

By acceptance of this article, the publisher or recipient acknowledges the U.S. Government's right to retain a nonexclusive, royalty-free license in and to any copyright covering the article.

**MASTER**

PHYSICS OF FUSION-FUEL CYCLES\*

J. Rand McNally, Jr.  
Fusion Energy Division  
Oak Ridge National Laboratory  
Oak Ridge, Tennessee 37830

Invited Paper Vugraphs  
1981 IEEE International Conference on  
Plasma Science  
May 18-20, 1981  
Santa Fe, NM

\*Research sponsored by the Office of Fusion Energy, U.S. Department of Energy, under contract W-7405-eng-26 with the Union Carbide Corporation.

**DISCLAIMER**

The work was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

JRS

## NOTES ON FIGURES

1. Title page.
2. Abstract of paper.
3. Nuclear fusion fuels.
4. Fusion fuel costs.
5. Normalized charged particle output power for various "pure" fusion fuels operating at  $n_e = 10^{20} \text{ e/m}^3$  and  $n_i/n_j = Z_j/Z_i$ . For neutron producing fuels the total power output is larger (e.g., factor of  $\sim 5$  for catalyzed D-T,  $\sim 1.6$  for catalyzed D-D).
6. Bibliography of useful reaction tables and graphs of  $\langle \sigma v \rangle$  vs T.
7. Progress in thermal energy utilization factor, f, for toroidal devices vs year. Fast fusion factor,  $\epsilon$ , for 1978 PLT point would improve Q (= DT fusion power/plasma power throughput) by about a factor of 3. Consult Nucl. Fusion 17, 1273 (1977); ORNL/TM-6362, July 1978.
8. Progress in f for mirrors and toroidal devices vs year. Exponential build-up of proton density in mirrors has not been exploited fully.
9. Approximate evaluation of tritium burnup vs  $n\tau$  and T demonstrating need for large  $n\tau$  and high T to reduce tritium recycle.
10. Energy transfer from an ion to a Maxwellian sea of electrons at T = 10 keV for various electron velocities. Note that the dominant energy loss is to electrons which are slower than the test ion.
11. Rosenbluth depletion effect for pumping electrons out of "slow" region and thus reducing stopping power.
12. Effect of magnetic field on stopping power of electrons for a test ion moving parallel to magnetic field.
13. Slowing down rates for fast fusion product ions due to nuclear elastic collisions (after Devaney and Stein).
14. Comparison of nuclear elastic slowing down rate and Coulomb (FP energy to ions and electrons) slowing down rates in deuteron plasma at  $n_e = n_d = 10^{14} \text{ particles/cm}^3$ . l designates test ion,  $A_r$  designates reduced atomic mass number.

15. Bibliography of useful nuclear data information.
16. Probability of propagation chain reaction in pure DT burn vs plasma electron temperature. Actual proton consumption depends on fuel/ash mix but may be about 10%.
17. Effects of magnetic field correction, Rosenbluth effect, nuclear scattering correction and  $p + t \rightarrow n + {}^3\text{He}$  reactions on plasma reactivity.
18. Tritium production in and required tritium breeding ratios for DT reactors having depleted tritium. Excess neutrons can be used for energy multiplication or fissile fuel breeding. Catalyzed DD burn would have about 1-2% tritium.
19. Advantages and disadvantages of catalyzed DD reactors. Such reactors are probably the most promising for a viable fusion economy. Grossly reduced tritium inventory, no Li breeding region.
20. Nuclear effects in fusion plasmas. Further study of these phenomena may add or detract from present projections of fusion reactivity.
21. Fast proton reactions with  ${}^6\text{Li}$  in high temperature reactors,  $\sigma v(p, {}^3\text{He})$  is  $\sigma v$  for  $p + {}^6\text{Li} \rightarrow {}^3\text{He} + \alpha + 4.0 \text{ MeV}$ ;  $\sigma v(p, p')$  is  $\sigma v$  for  $p + {}^6\text{Li} \rightarrow p' + {}^6\text{Li}^* - 2.2 \text{ MeV} \rightarrow p' + d + \alpha - 1.7 \text{ MeV}$ . Fast fusion reaction probability curves for  $p({}^6\text{Li}, {}^3\text{He})\alpha$  and  $p({}^6\text{Li}, p'){}^6\text{Li}^*$  are shown for several electron temperatures. Electron temperatures will probably not exceed 150 keV in realistic plasmas.
22. Possibility of in situ ICRH coupling between fusion product charged particles and fuel ions.
23. Status of  $p - {}^{11}\text{B}$  fusion fuel prospects.
24. Radioactive  ${}^7\text{Be}$  production in enriched  ${}^{11}\text{B}$  fuel (97%  ${}^{11}\text{B}$ , 10%  ${}^{10}\text{B}$ ).
25. Problem of thermal and density excursions in ignited plasmas.
26. Summary.

PHYSICS OF FUSION FUEL CYCLES\*

J. Rand McNally, Jr.  
Fusion Energy Division  
Oak Ridge National Laboratory  
Oak Ridge, TN 37830

1981 IEEE International Conference on  
Plasma Science  
May 18-20, 1981  
Santa Fe, NM

\*Research sponsored by the Office of Fusion Energy, U.S. Department of Energy, under contract W-7405-eng-26 with the Union Carbide Corporation.

Physics of Fusion Fuel Cycles. J. RAND

McNALLY, Jr., Oak Ridge National Laboratory.\* The evaluation of nuclear fusion fuels for a magnetic fusion economy must take into account the various technological impacts of the various fusion fuel cycles as well as the relative reactivity and the required  $\beta$ 's and temperatures necessary for economic steady-state burns. This paper will review some of the physics of the various fusion fuel cycles (D-T, catalyzed D-D, D-<sup>3</sup>He, D-<sup>6</sup>Li, and the exotic fuels: <sup>3</sup>He<sup>3</sup>He and the proton-based fuels such as P-<sup>6</sup>Li, P-<sup>9</sup>Be, and P-<sup>11</sup>B) including such items as:

1) Tritium inventory, burnup, and recycle, 2) Neutrons, 3) Condensable fuels and ashes, 4) Direct electrical recovery prospects, 5) Fissile breeding, etc. The advantages as well as the disadvantages of the different fusion fuel cycles will be discussed. The optimum fuel cycle from an overall standpoint of viability and potential technological considerations appears to be catalyzed D-D, which could also support smaller relatively "clean", lean-D, rich-<sup>3</sup>He satellite reactors<sup>1,2</sup> as well as fission reactors.<sup>3</sup>

---

\*Research sponsored by the Office of Fusion Energy U.S. Department of Energy, under contract W-7405-eng-26 with the Union Carbide Corporation.

