2
ornl

Received by OSTI

MAY 08 1986

ORNL/TM-9958

**OAK RIDGE
NATIONAL
LABORATORY**

MARTIN MARIETTA

**The Reduction of Plutonium(IV)
Using Photochemically Generated
Uranium(IV)**

L. M. Toth
L. K. Felker

**DO NOT MICROFILM
COVER**

OPERATED BY
MARTIN MARIETTA ENERGY SYSTEMS, INC.
FOR THE UNITED STATES
DEPARTMENT OF ENERGY

DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency Thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

DISCLAIMER

Portions of this document may be illegible in electronic image products. Images are produced from the best available original document.

Printed in the United States of America. Available from
National Technical Information Service
U.S. Department of Commerce
5285 Port Royal Road, Springfield, Virginia 22161
NTIS price codes—Printed Copy: A03 Microfiche A01

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

**DO NOT MICROFILM
COVER**

ORNL/TM--9958

DE86 009911

Chemical Technology Division

THE REDUCTION OF PLUTONIUM(IV) USING PHOTOCHEMICALLY GENERATED URANIUM(IV)

L. M. Toth
L. K. Felker

MASTER

Date of Issue: May 1986

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

Prepared by
OAK RIDGE NATIONAL LABORATORY
Oak Ridge, Tennessee 37831
operated by
MARTIN MARIETTA ENERGY SYSTEMS, INC.
for the
U.S. DEPARTMENT OF ENERGY
under Contract No. DE-AC05-84OR21400

DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

ps

CONTENTS

	<u>Page</u>
ABSTRACT	1
1. INTRODUCTION	1
2. EXPERIMENTAL METHODS	3
2.1 UO_2^{2+} PHOTOLYSIS SYSTEM	4
2.2 $\text{UO}_2^{2+}/\text{Pu}$ PHOTOLYSIS SYSTEM	5
2.3 MATERIALS AND PREPARATIONS	10
2.4 ACTINOMETRY	11
2.5 SPECTROPHOTOMETRY	12
2.6 COUNTING TECHNIQUES	14
3. RESULTS AND DISCUSSION	15
3.1 PHOTOLYSIS OF URANYL ION	16
3.2 URANIUM/PLUTONIUM PHOTOLYSIS	19
4. CONCLUSIONS	31
5. REFERENCES	32

THE REDUCTION OF PLUTONIUM(IV) USING PHOTOCHEMICALLY GENERATED URANIUM(IV)

L. M. Toth and L. K. Felker

ABSTRACT

The reduction of Pu(IV) using photochemically generated U(IV) has been evaluated as a procedure for possible inclusion in the Savannah River Plant's nuclear fuel reprocessing facility. The "Purex 2nd Uranium Cycle" with feed conditions of 400 g/L U, 1 M HNO₃, and 10⁻⁸ to 10⁻⁶ M Pu was identified previously as the most promising stage for application of a photochemical method. Laboratory tests were conducted under similar conditions to determine if the plutonium could be successfully reduced and separated in a two-phase flowing system. The laboratory scale tests, which used primarily a 0.01 M butanol/0.01 M hydrazine reductant combination, demonstrated that reductive stripping of Pu(IV) using photochemically generated U(IV) is a practical method for removing plutonium from the organic phase. The kinetics of the reductive stripping procedure were found to be determined by the mixing rates of the organic and aqueous phases in this simple laboratory-scale system.

1. INTRODUCTION

The possibility of using photochemical techniques in nuclear fuel reprocessing operations has been an attractive idea for at least 15 years. The impetus for this application comes largely from a compelling need to reduce the amount of waste solution generated during reprocessing. A photochemical process could eliminate customary chemical redox agents, such as ferrous sulfamate, that contribute substantially to the waste.

During the past decade, we have accumulated much information on the fundamental photochemistry of uranium, plutonium, and neptunium in aqueous acid solutions;¹⁻⁴ also, patents for several attractive photochemical

processes have been filed or granted.⁵⁻⁷ The most technically promising of these processes involves the separation of neptunium, using nitrite ion generated by the photochemical reduction of some of the HNO_3 solvent. We have examined this chemistry to some extent, and the results are described in an earlier report on photochemical applications to the Savannah River Plant (SRP) reprocessing facility.⁸

The more immediate requirement appears to be that of plutonium separation in the Purex 2nd Uranium Cycle feed, as described in ref. 8. A research effort was therefore initiated to determine the feasibility of incorporating a photochemical redox process into the SRP plutonium separation scheme and also to determine the optimum conditions for the use of the process. The best reductant for Pu^{4+} for these purposes is U^{4+} , which is generated by the photochemical reduction of UO_2^{2+} using a suitably chosen secondary* reductant. The advantage gained by this two-step process.

is that uranyl ion is reduced with a higher quantum efficiency in nitric acid solutions than is Pu^{4+} .

*The secondary reductant, designated as "R" in Eq. (1), is defined here as that which is effective in photochemically reducing UO_2^{2+} to U^{4+} , the primary reductant which, in turn, reduces the Pu^{4+} to Pu^{3+} . The designation U(VI) and U(IV) will be used to denote oxidation states with no definition of the species. The ions involved, however, will be indicated as UO_2^{2+} and U^{4+} , respectively.

Although the interaction of light with uranyl ion has been extensively studied over the past several decades, relatively little has been done regarding direct photochemical reduction of uranyl ion in nitric acid solutions. We have sought to remedy this omission by measuring the formation of U^{4+} from the photochemical reduction of UO_2^{2+} in 1 M HNO_3 . These data were necessary in order to determine the best conditions (i.e., identity and concentration of secondary reductant) for U^{4+} production that were compatible with the SRP reprocessing system.

