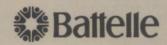
# WESF Cesium Capsule Behavior at High Temperature or During Thermal Cycling

G. L. Tingey W. J. Gray R. J. Shippell Y. B. Katayama

June 1985

Prepared for the U.S. Department of Energy under Contract DE-AC06-76RLO 1830

Pacific Northwest Laboratory
Operated for the U.S. Department of Energy
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Printed in the United States of America Available from National Technical Information Service United States Department of Commerce 5285 Port Royal Road Springfield, Virginia 22161

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#### SUMMARY

A Hanford Waste Encapsulation and Storage Facility (WESF) capsule was filled with a nonradioactive salt mixture to simulate cesium chloride that had decayed for 1 half-life (30 years). The impurities found in the radioactive WESF capsules were also included in the salt mixture. This capsule, which was filled to a loading factor of 3.11 g of salt/cm<sup>3</sup> of inner capsule volume, was heated in an 800°C furnace for 90 min to examine the possible effects from an accidental fire. The end cap weld of the inner capsule failed, opening a small crack and allowing the molten salt mixture to flow into the annulus between the inner and outer capsules. The salt, however, was contained in the outer capsule, which has the same wall thickness as the inner capsule. Corrosion of the inner capsule wall by the salt was quite high (0.005 to 0.010 in./h), as expected at this temperature; however, corrosion should not be of consequence in a short-duration fire.

Based on the results of this test, WESF capsules can be used safely even in the unlikely event of a short-duration  $800^{\circ}\text{C}$  temperature excursion. Capsules with loading factors equal to or less than 2.71 g/cm<sup>3</sup> are not expected to be stressed on either the inner or outer capsule walls, thus assuring capsule integrity. About 40% of the capsules in the WESF pool have loading factors higher than 2.71 g/cm<sup>3</sup> and could potentially have inner capsule weld failures under the conditions of this test, but none of the capsules planned for irradiator use has a loading factor high enough to produce stress from salt expansion on the outer capsule. Since the most highly loaded capsules may have inner end cap weld failure when subjected to a fire, consideration should be given to removing them from service if they are ever subjected to such high temperatures.

In a wet storage/dry operation irradiator, capsules are thermally cycled as they are transferred from the water storage pool to the air environment. The results of a finite-element analysis of thermal cycling indicate that the most severe thermal stresses are placed on the outer capsule bottom end cap weld. This weld is subjected to temperature differences up to 36°C across the weld when the capsules are lowered into the water. The stresses generated by

such temperature differences are projected to produce a crack growth rate of  $2.9 \times 10^{-9}$  in./cycle. Thus, a weld crack of only 0.0001 in. would be expected from a capsule after  $3.5 \times 10^4$  cycles (94 years if cycled once each day).

An experiment was conducted to verify the theoretical calculations using WESF capsules filled with radioactive cesium chloride. After 3845 cycles, no evidence of cracking or thermal stress was observed on either of the outer capsule welds during high-resolution metallographic examination. Although this short-term test does not quantitatively verify the projection, it does indicate that no unanticipated, observable thermal effects occur in the first 3800 cycles. The test simulates the stresses expected during 10.5 years of operation with the capsules cycled once each day during irradiator operation.

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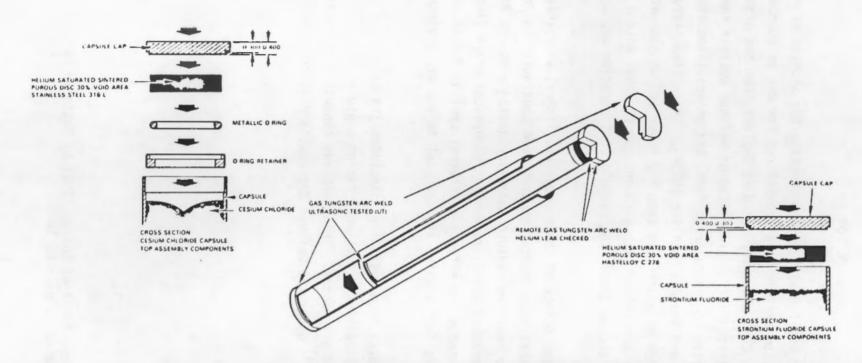
#### INTRODUCTION

Double-walled stainless steel (SS) capsules prepared for storage of radio-active <sup>137</sup>Cs from defense waste are now being considered for use as sources for commercial irradiation. Cesium was recovered at B-plant from the high-level radioactive waste generated during processing of defense nuclear fuel. It was then purified, converted to the chloride form, and encapsulated at the Hanford Waste Encapsulation and Storage Facility (WESF). The molten cesium chloride salt was encapsulated by pouring it into the inner of two concentric SS cylinders. Each cylinder was fitted with a SS end cap that was welded in place by inert gas-tungsten arc welding. The capsule configuration and dimensions are shown in Figure 1.

In a recent review of the safety of these capsules, Tingey, Wheelwright, and Lytle (1984) indicated that experimental studies were continuing to produce long-term corrosion data, to reaffirm capsule integrity during a 90-min fire where capsule temperatures reached 800°C, to monitor mechanical properties as a function of time, and to assess the effects of thermal cycling due to periodic transfer of the capsules from a water storage pool to the air environment of an irradiator facility.

This report covers results from tests that simulated the effects of the 90-min fire and from thermal cycling actual WESF cesium capsules for 3845 cycles over a period of six months. This work was conducted by Pacific Northwest Laboratory (PNL)<sup>(a)</sup> under the Defense Byproducts Production and Utilization Program.

<sup>(</sup>a) Operated for the U.S. Department of Energy (DOE) by Battelle Memorial Institute under Contract DE-ACO6-76RLO 1830.



					CAP	SULE				
			INNER					OUTER		
	MATERIAL	WALL THICKNESS	DIAMETER	LENGTH	TOTAL CAP	MATERIAL	THICKNESS	DIAMETER	TOTAL	TOTAL CAR
STRONTIUM	HASTELLOY C 276	0 136	2 260	19 050	0 400	STAINLESS STEEL 316 L	0 136	2 025	20 100	0 400
CESIUM	STAINLESS STEEL 316 L	0 138	2 250	19 725	0 400	STAINLESS STEEL 316 L	0 136	2 025	20 775	0 400

RCP8303 24A

FIGURE 1. Detail of WESF Capsules

## CAPSULE SWELLING DUE TO THERMAL EXPANSION OF CESIUM CHLORIDE AT 800°C

Pure cesium chloride expands to about 1.5 times its initial volume when heated from 25 to 800°C. Components of this expansion include thermal expansion of the solid and liquid phases, an 18% expansion during the crystalline phase transition from body-centered cubic to the face-centered cubic structure that occurs at 469°C, and a 10% expansion during the solid-to-liquid transition at 645°C. In the event of a fire, which could increase the temperature of the capsule and its contents to 800°C, volume expansion has the potential for stressing the WESF capsules.

The radioactive WESF capsules were filled with salt by pouring molten cesium chloride into the inner container at a salt temperature of 700 to 750°C. During cooling to its steady-state temperature, the salt shrinks, leaving a substantial void volume at the top. This void volume would normally be sufficient to accommodate any expansion occurring at salt temperatures up to 750°C. In some cases, however, additional molten cesium chloride was added after cooling to "top off" those capsules that had been only partially filled. Additional molten cesium chloride may have been allowed to flow into the capsule during cooldown. Thus, the high-temperature salt volume may be greater than the inner capsule internal volume, which would cause the inner capsule to swell or even rupture if it were subjected to high temperatures after closure.

Capsule loading factors (salt weight/capsule volume at 25°C) were determined for WESF capsules using the measured salt mass and the nominal capsule volume. Using these data, the extent of swelling required to accommodate the salt volume at any temperature (if the salt were pure cesium chloride) can be estimated. However, since there is 10% to 20% impurity in the salt initially and since <sup>137</sup>Cs decays to <sup>137</sup>Ba, the specific volume of the salt mixture at temperature cannot be calculated directly. An experiment was conducted to measure the potential for swelling of the capsules using a nonradioactive salt mixture to simulate the WESF salt after 1 half-life (30 years) of decay time. The following paragraphs describe the test and the results of the study.

#### TEST PROCEDURE AND RESULTS

Test procedures (including the salt preparation and characterization), capsule characterization, loading factor, closure, heat treatment, and post-heat-treatment analysis are described in this section. Test results are also presented.

#### Salt Preparation and Characterization

The salt composition was chosen to be representative of the WESF salt after 1 half-life (30 years) of \$^{137}Cs decay. Thus, barium was added assuming that 33% of the original Cs was \$^{137}Cs (Tingey, Wheelwright, and Lytle 1984, p. 17). In addition, the salt mixture included impurities based on the most likely composition from previous analyses (Tingey, Wheelwright, and Lytle 1984, p. 15). Table 1 gives the nominal and analyzed compositions of the salt that was prepared and lists the actual compounds that went into the mixture.

When  $^{137}\text{Cs}$  decays to  $^{137}\text{Ba}$ , a disproportionation reaction such as the following is postulated:

Therefore, half the Ba was included in metallic form and half in chloride form. The metallic Ba was a solid rod that was cut into roughly 1 mm x 3 mm x 3 mm chunks in an argon glove box; the oxygen concentration was maintained below 10 ppm. All the chemicals were in anhydrous form, either purchased or dried in a laboratory oven before use. Approximately 4 kg of the mixture and some  $Al_2O_3$  balls were sealed in a large polyethylene bottle in the argon glove box. The bottle was then removed from the glove box and rolled on a ball mill for ~1 h to thoroughly mix the chemicals.