<sup>1</sup>G. H. Miley et al., EPRI ER-536-SR (1977), p. 39.

<sup>2</sup>J. Rand McNally, Jr., Nucl. Fusion 18, 133 (1978).

<sup>3</sup>M. J. Saltmarsh, W. R. Grimes, R. T. Santoro, ORNL/PPA-79/3 (1979).

## NUCLEAR FUSION FUELS

### "Classical" Fusion Fuels

DT -- 20% charged particles, 80% 14 MeV n's.  
Must breed T from Li (DT-Li reactor).  
Radioactive T (~100 megacuries).

### "Conventional" Advanced Fusion Fuels

DD } Practical advanced fusion fuels for steady-state,  
D<sup>6</sup>Li } moderate  $\beta$  plasmas.

D<sup>3</sup>He Relatively "clean" fuel burn.  
Dependent on n-T, or DD or D<sup>6</sup>Li economy.

### "Exotic" Advanced Fusion Fuels

${}^3\text{He}{}^3\text{He}$  }  
 ${}^6\text{Li}$  } Need more study before acceptance as  
 ${}^9\text{Be}$  } "conventional" fusion fuels.  
 ${}^{11}\text{B}$  }  $Q \lesssim 1$ .

FUSION FUEL COSTS<sup>1,2</sup>

<u>Fuel</u>	<u>Supplier</u>	<u>Purity</u>	<u>Cost</u>	<u>Unit Fuel Cost (FBU = 1.0)</u>
D	S.R.L.	99.1%	1063 \$/kg	0.008 mil/kwth hr.
T	M.L.	(>94%)	7.5 x 10 <sup>6</sup>	42.
<sup>3</sup> He	M.L.	99.9	7.35 x 10 <sup>5</sup>	4.5
<sup>6</sup> Li	O.R.N.L.	95	1250	0.03
<sup>11</sup> B	E.P.	97	36,000	1.7

<sup>1</sup>DOE established prices provided by J. Ratledge, C. Benson (ORNL).

<sup>2</sup>Prices and purities subject to revision based on demand and technological improvements.

