I modifications are as follows:

- 1. New ventilation ports of 12 square feet cross sectional area will be provided into the ventilation tunnel for Cells 5 through 12, inclusive. Special temporary dust collection equipment may be required to control dust when the new holes are cored through.
- Sheet metal or other restrictors will be attached to cell covers to limit air flow to each operating cell at 3700 CFM during normal operations. Cell covers for all non-operating cells in Phase I, except Cell 40, will be sealed for Phase I operations.
- 3. Repairs will be made to the roof of the sand filter to seal detectable leaks.
- 4. The present emergency steam engine and exhaust fan will be replaced prior to the start of Phase I operations with a new steam turbine unit designed for the new Phase II ventilation system. This new emergency blower which will handle 37,500 CFM at 12-inches static pressure, will be relocated eventually to the Phase II facility.
- 5. A new 6 ft. x 8 ft. concrete stub will be provided in the exhaust tunnel for the new Phase II exhaust filter and fans.
- 6. The 224-B Building exhaust duct leading into the 221-B ventilation tunnel will be blocked off at the 224-B Building.
- 7. An extensive overhaul of 5 of the Carrier air supply units will be performed.
- 8. Present exhaust fan dampers will be repaired or replaced with low pressure-drop dampers.

# K. Emergency Raw Water Supply

An emergency raw water system shown on Drawing SK-2-18573 is provided to prevent bolling of solution in storage and process tanks in case of failure of raw water supply. Without cooling, full storage tanks would start to boil within one hour. Emergency repairs to existing underground lines between the 282-E Reservoir and B-Plant would, in general, require considerable in excess of one hour for restoration of service. On failure of the Export Water Line, Purex would have first call on available reservoir storage. Insufficient time would be available to jet full tanks at B-Plant to waste.

A new well is provided near the southwest corner of B-Plant for emergency raw water supply. A 200 gpm, 500 foot head, 25 brake horsepower pump, driven by a combustion engine would be required. Although a flow of 120 gallons per minute would suffice for Phase I emergency flow, the 200 gpm capacity should provide for emergency service for future Phase II and III operations, as





well as Phase I. The overall system includes a 4-inch tie-in, with check values, into the 10-inch raw water line and a new 3-inch line, routed to the 24-inch sewer, for routine standby testing of the pump and combustion engine. Present data from Chemical Effluents Technology indicate activity less than  $10^{-8}$  micro-curies per cubic centimeter in ground water at the site indicated for the new well.

#### L. Other Phase I Requirements

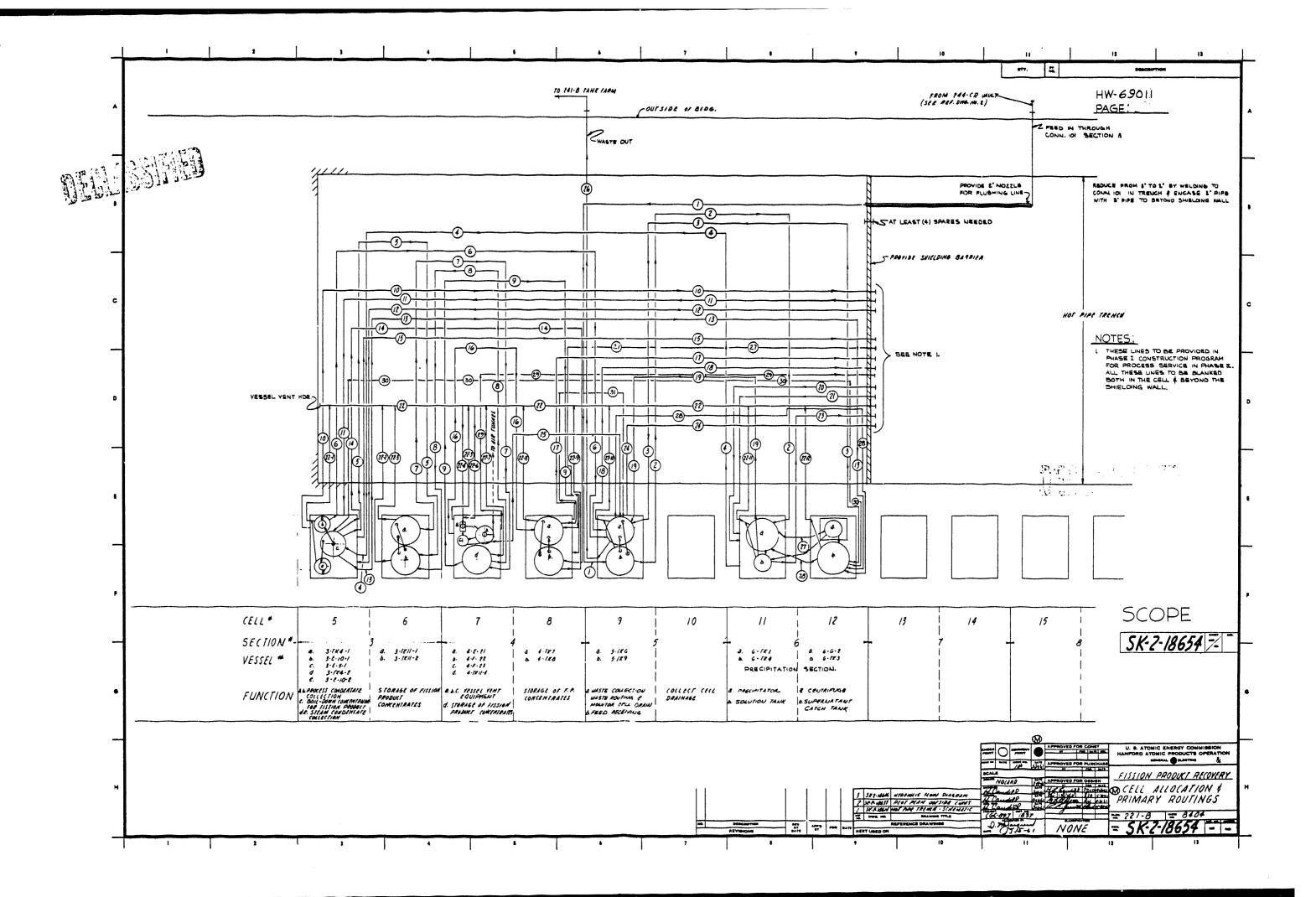
Other B-Plant modifications required for Phase I and not covered by scope drawings are listed as follows:

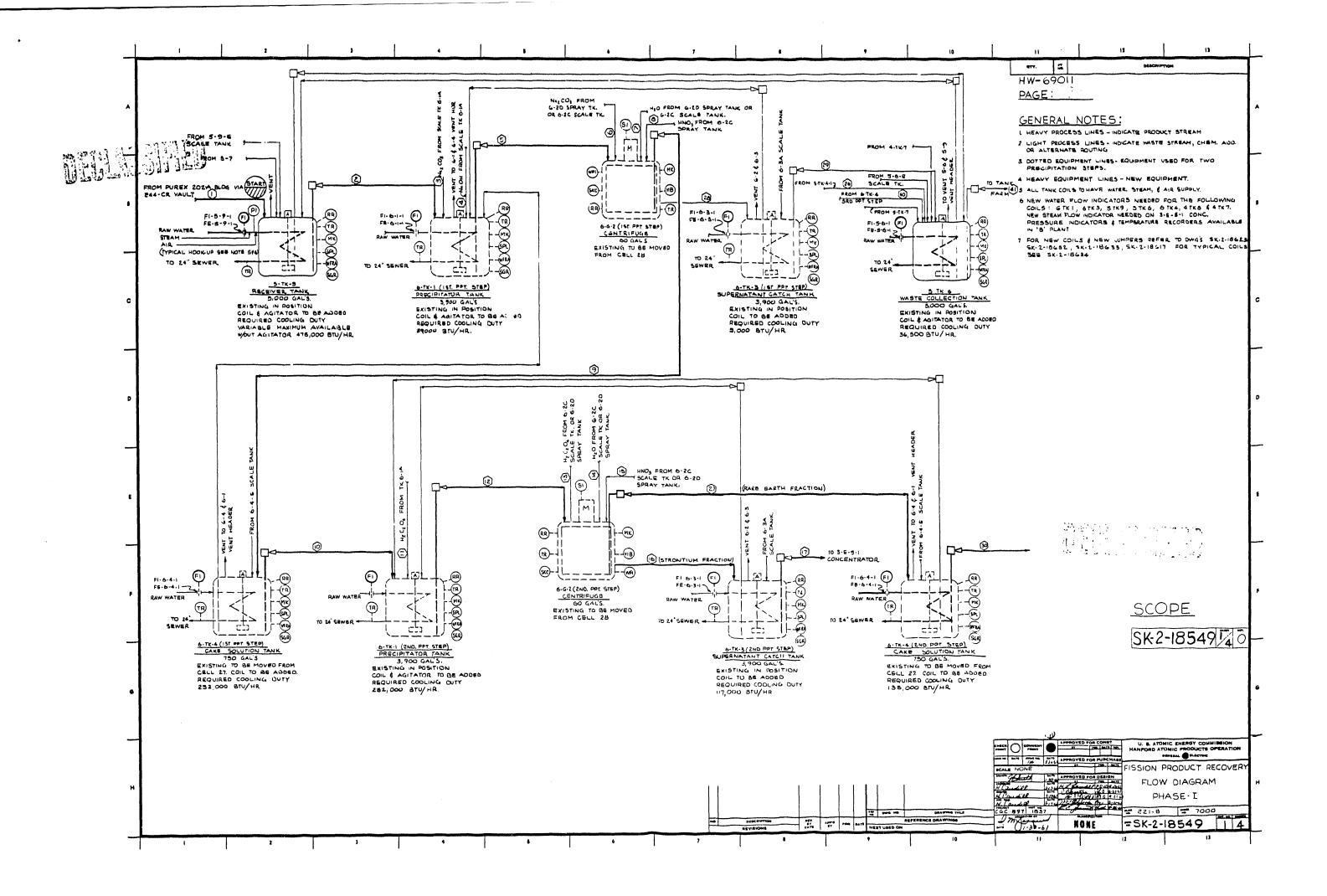
- 1. Mechanical Maintenance Shop
- 2. Instrument Maintenance Shop
- 3. Decontamination Room
- 4. SWP Lobby
- 5. SWP Locker Room
- 6. PAX Telephone System
- 7. Radiation Instruments
- 8. Fresh Air Supply
- 9. Essential Materials Handling Equipment
- 10. Area Barricades
- 11. Miscellaneous Requirements