After mixing, the chemicals were returned to the argon glove box, melted, and poured into the inner capsule. Each melt consisted of about 500 g of salt that was placed in a SS crucible and then in a furnace preheated to 800°C. As the mixture began to melt, the Ba metal chunks lying near the surface were observed to undergo some type of rather violent reaction that caused some material to splatter from the crucible. Bubbling and frothing also occurred, indicating gas release. However, these reactions were of short duration and little

TABLE 1. Salt Composition

Nomina				Analyzed	
Compound Formula	wt%	Symbol Elem	wt%	First Melt, wt%	Last Melt, wt%
A1203.10Si02	4.84	A1	0.37	0.50	. 0.31
B203	0.91	В	0.28	0.39	0.15
BaC12	7.75	Ba	10.24	8.8	9.6
Ba	5.11				
CaC12	0.13	Ca	0.048	0.10	0.10
CdC12	0.049	Cd	0.030	0.03	0.03
CsC1	63.45	Cs	50.08	45.9	47.6
CoCl	0.20	Co	0.091	0.076	0.065
CrC1 <sub>3</sub>	3.92	Cr	1.29	1.3	1.1
CuCl <sub>2</sub>	0.38	Cu	0.18	0.17	0.17
FeC12	4.26	Fe	1.87	2.0	1.8
KC1	1.21	K	0.63	0.5	0.5
Mn0 <sub>2</sub>	0.074	Mn	0.046	0.05	0.05
Na <sub>2</sub> SO <sub>4</sub>	6.53	Na	2.34 <sup>(a)</sup>	2.9	2.4
NiC12	0.20	Ni	0.092	NA(b)	NA
Na <sub>3</sub> PO <sub>4</sub>	0.53	Р	0.10	0.1	0.1
PbC1 <sub>2</sub>	0.17	Pb	0.13	0.076	0.071
PdC12	0.031	Pd	0.019	NA	NA
RbC1	0.026	Rb	0.019	NA	NA
SrC12	0.034	Sr	0.019	0.053	0.057
TiO <sub>2</sub>	0.11	Ti	0.067	0.088	0.067
ZnC1 <sub>2</sub>	0.090	Zn	0.044	0.058	0.043
The latest acres		C1	22.24	23.0	22.6
7.470		0	5.65	NA	NA
Date Halling		S	1.47	1.1	1.0
		Si	1.94	2.7	1.5
	100.0		99.3	89.9	89.3

<sup>(</sup>a) Includes Na from Na<sub>3</sub>PO<sub>4</sub>.(b) NA = not analyzed.

material was lost from the crucible. After melting appeared to be complete, the mixture was maintained at 800°C for 20 to 30 min before pouring. Six melts were required to fill the capsule. In addition, the first and last melts were each poured into separate cans for subsequent characterization; thus, eight melts were prepared.

The densities of two salt samples from the first melt were measured by immersion in kerosene whose density had been measured by immersion of a machined aluminum cylinder. The sample densities were 3.291 and 3.297 g/cm<sup>3</sup>, respectively; the density of pure CsCl is 4.04 g/cm<sup>3</sup> (Tingey, Wheelwright, and Lytle 1984, p. 28). The close agreement of the measured densities for the two samples suggests a lack of large closed pores since it would be unlikely that both samples would have equal volumes of such pores. A uniform distribution of fine porosity cannot be excluded, however.

Thermal expansion coefficients were measured for pure CsCl and for a sample of the salt mixture from the first melt using a Perkin-Elmer TMS-2 thermomechanical analyzer. A value of 1.23 x  $10^{-4}/^{\circ}$ C in the range from 140 to 325°C was obtained for the impure mixture. Results for pure CsCl were 1.70 x  $10^{-4}/^{\circ}$ C over the range from 200 to 450°C; a value of 1.89 x  $10^{-4}/^{\circ}$ C over the range from 200 to 460°C has been published by Tingey, Wheelwright, and Lytle (1984, p. 28). Although the measured value for pure CsCl is slightly lower than the published coefficient, it is clear that the salt mixture has a lower thermal expansion coefficient than pure salt. The thermal expansion data indicated a melting point of ~430°C for the salt mixture but gave no indication of a crystallographic transformation such as exists for pure CsCl.

The liquid density of the salt mixture was determined at  $800^{\circ}\text{C}$  by measuring the bouyancy of a platinum bob in a sample of salt taken from the capsule after the high-temperature test. The measured density was  $2.57~\text{g/cm}^3$ , which is close to the value of  $2.65~\text{g/cm}^3$  given for pure CsCl (Tingey, Wheelwright, and Lytle 1984, p. 28) despite the fact that the two densities are much different at  $25^{\circ}\text{C}$  ( $4.04~\text{g/cm}^3$  for pure salt and  $3.29~\text{g/cm}^3$  for the salt mixture).

Both the first and last melts were chemically analyzed for all but a few constituents, the most notable exception being oxygen. The composition of the two samples was essentially the same and in good agreement with that

calculated from the compounds added in the preparation (Table 1). The lower fraction of cesium was expected due to vaporization losses during melting. Barium may also have vaporized somewhat, and some may have been lost due to splattering that occurred during the violent reaction during initial melting. If the nominal values for 0, Ni, Pd, and Rb are added to the total analyzed constituents, the sum is greater than 95% of the sample weight, indicating a reasonably complete mass balance.

## Capsule Characterization and Loading Factor

Tables 2 and 3 list the measured dimensions for the inner and outer capsules used in this study. The locations of the dimensional measurements, the end caps, the sintered disk, and the 0-ring and retainer that are placed in the ends of the capsules are shown in Figure 2. The inner wall at the ends of the capsules was machined from 0.14 to 0.08 in. to accommodate the 0-ring and retainer. Additional capsule dimensions are given in Table 4.

The capsule loading factor is defined as the weight of the salt divided by the capsule volume at 25°C. The loading factor selected for this test was calculated to cause the inner capsule diameter to swell just enough at 800°C to fill the annulus between the inner and outer capsules. Since the density and thermal expansion of the salt mixture were not known, the values for pure CsCl were used even though it was recognized that this might lead to considerable error. The volume of the inner capsule available to the salt was assumed to be the total free volume excluding the porosity in the sintered metal disk. To this volume, the volume of the annulus between the inner and outer capsules over the inside length of the inner capsule was added. After adjusting for thermal expansion, this volume at 800°C was determined to be 1120 cm<sup>3</sup>. Since the density of CsCl at 800°C is 2.65 g/cm<sup>3</sup> (Tingey, Wheelwright, and Lytle 1984, p. 28), the desired weight of the salt was taken to be 2967 g. The room temperature volume of the inner capsule up to the bottom of the sintered metal disk (see Table 4) was 918 cm3. Thus, the desired loading factor was  $2967/918 = 3.23 \text{ g/cm}^3$ .

TABLE 2. Inner Capsule Dimensions

Measurement	Dimensions, in.					
Position(a)	Initial	After Welding	After Heat Treatment(b)	Difference(c)		
1A	2.2533	2.2537	2.4054	0.1517		
2A	2.2537	2.2537	2.2859	0.0322		
3A	2.2541	2.2543	2.2848	0.0305		
4A	2.2523	2,2532	2.2880	0.0359		
5A	2.2528	2.2540	2.2899	0.0359		
6A	2.2530	2.2530	2.3000	0.0470		
1B	2.2538	2.2525	2.4021	0.1496		
2B	2.2548	2.2545	2.2880	0.0335		
3B	2.2538	2.2537	2.2897	0.0360		
4B	2.2535	2.2531	2.2862	0.0331		
5B	2.2534	2.2531	2.2884	0.0353		
6B	2.2530	2.2523	2.3000	0.0477		
10	19.7172	19.7090	19.7350	0.0260		
20	19.7171	19.7098	19.7455	0.0357		
3C	19.7191	19.7083	19.7341	0.0258		
4C	19.7185	19.7090	19.7283	0.0193		
5C	19.7188	19.7093	19.7357	0.0264		

<sup>(</sup>a) See Figure 2 for position locations; A and B positions are diameters, 90° apart; C dimensions are lengths.
(b) 90-min 810°C heat treatment.
(c) Difference was taken between "after heat treatment" and "after welding"

measurements.

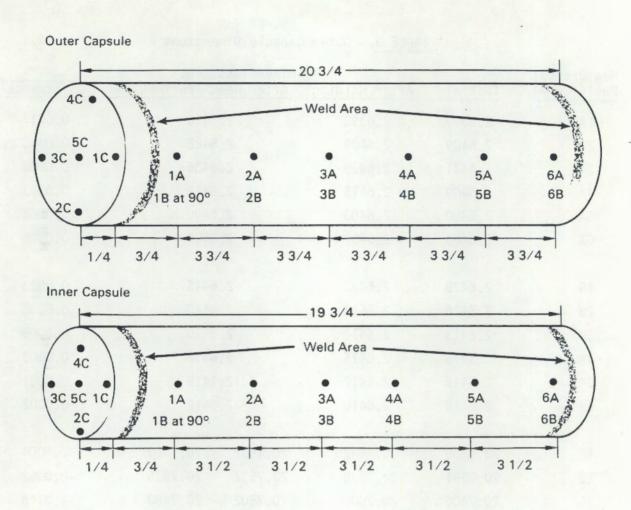
TABLE 3. Outer Capsule Dimensions

Measurement			Dimensions, in.		
Position(a)	Initial	After Welding	After Hea	t Treatment(b)	Difference(C)
. 1A	2.6399	2.6398	2.	6415	0.0017
2A	2.6409	2.6409	2.	6422	0.0013
3A	2.6431	2.6428	2.	6436	0.0008
4A	2.6409	2.6415	2.	6418	0.0003
5A	2,6400	2.6403	2.	6405	0.0002
6A	2.6400	2.6403	2.	6409	0.0006
18	2.6420	2.6422	2.	6445	0.0023
2B	2.6416	2.6512	2.	6420	0.0008
3B	2.6413	2.6412	2.	6420	0.0008
4B	2.6416	2.6415	2.	6418	0.0003
5B	2.6415	2.6417	2.	1418	0.0001
6B	2.6410	2.6410	2.	6412	0.0002
10	20.7589	20.7592	20.7592	20.7600	0.0004
2C	20.7591	20.7598	20.7537	20.7535	-0.0062
3C	20.7606	20.7609	20.7502	20.7480	-0.0118
4C	20.7552	20.7558	20.7540	20.7528	-0.0024
5C	20.7575	20.7568	20.7513	20.7510	-0.0057

<sup>(</sup>a) See Figure 2 for position locations; A and B positions are diameters, 90° apart; C positions are lengths.

<sup>(</sup>b) Lengths were measured twice after the 90-min 810°C heat treatment as a measure of the reproducibility of the measurements.(c) Difference was taken between "after heat treatment" and "after welding"

measurements.



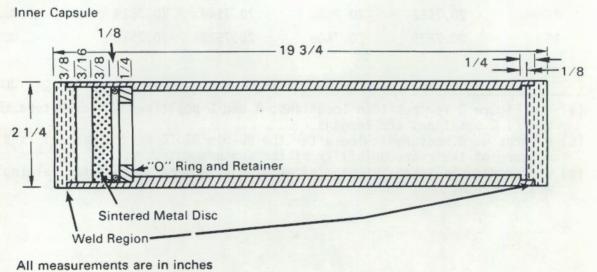


FIGURE 2. WESF Capsules Showing Measurement Locations and Other Parameters (A and B diameter measurement locations are 90° apart)

TABLE 4. Additional Capsule Parameters

	Inner Capsule	Outer Capsule
Inside diameter, cm	5.00(a)	6.10
Internal volume, cm <sup>3(b)</sup>	918 <sup>(c)</sup>	1480
External volume, cm <sup>3(d)</sup>	1290	Carl Carl Carl Carl

(a) Calculated based on measured internal volume.

(b) Determined by the weight of water required to fill the capsules.

(c) Includes only that volume up to the bottom of the sintered metal disk.