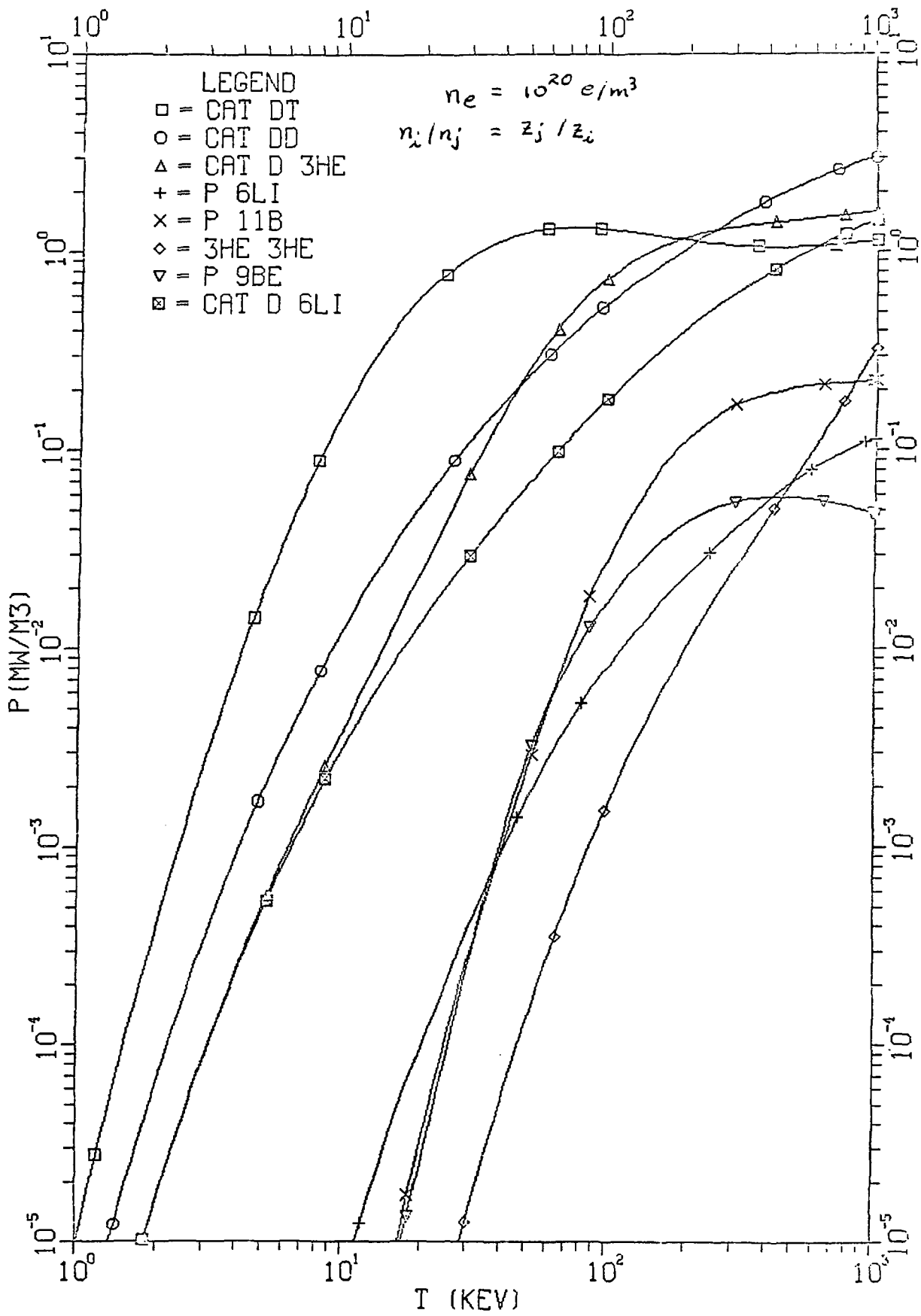


Fig. 5

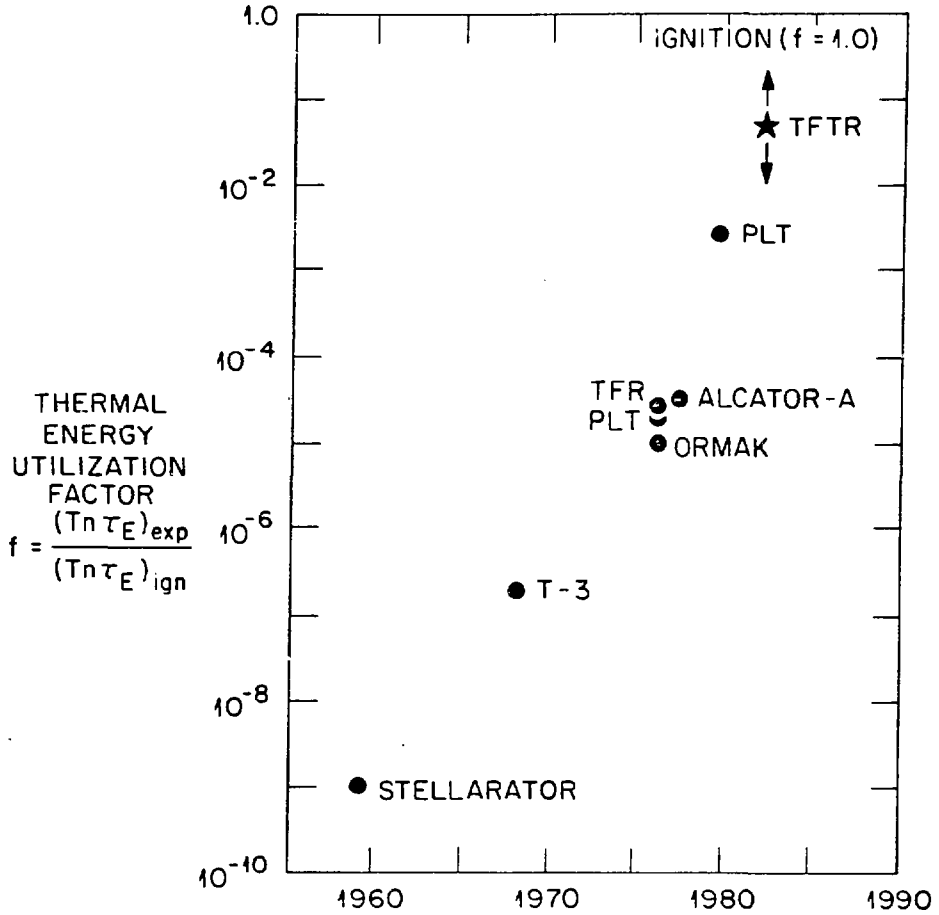


## REACTION TABLES AND GRAPHS

1. J. R. McNally, Jr., K. E. Rothe, R. D. Sharp,  
"Fusion Reactivity Graphs and Tables for  
Charged Particle Reactions," ORNL/TM-6914  
(1979); update October, 1980. (37 reactions).
2. R. J. Howerton, "Maxwell-Averaged Reaction  
Rates ( $\bar{\sigma v}$ ) for Selected Reactions between  
Ions with Atomic Mass  $\leq 11$ ," UCRL-50400, 21,  
Part A (1979). (24 reactions).

SUBSTANTIAL PROGRESS  
HAS BEEN MADE IN ACHIEVING  
THE CONDITIONS NECESSARY FOR FUSION

ORNL-DWG 80-3189    FED



THERMAL  
ENERGY  
UTILIZATION  
FACTOR

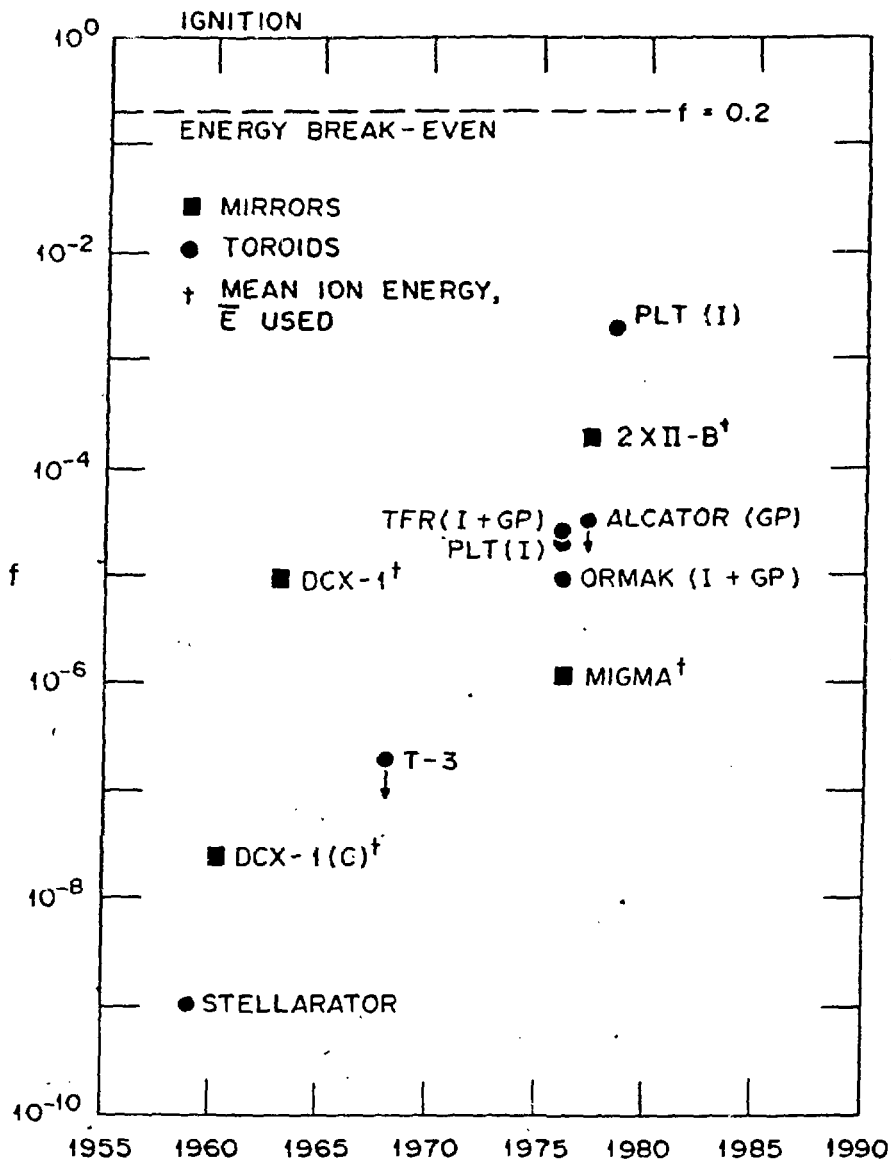
$$f = \frac{(Tn\tau_E)_{exp}}{(Tn\tau_E)_{ign}}$$