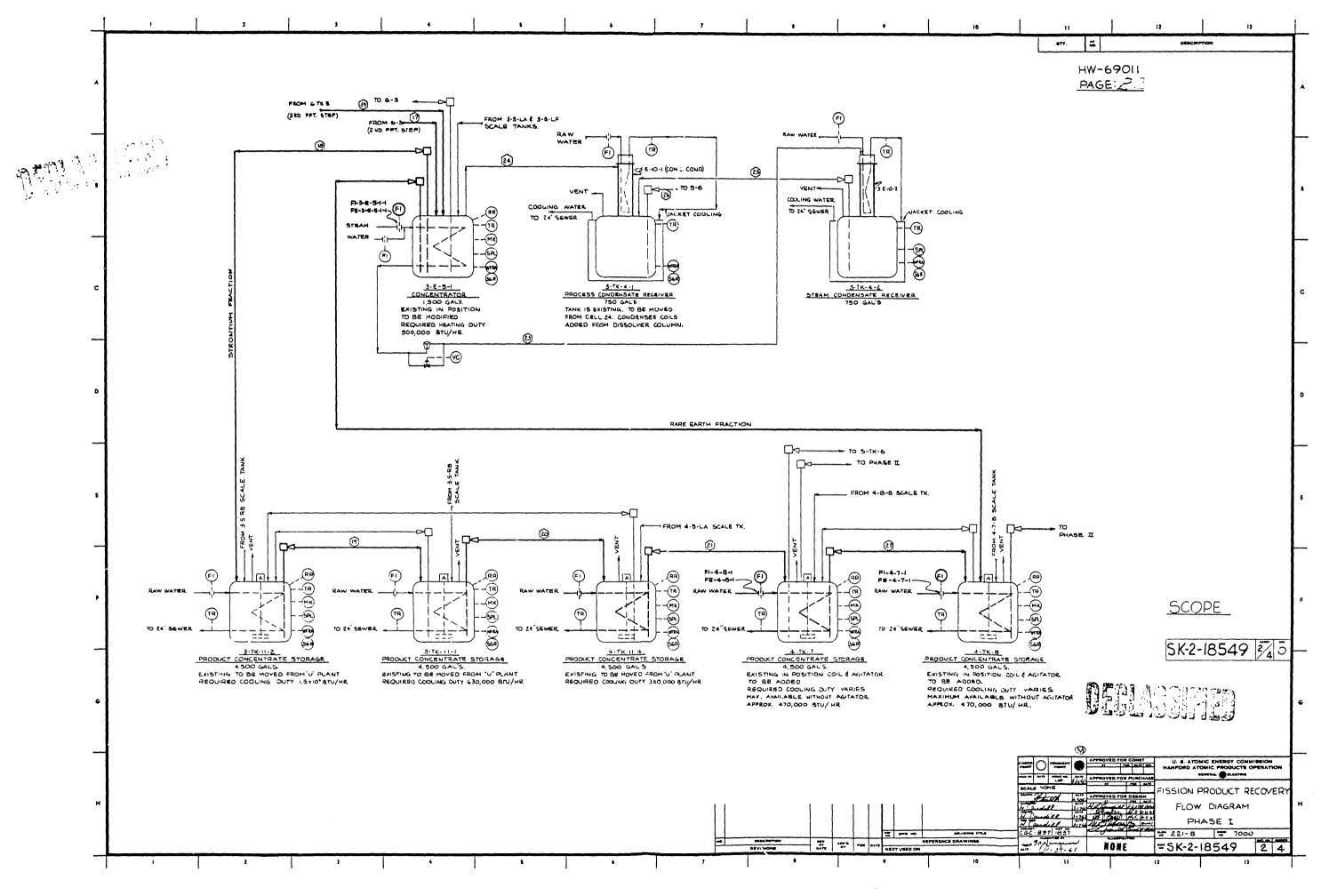
The above modifications are described in Reference 1. However, the exact nature of these modifications will be established during Title II design.

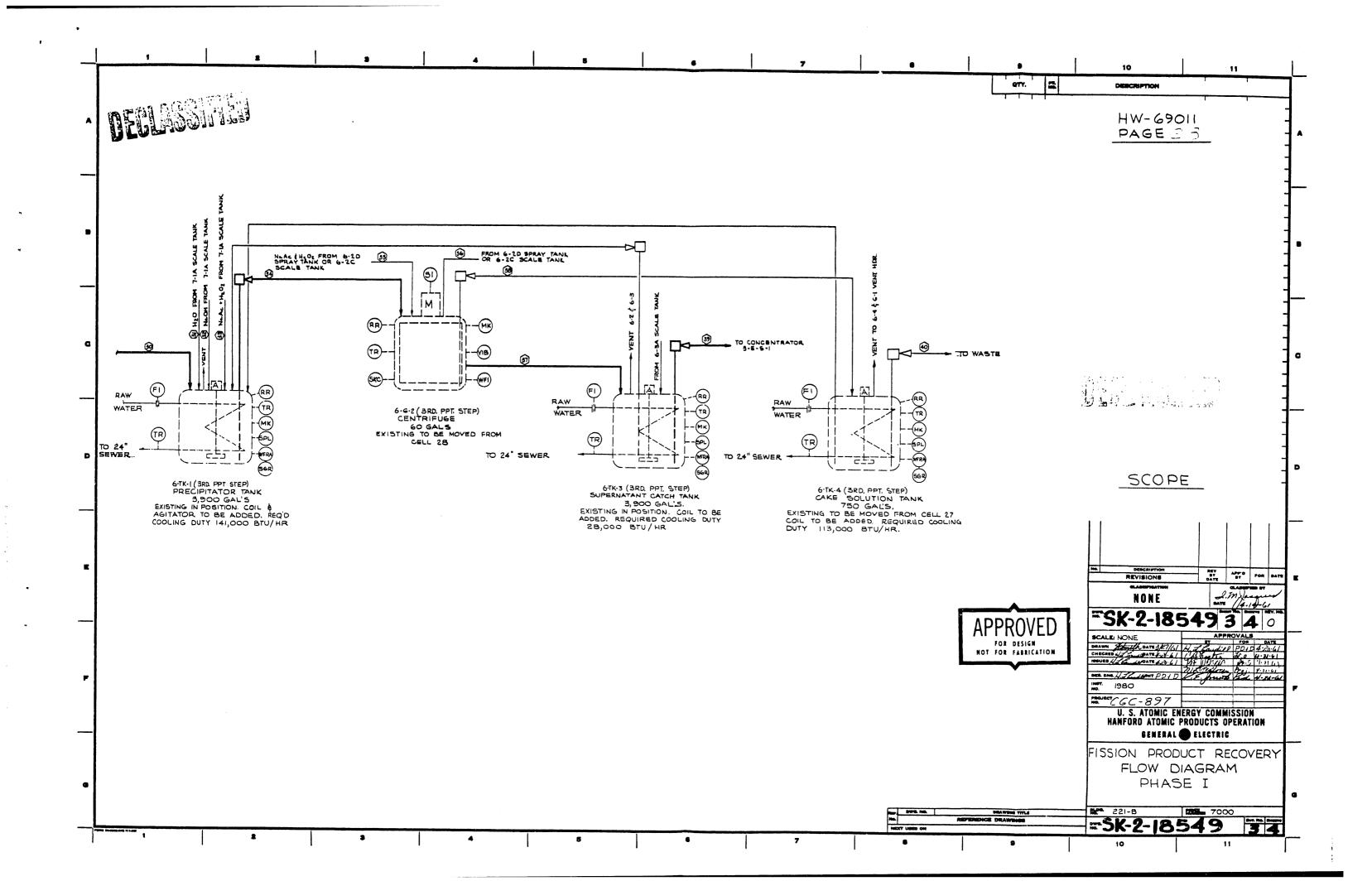
Extraction Design and Development Facilities Engineering CHEMICAL PROCESSING DEPARTMENT

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L BASED ON PUREX PROCESSING MATERIAL AGED 90 DAYS.

L TRE - TRI-VALENT RARE EARTH.

A BASED ON 600 MWD EXPOSURE MATERIAL PROCESSED

THROUGH THE PUREX PLANT,

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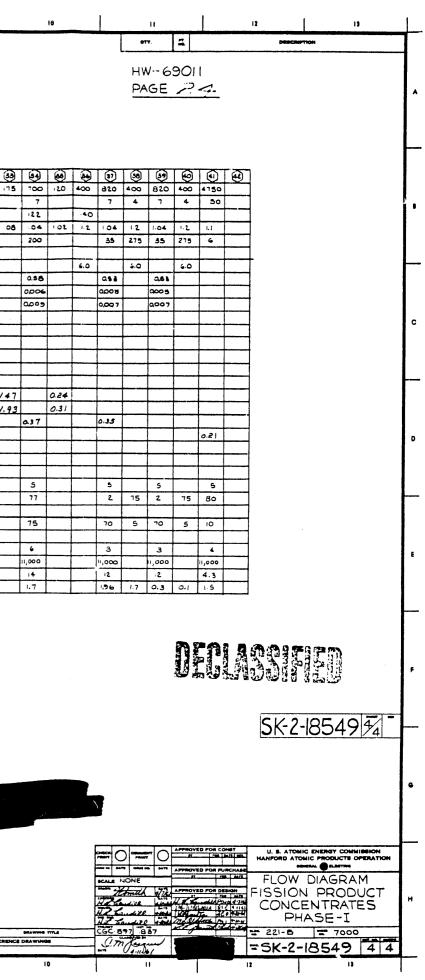
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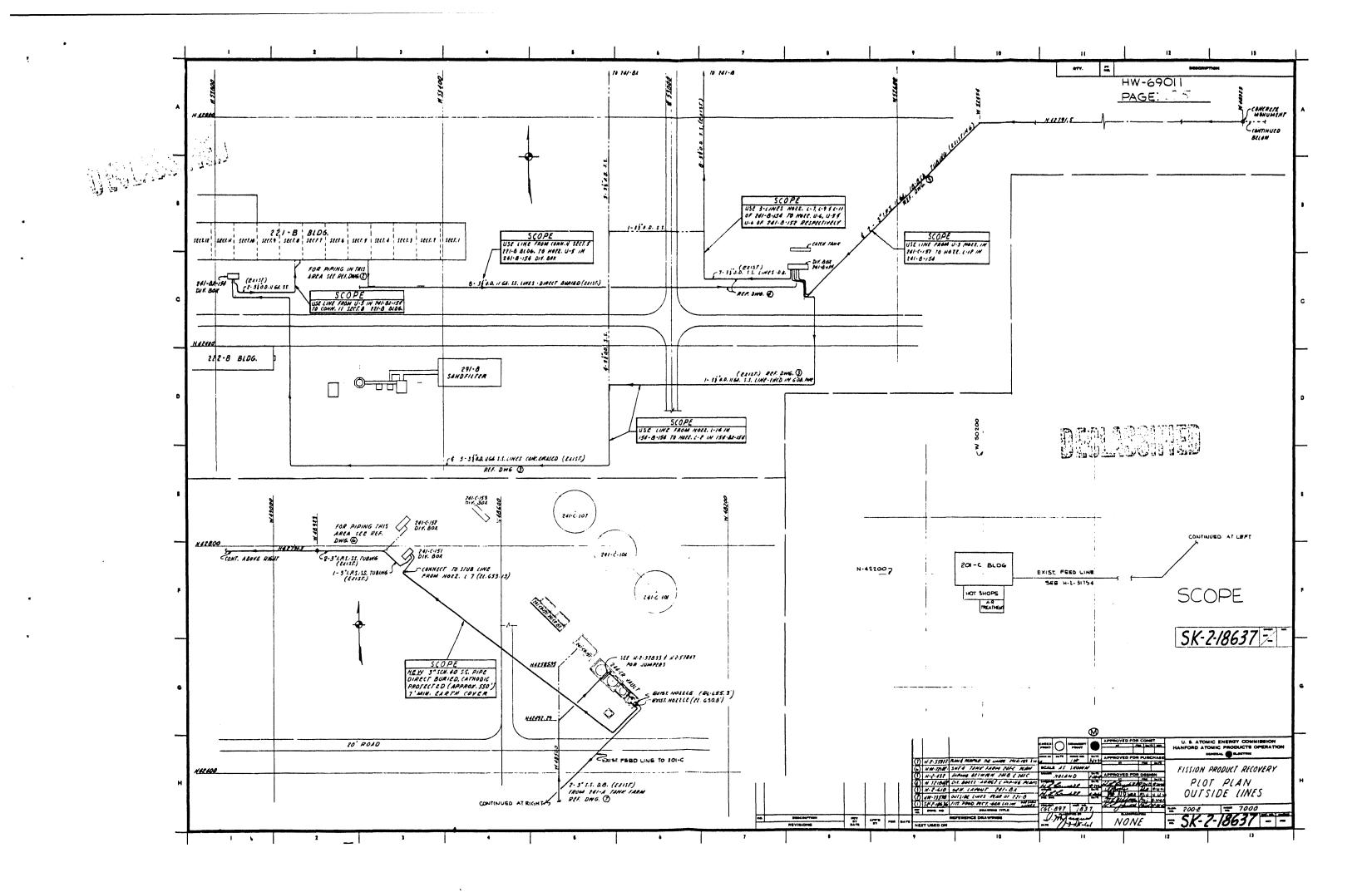
4 TOTAL WASTE ACCUMULATED PER BATCH DOES NOT INCLUDE CHEMICALS NEEDED FOR FLUSHING VESSELS