(d) Determined by measuring the amount of water displaced by the capsule.

The salt mixture was melted and poured into the inner capsule inside an argon glove box; the oxygen concentration was kept below 10 ppm. Even though the capsule was as full as possible, the total weight of salt was only 2854 g (113 g short of the desired amount), resulting in a loading factor of 3.11 g/cm $^3$ . This lower-than-anticipated loading factor occurred because the salt mixture had a lower density (3.29 g/cm $^3$ ) than pure CsCl (4.04 g/cm $^3$ ).

## Capsule Closure and Heat Treatment

After the inner capsule was filled, a plastic bag was taped over the open end to prevent air from contacting the salt. The capsule was then transferred to a helium glove box where it was evacuated and backfilled with helium to allow for leak checking after welding the end caps in place. While in the helium glove box, the sintered metal frit was inserted and the end cap was placed on the capsule. Again, a plastic bag was taped over the end to minimize exchange of air for He during transport of the capsule to the welding facility. The end cap was welded in place by Rockwell Hanford Company personnel using the same technique established for radioactive WESF capsules (Tingey, Wheelwright, and Lytle 1984, pp. 45-46). No leaks were detected when the entire capsule was checked at a sensitivity of 2 x 10<sup>-10</sup> std. cm<sup>3</sup> of helium/s.

The inner capsule was placed inside the outer capsule in a helium glove box, the end cap was set in place, and a plastic bag was taped over the end. The double capsule was then transported to the Rockwell welding facility where the end cap was welded on the outer capsule. The entire outer capsule was leak checked; again, no leaks were detected at a sensitivity of  $2 \times 10^{-10}$  std. cm<sup>3</sup> of helium/s.

The capsule was placed horizontally in a large furnace that had been preheated to 810°C. The temperature and temperature uniformity were established by seven thermocouples attached to a 2-3/4-in. diameter x 24-in. long pipe lying beside the capsule. The end-to-end temperature varied from 810 to 815°C at the beginning of the test and from 808 to 812°C at the end of the test. After 90 min in the 810°C furnace, the capsule was removed and allowed to cool in air while standing on end on a fire brick. Thermal calculations indicated that a capsule initially at 25°C would require about 30 min to reach 800°C when placed in a preheated furnace. Radioactive decay heat in a WESF capsule would make a negligible contribution to the heating rate but could increase the steady-state temperature of the inner capsule by as much as 80°C. This increase was calculated to be only 10°C if the inner capsule swelled or if salt filled the annulus. Therefore, the 90-min heating period should exceed the time at temperature from a 30-min fire by more than a factor of two.

## Post-Heat-Treatment Analyses

After heat treatment, the outer capsule was again leak checked at  $2 \times 10^{-10}$  std. cm<sup>3</sup> of helium/s and found to be leak free. Capsule dimensional measurements given in Table 3 show that, except for a very slight reduction in length, the outer capsule suffered essentially no dimensional changes as a result of the heat treatment.

Following visual observations and careful dimensional measurements, the ends of the outer capsule were cut off and the inner capsule was removed. Considerable force was required to push the inner capsule from the outer capsule because the thin-wall region of the inner capsule had bulged against the outer capsule wall. The end cap weld had developed a small crack, allowing 150 g of salt to flow into the annulus between the inner and outer capsules. Figure 3 shows the bulged end of the capsule encrusted with salt just as it was pushed from the outer capsule. Figure 4, which is the same area after the salt was cleaned off, shows the failed region of the weld covering an arc of nearly 90°.

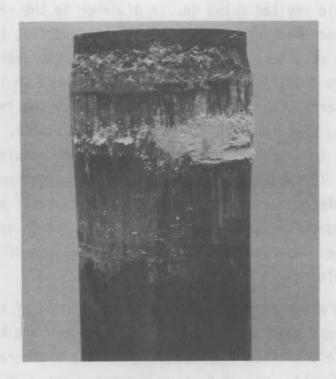


FIGURE 3. Inner Capsule Showing Bulged End, Failed Weld Area, and Salt Incrustation

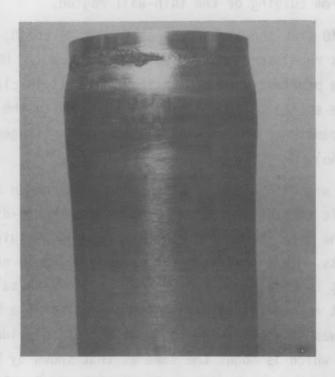


FIGURE 4. Inner Capsule After Salt Removal (note bulge and failed weld)

The inner capsule swelled 0.150 in. in diameter in the region of the bulge where the wall had been machined to accommodate the end cap, the sintered metal disk, and the 0-ring retainer. Other than the bulge, the average swelling of the inner capsule was 0.037 in. in diameter and 0.027 in. in length (see Table 2), which compares with the diametral gap between the inner and outer capsules of 0.149 in. (see inner diameter listed in Table 4). Obviously, the bulged region of the inner capsule contacted the outer capsule, preventing additional bulging. If the weld had not cracked, allowing the salt to flow from the inner capsule, the diameter would have had to swell an average of 0.092 in. to accommodate the salt volume. This calculation is based on extrapolation of the swelled inner capsule volume and the weight of salt extruded from the inner capsule.

Figures 5 and 6 are micrographs of the failed region of the weld; Figures 7 and 8 are micrographs taken 180° from the crack where the weld remained intact. Considerable corrosion is evident in all the micrographs, even in areas 1 in. from the weld. Figure 8 shows corrosive attack 0.010 in. deep near the intact weld region, apparently caused by the combination of corrosion and mechanical stress from bulging of the thin-wall region.

Figures 9 and 10 show the same areas as Figures 6 and 8, respectively, after the sample was repolished and etched (using a 1 HC1/2  $HN0_3/3$  glycerin mixture) to show the penetration depth of the weld. It is clear that the weld fully penetrated the entire thickness of the wall despite the requirement that the weld need only penetrate 55% of the wall thickness (Tingey, Wheelwright, and Lytle 1984, p. 45).

Figure 11 is a scanning electron micrograph (SEM) of an area near the center of the capsule showing the SS/salt boundary. Energy-dispersive x-ray (EDX) analyses of the area showed that the dark regions within the salt were SS corrosion products. This corrosion was intergranular in nature, and EDX analysis showed that the grain boundaries were filled with salt. The prominant white spots are salt crystals that have pushed up out of the boundary regions after the specimen was polished. The depth of corrosion evident in Figure 11 is about 0.008 in., which is about the same as that shown by Figure 8 (exclusive of the cracks) near the weld area. Figure 12 is a 500X magnification



 $\underline{\mathsf{FIGURE}\ 5}.$  Macrographic Cross Section of Inner Capsule in Vicinity of Failed Weld

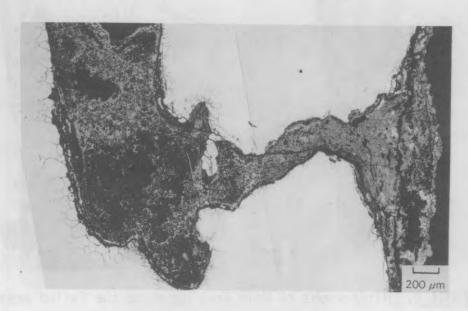


FIGURE 6. Micrograph of Failed Weld

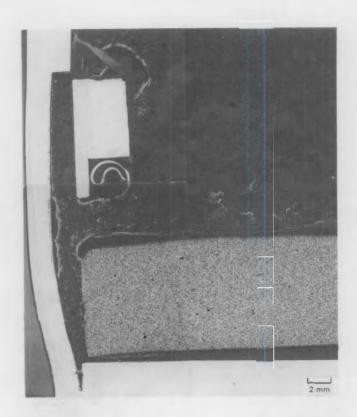


FIGURE 7. Macrographic Cross Section of Inner Capsule Opposite the Failed Weld Area

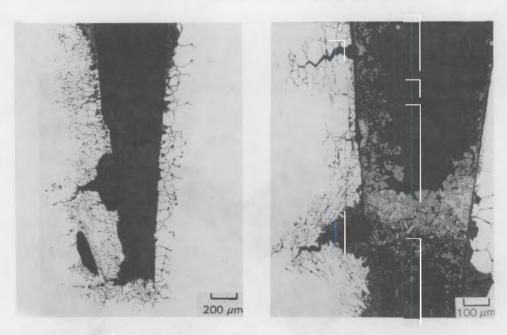


FIGURE 8. Micrographs of Weld Area Opposite the Failed Area

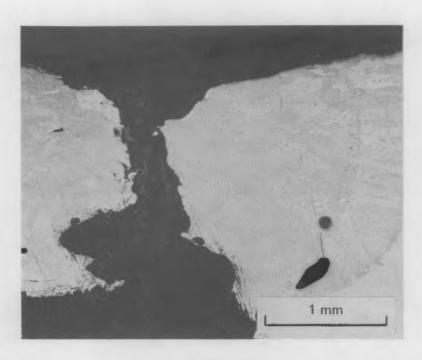


FIGURE 9. Failed Weld Area (Figure 6) After
Repolishing and Etching

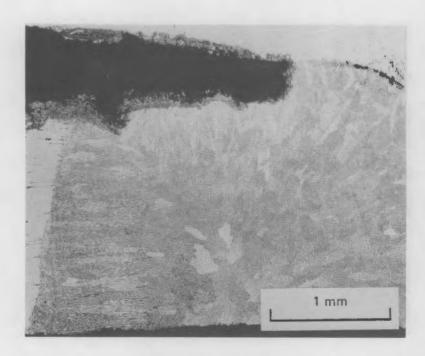


FIGURE 10. Weld Area Opposite the Failed Area (Figure 8)
After Repolishing and Etching



FIGURE 11. Micrograph of Center Region of Inner Capsule Showing the Salt/Stainless Steel Boundary

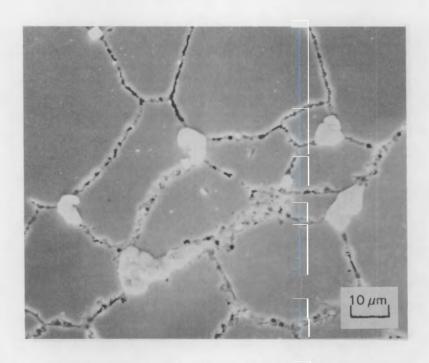


FIGURE 12. Magnification of the Stainless Steel Region in Figure 11

of the stainless steel area in Figure 11. Figure 13 is an SEM of the sintered metal disk. EDX analyses showed that the open porosity within the disk was essentially completely filled with salt.