This report describes our preliminary studies of uranyl nitrate photochemistry in 1 M HNO_3 . Upon selection of the secondary reductant, Pu^{4+} was added and the combined solution was tested for plutonium reduction in both one-phase (aqueous) and two-phase (aqueous/organic) operations. Although the experiments were conducted in a small-scale laboratory apparatus, the results provide much information that is required for testing on a larger scale.

The analytical determination of actinide concentrations involved both scintillation counting in the aqueous phase combined with absorption spectrophotometry (when the concentrations were high enough) and scintillation counting alone in the organic phase. The PERALS^{9,10} scintillation counting technique allowed us to obtain meaningful data at the low concentrations reported here.

2. EXPERIMENTAL METHODS

The experimental approach was progressive in nature, beginning with the determination of the best uranyl photolysis conditions. Next, Pu^{4+} , in relatively high concentration, was added to the one-phase aqueous

system, and the reduction of the Pu^{4+} to Pu^{3+} was monitored spectrophotometrically. This reduction was then followed by a two-phase (TBP, n-dodecane organic/ HNO_3 aqueous) flow system where the aqueous phase was photolyzed after prior equilibration of the various constituents. After the system had become operational at relatively high $[\text{Pu}^{4+}]$, the concentration was decreased in subsequent experiments until the detection limits of the spectrophotometer were reached and the need for a more sensitive detection system arose. Two alpha scintillation counting procedures were used to provide more quantitative results for the very low $[\text{Pu}^{4+}]$ in the presence of large amounts, 1.0 M, of uranyl ion. The various steps in this procedure are described in the following paragraphs.

2.1 UO_2^{2+} PHOTOLYSIS SYSTEM

The initial uranium photoreduction tests were made using a Scoeffel 1000-W mercury-xenon vapor arc lamp. The light output of the lamp was filtered through a combination of Pyrex and Plexiglas plates and focused on a spectrophotometer cell fitted into a Cary 14H spectrophotometer. The use of the filters was necessary to remove the ultraviolet (uv) and the high-energy-visible light which would photolyze the nitrate ion and thereby produce undesirable side reactions. The Pyrex-Plexiglas combination gave a sharp cutoff at 350 nm. A polycarbonate filter that passed light of a >400 -nm wavelength was also used to test the effect of wavelength on the uranyl photolysis. These filters proved to be more satisfactory than commercially available Corning glass filters because they had cutoff bands at preferred wavelengths and/or gave sharper cutoffs. It was necessary to air-cool the plastic filters to prevent

them from melting. The distance from the lamp to the cell was approximately 30 cm. The photochemical cell (see Fig. 1) was fabricated from silica and consisted of a 1.5-cm-diam by 5-cm-pathlength body fitted with a magnetic stir-bar pump and water jacket. The total volume of the cell and stirring leg was 17 mL. The temperature for each of these tests was 25°C, except where noted otherwise. In a typical experiment, the uranium solution was added to the cell and a spectrum was obtained. The reductants were then added, and the arc lamp was used to photolyze the solution for a measured time while the growth of the U^{4+} band was monitored with the spectrophotometer. This allowed for determination of the U^{4+} production rates. Several tests were performed with this system to determine the concentrations and types of reductants that would provide for maximum U^{4+} production.

2.2 UO_2^{2+}/Pu PHOTOLYSIS SYSTEM

The photoreduction of samples containing plutonium required the use of a modified glove box-spectrophotometer arrangement, as shown in Fig. 2. An appendage attached to the glove box allowed the light from the spectrophotometer to pass through silica windows on the appendage. Since the UO_2^{2+}/Pu^{4+} photolysis would incorporate a two-phase extraction system, a new cell configuration (see Fig. 3) was designed and then fabricated from Pyrex. Again, the spectrophotometer cell had a 5-cm pathlength; however, it now had a 2.0-cm-diam body.

Above the cell was a water-jacketed chamber (2.0 × 15 cm) for the organic phase which provided an organic/aqueous ratio of 1:1. A similar configuration with a 4.5- × 15-cm chamber for the organic phase was fabricated to provide higher organic/aqueous ratios (4:1 to 5:1). The

Fig. 1. Compact photochemical cell that permits in-line absorption spectrophotometric measurements.

Fig. 2. Photograph of glove-box appendage that permits absorption spectral measurements on alpha-active materials without contaminating the glove box.

Fig. 3. Photograph of two-phase extraction system. The organic phase was confined to the water-jacketed column above the 5-cm pathlength spectrophotometric cell. The photolysis compartment is to the right.

photolysis chamber (4-cm diam $\times$ 1-cm pathlength) was used to provide a large-area window for the light. The short pathlength of the chamber was necessary to keep volumes at a minimum, although the design sacrificed some light absorption in the more-dilute uranyl nitrate solutions. The aqueous phase was circulated through the system with an FMI piston pump at 22 mL/min. The aqueous phase was drawn from the top of the photolysis chamber and discharged at the top of the organic phase, where it dispersed and fell through the organic solution. The pump discharge at the top of the organic phase provided moderate mixing in the organic phase; but in the higher organic/aqueous ratios, the organic phase was stirred occasionally with a glass rod to ensure complete mixing. In spite of the rather gentle agitation, we were sometimes troubled with the appearance of the organic phase in the spectrophotometric cell which caused very noisy aqueous solution spectra.

The same mercury-vapor arc lamp was used in this series of photoreductions. The Pyrex and Plexiglas filters that remove the high-energy-visible and uv light were mounted on the outside of the glove-box appendage. The output of the lamp passed first through the Pyrex and Plexiglas filters, then through the silica windows of the glove-box appendage, and finally into the photolysis chamber containing the aqueous $\text{UO}_2^{2+}/\text{Pu}^{4+}$ nitrate solution. Aluminum foil was used to shield the rest of the apparatus from the light. The production of U^{4+} or the reduction of Pu^{4+} to Pu^{3+} was monitored on a Cary 14H spectrophotometer or sampled for alpha scintillation counting.