- TFTR = TOKAMAK FUSION TEST REACTOR
- PLT = PRINCETON LARGE TORUS
- TFR = TOKAMAK FONTENAY-AUX-ROSES
- ORMAK = OAK RIDGE TOKAMAK
- T-3 = TOKAMAK - 3

Fig. 7

# ENERGY UTILIZATION FACTOR, $f$ VERSUS YEAR

ORNL-DWG 80-2158 FED



$$f = \frac{(T n_e \tau_E)_{EXP}}{(T n_e \tau_E)_{IGN}} = \frac{DT \propto \text{POWER}}{\text{INPUT POWER}} \propto \frac{\beta^2 B^4}{\text{INPUT POWER}}$$

Fig. 8

TRITIUM BURN-UP

$$S_T = \frac{n_T}{\tau_T} + n_D n_T \langle \sigma v \rangle$$

$$\text{FBT} = \frac{n_D n_T \langle \sigma v \rangle}{S_T} = \frac{n_D \tau_T \langle \sigma v \rangle}{1 + n_D \tau_T \langle \sigma v \rangle}$$

$n_D \tau_T$ ( $\text{cm}^{-3} \text{sec}$ )	$T =$	10 keV	20 keV	30 keV
	$\langle \sigma v \rangle =$	$1.1 \times 10^{-16}$	$4.3 \times 10^{-16}$	$6.7 \times 10^{-16} \text{ cm}^3 \text{sec}^{-1}$
$3 \times 10^{14}$		0.03	0.11	0.17
$6 \times 10^{14}$		0.06	0.20	0.29
$1 \times 10^{15}$		0.10	0.30	0.40