During examination of the salt after the heat treatment, it was evident that some separation had occurred. Two different regions were observed by an obvious color difference. The top 2 in.  $^{(a)}$  of salt was a very light green color; the rest of the salt was gray. Samples were taken from both regions and analyzed; the results are shown in Table 5. The mass balance is not as good as the earlier analyses (Table 1) but shows that the composition difference between the two regions is small.

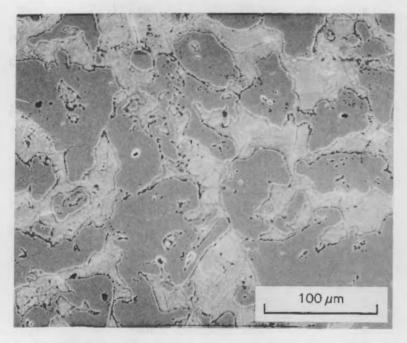


FIGURE 13. Micrograph of Sintered Disk

<sup>(</sup>a) The top of the capsule as it stood on end during cooling was the bottom of the capsule as filled and is the opposite end from the weld failure.

TABLE 5. Composition of Salt in Capsule After Heat Treatment

		Composition, w	t%
Element	Nomi nal	Green Salt	Gray Salt
A1	0.37	ND(a)	0.34
В	0.28	0.02	0.23
Ba	10.24	7.81	9.49
Ca	0.048	0.08	0.08
Cd	0.030	0.04	0.03
Cs	50.08	50.2	42.2
Co	0.091	0.03	nonger 0.12 and maswind some
Cr	1.29	0.04	1.76
Cu	0.18	0.10	0.17
Fe	1.87	1.23	2.91
K	0.63	NA(b)	NA
Mn	0.046	0.07	0.06
Na	2.34	2.91	2.15
Ni	0.092	NA	NA
Р	0.10	ND	0.14
Pb	0.13	0.18	0.16
Pd	0.019	NA	NA
Rb	0.019	NA	NA
Sr	0.019	0.05	0.07
Ti	0.067	ND	0.09
Zn	0.044	0.06	0.06
C1	22.24	19.1	20.5
0	5.65	NA	NA
S	1.47	NO	1.1
Si	1.94	0.07	1.82
	99.3	82.0	83.5

<sup>(</sup>a) ND = not detected.

<sup>(</sup>b) NA = not analyzed.

#### DISCUSSION

The primary purpose of this part of the study was to determine the behavior of a WESF cesium capsule with a high loading of salt. This objective was completed. In addition, more basic data such as salt density at 800°C, loading factors of capsules that can be accommodated, and capsule corrosion rates can be inferred.

The results of this test indicated that initial capsule swelling will occur in the section of the capsule wall that was machined to 0.080 in. to allow for insertion of the end cap, sintered disk, and 0-ring retainer. Expansion of this area of the capsule continued until it reached the outer capsule wall and apparently placed considerable torque and/or other modes of stress on the top weld. Additional expansion of the salt would be accommodated by either expansion of the thicker area of the wall or fracture of the weld. The actual behavior will depend on the ductility of the capsule wall and the strength of the weld. In the test conducted for this study, only minimal swelling occurred (0.037 in. in diameter and 0.027 in. in length) before a small crack developed in the weld.

A measure of the inner capsule expansion and the amount of salt extruded into the annulus can be used to estimate the salt density at  $800^{\circ}\text{C}$ . The internal volume of the inner capsule (excluding the volume of the sintered disk) after heat treatment was determined to be about  $960 \text{ cm}^3$  at  $25^{\circ}\text{C}$ , both from dimensional measurements and from water displacement of the inner container (expansion of the internal volume was assumed to be equal to the expansion of the external volume). Thus, at  $800^{\circ}\text{C}$ , the capsule volume is calculated to be  $1005 \text{ cm}^3$  due to thermal expansion of the stainless steel. In addition, the open pore volume of the sintered disk ( $^{\circ}6 \text{ cm}^3$ ) was also filled with salt, giving a total volume at  $800^{\circ}\text{C}$  of  $1011 \text{ cm}^3$ . The initial salt content of the capsule was 2854 g; but after the high-temperature treatment, only 2704 g remained in the inner capsule. Thus, the density of this salt mixture at  $800^{\circ}\text{C}$  is  $2704 \text{ g}/1011 \text{ cm}^3$  or  $2.67 \text{ g/cm}^3$ , which is in reasonable agreement with the measured value of  $2.57 \text{ g/cm}^3$ . Because these values are close to the density of pure CsCl  $(2.65 \text{ g/cm}^3)$ , the uncertainty in density at  $800^{\circ}\text{C}$  due to

uncertainty in salt composition is minimized. Nevertheless, the more conservative value (2.57  $g/cm^3$ ) is used in the following calculations.

Because the weld of the inner capsule failed in this test, it is worth considering capsule loading factors below which the welds would not fail. In this test, where 2704 g of salt remained in the capsule, it would appear that any loading factor below 2704  $q/918 \text{ cm}^3 = 2.95 \text{ g/cm}^3$  would not have resulted in weld failure. However, the capsule did swell somewhat prior to failure. A more conservative assessment indicates that weld failure might occur after only minimal swelling. The salt weight where swelling is expected to begin is  $2.57 \text{ g/cm}^3 \times 968 \text{ cm}^3 = 2488 \text{ g}$  (the volume of the unswelled capsule below the sintered disk at 800°C is 968 cm<sup>3</sup>). Thus, the maximum room temperature loading factor that would not produce capsule swelling is 2488 g/918 cm<sup>3</sup> = 2.71 g/cm<sup>3</sup>. Of the 1500 cesium capsules stored in the WESF pool, about 600 (40%) have loading factors above this value (Tingey, Wheelwright, and Lytle 1984, p. 35) and, thus, have potential for inner capsule weld failure if subjected to a fire that results in a salt temperature of 800°C. The number of capsules with potential weld failure is reduced to about 300 (20%) if a salt density at 800°C of 2.67 g/cm<sup>3</sup> is used. It would be further reduced if the salt penetrated the sintered disk and if capsule swelling occurred prior to weld failure.

Inner capsule weld failure would result in the flow of molten radioactive salt into the annulus of the outer capsule. The outer capsule has the same wall thickness and weld parameters as the inner capsule but has had only minimal machining of the wall to allow for insertion of the end caps. Furthermore, this capsule was not subjected to high-temperature salt during filling, storage, or use and has met quality control standards for the welds including ultrasonic weld inspection. Therefore, the radioactive salt would be contained in a high-quality outer capsule, although it would no longer be doubly encapsulated. Thus, the capsules would not be expected to meet regulatory requirements for continued use in an irradiator after being subjected to an 800°C fire; it would be necessary to return them for reencapsulation and/or disposal.

It is also instructive to assess the potential for outer capsule swelling. If the annular volume were available to the salt, either from inner capsule swelling or from weld failure, the salt volume could reach 1120 cm<sup>3</sup> at 800°C

before mechanical stress on the outer capsule would occur. Therefore, 2.57  $\rm g/cm^3 \times 1120~cm^3 = 2878~g$  of salt could be accommodated. Thus, any loading factor below 2878  $\rm g/918~cm^3 = 3.14~g/cm^3$  would not produce outer capsule stress if the salt composition were the same as used in this study.

Of the present capsules in the WESF pool, 21 have loading factors above  $3.14~\rm g/cm^3$ . However, all of these (about 1 MCi of  $^{137}\rm Cs$ ) have been reserved for reencapsulation for other source needs (Tingey, Wheelwright, and Lytle 1984, p. 37) and, thus, will not be available for irradiator use. The loading factor required to produce outer capsule stress increases to  $3.28~\rm g/cm^3$  if  $2.67~\rm g/cm^3$  is used as the salt density at  $800\,^{\circ}\rm C$ ; only one WESF capsule has a higher loading factor.

As expected, the rate of corrosion that was observed on the inner surface of the inner capsule wall was much higher than the rate observed by Bryan (1983) at 450°C. Thus, corrosion rates at high temperatures could be of concern for long periods of time; but for a short-duration fire (1 to 3 h), they should be of little consequence.

The test performed used a nonradioactive salt mixture prepared to simulate the contents of a typical WESF capsule. The impurity content of the salt in WESF capsules is somewhat variable and is not well known. The composition selected for this test was based on the limited available data. Some differences in density are expected for different compositions. However, as mentioned earlier, the salt volume at 800°C is not expected to be vastly different because the measured density of the salt mixture used was close to that of pure CsCl at 800°C even though the room temperature densities were quite different. The apparent separation of the salt was observed by color change only and only minor compositional differences noted; this separation is not expected to be of consequence in the capsule behavior. The absence of radioactivity in the simulated salt is not expected to have a major effect on either swelling or corrosion results. In the 800°C fire, the heat flow will be mainly from the outside toward the inside; therefore, the relatively small heat generation rate from the salt should have little effect on the salt temperatures. This effect was accommodated by heating the capsule to 810°C rather than 800°C required for the standard tests. The effect of gamma irradiation on the corrosion was examined

to a limited extent earlier at low temperatures (Fullam 1972) with little observable effect. At high temperatures, this effect is expected to be negligible because the radiolytic impact of the rate is not markedly affected by temperature. Thus, the thermal rate would be much greater than the radiolytic rate.

### THERMAL STRESS DURING THERMAL CYCLING

Initial studies on WESF capsules considered primarily underwater storage and ultimate disposal either near the surface of the earth or in a deep geologic repository. In both cases, the capsules reach a quasi-steady-state temperature that decreases slowly with time as the isotope decays. A similar environment is encountered in the Sandia Irradiator for Dried Sewage Solids (SIDSS) or any dry storage/dry operation system where the capsules are maintained in air during both irradiation and storage. In these instances, the capsules are not subjected to large temperature cycling.

However, most irradiators in use today store the capsules in a water basin when not in use and lift them from the water during the irradiation processing. This concept cycles the source capsules from the water basin to air one to several times each day. This periodic removal and reinsertion process cycles the temperature and, thus, might induce thermal stresses in the upper and lower end cap welds. The long-term effect of this cyclic thermal loading is possible crack initiation and subsequent fatigue crack growth, which may eventually result in loss of outer capsule integrity. The inner capsule is subjected to much smaller temperature variations; thus, thermal stresses are also smaller. Therefore, a study was undertaken to examine the effects of the thermal stresses generated during this thermal cycling of WESF cesium capsules. The results of a finite-element analysis of these thermal stresses and an experimental program in which a WESF capsule was cycled for 6 months (3845 cycles) to verify the calculations are discussed in the following sections.