The quantum yield (QY) for Pu(III) production was determined for the reference solution (run 60, described in Sect. 3) by measuring the amount of time required to reach nearly complete (i.e., 95%) reduction of the

Pu(IV) to Pu(III) as determined by monitoring the absorbance in the aqueous phase. No dark reduction of Pu(IV) was ever observed; thus, the only error in this determination would be caused by a delay in the reaction of the reduced uranium (principally U^{4+} , but see Sect. 3) with the Pu^{4+} as a result of slow kinetics or slow mixing of the solutions.

Therefore, any error in the QY value measured by this approach would be on the low side of the true value. We believe that the QY value for the reference solution is accurate because it corresponds to the value found in the single-phase solution; however, the values for solutions containing lower concentrations of butanol/hydrazine ($BuOH/N_2H_4$) reductant are apparently lower as a result of the slow rate of solution mixing by the FMI pump. Obtaining more accurate QY values for the other solutions would have required interruption of the photolysis to allow for mixing, a procedure that would have unduly complicated the photolysis procedure.

2.3 MATERIALS AND PREPARATIONS

All chemicals used in this study were analytical-grade reagents. The uranium, as uranium dinitrate oxide hexahydrate (IUPAC* name for uranyl nitrate hexahydrate), and the hydroxylamine nitrate, as a 24% solution, were obtained from Alfa Products. The tributyl phosphate, n-dodecane, and hydrazine were obtained from Fisher Scientific. The phenylhydrazine was from Eastman Chemicals Products, Inc. The plutonium was obtained on-site as a 0.1 M plutonium(IV) nitrate solution which had been purified by standard ion-exchange techniques. Analytical-grade nitric acid and ultrapure water were used in the preparation of all solutions.

*International Union of Pure and Applied Chemistry.

The 30% TBP-NDD solution was pre-equilibrated with 1.0 M HNO_3 before use. All the solution components were added to the cell, except for the reductants which were introduced just before the photolysis to prevent any photolysis from stray room light. The concentrations reported for the solutions in the two-phase system are those in the aqueous phase prior to the extraction of uranium and plutonium into the organic phase. The individual concentrations of the species in the different phases will be noted where necessary.

2.4 ACTINOMETRY

A measure of the light absorption by the solution was needed in order to determine quantum efficiencies (or yields: $\text{QY} = \text{mol of product formed/mol of light absorbed}$). The UO_2SO_4 solution with ethanol (EtOH) reductant was ideal for our study, since the amount of light absorbed by the uranyl ion was the quantity in question. The key difference between the sulfate and the nitrate systems was the stability of the U^{4+} product, $\text{QY} = 0.5$,* in the former, making it ideal for actinometry. A solution containing 1.25 M UO_2SO_4 , 5.0 M H_2SO_4 , and 1.0 M ethanol was used as the actinometer. In the two-phase system, the amount of absorbed light was determined by the photolysis of similar solutions of 0.14, 0.07, and 0.035 M UO_2SO_4 which were representative of the uranyl concentrations in the aqueous raffinate after extraction. Typically, for 0.035 M uranyl solutions, the absorbed wattage in the photolysis cell using the Pyrex-Plexiglas filter was 0.618 W, while that using the polycarbonate filter was 0.362 W.

*This value has been reported¹¹ to be as high as 0.6, but for the sake of this discussion, the theoretical value of 0.5 is sufficient and does not alter the conclusions that follow in the discussion of results.

2.5 SPECTROPHOTOMETRY

As stated above, the spectrophotometric analysis was performed with a Cary 14H system, which has an open sample and reference compartment ($42 \times 40 \times 30$ cm). The appendage from the glove box was designed to fit into this compartment (see Fig. 2), thus allowing spectrophotometric measurements to be made of solutions inside the glove box. All of the measurements were recorded on a strip-chart recorder. Good spectra were obtained during the uranyl-plutonium photolysis when the plutonium concentration was $>1 \times 10^{-3}$ M. When the concentration was $<1 \times 10^{-3}$ M, a more sensitive recorder slide-wire (0 to 0.2 abs vs the standard 0 to 2.0 abs) was utilized. The 0- to 0.2-abs slide-wire gave good results, but the noise level of the recorder was significantly increased. For plutonium concentrations $<1 \times 10^{-4}$ M, the intensity of the spectrum was too small to measure accurately, thus mandating counting techniques.

The photolysis of UO_2^{2+} was monitored spectrophotometrically by following the growth of the U^{4+} absorption band at either 545 or 648 nm. Representative spectra of the uranium species in 1.3 M HNO_3 are shown in Fig. 4. (This figure was adapted from an earlier report.¹² Under the experimental conditions used in this study, the uranyl peak at 414 nm was typically very intense and therefore too far off scale to measure; these conditions were necessary to permit the accurate measurement of the weaker U^{4+} product spectrum. The weak uranyl band at 485 nm with a molar absorptivity of $0.505 \text{ M}^{-1} \text{ cm}^{-1}$ was the only band available under these conditions.

ORNL DWG 83-288

Fig. 4. Absorption spectra¹² of 6.4 g/L uranyl nitrate (dashed line) and 6.4 g/L U^{4+} nitrate in 1.3 M HNO_3 (solid lines).

2.6 COUNTING TECHNIQUES

Counting techniques were utilized to determine the distribution of plutonium between organic and aqueous phases in solutions with plutonium concentrations $<10^{-4}$ M. An LKB-Wallac RackBeta counter was used for alpha scintillation counting. Results were good for higher concentrations where plutonium gave the dominant signal, but poor resolution of the uranium and plutonium bands at plutonium concentrations $<1 \times 10^{-4}$ M precluded the usage of this technique. An improved scintillation procedure, Photon-Electron Rejecting Alpha Liquid Scintillation (PERALS)^{9,10} developed by McDowell and Case of ORNL, was then selected. This method permitted discrimination between the alpha and beta-gamma energies while providing the resolution to separate the uranium and plutonium alpha scintillation bands, thus giving quantitative results for concentrations as low as 10^{-7} to 10^{-8} M plutonium in 0.1 M uranium. With this method, analyses of samples were accomplished within a few hours of the experiment, and no separation of the uranium and plutonium was necessary.