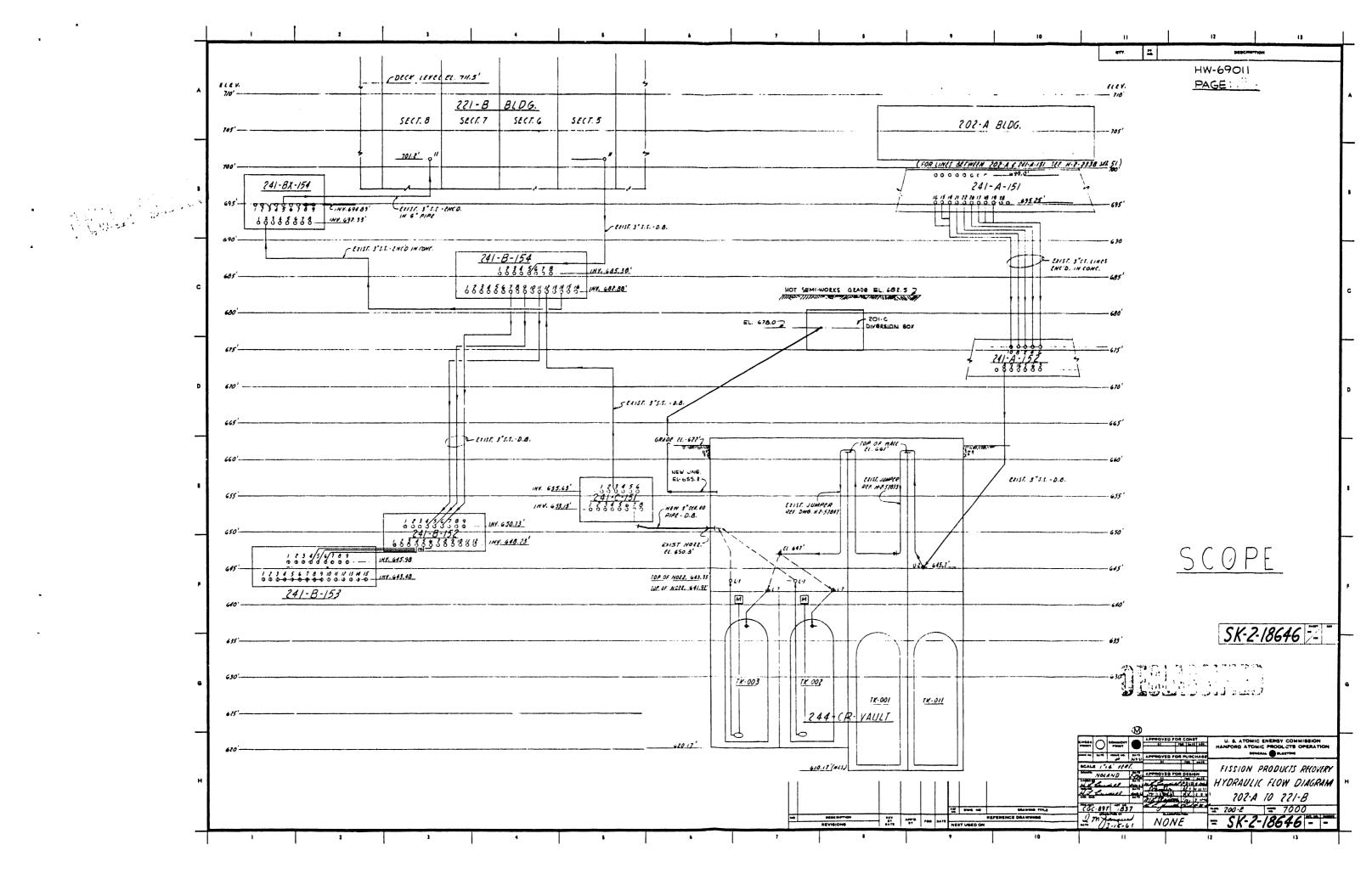
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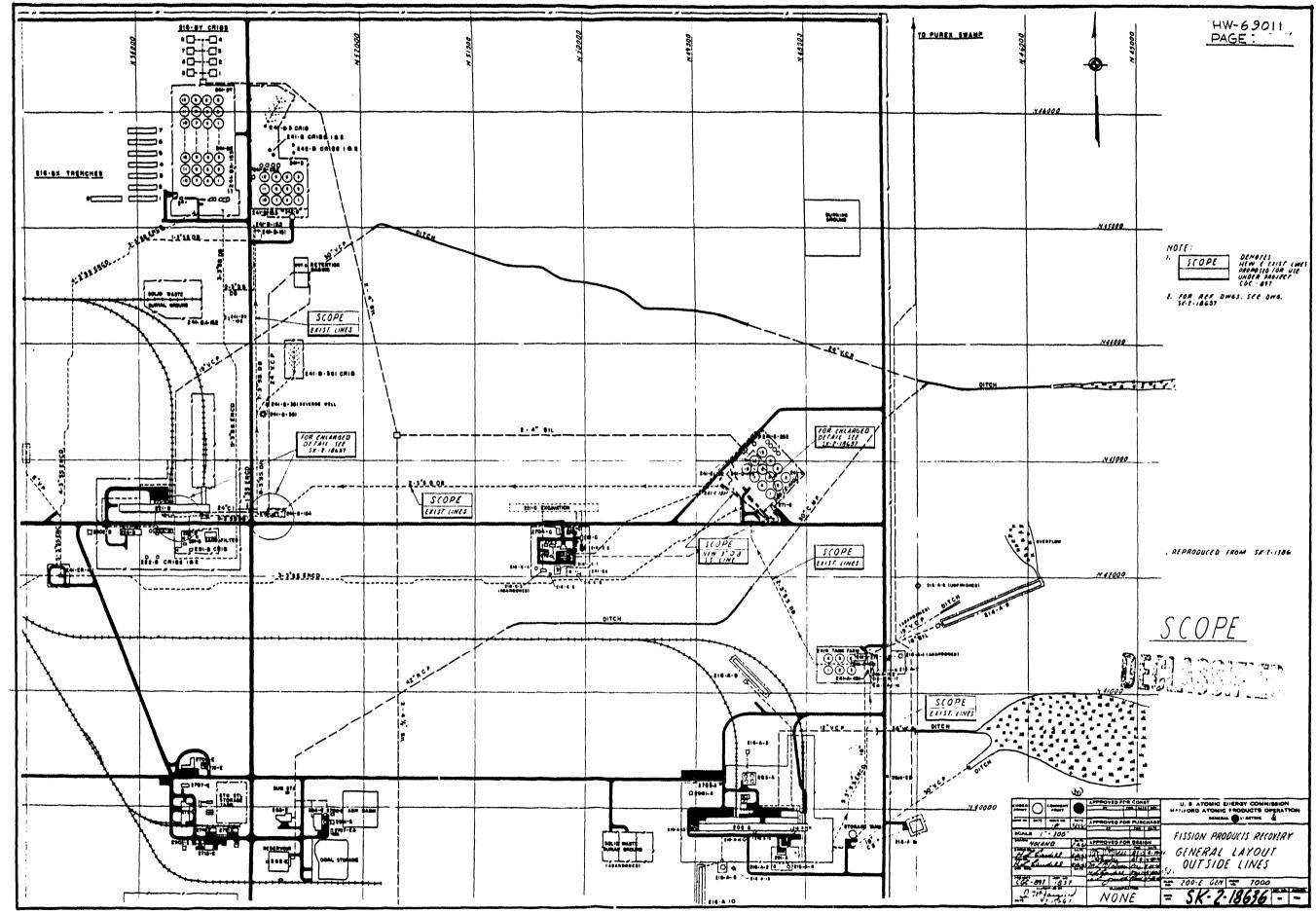
NO SERCENTION ATT ATT DE LAT REFE REVISIONE SIN ATT CE LAT REFE REVISIONE SIN ATT CE LAT REFE REVISIONE SIN ATT CE LAT REFE NEXT USED ON