Note: For 50:50 DT mixture  $n\tau \approx 2 n_D \tau_T$

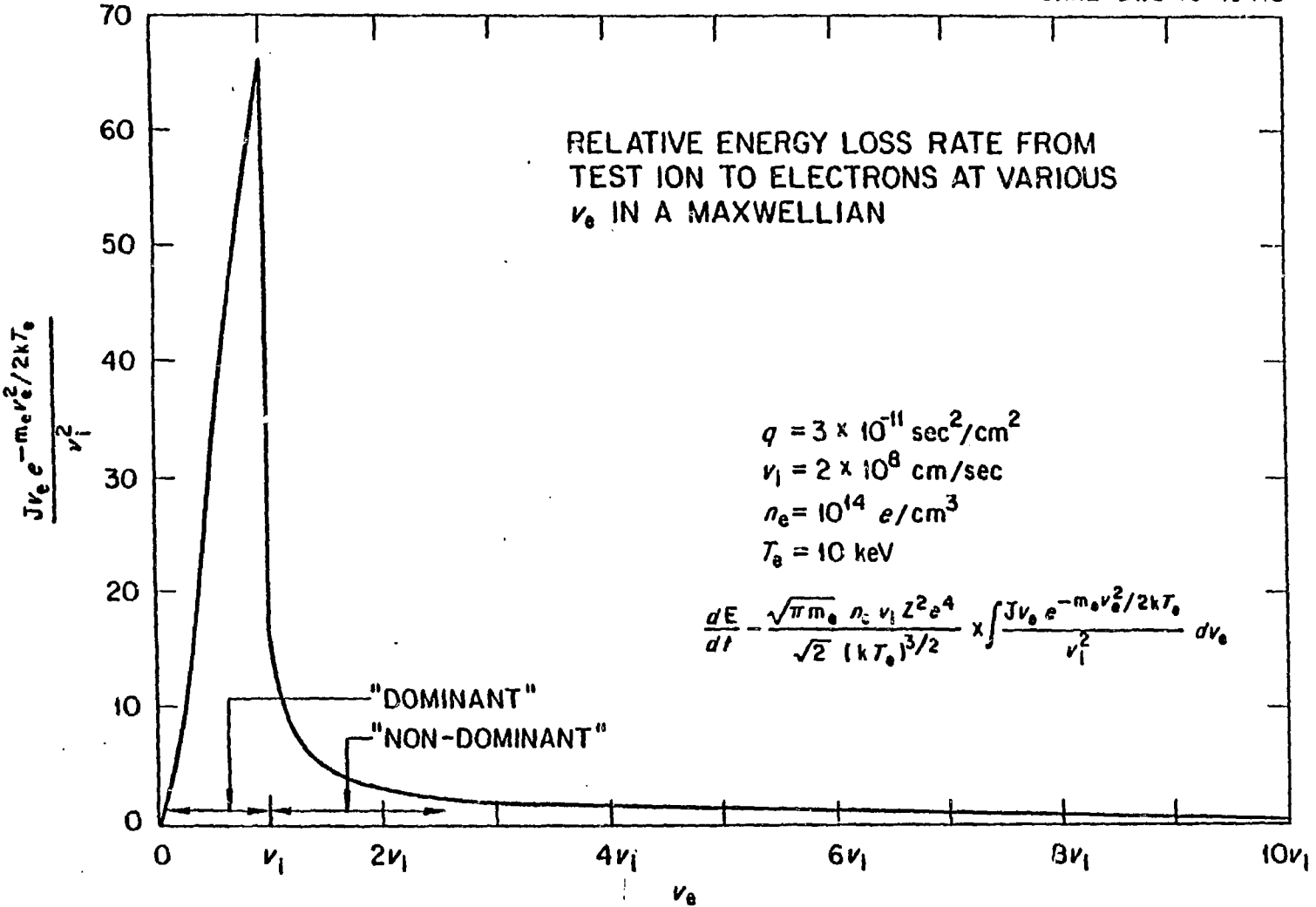


FIG. 10

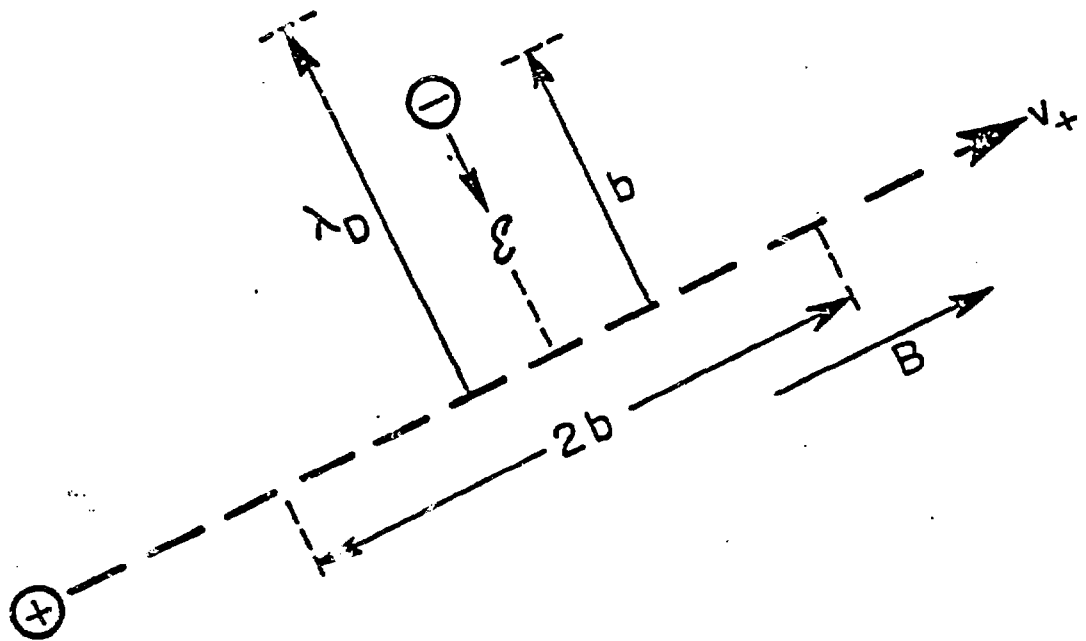
# EFFECT OF ROSENBLUTH CORRECTION FOR DEPLETION OF COLD ELECTRONS IN STEADY-STATE CATALYZED DD PLASMAS

$$\text{Input } \left\{ \begin{array}{l} B = 5.0 \text{ T} , R_{\mu} = 0.9 , a = 5 \text{ m} , 0 - D \\ n_e = 1.0 \times 10^{14} \text{ cm}^{-3} , n_d/n_e = 0.55 \end{array} \right.$$

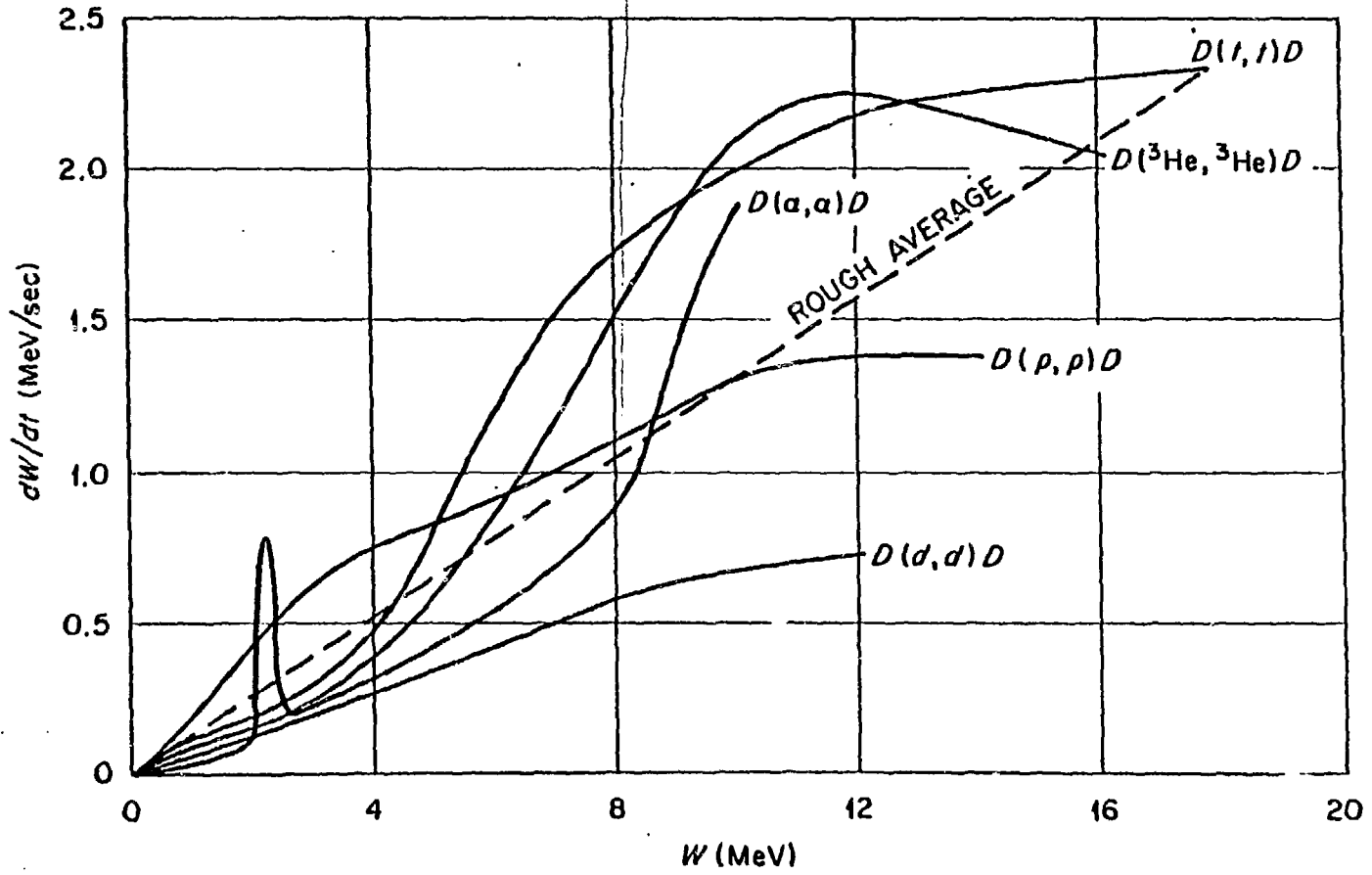
$$\left. \frac{dE}{dt} \right)_{i-e} = \left. \frac{dE}{dt} \right)_{\text{spitzer}} \times \left[ 1 - \left( \frac{2\pi^2}{3^{5/4}} \frac{m_e}{m_i} \frac{ZT_i}{T_e} \right)^{2/3} \right]$$