Both of the counting techniques required samples to be removed from the glove box. When the RackBeta counter was used, 10 mL of ACS liquid scintillation fluid was added to scintillation vials, which were put in individual plastic bags and bagged into the glove box. A 10- μ L sample was carefully pipetted into the scintillation fluid so that neither the glove nor any contaminated surface contacted the vial. The vials were then bagged out in a clean bag and checked for external contamination. If PERALS counting was to be used, a 15- μ L sample was pipetted into a 1-dram vial in a plastic bag. The vials were bagged out and checked for external contamination. In a hood, a 5- μ L sample was pipetted into 1 mL of PERALS^{9,10} scintillation fluid for counting.

3. RESULTS AND DISCUSSION

The experimental aspects of this study have focused, first, on the photochemical reduction of uranyl ion alone — to determine the optimum conditions for the U^{4+} generation process, and, second, on the photochemical reduction of uranyl ion in the presence of Pu^{4+} to demonstrate the practicality of the photoseparations process.

The development of a system for the photochemical reduction of Pu^{4+} to Pu^{3+} requires a reductant that is effective with respect to both quantum efficiency and kinetic speed. (The reason for separating these apparently redundant characteristics should become evident in the following discussion.) We have not found a reagent that satisfies these requirements directly since the photochemical reduction of Pu^{4+} is not very efficient; however, the dark reaction of U^{4+} with Pu^{4+} is so rapid and, thus, so attractive that a photochemical means of reducing UO_2^{2+} could also provide a very satisfactory, indirect method for reducing Pu^{4+} , as shown in Eqs. (1) and (2).

Although some of the uranyl photolysis work was presented earlier,⁸ we have included an expanded version of it here (with additional discussion) to provide continuity with subsequent experiments where uranium and plutonium were combined in a two-phase flow system. Following the photolysis experiments with pure uranyl ion, tests were conducted using $UO_2^{2+}/-Pu^{4+}$ mixtures to determine the efficacy of the overall photochemical scheme.

3.1 PHOTOLYSIS OF URANYL ION

The quantum efficiencies given in Table 2 of ref. 8 indicate that a maximum value of 0.5 for conversion of UO_2^{2+} to U^{4+} is feasible. Furthermore, we know¹³ that the uranyl ion can be reduced with a variety of reductants by using light of relatively low photon energy (i.e., visible light). Therefore, we sought to determine the best conditions for uranyl ion reduction, thereby providing U^{4+} which could be used to reduce Pu^{4+} in an appropriate demonstration facility. The reductants that were considered for UO_2^{2+} were methanol, ethanol, BuOH , hydroxylamine, and N_2H_4 . The first two were selected because they are favorites in fundamental systems; the last three were chosen because they are possible constituents in fuel reprocessing streams.

The UO_2^{2+} photolysis in HNO_3 is very different from that in less oxidizing media (e.g., H_2SO_4 since the U^{4+} in HNO_3 is rapidly oxidized via the catalytic effect of the NO_2^- that is almost always present in these solutions. (In contrast, the U^{4+} ion is so stable in H_2SO_4 solutions that the UO_2^{2+} photolysis in 5 M H_2SO_4 provides an excellent actinometer, as described in Sect. 2.4.) In order to stabilize the U^{4+} in HNO_3 solution, it is necessary to add a holding reductant such as hydrazine, N_2H_4 . Nevertheless, the stability of the U^{4+} ion is still far less than that in H_2SO_4 solutions. The results described in the following paragraphs will emphasize these differences.

We have considered the possibility that maintaining the U^{4+} stability during the actual reduction of Pu^{4+} might be unnecessary since U^{4+} may react before being reoxidized by any impurity ions; however, similar stability concerns exist with respect to Pu^{3+} , and the stabilizing agent

that is added for Pu^{3+} would probably also be useful for U^{4+} . During the development of a procedure, we attempted to minimize the amounts of reductant and holding agents necessary and to obtain the maximum conversions and rates for a given light input (which was found to be ~ 0.7 W of absorbed power). The acid concentration was fixed at 1 M HNO_3 to be consistent with conditions as stated for the first priority in Table 4 of ref. 8. The results of some of these experiments are summarized in Table 1.

The data demonstrate that BuOH is the most effective of the reducing agents tested. Hydrazine was approximately ten times less effective, while hydroxylamine nitrate (HAN) was found to be totally ineffective. Butanol was even more effective than a comparable concentration of ethanol. Because tributyl phosphate (TBP) has been demonstrated⁷ to be a reductant for UO_2^{2+} , an aqueous nitric acid solution saturated with TBP was tested. (Saturation was achieved by shaking with 30% TBP in n-dodecane to achieve the approximate 1 mM TBP concentration in the aqueous phase.) We found that under these conditions the TBP was ineffective in reducing any of the UO_2^{2+} present. In all cases, the U^{4+} product was unstable unless N_2H_4 was present. (Consideration of other holding agents was beyond the scope of this work.) A concentration of 0.01 M N_2H_4 was required to hold the U^{4+} for a period of several hours, whereas 0.001 M N_2H_4 was effective for only a few minutes.

These results suggest that optimal conditions for the photochemical reduction of UO_2^{2+} would include 0.01 M BuOH/ 0.01 M N_2H_4 and would result in a U^{4+} production rate of 4×10^{-6} mol/min. In our particular system of 0.015-L volume, this would net 11 mM U^{4+} after 48 min of photolysis - an amount we consider adequate to reduce any Pu^{4+} that might be present.