### METHOD OF ANALYSIS

An analytical estimate of possible fatigue crack growth in partial penetration end welds was performed using Revision 4.0 of the finite-element code ANSYS. (a) ANSYS is capable of solving for the thermal transients that correspond to the in-air heatup and in-water quench associated with capsule removal and return to storage. The thermal pass solution is used to define

<sup>(</sup>a) ANSYS is a proprietary engineering analysis program owned, distributed, and maintained by Swanson Analysis Systems, Inc., Houston, Pennsylvania.

the time-temperature history for the stress solution. The crack opening and crack tip forces from the stress pass solution can be used to estimate the peak-to-peak stress intensity factor (K) corresponding to the peak-to-peak thermal effects. This change in K can be related to the amount of crack growth per cycle and the growth expected during the design life of the capsule. The results include a discussion of the one-dimensional (1-D) verification study, the 2-D temperature profiles, and the fatigue crack propagation rate predictions.

### Assumptions

A few simplifying assumptions are necessary to make any analysis practical within time and budget limits. Such general assumptions are mentioned here. Several more assumptions appear in the remainder of this section.

The capsule was assumed to be axisymmetric and horizontally oriented. The nonaxisymmetric features were addressed in the 1-D scoping analyses, ignored, or their effects were averaged into the local element properties. The ambient fluid was assumed to extend axisymmetrically for a large radius around the capsule. This assumption omitted the effects of neighboring capsules and nonaxisymmetric fluid temperatures and thermal properties. Natural convection and radiation were the exchange mechanisms at the capsule surface to the isothermal sink.

The CsCl salt was in intimate contact with the inner wall of the inner capsule and was assumed to have a uniform thermal power density. Intimate contact was also assumed to exist between the end caps of the inner and outer capsule at the bottom (away-from-fill) end of the assembly. This resulted in the coupling of the temperature degrees-of-freedom at the inner and outer capsule end cap interface and the inner capsule and salt interface, respectively. No attempt was made to model voids or nonuniformities at any interface.

When capsules are returned to storage pools, thermal cycling and the difference in thermal mass from the end cap to the sidewall induce a thermal stress in the vicinity of the weld. The transition from the in-air to in-water environment was assumed to occur instantaneously, without boiling.

### Geometry

The 1-D model is shown in Figure 14. The fuel and capsule walls were modeled using four-node quadrilateral continuum elements (STIF 55). The ANSYS element type 55 is capable of 2-D steady-state or transient thermal analysis. The element has four nodal points with a single degree-of-freedom (temperature) at each node. The first 10 elements modeled the fuel; the next three, the inner capsule wall. A series of radiation links (STIF 31) and conduction links (STIF 32) were used to thermally connect the inner and outer walls through the annulus. Element type 31 is capable of modeling 2- or 3-D steady-state or transient radiation. The element is uniaxial with two nodes and a single degree-of-freedom (temperature) at each node. Element type 32 is capable of 2-D steady-state or transient conduction. It also has two nodes and temperature associated with its degree-of-freedom. The next three elements represented the outer capsule wall. The outer three links were used to allow radiation from the capsule wall to the environment. A total of 38 nodes and 25 elements were used in the 1-D scoping study.

The geometry for the 2-D finite-element mesh was taken directly from Figure 1. Amplified views of the mesh in the weld areas are shown in Figures 15 and 16 for the upper and lower welds, respectively. The element types were the same as those used in the 1-D model except that the annulus conduction links were replaced with the STIF 55 quadrilateral. A total of 2140 nodes and 2424 elements were used for the capsule assembly.

# Boundary Conditions

Convection and radiation surfaces were established between the outer capsule and the air. Convection coefficients of 10 and 13  $\text{W/m}^2$ -°C were used for the horizontal sidewall and vertical end caps, respectively. A constant temperature of 27°C (80°F) was assumed for air. Convection was the principal cooling mode for the water quench portion of the thermal cycle. Convection coefficients of 568 and 738  $\text{W/m}^2$ -°C were used for the quench portion of the thermal cycle. The water was assumed to remain at a constant 21°C (70°F). The annulus between the inner and outer capsules was too small for convection

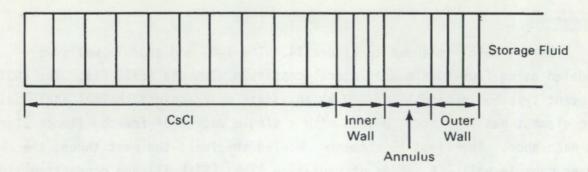


FIGURE 14. Finite-Element Mesh of the One-Dimensional Thermal Model

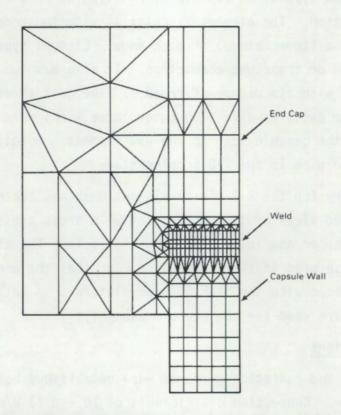


FIGURE 15. Finite-Element Mesh of the Upper Weld Region - Two-Dimensional Model

to occur. This was noted from previous studies (Campbell 1981) and was also confirmed by the approach presented by Holman (1976). The calculation for the nominal annulus geometry is presented in Appendix A. All other unspecified free surfaces were assumed to be adiabatic.

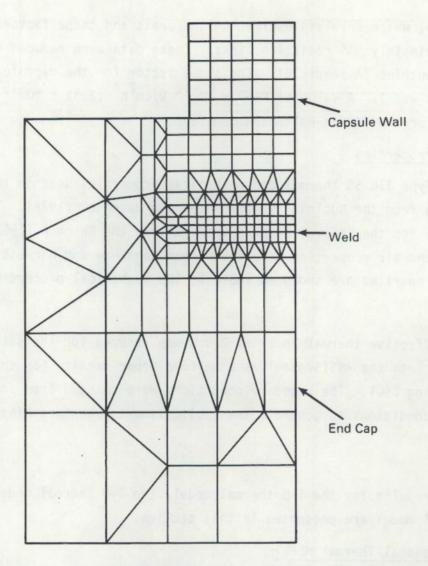


FIGURE 16. Finite-Element Mesh of the Lower Weld Region - Two-Dimensional Model

## Real Constants

The radiation links require area, shape factor, and emissivities in the form of real constant input to ANSYS. Three emissivities were needed for this analysis: the fuel to the inner capsule; the outer wall of the inner capsule to the inner wall of the outer capsule; and the outer wall of the outer capsule to air. These values were assumed to be 0.9, 0.5, and 0.65, respectively, based on observations of a capsule destructive examination. The nodal areas and element shape factors were dependent on the local finite-element mesh

resolution, which involved estimating the areas and shape factors for all of the approximately 535 radiation links. These data were reduced to 119 real constant entries (Appendix B). The shape factor for the capsule-to-air radiation links was 1. A value of 5.67 x  $10^{-12}$  W/cm<sup>2</sup>K<sup>4</sup> (3.31 x  $10^{-15}$  Btu/s-in.<sup>2</sup>-R<sup>4</sup>) was used for the Stefan-Boltzmann constant.

### Material Properties

The Type 316 SS thermal and mechanical properties used in this analysis were taken from the Nuclear Systems Materials Handbook (1978). The thermal properties for the CsCl were taken from Rimshaw and Ketchen (1967). The nitrogen and air properties were taken from Rohsenow and Harnett (1973). The thermal properties are shown in Table 6; the mechanical properties, in Table 7.

### Loads

An effective thermal power of 200 W was assumed for the salt. This value was input into the ANSYS model as a uniform power density for the elements representing CsCl. The boundary conditions were changed from in-air to in-water conditions to generate the cyclic time-temperature history.

# RESULTS

The results for the 1-D thermal model, the 2-D thermal model, and the 2-D mechanical model are presented in this section.

### One-Dimensional Thermal Model

The mechanisms for heat transfer in the 1-D model were uniform heat generation and conduction in the salt, conduction through the inner capsule wall, radiation and conduction across the annulus, conduction through the outer wall, and radiation and convection from the outer surface to the storage fluid. These steady-state calculations were performed using Revision 4.0 of the finite-element code ANSYS. To check the 1-D ANSYS solution, the temperature distributions were also generated using a closed-form approach. The 1-D solution provided verification for the 2-D solution scheme.

TABLE 6. CsCl Capsule ANSYS Thermal Properties

Material	terial Temperature		Conductivity, W/cm-°C	Density, g-s <sup>2</sup> /cm <sup>4</sup>	Specific Heat, W-cm/s-g-°C			
CsC1	20 150 260 370 480 600	70 300 500 700 900 1100	8.301 x 10-3 5.741 x 10-3 4.502 x 10-3 3.703 x 10-3 3.145 x 10-3 3.733 x 10-3	4.082 x 10 <sup>-3</sup> 4.004 x 10 <sup>-3</sup> 3.933 x 10 <sup>-3</sup> 3.853 x 10 <sup>-3</sup> 3.212 x 10 <sup>-3</sup> 3.160 x 10 <sup>-3</sup>	299 305 308 309 309 390			
316 SS	20 150 260 370 480 600	70 300 500 700 900 1100	1.336 x 10-1 1.551 x 10-1 1.731 x 10-1 1.905 x 10-1 2.070 x 10-1 2.230 x 10-1	8.103 x 10 <sup>-3</sup> 8.718 x 10 <sup>-3</sup> 8.002 x 10 <sup>-3</sup> 7.956 x 10 <sup>-3</sup> 7.910 x 10 <sup>-3</sup> 7.862 x 10 <sup>-3</sup>	444 497 527 546 559 569			
Air	20 150 260 370 480 600	70 300 500 700 900 1100	1.823 x 10-4 2.618 x 10-4 3.332 x 10-4 3.982 x 10-4 5.120 x 10-4 6.066 x 10-4	1.746 x 10-6 1.165 x 10-6 8.710 x 10-7 6.962 x 10-7 5.034 x 10-7 3.873 x 10-7				
Air	-70 30 130 230 430 630	-100 80 260 400 800 1160			1023 1021 1027 1036 1077 1125			

TABLE 7. CsC1 Capsule ANSYS Mechanical Properties

Material	Temperature,	Elastic Modulus, psi	Poisson's Ratio	Linear Coefficient of Thermal Expansion, in./in./°C				
316 SS	20	28.11 x 10 <sup>6</sup>	0.265	1.511 x 10-5				
	150	27.16 x 10 <sup>6</sup>	0.275	1.617 x 10-5				
	260	26.07 x 10 <sup>6</sup>	0.284	1.695 x 10-5				
	370	24.77 x 10 <sup>6</sup>	0.292	1.760 x 10-5				
	480	23.31 x 10 <sup>6</sup>	0.301	1.814 x 10-5				
	600	21.72 x 10 <sup>6</sup>	0.309	1.859 x 10-5				

### Radial Temperatures

ANSYS thermal results were generated for steady-state in-air and in-water conditions. The maximum radial temperature distribution corresponded to the steady-state in-air solution (Figure 17). The closed-form steady-state in-air solution is also shown in this figure. Closed-form sample calculations are presented in Appendix C. The maximum temperature of about 293°C was found at the centerline of the assembly. This temperature varied, as a function of power, through the salt to about 218°C at the salt/inner capsule interface. The canister wall acted as a thermal short and there was no noticeable temperature drop across it. There was, however, a substantial drop (about 98°C) across the inner annulus to the 149°C outer wall temperature.