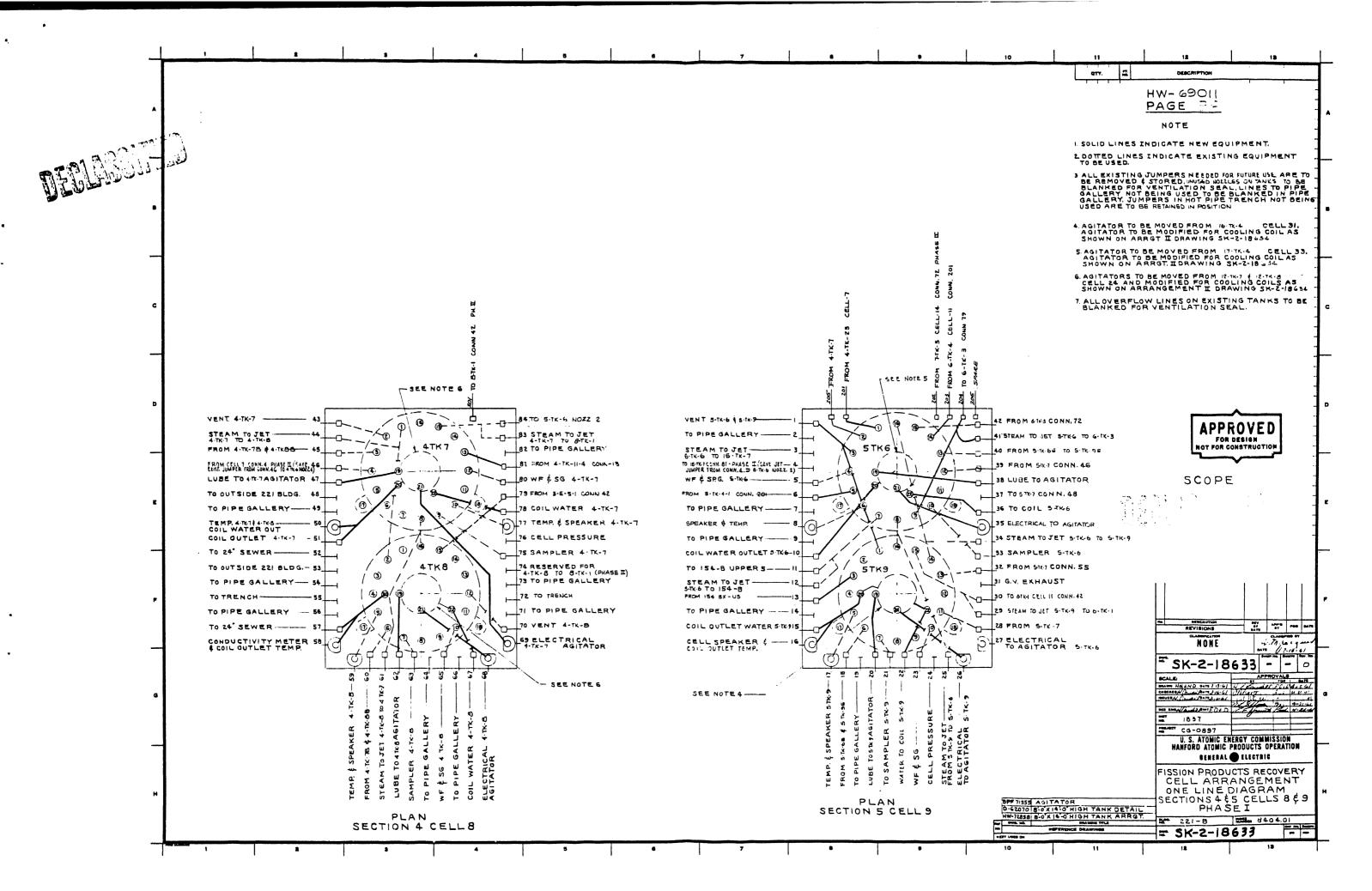




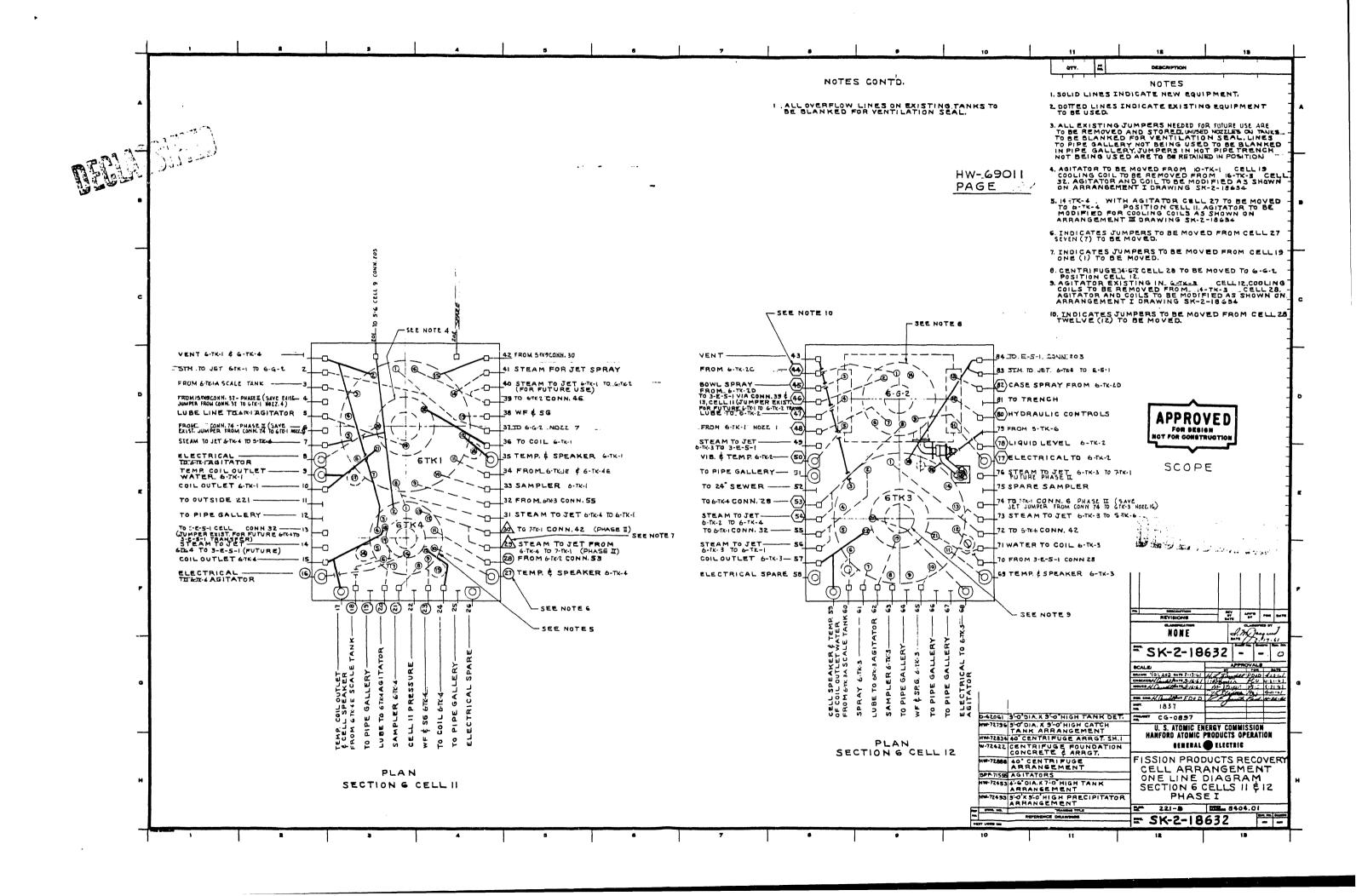








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August 8, 1961

To: Distribution From: H. L. Caudill

# PROJECT CGC-897

- References: (1) HW-69011, "Project CGC-897 Title I Design Fission Product Storage in B-Plant," by HL Caudill and LL Zahn, dated April 3, 1961.
  - (2) HW-69959, "Process and Detail Design Criteria - Heat Transfer Coil Installation Chemical Processing Department," by JB Fecht, dated June 16, 1961.

A copy of revised scope drawing SK-2-18634 is attached. This drawing should replace Page 30 of Reference 1 and become a part thereof.

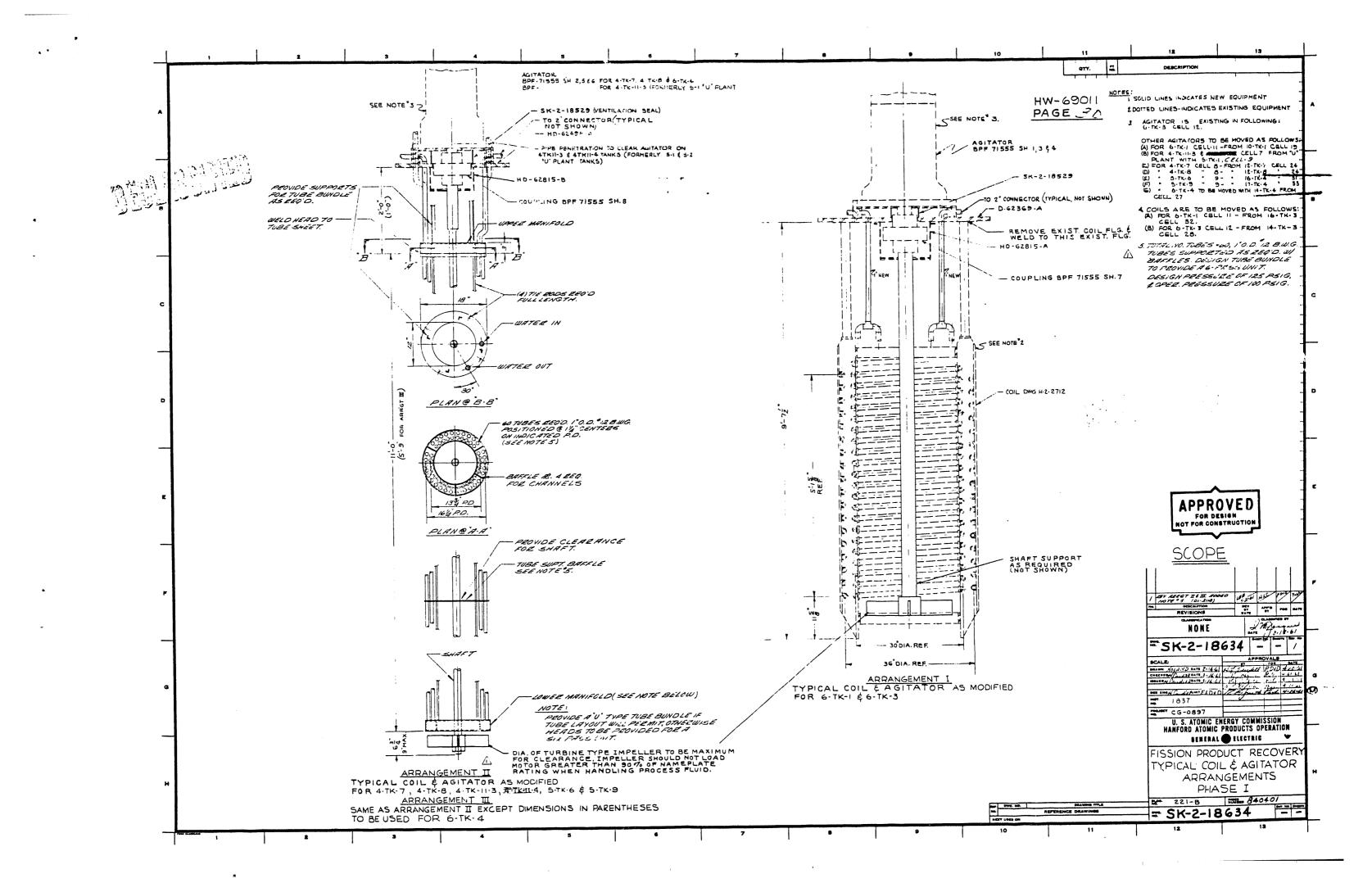
The typical coil and agitator arrangements II and III were revised as a result of the recently published coil design criteria, Reference 2, to provide a cooling system of greater integrity.