Table 1. Photochemical reduction of 1.25 M uranyl nitrate in 1 M HNO₃ using ~0.7 W of absorbed power from a 1000-W, Hg-Xe, high-pressure arc lamp^a

Run No.	Reductant	Photolysis time (min)	U(IV) production	
			Max. conc. (10 ⁻³ mol/L)	Rate (10 ⁻⁶ mol/min)
33	1 M N ₂ H ₄	60	33	9.5
34	0.1 M N ₂ H ₄	120	27	3.6
35	0.01 M N ₂ H ₄	60	1.4	0.37
36	0.1 M HAN	40	0	0
37	0.05 M HAN/0.05 M N ₂ H ₄	120	10	1.3
39	0.1 M BuOH	60	6.4	1.5
40	0.1 M BuOH/0.01 M N ₂ H ₄	9	26	43
41	0.01 M BuOH/0.01 M N ₂ H ₄	48	11	4.0
43	0.01 M BuOH/0.001 M N ₂ H ₄	40	9.4	4.1
44	0.01 M BuOH/0.005 M N ₂ H ₄	54	9.4	4.2
46	0.01 M BuOH/0.0001 M N ₂ H ₄	36	9.6	4.1
47	0.01 M EtOH/0.01 M N ₂ H ₄	56	7.5	2.4
50	0.001 M TBP	20	0	0

^aThe radiation was filtered to achieve a cutoff of all wavelengths below 350 nm. The volume of the photochemical cell was 15 mL.

The maximum quantum efficiency (determined by the slope of the U^{4+} growth curve divided by the rate of light input) for U^{4+} using 0.01 M BuOH/0.01 M N_2H_4 is 0.050. We find this to be an unusually low value since the theoretical value of 0.5 is easily achieved in the H_2SO_4 solutions. Even though the molar absorptivity of the U^{4+} is comparable to that of UO_2^{2+} in the 400- to 450-nm region (see Fig. 4), the small amount (<1%) of U^{4+} formed ensures that most of the photochemistry effected by these photons is with UO_2^{2+} . Photoinduced oxidation of U^{4+} may occur as a result of light absorption at longer wavelengths (>500 nm) and, thus, be responsible for the apparently lower QY for UO_2^{2+} reduction that we observe. However, we are unaware of any such photoactivity of U^{4+} in this region where the energy of the photon is considerably lower.

If the low value is indeed correct, then it indicates potential limitations in the nitric acid system. (Surprisingly, these limitations did not prove to be a problem. The reasons for this are discussed in the following section, which deals with the photolysis of the combined UO_2^{2+}/Pu^{4+} system.)

3.2 URANIUM/PLUTONIUM PHOTOLYSIS

Having determined that the combination of 0.01 M BuOH and 0.01 M N_2H_4 was the most satisfactory of the reductants tested, we sought to combine these two reagents in a system containing both UO_2^{2+} and Pu^{4+} . Although the SRP flowsheet specifications describe a stage with 1.3 M UO_2^{2+} and 10^{-7} M Pu^{4+} in 1 M HNO_3 , we chose a system that contained 1.0 M UO_2^{2+} and 4×10^{-3} M Pu^{4+} in 1 M HNO_3 for the initial tests. The choice was motivated by the realization that the higher concentration of Pu^{4+} would be easy to monitor spectrophotometrically and that, if the U^{4+}

reductant generated was sufficient to reduce all of the 4×10^{-3} M Pu^{4+} , it would probably be sufficient to handle the lower concentrations.

In the first test of the mixed $\text{UO}_2^{2+}/\text{Pu}^{4+}$ system, we merely added Pu^{4+} to the already established UO_2^{2+} reduction in the single-phase experiment. When the concentrations of the BuOH and N_2H_4 reductants were each 0.01 M, quantum efficiencies of 0.125 to 0.137 were obtained for the formation of the Pu^{3+} product; and all of the plutonium was apparently reduced to Pu^{3+} , as indicated by a final spectrum of the mixture. However, this system did not test the feasibility of photolyzing an aqueous phase in contact with an organic extractant which had removed a large amount of the two actinides. For this reason, we found it necessary to run a two-phase experiment.

The results of the two-phase photolysis experiments, shown in Table 2, demonstrate that good separation of Pu^{3+} into the aqueous phase can be achieved through this photo-redox stripping procedure. The final distribution coefficients given in the last column support this general conclusion. The quantum efficiencies for Pu^{3+} production are actually secondary values, resulting from the photochemical reduction of uranyl ion followed by the reaction of the reduction products with Pu^{4+} to form Pu^{3+} . (Note that the quantum efficiencies are generally higher than those for U^{4+} production, as given in Sect. 2). All of the experiments were performed with 1 M UO_2^{2+} , and the majority of those shown were run with 4×10^{-3} M Pu^{4+} for the reasons stated earlier. Subsequently, the Pu^{4+} concentration was decreased gradually, by decades, until the detection limit of the PERALS scintillation counting technique was reached at $\sim 4 \times 10^{-6}$ M.