The minimum radial temperature distribution corresponded to in-water steady-state storage. The ANSYS and closed-form predictions appear in Figure 18. A maximum temperature of about 193°C occurred at the capsule center.

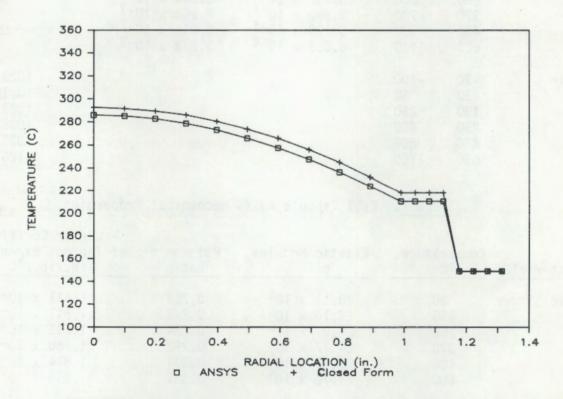


FIGURE 17. One-Dimensional Radial Temperature Profiles at Steady-State In-Air Condition

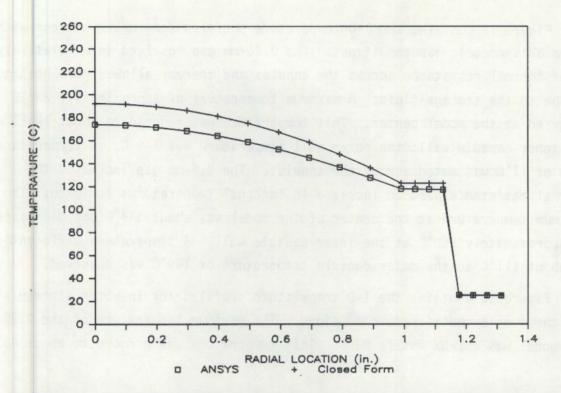


FIGURE 18. One-Dimensional Radial Temperature Profiles at Steady-State In-Water Condition

This temperature varied through the salt to about 121°C at the salt/inner capsule interface. The inner capsule wall temperature was essentially a constant 121°C through its thickness; the outer wall temperature was about 25°C. The temperature drop across the annulus was identical to that for air storage, approximately 98°C. The same profile was observed for the steady-state inwater condition, with the temperatures shifted downward from those for air storage.

#### Annulus Variation

The effect that nonconcentricity could have on the temperature profiles was estimated by analyzing several annulus widths. These widths included the full gap = 2.6 mm, nominal gap = 1.31 mm, and one-tenth full gap = 0.26 mm. The 1-D model real constants were adjusted to accommodate these values as an axisymmetric annulus between the inner and outer capsules. A true nonconcentricity study would have involved a series of 3-D analyses that were considered beyond the scope of this project.

Figure 19 contains the 1-D temperature profiles for in-air storage with three axisymmetric gap conditions. The 0.26-mm gap resulted in a relatively lower thermal resistance across the annulus and thereby allowed more heat to escape to the storage fluid. A maximum temperature of approximately 232°C occurred at the model center. This temperature was reduced to about 166°C at the inner capsule wall; the outer wall temperature was 149°C. A temperature drop of 17°C was noted across the annulus. The 2.6-mm gap increased the thermal resistance, and an increase in internal temperatures followed. The maximum temperature at the center of the model was about 343°C but decreased to approximately 260°C at the inner capsule wall. A temperature differential of about 111°C to the outer capsule temperature of 149°C was observed.

Figure 20 contains the 1-D temperature profiles for in-water storage with the three axisymmetric gap conditions. The maximum temperature of the 0.26-mm gap model was approximately 93°C. This temperature was reduced to about 46°C

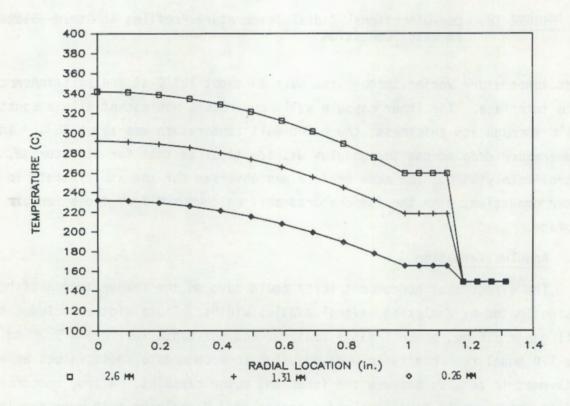


FIGURE 19. One-Dimensional Radial Temperature Profiles at Steady-State In-Air Condition with Varying Annulus Gap

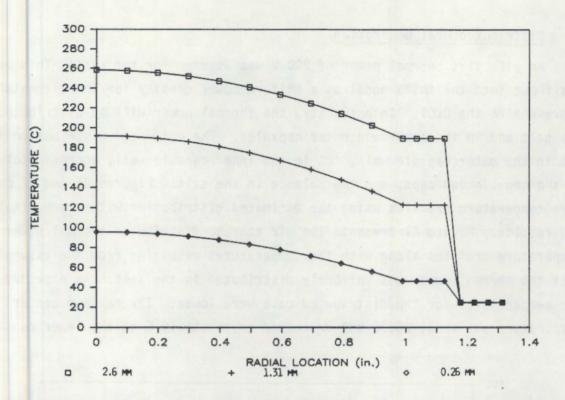


FIGURE 20. One-Dimensional Radial Temperature Profiles at Steady-State In-Water Condition with Varying Annulus Gap

at the inner capsule wall; the outer wall temperature was at 25°C. A temperature drop of 22°C was noted across the annulus. The maximum temperature at the center of the 2.6-mm annulus model was about 260°C but decreased to approximately 188°C at the inner capsule wall. A temperature differential of about 163°C to the outer capsule temperature of 25°C was observed. Since the total heat through the outer wall did not change, the outer skin temperatures remained constant for each gap condition.

In the actual configuration, the heat transfer rate around the skin of the capsule would be much greater than the transfer through the wall, which would tend to reduce or redistribute the nonconcentricity influence on the temperature profiles. Analysis of the nominal gap arrangement was considered adequate for this study.

### Distributed Heat Generation

An effective thermal power of 200 W was assumed for the salt. This value was input into the ANSYS model as a uniform power density for the elements representing the CsCl. In actuality, the thermal power will be distributed in the salt and in the inner and outer capsules. The estimated distribution is 15% in the outer capsule wall, 12% in the inner capsule wall, approximately 3% in the capsule end caps, and the balance in the salt. Figures 21 and 22 compare temperature profiles using the estimated distribution with the nominal 1-D results. Figure 21 presents the air storage distributed thermal power temperature profiles along with the temperatures resulting from the assumption that the thermal power was uniformly distributed in the salt. As expected, the temperatures for the distributed case were lower. The maximum center temperature was about 266°C but decreased to about 216°C at the inner capsule

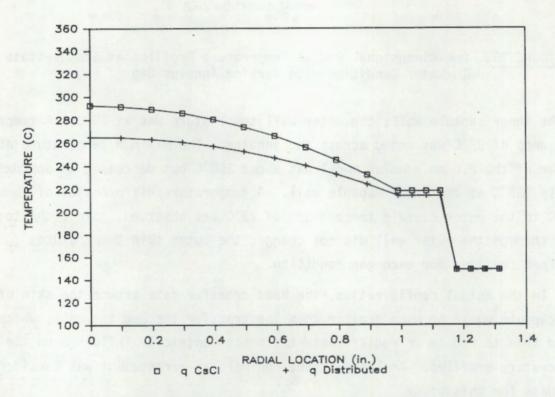


FIGURE 21. One-Dimensional Radial Temperature Profiles at Steady-State In-Air Condition with Distributed Thermal Power

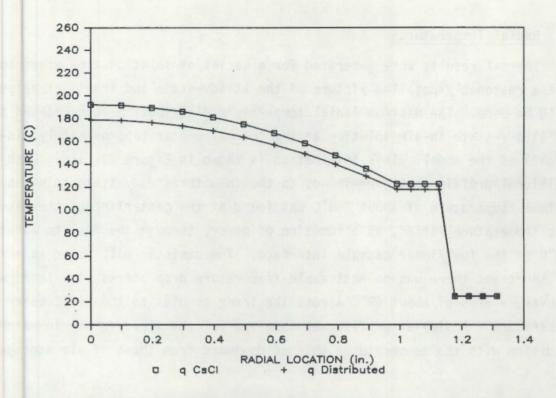


FIGURE 22. One-Dimensional Radial Temperature Profiles at Steady-State In-Water Condition with Distributed Thermal Power

wall. The outer capsule wall temperature was 149°C. The in-water steady-state results are shown in Figure 22. Similar trends were noted; temperatures of 177°C, 118°C, and 25°C occurred at the center, inner capsule wall, and outer capsule wall, respectively.

Since the two cases produced very little difference in temperatures for the inner capsule and no difference in the outer capsule, the assumption of uniform power in the salt was considered acceptable for this analysis.

### Two-Dimensional Thermal Model

The 2-D model allowed heat transfer to occur in both the radial (1-D case) and axial directions. Steady-state and transient solutions were examined using Revision 4.0 of ANSYS. The 2-D temperature profiles were used as input to the 2-D mechanical solution.

### Radial Temperatures

Thermal results were generated for a series of solution time steps so that a reasonably detailed picture of the steady-state and transient response could be made. The maximum radial temperature distribution corresponded to the steady-state in-air solution at the thermal center (approximately midheight) of the model. This information is shown in Figure 23, along with an additional profile that corresponds to the in-water steady-state solution. A maximum temperature of about 288°C was found at the centerline of the assembly. This temperature varied, as a function of power, through the fuel to about 218°C at the fuel/inner capsule interface. The canister wall acted as a thermal short and there was no noticeable temperature drop across it. There was, however, a drop of about 69°C across the inner annulus to the 149°C outer wall temperature. A similar profile was observed for the steady-state in-water condition with the temperatures shifted downward from those of air storage.