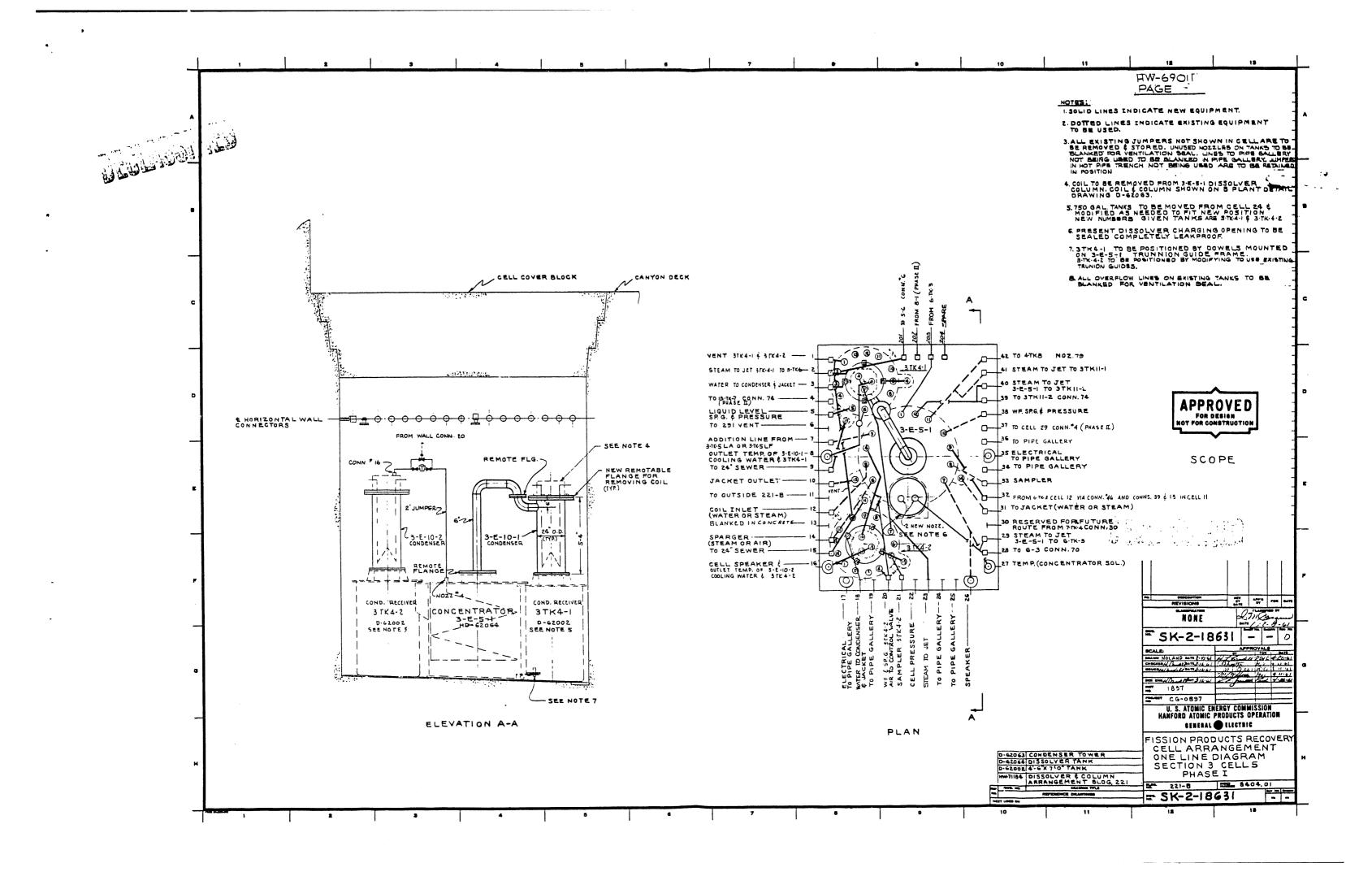
HL Caudill 11m

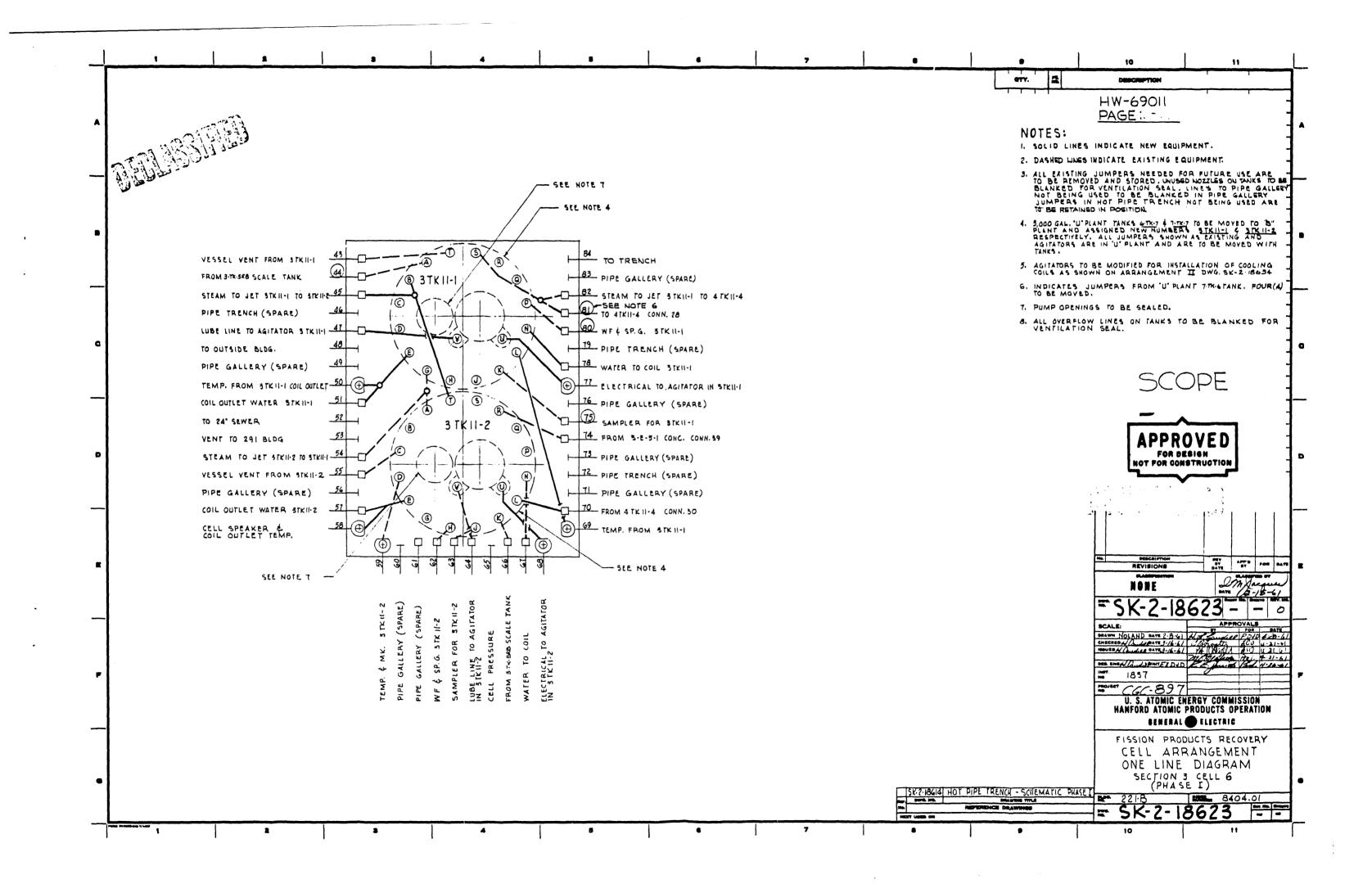
CR Ander LP Bupp GK Carpe HL Caudi VR Coope JB Fecht CB Foste RG Geier KG Grimm DR Gusta OF Hill ER Irish BF Judso TG LaFol JR LaRiv CA Lynei PR McMur	l son ette ere
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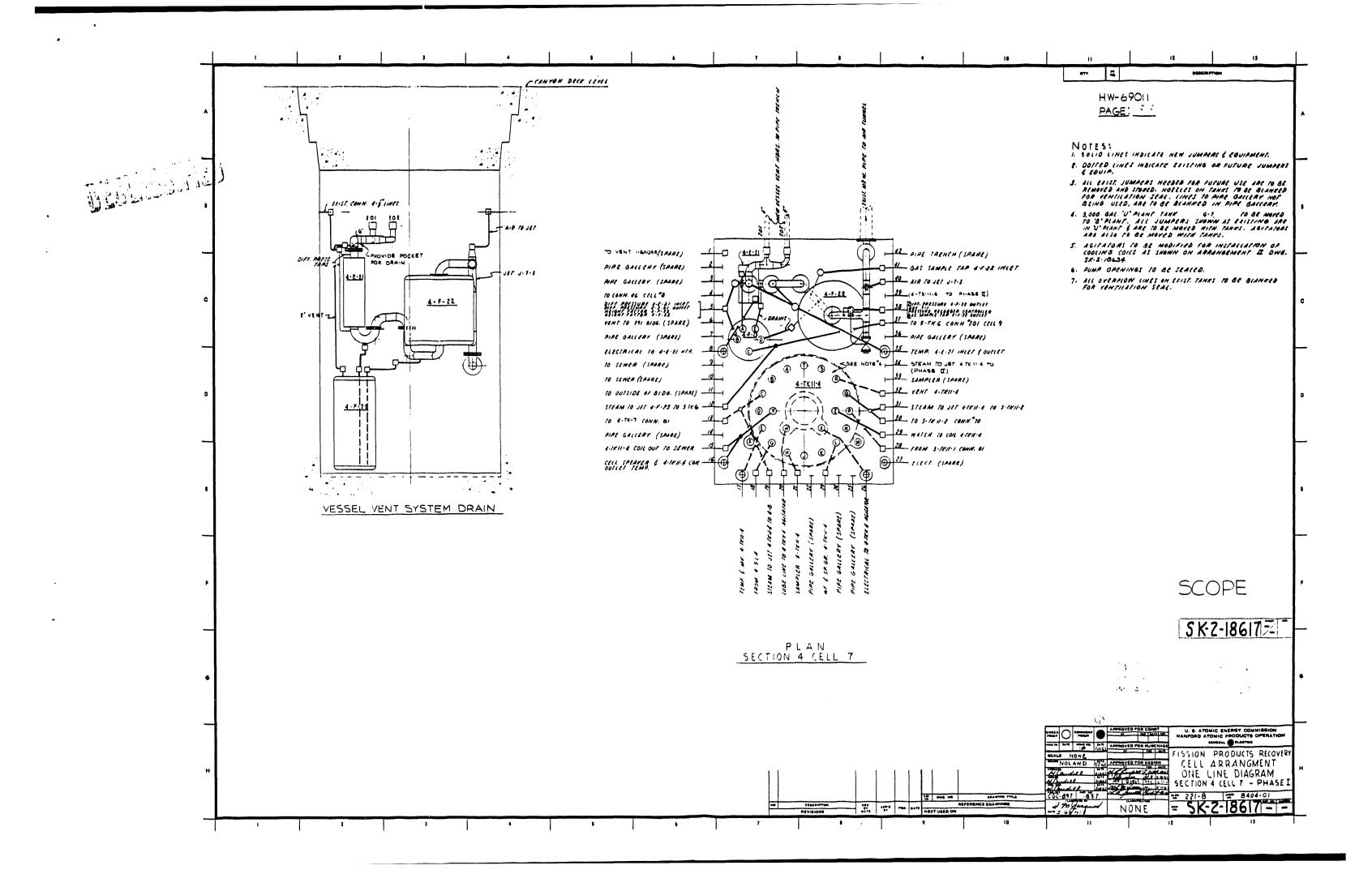
LR Michels ML Oldfather AM Platt PH Reinker HP Shaw SG Smolen WH Swift RE Tomlinson JH Warren LL Zahn Record Center 300 Files

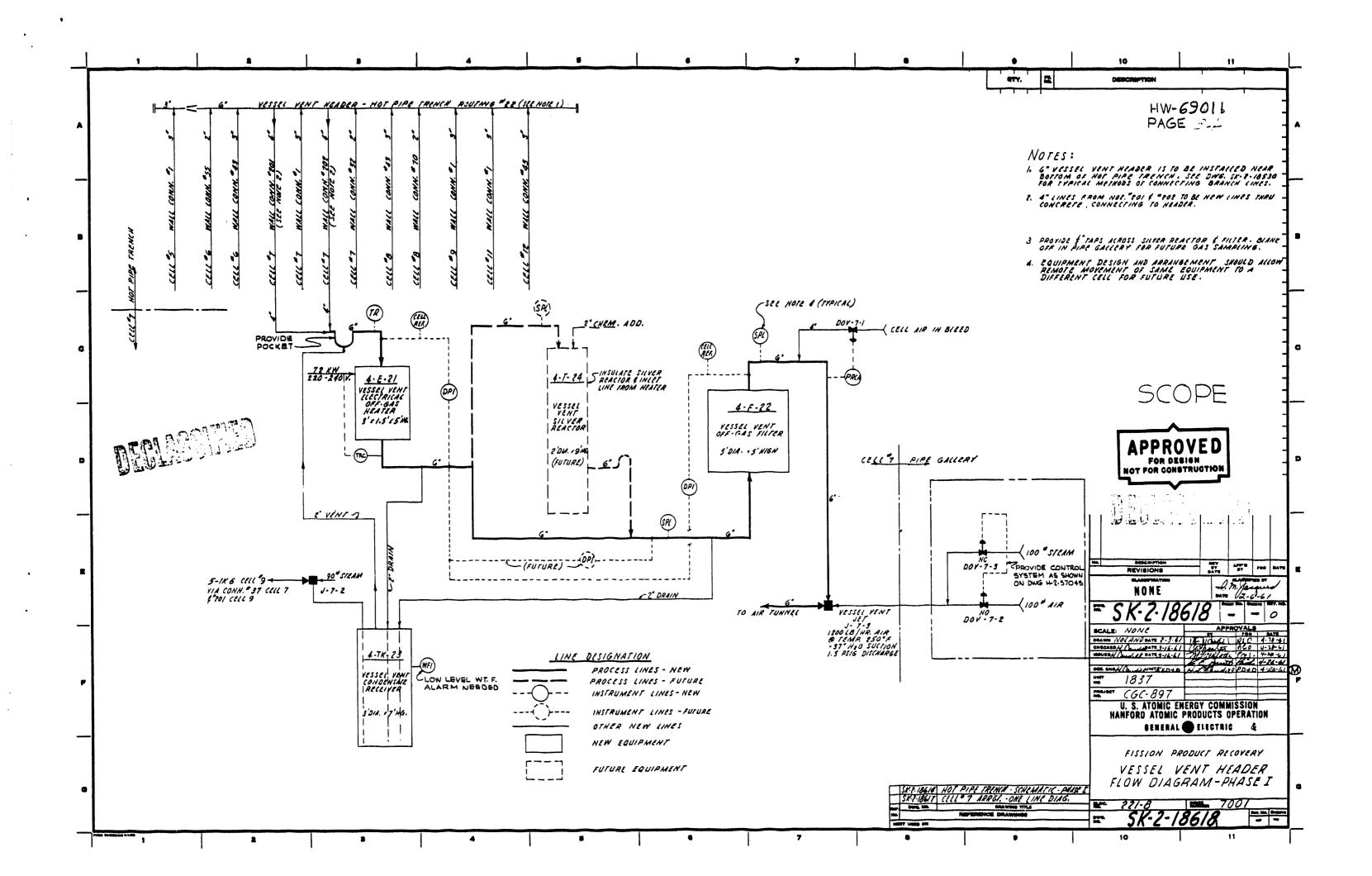
Extras (5)