Bull. Am. Phys. Soc. 21, 1114 (1976):

Ros. Corr.	$T_i$ (keV)	$T_e$ (keV)	$\beta$ (%)	$P_{\text{total}}$ (kW/m <sup>3</sup> )
No (1.0)	93.4	69.1	23.7	294
Yes (0.984)	103.0	73.2	25.6	326
Effect	+10.3%	+5.9%	+8.0%	+10.9%

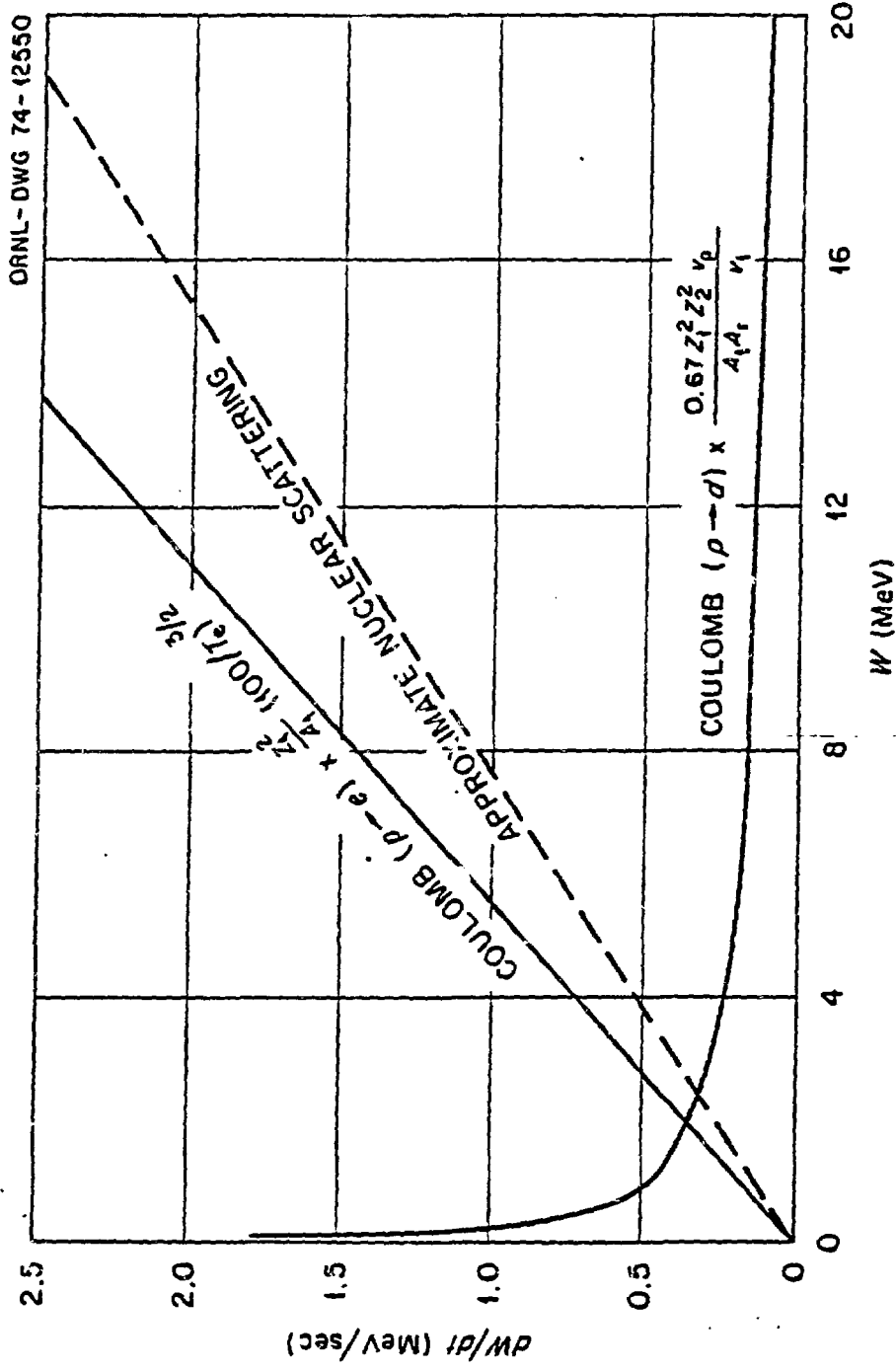


Bohr-Fermi method of analyzing maximum impact parameter for test ion moving at velocity  $v_+$  in an electron sea immersed in a magnetic field,  $B$ . When the Debye distance,  $\lambda_D$ , is larger than  $v_+/\omega_{ce}\sqrt{2}$  the latter defines an approximate maximum impact parameter.  $\mathcal{E} \sim Ze/b^2$  and approximate collision time is  $2b/v_+$ .



Slowing Down Rates for Nuclear Elastic Collisions with Deuterons.



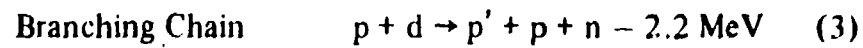
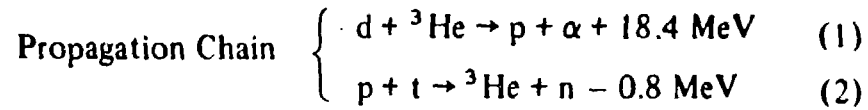


Energy Loss Rates for Test Ions in D-Plasma  
 ( $n_e = n_d = 10^{14}$  particles/cm<sup>3</sup>). Assumes  $\Lambda_{ie} \sim \Lambda_{ij} \sim 20$ .