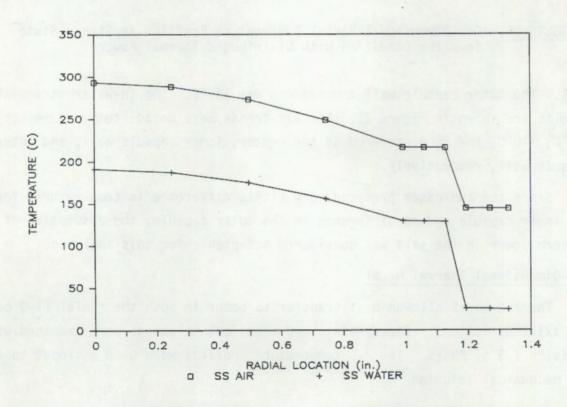


FIGURE 23. Two-Dimensional Maximum Radial Temperature Profiles at Steady-State In-Air and In-Water Conditions

### Salt/Wall Interface Temperatures

Figure 24 presents the temperature profiles for the inner capsule salt/wall interface for both steady-state in-air and in-water conditions. The highest temperatures were associated with in-air storage. At the assembly end, where both of the end caps were in contact, the salt/wall interface temperature was approximately 138°C. The temperature increased to a maximum of about 216°C at a distance of 7 to 8 in. from this end and remained constant for the next 5 to 6 in. At the fill end of the capsule, the temperature was about 154°C. The in-water temperatures were substantially lower. The temperature at the end of the assembly was approximately 38°C. The temperature increased to about 127°C at a distance of 7 in. from the end. The interface temperature remained 127°C from about 7 to 14 in. The temperature at the fill end of the capsule was approximately 88°C.

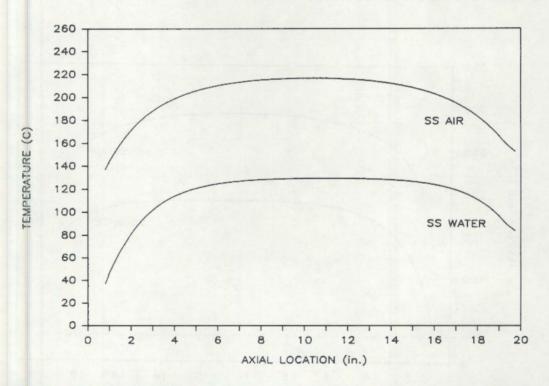


FIGURE 24. Two-Dimensional Salt/Wall Interface Axial Temperature Profiles at Steady-State In-Air and In-Water Conditions

### Salt Temperatures

The maximum assembly temperatures occurred at the centerline of the model in the CsCl (Figure 25). The steady-state in-air solution predicted an end temperature of approximately 135°C, increasing to a maximum of about 288°C. This temperature was essentially constant for the region 7 to 14 in. from the bottom. At the fill end, the salt temperature was about 218°C. The in-water temperatures, lower profile, were 32°C at the bottom. A maximum of 193°C was observed 7 to 14 in. from the end. The fill end temperature was slightly below 149°C.

### Temperature Differential

The potential for crack propagation is driven by the temperature differential between the end cap and near-weld region of the capsule. The temperatures associated with the end cap and near-weld area maximum differences (between the in-water and in-air transients) were used in the stress portion of this

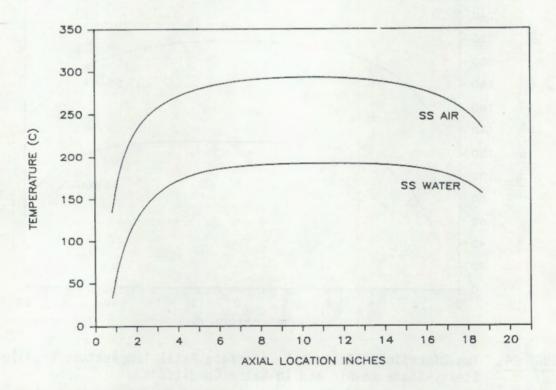


FIGURE 25. Two-Dimensional Maximum CsCl Axial Temperature Profiles at Steady-State In-Air and In-Water Conditions

study. The cooling curves for the upper and lower welds, from a steady-state in-air condition to the water quench, are shown in Figures 26 and 27, respectively. The temperatures for the center cap (CNTR Cap), corner cap (CNR cap), weld side (W side), and midheight (Mid HT) locations are shown in Figure 26. The midheight steady-state in-air temperature was approximately 143°C. The quench exponentially reduced the temperature until 29°C was reached after about 90 to 100 s. The trends were similar for the other portions of the capsule. The center end cap temperature dropped from about 88 to 29°C. The corner end cap and the weld side temperatures fell from about 88°C and 93°C to 24°C after the 90- to 100-s quench time. The differences in starting temperatures, cooling rates, and final temperatures were determined by the local heat transfer coefficients, heat load, and thermal mass.

The upper weld, end cap, and weld sidewall maximum temperature differences are shown in Figure 28. The steady-state in-air difference was -6°C. The negative difference means that the weld area sidewall was at a higher temperature

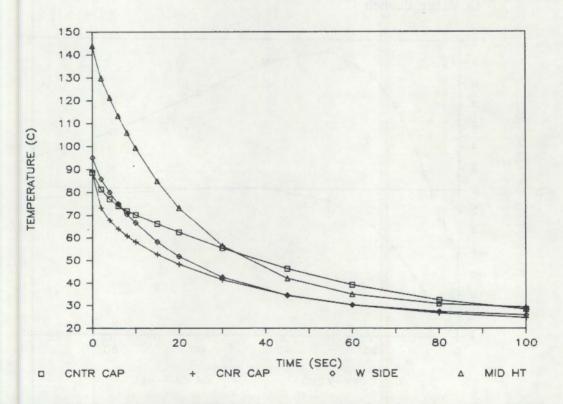


FIGURE 26. Two-Dimensional Temperature Profiles of Upper Weld Area Response to Water Quench

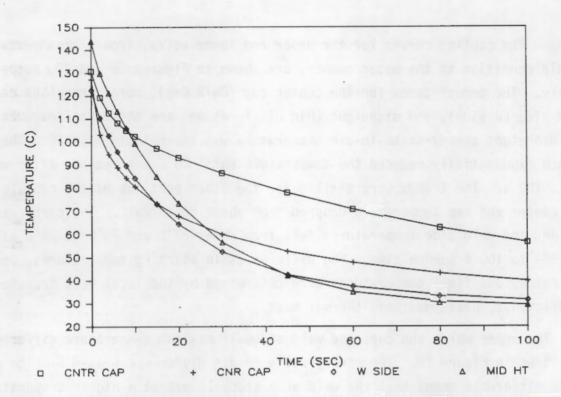


FIGURE 27. Two-Dimensional Temperature Profiles of Lower Welld Area Response to Water Quench

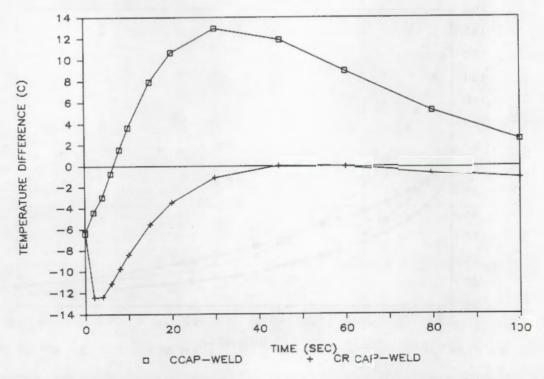


FIGURE 28. Two-Dimensional Temperature Differences in Upper Weld Area as Result of Water Quench

than either of the end cap locations. The corner of the end cap lost heat at a greater rate than the center of the cap, which accounted for the increase in negative difference to about -13°C for the corner cap-to-weld difference and a decrease in the center cap-to-weld difference. The extreme differences noted for the center cap-to-weld were approximately -6°C and 13°C. The corner cap-to-weld differences were about -13°C and 0°C.

Figure 27 shows the cooling response of the lower weld area. The center cap location had a steady-state in-air temperature of 129°C that decreased to about 63°C after 100 s in the water bath. The corner cap and weld location steady-state temperature of 121°C cooled to 46°C and 35°C after 100 s. Temperature difference curves (similar to those generated for the upper weld area) are shown in Figure 29. The extreme differences for the corner cap-to-weld and center cap-to-weld location are about -8°C and 8°C to 36°C, respectively.

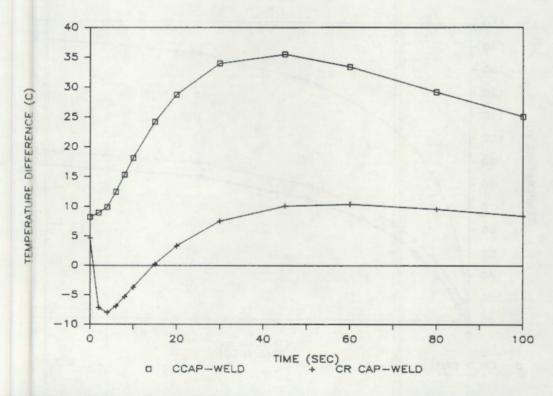


FIGURE 29. Two-Dimensional Temperature Oifferences in Lower Weld Area as Result of Water Quench

The above information covered the steady-state in-air to in-water transient portion of the thermal results. The second portion of the thermal cycle was concerned with the steady-state in-water to in-air transient results. These temperature contours are shown in Figures 30 and 31 for the upper and lower weld regions, respectively. From the upper weld area in Figure 30, the four locations on the capsule heat up in an exponential fashion. The midheight started at about 29°C and increased to approximately 141°C after 8000 s (2.2 h). The weld location temperature varied from 24°C to 93°C, while both the center and corner end cap temperatures increased from 24°C to approximately 88°C. The temperature differences for the upper weld area steady-state water to in-air transients are presented in Figure 32. The extremes were -0.9 to -6°C and -0.8 to -7°C for the corner cap-to-weld and center cap-to-weld locations, respectively.