Table 2. Summary of the two-phase experiments

Conditions: Circulation rate = 22 mL/min; $[\text{UO}_2(\text{NO}_3)_2] = 1 \text{ M}$;
 $[\text{HNO}_3] = 1 \text{ M}$; temperature = 25°C unless otherwise noted

Run No.	Concentration (M)			Volume, org/aq (mL)	QY	Method ^c	Final $D_{0/a}(\text{Pu})$
	Pu ^a	Pu ^b	N ₂ H ₄ /BuOH				
60	4×10^{-3}	3.97×10^{-3}	0.01/0.01	35/35	0.137	S	
62	4×10^{-3}	3.84×10^{-3}	0.001/0.001	35/35	0.054	S	
63	4×10^{-3}	3.97×10^{-3}	0.001/0.001	35/35	0.068	S,P	
67 ^d	4×10^{-3}	2.60×10^{-3}	0.001/0.001	35/35	0.076	S	
67 ^d	4×10^{-3}	4.0×10^{-3}	0.01/0.01	35/35	0.114	S	
73	4×10^{-3}	3.4×10^{-3}	0.01/0.01	180/45	0.093	S	
72 ^e	4×10^{-3}	2.6×10^{-3}	0.01/0.01	180/45	0.049	S	
82 ^f	4×10^{-3}	2.8×10^{-3}	0.01/0.01	180/45	0.121	S	
70 ^g	4×10^{-3}	n.a.	0.01/0.01	180/40	0.080	S	
64	4×10^{-4}	3.8×10^{-4}	0.001/0.001	35/35		S,P	0.050
69	4×10^{-4}	3.5×10^{-4}	0.001/0.001	180/45		P	0.037
65	4×10^{-5}	3.9×10^{-5}	0.001/0.001	35/35		P	0.025
74	4×10^{-5}	3.9×10^{-5}	0.01/0.01	180/45		P	0.006
76	4×10^{-6}	3.7×10^{-6}	0.01/0.01	180/45		P	0.02
77	4×10^{-6}	3.7×10^{-6}	0.01/0.01	180/45		P	0.018

^a[Pu] originally present in aqueous phase prior to extraction.

^b[Pu] final in aqueous phase after photoreductive stripping from the organic phase.

^cMethod: S = spectrophotometric; P = PERALS.

^dExperiment run at 50°C.

^eExperiment run at 45°C.

^fPolycarbonate filter used; wavelength >400 nm.

^gMethanol used in place of butanol.

n.a. = not available; however, total reduction to Pu³⁺ was apparent.

Most of the experiments were performed with 0.01 M of each reductant; however, a few were performed with 0.001 M to test the possibility of using smaller amounts of the reductants. As stated earlier, BuOH was selected because it is a product of TBP degradation and therefore would not necessarily be foreign to the system. Nevertheless, other reductants could function almost as well as shown by the data for methanol in experiment 70.

The noticeable drop in quantum efficiency with decreased reductant concentration should be noted. These values were determined from the slopes of the Pu^{3+} growth curves for continuous illumination and not from the ultimate values reached for a limited exposure (see Sect. 2). The slower growth of Pu^{3+} was probably caused by the mixing speeds since the reaction of U^{4+} is known to be rapid; therefore, these decreases in quantum efficiencies represent only the variations in reaction rates due to differences in the steady-state concentrations. Quantum efficiencies were not observed for the lower Pu^{4+} concentrations because, if sufficient reductant was present to reduce the 4×10^{-3} M Pu^{4+} solution, there would obviously be an excess for the lower Pu^{4+} concentrations.

Some of the experiments were conducted with equal amounts of organic and aqueous phases (i.e., the 35/35-mL volume ratios). Under such conditions, the organic phase was saturated with uranyl nitrate, attaining a maximum concentration of 0.5 M and leaving an equivalent amount of uranyl ion in the aqueous solution. However, because the SRP process conditions include a 5:1 organic/aqueous (o/a) volume ratio, we sought to approach these conditions with an increase in the amount of organic phase. The larger organic contactor cell described in Sect. 2 was used to run some

experiments with 5:1, 4.5:1, and 4:1 volume ratios. The contactor could not accommodate the 5:1 ratio satisfactorily because some of the organic phase was driven into the spectrophotometric cell compartment by the pumping operation, thus interfering with the measurement of the spectra. The o/a ratio was successively reduced until the aqueous solution could be circulated with no interference of organic in the optical path. The best results were obtained at the 4:1 ratio, but data from some of the earlier runs at the 4.5:1 ratio have also been included.

The higher o/a ratio system produced more complete extraction of the UO_2^{2+} , leaving only 0.045 M in the aqueous with a final distribution coefficient $D_{o/a} (\text{UO}_2^{2+}) = 5.3$ for the the 4:1 ratio. Under these conditions, the amount of light absorbed during photolysis was less because of the short pathlength of the photolysis chamber and the dilute uranyl nitrate solution. Since less light was being absorbed by the system, the steady-state concentration of the uranium reductant (mainly U^{4+}) was lower; thus, the measured quantum yield appears to be lower. However, an advantage of the dilute solution is that the generation of U^{4+} would be more uniform along the pathlength of the exciting light than in the corresponding solution containing 1 M UO_2^{2+} , where most of the light absorption occurs in the first few millimeters of solution.

The effect of temperature on the photolysis was also examined at $[\text{Pu}^{4+}] = 4 \times 10^{-3}$ M. Solutions with o/a ratios of 1:1 and 4:1 were examined at 50 and 45°C, respectively. The quantum efficiency appears to decrease from 0.137 (at 20°C) to 0.115 (at 50°C) for 1:1 solutions and from 0.093 (at 20°C) to 0.049 (at 45°C) for the 4:1 solutions. Most of this decrease is attributed to an increase in the rate of Pu^{3+} oxidation that we observed at the higher temperatures.

Distribution coefficients of Pu^{4+} and UO_2^{2+} were measured prior to photolysis and were found to be 0.41 ± 0.3 and 5.3 ± 0.8 , respectively, for the 1 M HNO_3 containing 1 M UO_2^{2+} prior to extraction with 4:1 ratio organic that had been pre-equilibrated with acid beforehand. These values agree favorably with those given earlier.^{14,15} It should be noted that the low acid value and the high UO_2^{2+} content cause the Pu^{4+} to favor the aqueous phase; hence, the effects of photochemical reduction and subsequent stripping are not as dramatic. Nevertheless, the final distribution coefficient for plutonium (as shown in the experiments where the counting techniques were employed) has been reduced by at least a factor of 10 to a value which is consistent with that predicted for Pu^{3+} .¹⁶ The range of distribution coefficient values in the last column of Table 2 is probably the result of incomplete equilibration in the solutions.