NUCLEAR ELASTIC PLUS INTERFERENCE  
CROSS SECTIONS

- J. J. Devaney and M. L. Stein, Nucl. Science and Engineering 46, 323 (1971). (5 graphs).  
p, d, t,  $^3\text{He}$ ,  $\alpha$  on d.
- S. T. Perkins and D. E. Cullen, Nucl. Science and Engineering 77, 20 (1981). (25 graphs).  
p, d, t,  $^3\text{He}$ ,  $\alpha$  with each other.
- S. T. Perkins and D. E. Cullen, UCRL-50400, Vol. 15, Part F (1980). (25 data).  
p, d, t,  $^3\text{He}$ ,  $\alpha$  with each other.

### PROBABILITY OF PROPAGATION CHAIN IN DT



$$P = \frac{1/\tau_{p,t}}{1/\tau_{p,t} + 1/\tau_{p,d} + 1/2\tau_{s,d}}$$

$$= \frac{1}{2.0 + \frac{10^{-12} n_e}{n_t \bar{\sigma} v T_e^{3/2}}}$$

$T_e$	$P$	
50 keV	0.13	} $n_e = 2n_d = 2n_t = 10^{14} \text{ cm}^{-3}$ $\bar{\sigma} v_{p,t} \sim \bar{\sigma} v_{p,d} \sim 10^{-15} \text{ cm}^3 \text{ sec}^{-1}$
100 keV	0.25	
200 keV	0.37	
300 keV	0.42	

CORRECTIONS TO CATALYZED D-D BURNS\*

<u>Parametric</u>	<u>Cat.</u> <u>D-D</u>	<u>Plus</u> <u>B Effect</u>	<u>Plus</u> <u>Rosen. Effect</u>	<u>Plus</u> <u>N. S.</u>	<u>Plus</u> <u>P + T</u>
$T_i$ (keV)	100	143	144	173	175
$T_e$ (keV)	82	95	96	101	102
$\beta$ (%)	27.3	35.3	35.5	40.3	40.7
$P_{\text{CHP}}$ (kW/m <sup>3</sup> )	235	334	337	397	407
$P_{\text{NAB}}$ (kW/m <sup>3</sup> )	209	296	299	352	351
$n_T/n_e$	0.0088	0.0140	0.0142	0.0181	0.0178

---

\* $n_e = 10^{20} \text{ m}^{-3}$ ;  $n_D/n_e = 0.65$ ;  $B_o = 5 \text{ T}$ ;  $R_\mu = 0.9$ ;  $a = 5 \text{ m}$ ;  
Blanket Energy Release = 4.8 MeV/n;  $\tau_E = \tau_P$ .

TRITIUM PRODUCTION IN  
D-T REACTORS WITH DEPLETED TRITIUM

Operating Temperature	T Produced/T Consumed*			Required T Breeding Ratios
	( $n_T/n_D=1$ )	( $n_T/n_D=0.1$ )	( $n_T/n_D=0.02$ )	
30 keV	0.36%	3.6%	18%	1.00/0.96/0.82
60	0.71	7.1	35.5	0.99/0.93/0.64
90	1.2	12	60	0.99/0.88/0.40
120	1.7	17	85	0.98/0.83/0.15
150	2.3	23	115	0.98/0.77/0.00
180	3.0	30	150	0.97/0.70/0.00

$$*T \text{ Produced} \approx \frac{1}{4} n_D^2 \langle \sigma v \rangle_{DDt}$$

$$T \text{ Consumed} \approx n_D n_T \langle \sigma v \rangle_{DT}$$

$$T \text{ Prod}/T \text{ Cons} \approx \frac{n_D \langle \sigma v \rangle_{DDt}}{2n_T \langle \sigma v \rangle_{DT}}$$

## CAT-DD FUELED REACTORS

### ADVANTAGES:

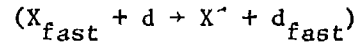
1. Lowest fuel cost; gaseous fuel and ashes.
2. Modest total tritium inventory ( $\sim 1$  g); no Li blanket.
3. Optimal selection of primary heat exchanger and structures.
4. Fissile and  $^3\text{He}$  fuel breeding.
5. About 45% 14 MeV neutrons as DT.
6. Steady-state burn prospect.

### DISADVANTAGES:

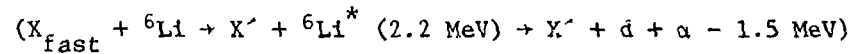
1. Rapid isotopic separation and fuel make-up.
2. Total neutron flux comparable to DT.
3. Requires higher temperatures and  $n\tau$ 's than DT.
4. Requires  $\beta \gtrsim 20\%$  for economic burn.
5. Major safeguards problem (neutrons are "free").

## NUCLEAR EFFECTS

1. Nuclear elastic scattering of fuel ions to suprathreshold energies

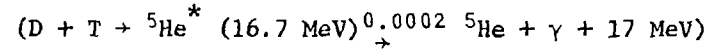


2. Nuclear dissociation events

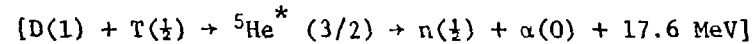


3. Partition of nuclei among excited nuclear states?<sup>1</sup>

4. Gamma ray production



5. Nuclear "spin" conservation?



<sup>1</sup>N. A. Bachall, W. A. Fowler, A. J. 157, 645 (1969); 161, 119 (1970).