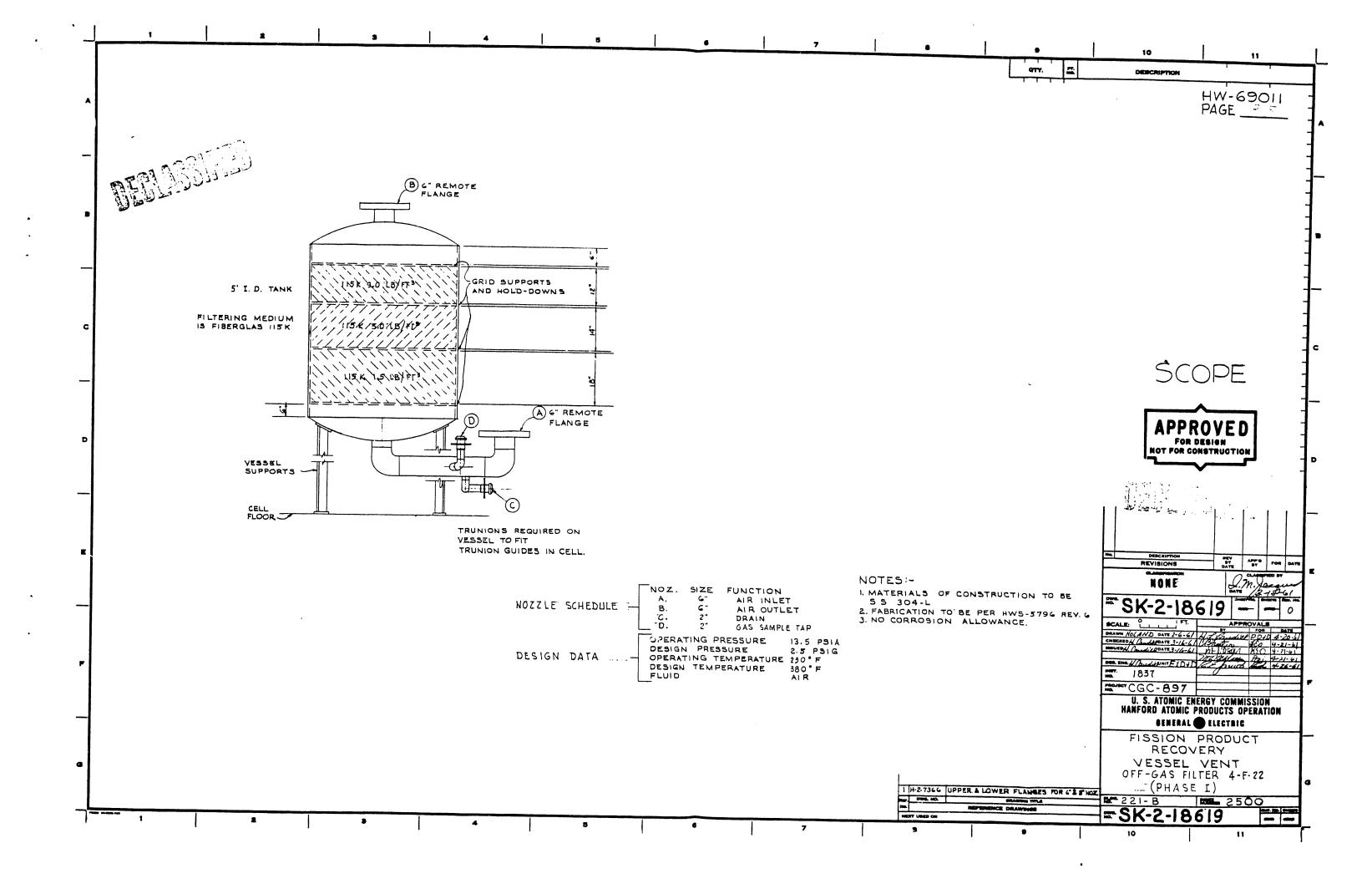


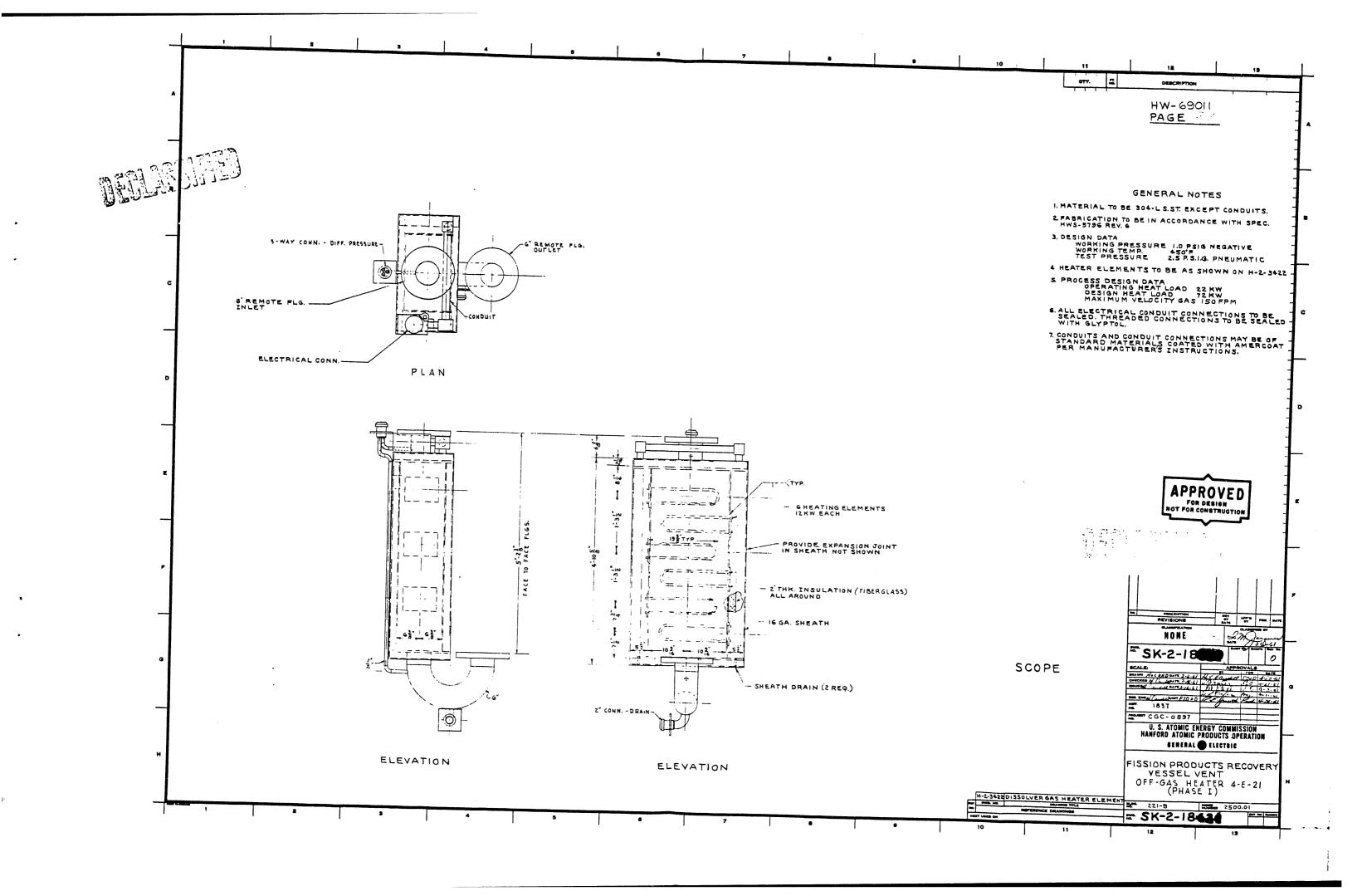


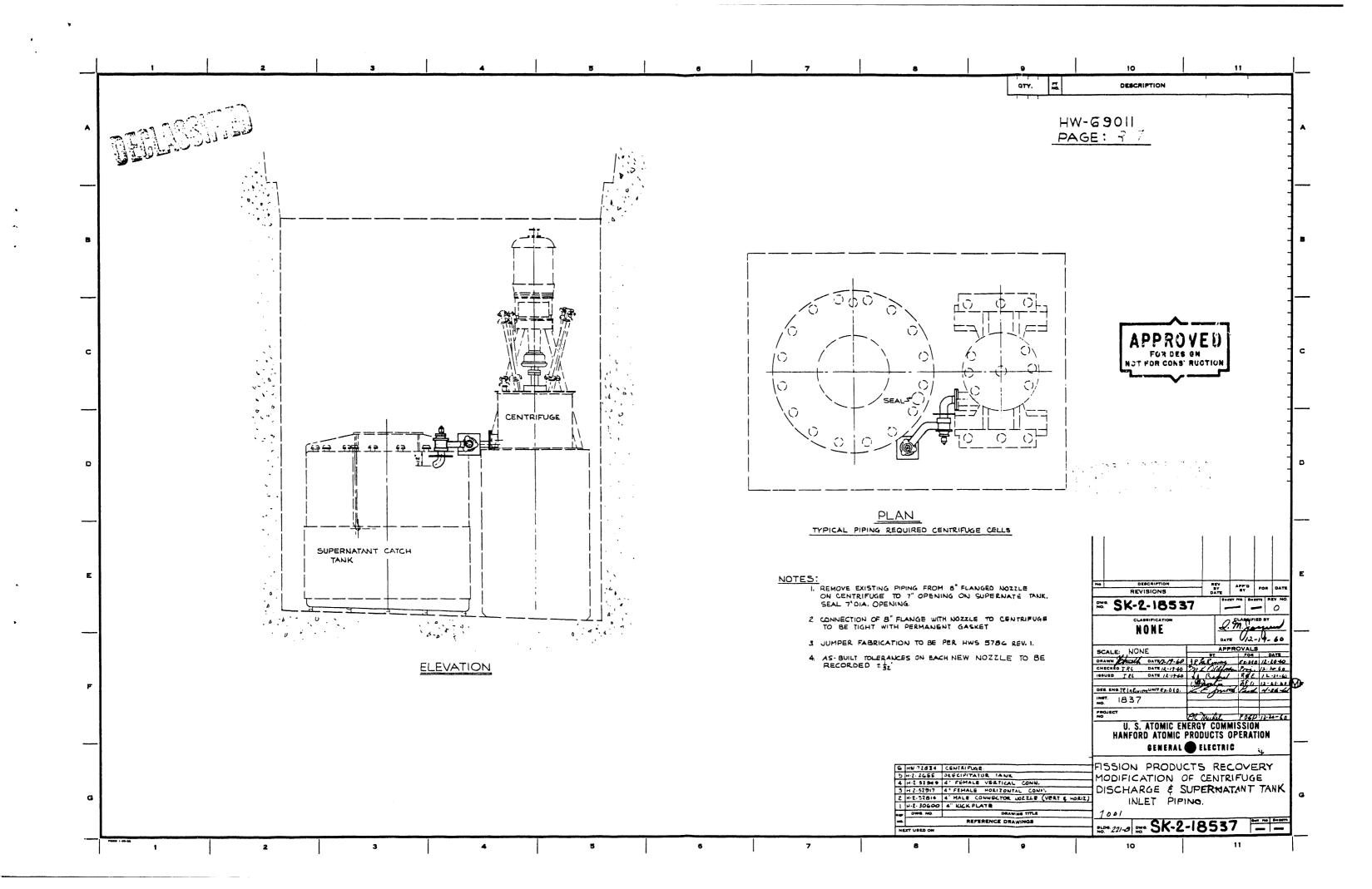


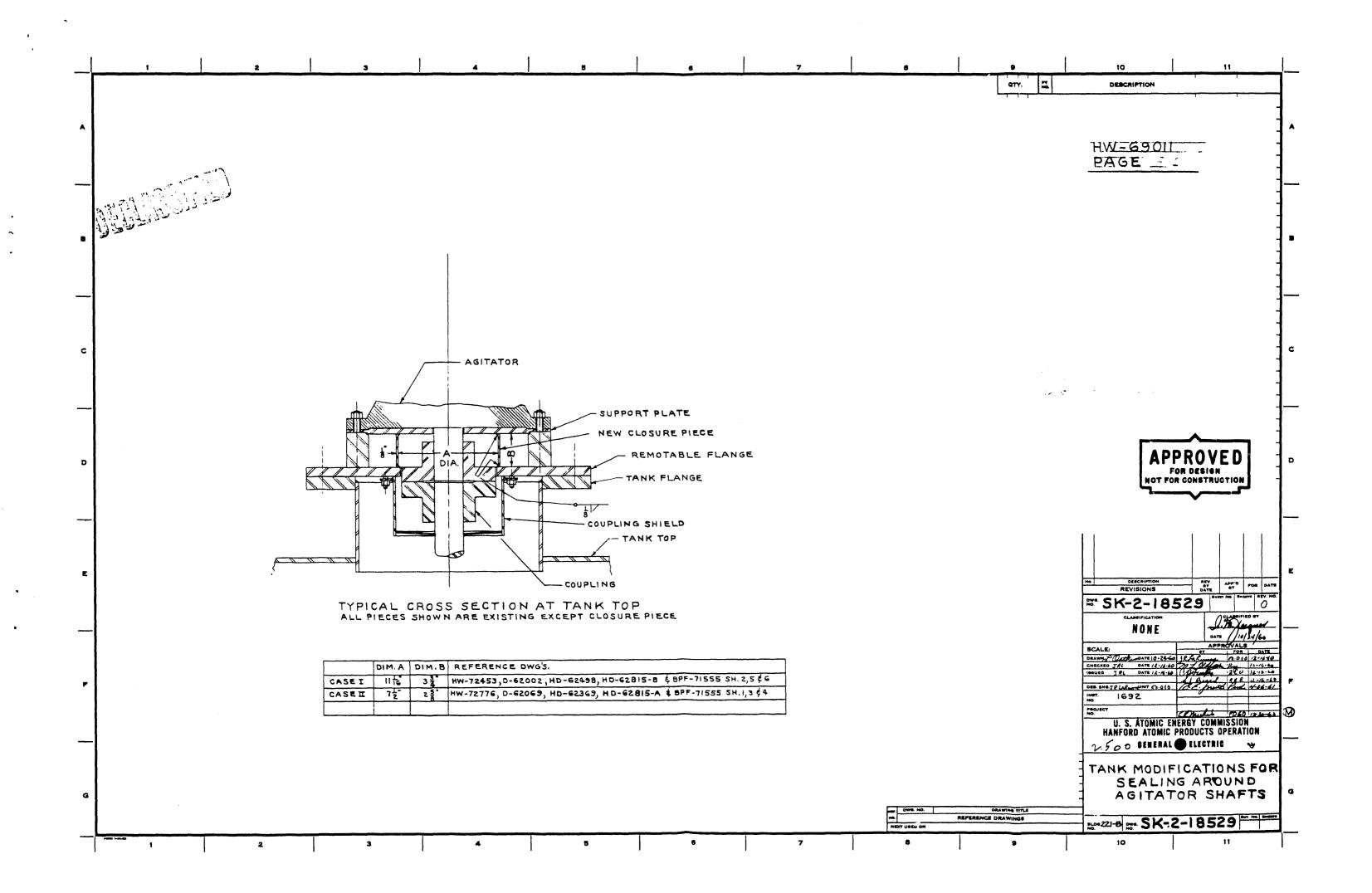