The final concentration of plutonium in the aqueous phase was found to be at least 87.5% (determined from columns two and three of the table) that of the starting value in all cases except those where the higher temperatures or the polycarbonate filter were used (see Table 2, columns 2 and 3). As stated earlier, the higher temperatures caused some loss in Pu^{3+} stability and, in turn, the lower quantum efficiencies. The decreased stability of the Pu^{3+} is not believed to be a function of the photochemical process; instead, it appears to be related to the nature of the system (i.e., oxidation by NO_x species) and should not reflect on the merits of the photochemical method.

We sought to evaluate the efficacy of using longer-wavelength radiation in the uranium photoreductive stripping of plutonium, since photodegradation of the solvent is of concern. (However, no solvent degradation was observed for any of the experiments described in this report.)

The effect of the polycarbonate filter, which excluded all light of wavelengths shorter than 400 nm, was seen in the amount of Pu^{3+} product formed. It appears that the longer-wavelength light is not satisfactory for complete reduction of Pu^{4+} ; however, we have no explanation for this observation, because our actinometry measurements demonstrate that no less than half of the original number of photons (i.e., those which were passed by the 350-nm cutoff filter) are still being absorbed, and this number should be sufficient for eventual reduction of all the Pu^{4+} .

We have also considered the possible reflux oxidation of the Pu^{3+} that has been identified in the $\text{U}^{4+}/\text{Pu}^{4+}$ reduction process.¹⁷ This procedure was tested in the experiment with 4×10^{-6} M Pu^{4+} where phenylhydrazine was added in place of the H_2N_4 , anticipating that its greater solubility in the organic phase would eliminate any back-oxidation of Pu^{3+} occurring there. No significant difference was achieved with this reductant to warrant the conclusion that these experiments were being limited by this phenomenon.

Figures 5 through 8 summarize some of these results in a manner that includes the effect of time on the concentration of plutonium in the organic phase as Pu^{4+} is being reductively stripped into the aqueous phase. It should be stressed that the methods of circulation and mixing were very rudimentary by reprocessing standards; therefore, the 20-min rates are a function of the mixing system. Figure 5 gives the results for the 4×10^{-3} M plutonium reductive stripping. The horizontal line at the top of the figure is the Pu^{4+} concentration in the aqueous phase prior to extraction; the $[\text{Pu}^{4+}]$ at $t = 0$ is the equilibrium value of the Pu^{4+} after extraction. The two lines are for the same experiment: the line

Fig. 5. Change in the [Pu] as a function of photolysis time in the two-phase extraction system for 4×10^{-3} M total plutonium.

ORNL DWG 86-105

Fig. 6. Change in the [Pu] as a function of photolysis time in the two-phase extraction system for 4×10^{-4} M total plutonium.

ORNL DWG 86-107

Fig. 7. Change in the [Pu] as a function of photolysis time in the two-phase extraction system for 4×10^{-5} M total plutonium.

Fig. 8. Change in the [Pu] as a function of photolysis time in the two-phase extraction system for 4×10^{-6} M total plutonium.

marked "organic" represents values determined by counting techniques; the line marked "spectrophotometric difference" represents values determined by subtracting the concentration found in the aqueous by absorption spectrophotometry from the total concentration. The great divergence of the two lines is due to the error involved in the spectrophotometric measurement at the end of the photolysis - the difference between this measured Pu^{3+} and the starting concentration is a small number with considerable error.

In considering the same type of plot for 4×10^{-4} M (Fig. 6), 4×10^{-5} M (Fig. 7), and 4×10^{-6} M plutonium (Fig. 8), we see similar basic trends. In fact, these figures could be superimposed on each other if the differences in the $[\text{Pu}_T]$ were ignored. The $[\text{Pu}^{4+}]$ starts at a value, when $t = 0$, which is representative of the distribution coefficient for Pu^{4+} . However, we cannot explain the apparent decrease in $D_{O/a}$ (Pu^{4+}) as the concentration of plutonium decreases. The total plutonium concentration decreases at the same rate during the photolysis and to the same relative limits in all of the figures, reaching a limiting value equivalent to $\sim 0.08/4.0$ of (as read from Figs. 5-8, i.e., 2%) the original value. This limiting value is indicative of a Pu^{3+} distribution coefficient, $D_{O/a}$, of 0.02 and is consistent with what is expected.¹⁴

The fact that all of these curves almost overlay each other suggests that the rates are not being controlled by the chemistry of the reduction process, but, instead, by the rate of circulation and mixing (i.e., organic to aqueous transfer). If they were controlled by the chemistry, they would be progressively slower as the total $[\text{Pu}]$ was decreased. These data are encouraging from the standpoint that the rate of separation by reductive stripping in the system tested here is controlled by

the circulation and mixing - factors that would obviously be improved in a more sophisticated reprocessing system.

In support of the conclusions regarding mixing, the results of many of our early experiments gave data points that scattered so badly that the trends indicated in the figures were indiscernible. Observations of the circulating solution revealed that the organic phase was not being adequately agitated by the 22-mL/min stream emptying onto the top surface. Manual stirring prior to sampling dramatically decreased the scatter of points.

The apparent anomaly in the low quantum efficiency for U^{4+} production and the much higher "secondary" values for Pu^{3+} production were of considerable concern since they implied that another reductant besides the U^{4+} was being generated. Realizing the stoichiometry of Eq. (2), we can explain the discrepancy if the values differed by a factor of 2; however, we found that they differ by a factor of 2.5, suggesting the presence of some remaining UO_2^+ [i.e., $U(V)$] to account for the additional reduction of Pu^{4+} . The intermediate UO_2^+ is probably an even more effective reductant than U^{4+} (since an electron is transferred without molecular rearrangement). Nevertheless, the relatively high quantum efficiencies for Pu^{3+} generation satisfy the needs that we are addressing.