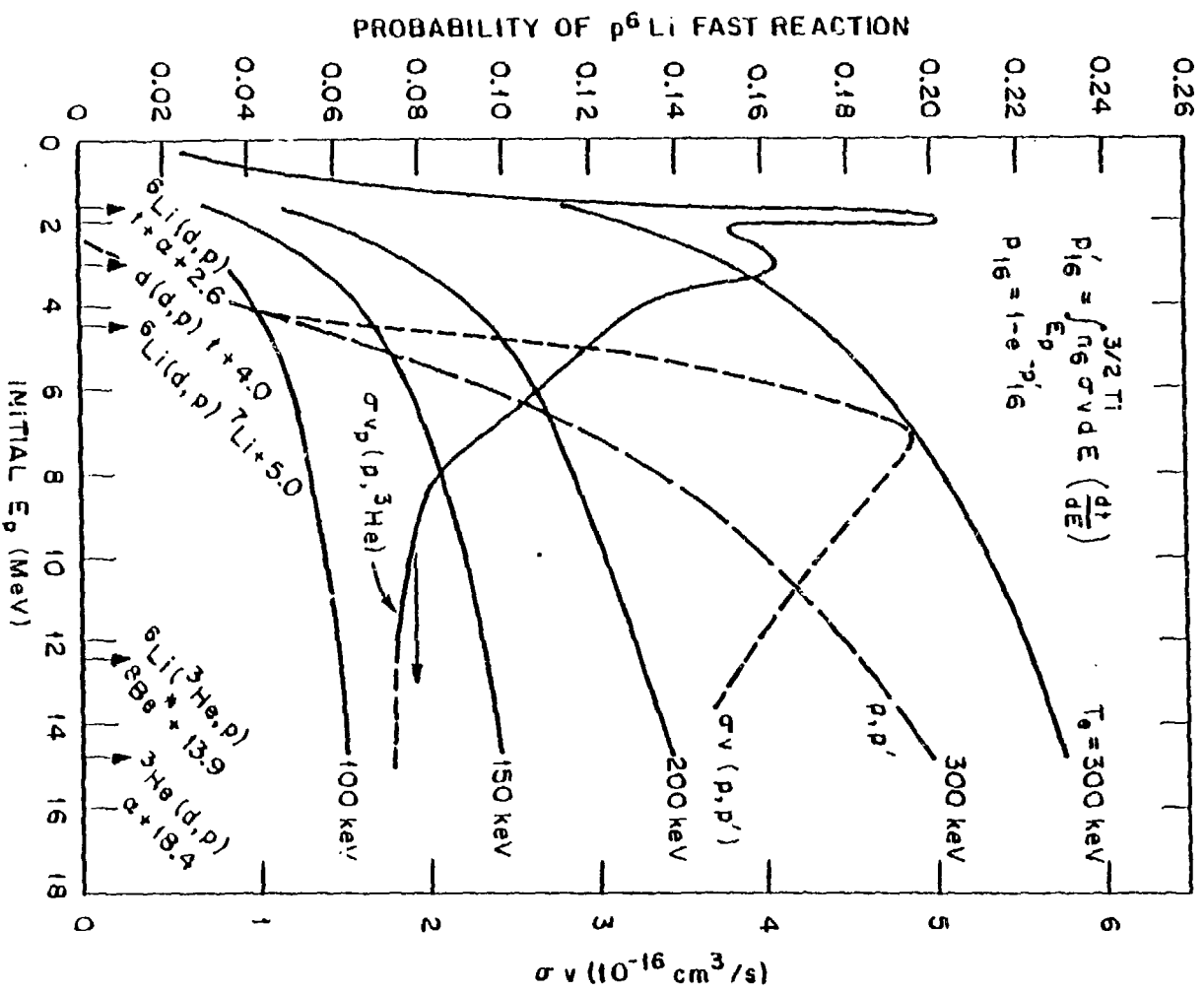


Fig. 21

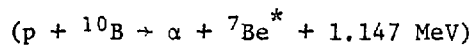


## ICRH COUPLING BETWEEN FUSION PRODUCT CHARGED PARTICLES AND FUEL IONS

- ICRH Heating of plasmas has been demonstrated (primarily by minority species coupling in dense plasmas).
- Selective heating of fuel ions will drive  $T_i > T_e$  and enhance reactivity.
- Alphas ( $Z/A = 1/2$ ), tritons ( $Z/A = 1/3$ ),  $^3\text{He}$  ( $Z/A = 2/3$ ), deuterons ( $Z/A = 1/2$ ), protons ( $Z/A = 1$ ).
- What is role of coherent bunching of fusion product ions in ion cyclotron motion?
- Conclusion: Need for in depth study of in situ ICRH coupling in burning plasmas.

## P-<sup>11</sup>B FUEL

1. Ignition prospects but no steady-state burns yet.
2. Problems of <sup>10</sup>B content (<sup>10</sup>B/<sup>11</sup>B: 18.7/81.3) and <sup>11</sup>B cost.



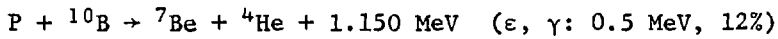
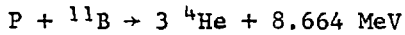
3. Problems of condensable ashes (debris)

(~ 2 tons/GW th y)

4. Synchrotron radiation problem

(FRM, Ion Layer, SURMAC)

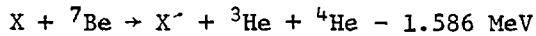
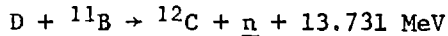
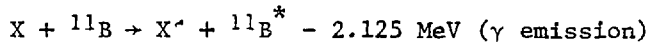
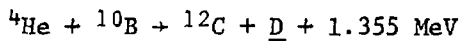
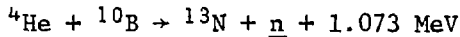
RADIOACTIVE  ${}^7\text{Be}$  PRODUCTION IN P- ${}^{10,11}\text{B}$  FUEL CYCLE\*



T	$\langle\sigma v\rangle(\text{m}^3/\text{s})$		${}^7\text{Be}$
	P + ${}^{11}\text{B}$	P + ${}^{10}\text{B}$	Production <sup>†</sup>
200 keV	$1.67 \times 10^{-22}$	$2.45 \times 10^{-23}$	0.0045
300	$2.43 \times 10^{-22}$	$5.96 \times 10^{-23}$	0.0076
400	$2.76 \times 10^{-22}$	$9.46 \times 10^{-23}$	0.0106

$${}^{\dagger}{}^{10}\text{B}/{}^{11}\text{B} = 3/97$$

\*Other side reactions needing evaluation:



## THERMAL AND DENSITY EXCURSIONS IN IGNITED PLASMAS

- Fusion plasmas have positive temperature coefficients (unstable) at ignition point.
- Fusion plasmas have negative temperature coefficient (stable) at burning temperature.
- Fusion plasmas have positive density coefficient at burning temperature.
- Power excursion following ignition may unload fuel absorbed on and adsorbed in first wall leading to further power excursion.
- Conclusion: Need for in depth study of transients in ignited plasmas.

## SUMMARY

1. Our understanding of reacting fusion fuels shows remarkable progress.
2. There is a need for more plasma and nuclear physics input to improve our understanding of reacting fusion fuels.