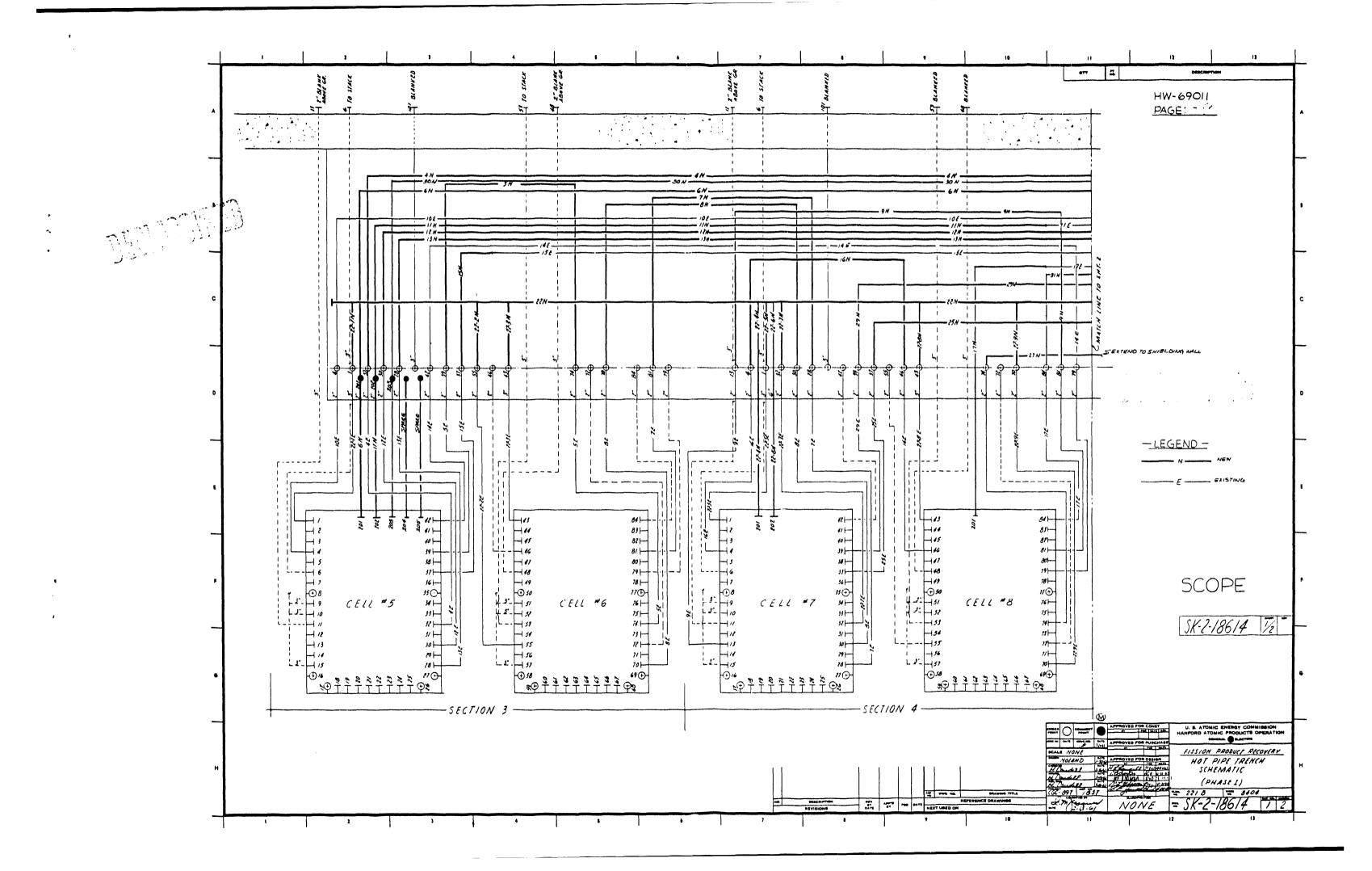


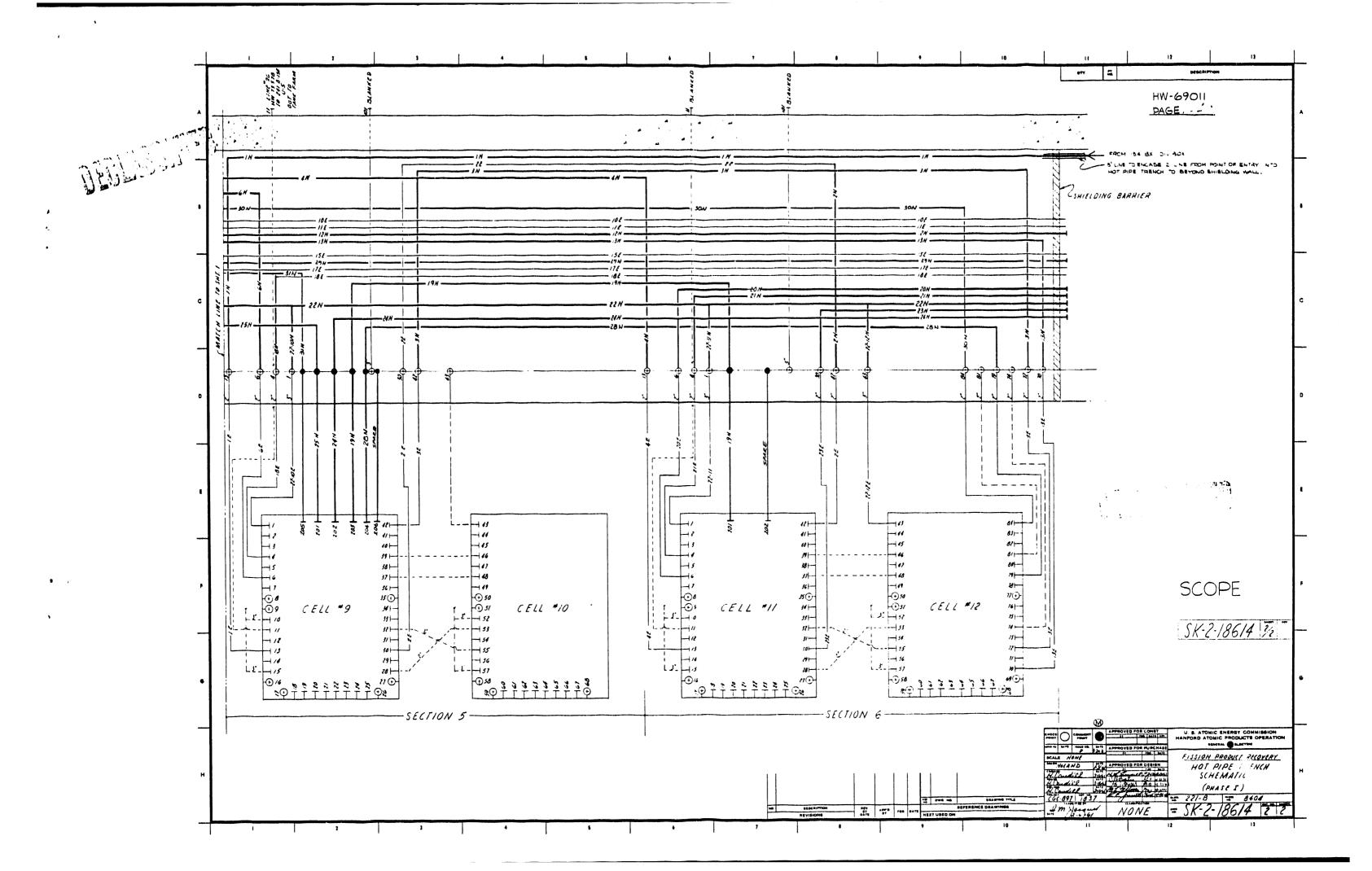


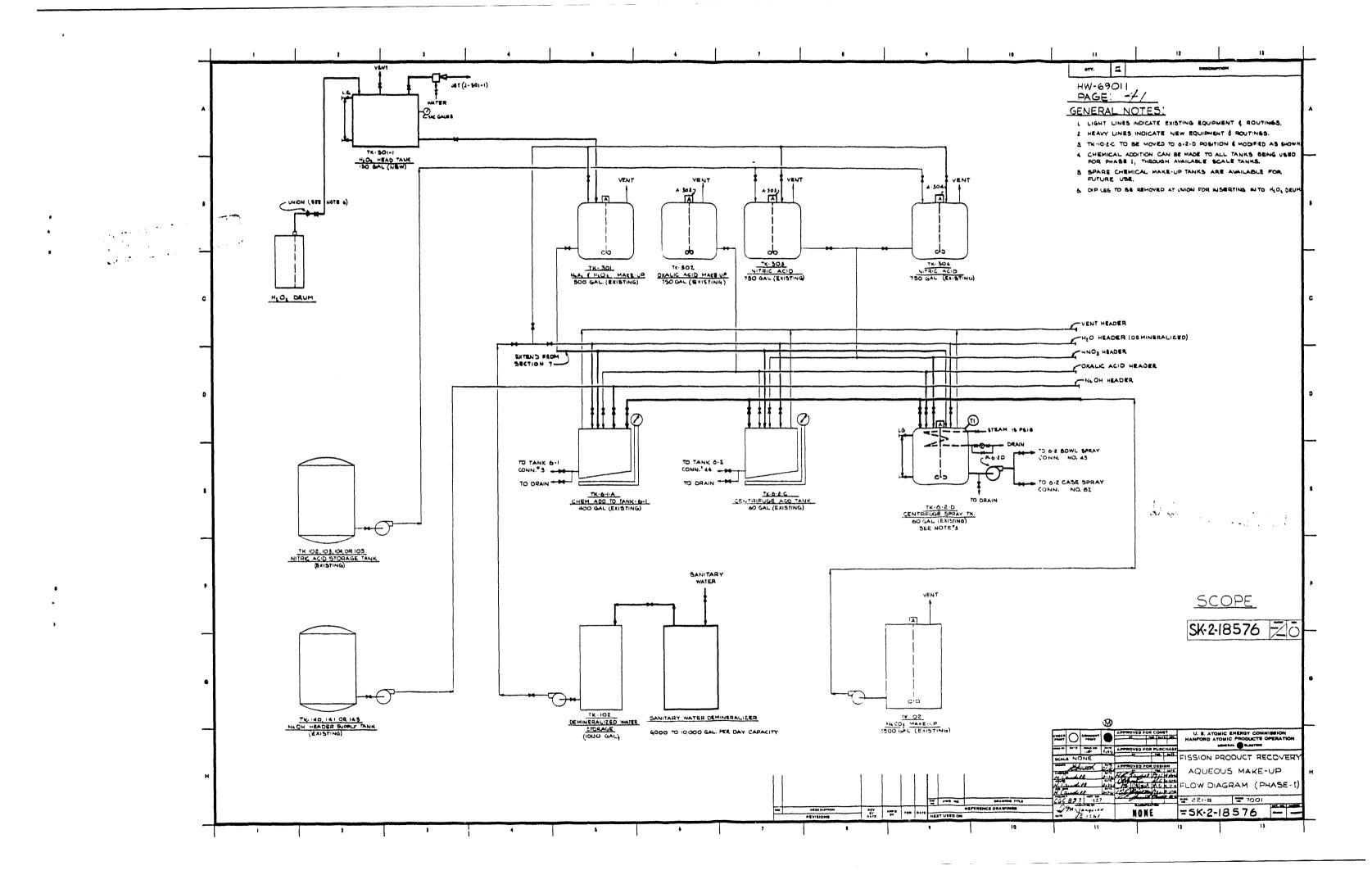


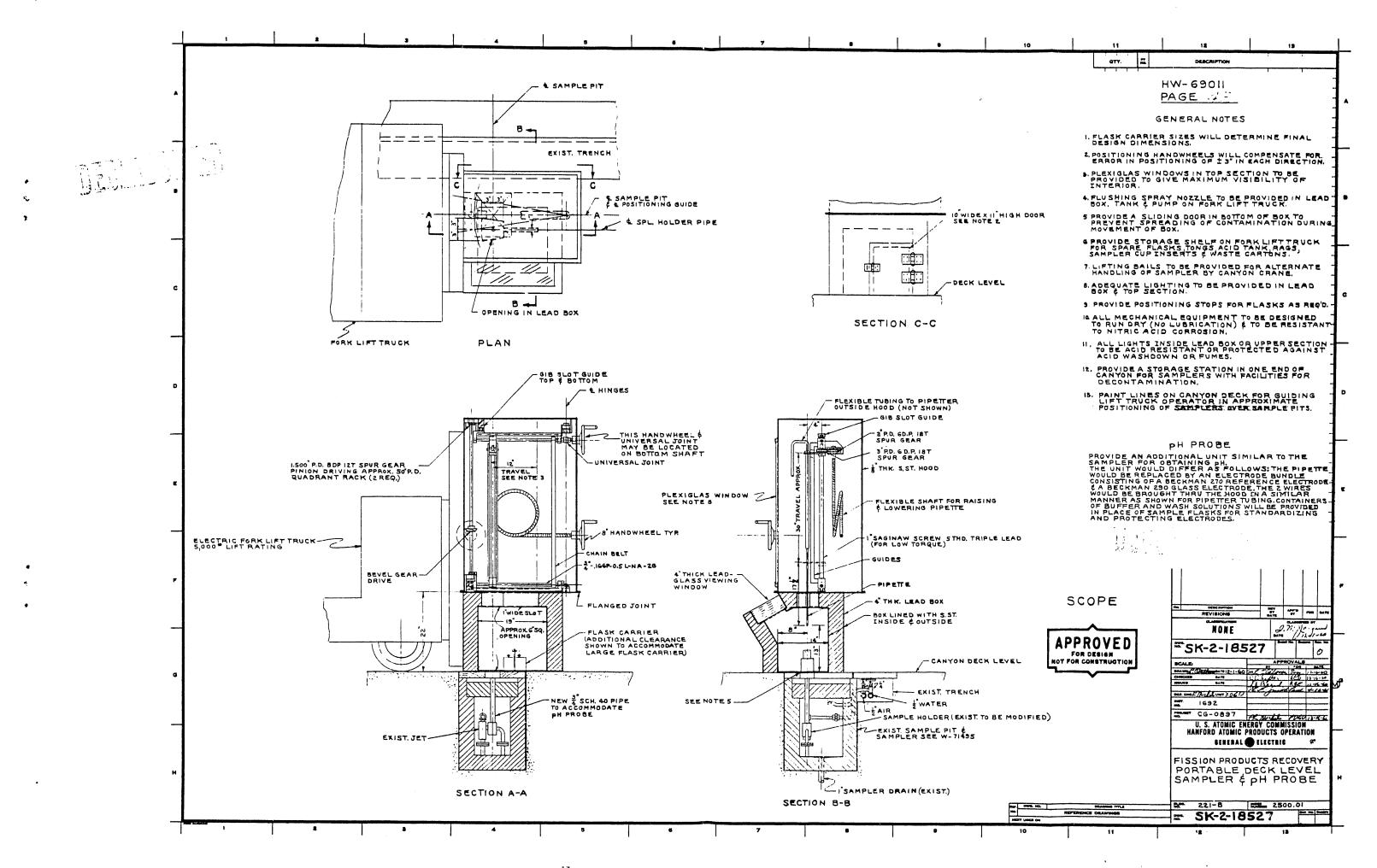