4. CONCLUSIONS

The reduction of Pu^{4+} using photochemically generated U^{4+} has been shown to be an efficient process, reaching "secondary" quantum efficiencies as high as 0.137 by using the reductant combination of 0.01 M BuOH

and N_2H_4 . Concentrations of Pu^{4+} as low as $4 \times 10^{-6} \text{ M}$ have been successfully stripped from a TBP/n-dodecane phase with no apparent loss of efficiency. We can then infer that still lower concentrations could be separated with comparable efficiency. The only limiting factor in our testing procedure was the simple method used for stirring and mixing. With improved mixing, a faster rate of reductive separation could be expected.

No degradation of the organic phase was observed as a result of the photolysis with wavelengths $>350 \text{ nm}$. Therefore, the use of light with a wavelength $>400 \text{ nm}$ would be of no significant advantage. The photoreduction was shown to proceed successfully at higher temperatures; the only limitation was a decreased stability of the Pu^{3+} produced. Although the experiments performed in this study focused on the $\text{BuOH}/\text{N}_2\text{H}_4$ reductant combination, a number of other reductants could be used, if some means of stabilizing the Pu^{3+} were provided.

5. REFERENCES

1. J. T. Bell and H. A. Friedman, J. Inorg. Nucl. Chem. 38, 831 (1976).
2. H. A. Friedman, L. M. Toth, and J. T. Bell, J. Inorg. Nucl. Chem. 39, 123 (1977).
3. H. A. Friedman, L. M. Toth, and M. M. Osborne, J. Inorg. Nucl. Chem. 41, 1339 (1979).
4. L. M. Toth and H. A. Friedman, Radiochim. Acta 27, 173 (1980).
5. H. A. Friedman, L. M. Toth, and J. T. Bell, "Method for Selectively Reducing Plutonium Values by a Photochemical Process," U.S. Patent 4,131,527 (1978).

6. H. A. Friedman and L. M. Toth, "Method for Selectively Reducing Neptunium Values by a Photochemical Process," U.S. Patent application, May 12, 1983.
7. G. L. DePoorter and C. K. Rofer-DePoorter, "Method for Photochemical Reduction of Uranyl Nitrate by TBP and Application of This Method to Nuclear Fuel Reprocessing," U.S. Patent 4,080,273.
8. L. M. Toth, J. T. Bell, J. C. Mailen, and K. E. Dodson, An Assessment of Photochemical Applications to Specific Stages in Savannah River Plant Actinide Reprocessing Streams, ORNL/TM-9755, Oak Ridge National Laboratory, December 1985.
9. W. J. McDowell, Alpha Liquid Scintillation Counting: Past, Present, and Future, in Liquid Scintillation Counting, Recent Applications and Development, C. T. Peng, D. L. Horrocks, and E. L. Alpen, eds., Academic Press, New York, 1980.
10. W. J. McDowell, "Simple, Accurate Alpha Assay by Liquid Scintillation Methods," paper presented at the American Nuclear Society Meeting in San Francisco, Calif., Nov. 29-Dec. 5, 1981.
11. J. T. Bell and M. R. Billings, J. Inorg. Nucl. Chem., 37, 2529 (1975).
12. D. T. Bostick and J. E. Strain, The Electroreduction and Photometric Detection of Low-Level Uranium in Aqueous Purex Solutions, ORNL/TM-8646, Oak Ridge National Laboratory, April 1983.
13. J. T. Bell and L. M. Toth, Radiochim. Acta 25, 225 (1978).
14. O. J. Wick, Plutonium Handbook, Vol. 1, American Nuclear Society, LaGrange Park, Illinois, 1980.

15. J. B. Knauer, M. H. Lloyd, and J. T. Barker, Plutonium, Uranium, and Thorium Distribution Coefficients in the 30 vol % TBP-NDD-Nitric Acid System at 25 and 50°C, ORNL/TM-7804, Oak Ridge National Laboratory, April 1982.
16. W. D. Bond, private communication, Oak Ridge National Laboratory, 1985.
17. P. Biddle, H. A. C. McKay, and J. H. Miles, "The Role of Nitrous Acid in the Reduction of Plutonium(IV) by Uranium(IV) in TBP Systems," in H. A. C. McKay, T. V. Healy, I. L. Jenkins, A. Naylor, eds., Solvent Extraction Chemistry of Metals, MacMillan, London, 1965, p. 133.

INTERNAL DISTRIBUTION

- 1-5. J. T. Bell
- 6. W. D. Burch
- 7. D. O. Campbell
- 8-12. K. E. Dodson
- 13. R. L. Fellows
- 14. D. E. Ferguson
- 15. W. S. Groenier
- 16. J. R. Hightower
- 17. E. K. Johnson
- 18. R. E. Leuze
- 19-23. J. C. Mailen
- 24. F. R. Mynatt
- 25. D. J. Pruett
- 26. R. H. Rainey
- 27. J. H. Shaffer
- 28-30. F. M. Scheitlin
- 31-33. M. G. Stewart
- 34-38. L. M. Toth
- 39. M. E. Whatley
- 40. R. G. Wymer
- 41. Central Research Library
- 42. ORNL/Y-12 Technical Library,
Document Reference Section
- 43-44. Laboratory Records
- 45. Laboratory Records, R.C.
- 46. ORNL Patent Section

EXTERNAL DISTRIBUTION

- 47-51. S. T. Augsberger, Savannah River Laboratory, Aiken, SC 29808-0001
- 52. H. D. Harmon, Savannah River Laboratory, Aiken, SC 29808-0001
- 53. R. T. Walters, Savannah River Laboratory, Aiken, SC 29808-0001
- 54. Office of Assistant Manager for Energy Research and Development,
DOE-ORO, P.O. Box E, Oak Ridge, TN 37831
- 55-56. Technical Information Center, Oak Ridge, TN 37830