HOC-1650

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			• •	February	17, 1950	at a series of its and under the
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	TO:	J. M. Frame				Grant
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	FROM:	J. L. Wceks	Ŵ		9. RE	Smith
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	SUBJECT:	RADIATION INTE	NSITY AT CENTER	R 42" RISER	ON 11. JM	Frame - LB
.e 300	£x×	WASTE STORAGE	TANK (MJ-4)		12. 3	From Gelland
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	to_dam_n	howe the liquid	hemonguned hef	ore recomme	existing are	
	made on	cutting through	the concrete s	eal.	idations are	
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	Ad	ctailed statement	nt of work on t	his problem	rollows.	
	The calc	ilations have b	een liiea.		i N	
	Tn	the study of th	e fessibility o	f outting t	brough the	
	concrete	seal on the ce	nter riser of t	he waste st	orage tank	JUL 30 1956
	the radi	ation intensity	that would be	encountered	is critical	200 0050
	because	direct operator	contact would	be required		SUU AREA
					C C	LASSIFIED HLES
	The	question of th	e radiation int	ensity to b	e expected	
	was appr	oached from two	standpoints:	(a) A stud	y of previou	8
	measurem	ents of sludge	level and activ	rities in th	e storage	
	tanks (7	-5537. HEW-7402	. HW-7776. HW-1	10.154. HW-1	1,235). and	
	(h) Dim	ent algulation	,			•

(a) Measurements reported in the above documents were obtained by lowering a special ionization chamber into a 4" chamber well on the storage tank. In a test on December 5, 1946 in the 241-T area on Tank 101-T an ionization chamber reading of 25,600 mr/hr was obtained when the chamber was just below the top of the tank and 7.25 ft. above the surface of the liquid. This reading is probably quite a bit lower than the actual intensity existing at this point due to the construction of the chamber.

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The chamber was constructed in the form of a cylinder whose overall dimensions were approximately 4" diameter by 6" length. The ends of the cylinder were shielded with approximately 1" of lead. The electrodes were metal discs about 1" diameter and separated by about $\frac{1}{2}$ ". They were supported in the center of the cylinder. Thus the chamber was most sensitive to radiation entering at right angles to the axis of the cylinder.

(b) The calculations were based on the following assumptions:

1. Metal was irradiated to 200 MWD/ton.

2. Total metal in tank was 315 tons.

3. Average age was 2 years.

- 4. Ten percent of activity was contained in supernate.
- 5. Gamma radiation was due to Cs137, Pr144;Rh106only.
- 6. Distance from liquid level to receptor was taken as 16 feet.
- 7. Concrete seal on riser was assumed to be 12" thick.

On the basis of these assumptions the calculated value of the radiation intensity, with no shielding was 6000 mr/hr; and, with 12¹⁷ of concrete shielding; 25 mr/hr:

The difference between the calculated value of the radiation intensity and the measurements obtained with the ionization chamber indicates that it would be hazardous to make recommendations on the basis of calcualtions based on the simplifying assumptions used. Rigorous analysis of the problem would be time-consuming, if not impossible. Several pertinent questions can not be answered at present. For example, what is the exact distribution in the tank of certain fission products?

It is recommended that a request be made for measurements of the radiation intensity in the air space above the liquid with a chamber that will measure the total radiation.

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