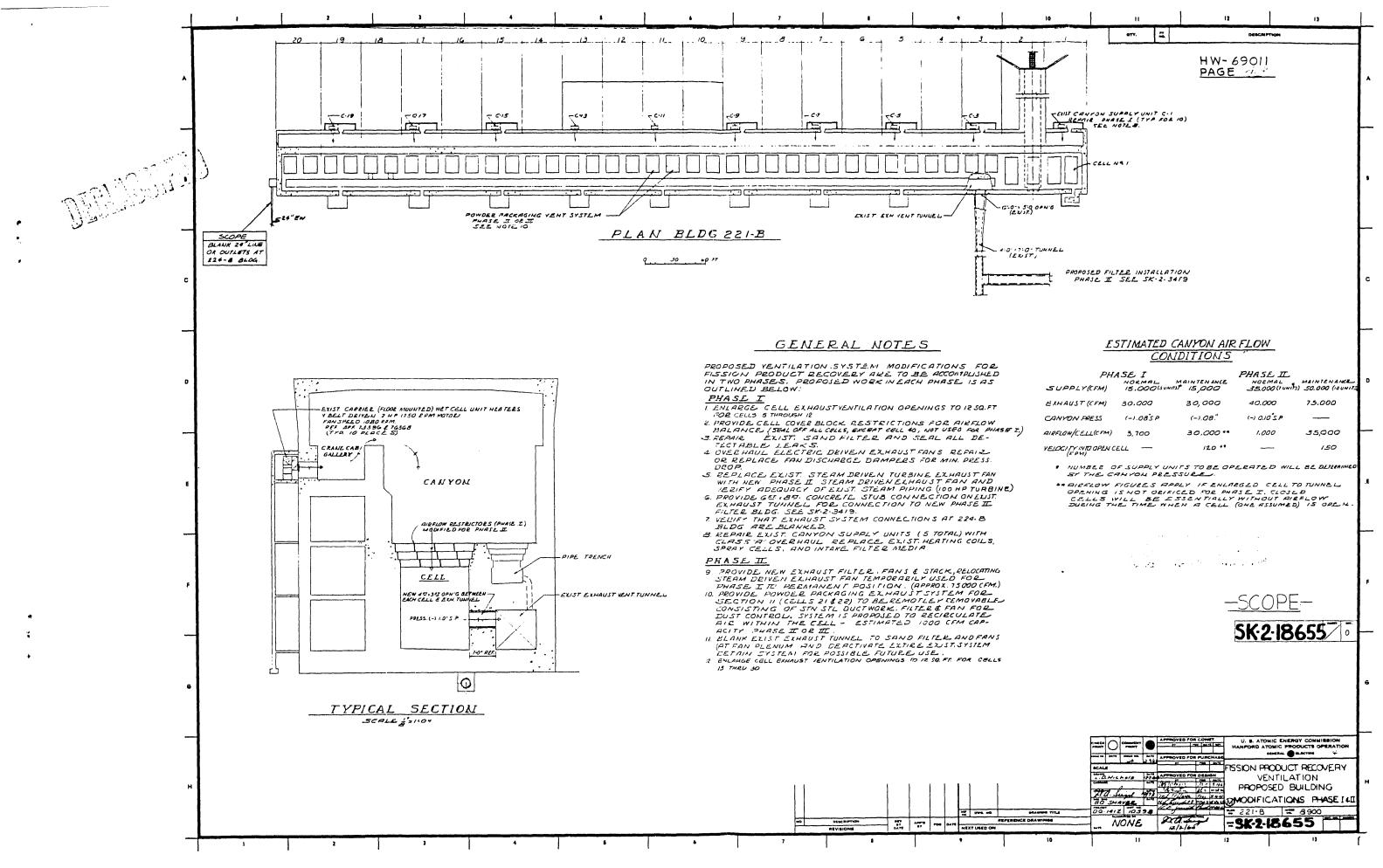




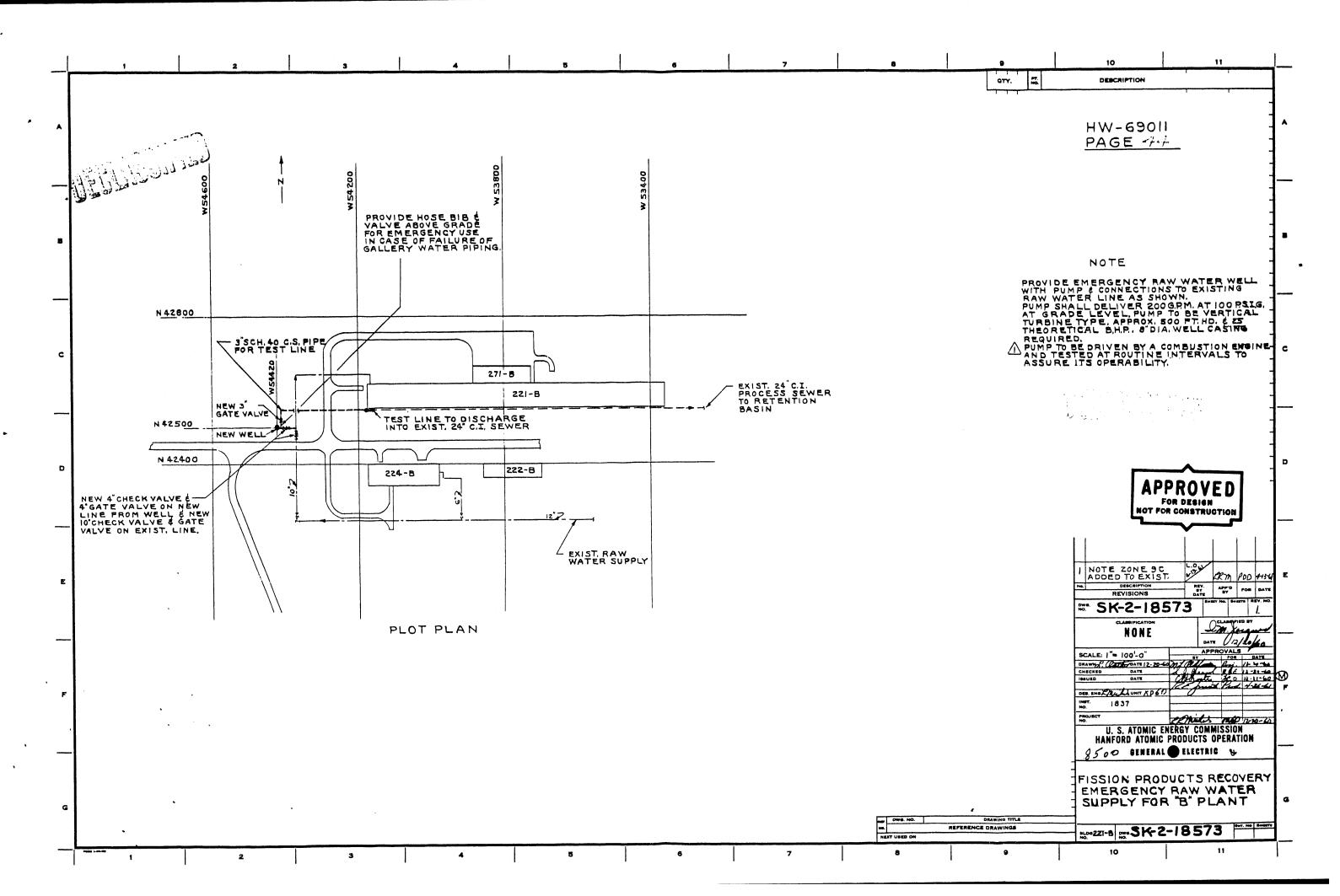




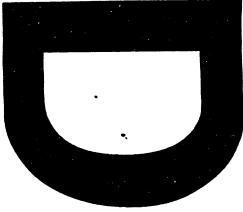




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