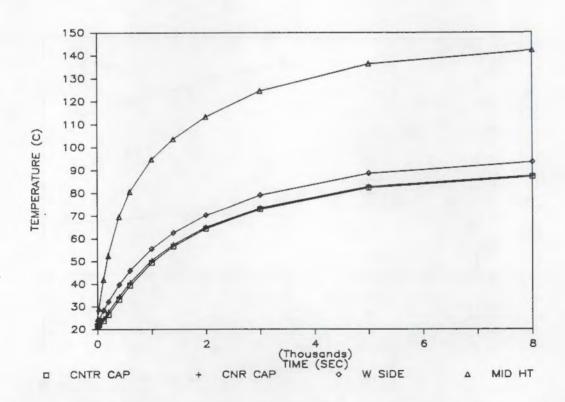


FIGURE 30. Two-Dimensional Temperature Profiles of Upper Weld Area Response to Air Heatup

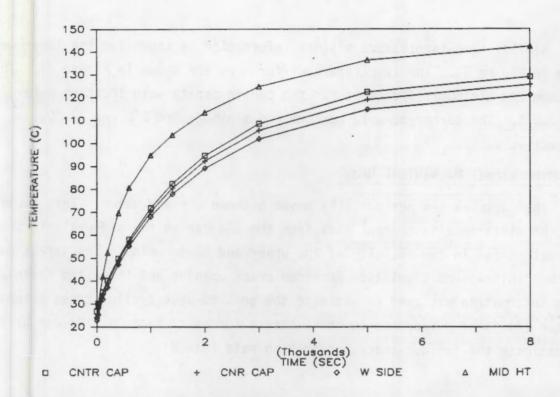


FIGURE 31. Two-Dimensional Temperature Profiles of Lower Weld Area Response to Air Heatup

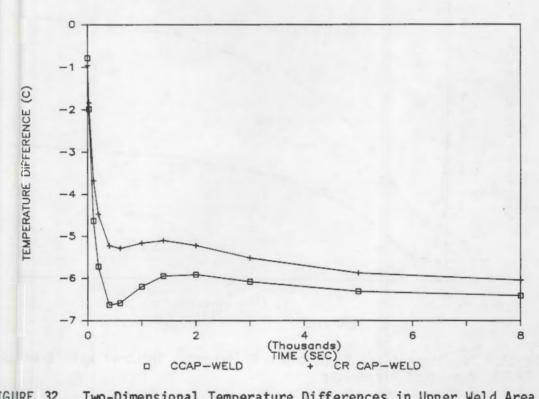


FIGURE 32. Two-Dimensional Temperature Differences in Upper Weld Area as Result of Air Heatup

Similar time-temperature history information is shown for the lower weld area in Figure 31. The temperature differences are shown in Figure 33. The maximum and minimum differences for the center cap-to-weld location were 8°C and 2.5°C. The corner-to-weld maximum and minimum were 5°C and 1.4°C, respectively.

#### Two-Dimensional Mechanical Model

The capsules are periodically moved between air and water. This cycling, and the difference in thermal mass from the end cap to the sidewall, induces a thermal stress in the vicinity of the upper and lower welds. The stress pass of the finite-element solution provided crack opening and crack tip force data. This information was used to estimate the peak-to-peak cyclic stress intensity factor (K). The difference in these stress intensity factors ( $\Delta$ K) can be used to estimate the fatigue crack propagation rate (da/dN).

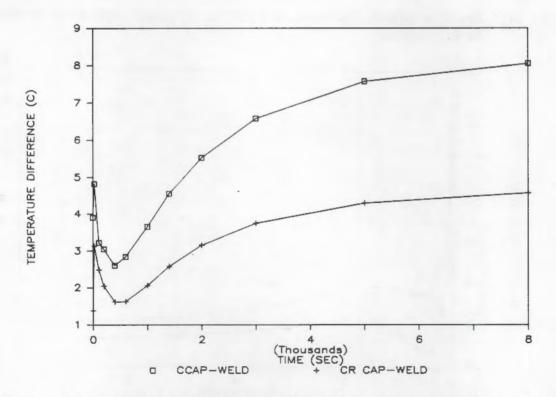


FIGURE 33. Two-Dimensional Temperature Differences in Lower Weld Area as Result of Air Heatup

### Stress Intensity Factor

One technique for estimating K, the displacement method, involves examining the nodal displacements that correspond to the maximum temperature differences. The displacement solution approach used in this portion of the study was based on the work by Tada (1973). The plane strain crack opening (v) is given by the following equation:

$$v = \frac{K_{I}}{G} \left[ \frac{r}{2\pi} \right]^{\frac{1}{2}} \sin \frac{\theta}{2} \left[ 2 - 2\nu - \cos^{2} \frac{\theta}{2} \right]$$

where K<sub>T</sub> = stress intensity

G = E/2(1 + v)

r = distance from crack tip to displacements

0 = 180°

v = Poisson's ratio

E = elastic modulus.

This equation was applied to the near-crack tip displacement field for the upper and lower welds (Appendix D). The extreme stress intensity factors for the upper weld were approximately 0.43 ksi  $\sqrt{i}$ n. and -2.1 ksi  $\sqrt{i}$ n., for a change in K of about 2.5 ksi  $\sqrt{i}$ n. The lower weld stress intensity factor varied between -0.31 ksi  $\sqrt{i}$ n. to -5.5 ksi  $\sqrt{i}$ n. The thermal loads in the lower weld area tend to close the crack. However, a conservative technique for accommodating some crack opening due to possible residual welding stress was to consider the difference as a change in reversing K. A value of 5.2 ksi  $\sqrt{i}$ n. was used for the lower weld.

The second estimate of the stress intensity factor was based on an energy method using the ANSYS-generated nodal forces and displacements. The crack closure work was calculated from:

$$W = 1/2 (f_X du_X + f_y du_y)$$

where  $f_X$  and  $f_y$  were components of the nodal reaction forces across the crack plane in the uncracked state. The components  $du_X$  and  $du_y$  were calculated from

the relative nodal displacements. For this study, the mode I stress intensity factor was calculated from the crack closure work (Tada 1973) by

$$K_{I} = \left[\frac{E(f_{y}u_{y})}{2R(a_{2} - a_{1})(1 - v^{2})}\right]^{1/2}$$

where R = radius to crack tip

a<sub>2</sub> = crack length after node release

a<sub>1</sub> = original crack length.

The crack closure equations were applied to the upper and lower welds. These calculations are shown in Appendix E. The extreme stress intensity factors for the upper weld were approximately 0.43 ksi  $\sqrt{\text{in}}$ . and -2.05 ksi  $\sqrt{\text{in}}$ ., for a change in K of about 2.5 ksi  $\sqrt{\text{in}}$ . The lower weld stress intensity factor was -0.32 ksi  $\sqrt{\text{in}}$ . and -5.26 ksi  $\sqrt{\text{in}}$ . A change in K of approximately 5.26 ksi  $\sqrt{\text{in}}$ . was used for the lower weld.

The two solution approaches were in good agreement between the stress intensity factor predictions for the upper and lower welds.

#### Crack Acceleration

The preceding section deals with estimating the stress intensity factor for the nominal weld geometry. During service, the periodic opening and closing of the welds will cause crack growth and possible increase in K. This effect was modeled by moving the crack tip along the crack plane and allowing the crack to grow from node to node in a controlled manner. At each new crack location the displacement and crack closure methods were used to estimate the new factors. The difference between these peak-to-peak stress intensity factors ( $\Delta$ K) is shown in Figures 34 and 35. The upper weld response to changing crack length is shown in Figure 34. The  $\Delta$ K for the nominal weld penetration was about 2.5 ksi  $\sqrt{i}$ n. Any growth in crack length reduced this. A  $\Delta$ K between 1.9 and 2.1 ksi  $\sqrt{i}$ n. was observed for an increase in crack length of about 0.028 in. A value of 2.5 ksi  $\sqrt{i}$ n. was assumed as a conservative level of  $\Delta$ K for the upper weld. Figure 35 presents the relative change in stress intensity as a function of crack length for the lower weld. Both methods predicted a

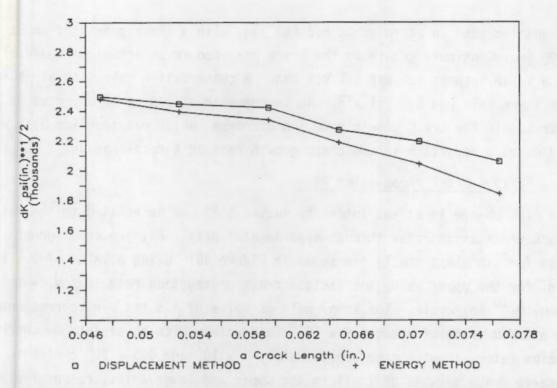


FIGURE 34. Effect of Crack Growth on the Relative Change in Stress Intensity Factor for the Upper Weld

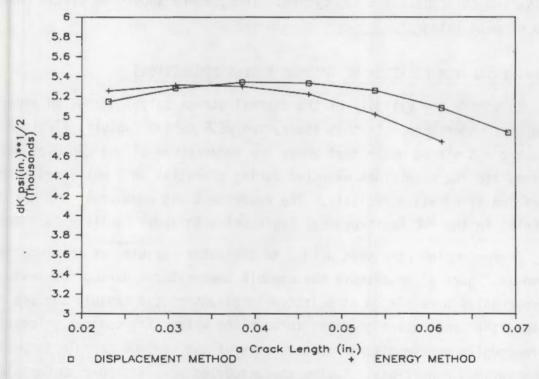


FIGURE 35. Effect of Crack Growth on the Relative Change in Stress Intensity Factor for the Lower Weld

slight increase in  $\Delta K$  to about 5.3 ksi  $\sqrt{\text{in}}$ . with a crack growth of about 0.02 in. Continued growth in the crack resulted in an actual decrease of  $\Delta K$  to a value between 4.8 and 5.1 ksi  $\sqrt{\text{in}}$ . A conservative selection of  $\Delta K$  for the lower weld was 5.3 ksi  $\sqrt{\text{in}}$ . An increase in  $\Delta K$  can be interpreted as an increase in the crack growth rate. A decrease, which was seen for both welds, indicates a reduction in the crack growth rate or a deceleration.

### Fatigue Crack Propagation Rate

The change in stress intensity factor ( $\Delta K$ ) can be related to the fatigue crack propagation rates through experimental data. Fatigue crack growth rate data for stainless steels are shown in Figure 36. Using a value of 2.5 ksi  $\sqrt{in}$ . for the upper weld, the fatigue crack propagation rate (da/dN) was less than  $10^{-10}$  in./cycle. The lower weld  $\Delta K$  value of 5.3 ksi  $\sqrt{in}$ . corresponded to a da/dN value of about 2.9 x  $10^{-9}$  in./cycle. With these fatigue crack propagation rates, it will take approximately 1 x  $10^7$  and 3.5 x  $10^5$  cycles to observe crack growths of 1 mil in the upper and lower welds, respectively. If the service life of the capsule is 30 years with one cycle per day, this is equivalent to about 1.1 x  $10^4$  cycles. Thus, there should be little chance of loss of weld integrity.

## EXPERIMENTAL VERIFICATION OF THERMAL STRESS PROJECTIONS

To verify the validity of the thermal stress calculations, an experimental study was undertaken. In this study, two WESF cesium capsules (Table 8) were selected and placed under test where the temperature of the capsules was cycled to simulate the conditions expected during operation in a wet storage/dry operation irradiation facility. The experiment was conducted in a hot cell (C-Cell) in the PNL Radiochemical Engineering Shielded Facility (324 Building).

Thermocouples were spot welded to the outer capsules at the locations shown in Figure 37 to measure the capsule temperatures during the test. Ten thermocouples were placed at selected locations on the capsule surface to measure the temperature gradient through the weld. The centrally located thermocouples were monitored as a measure of the maximum capsule temperature under various conditions. Twelve thermocouples were attached to Capsule C-105

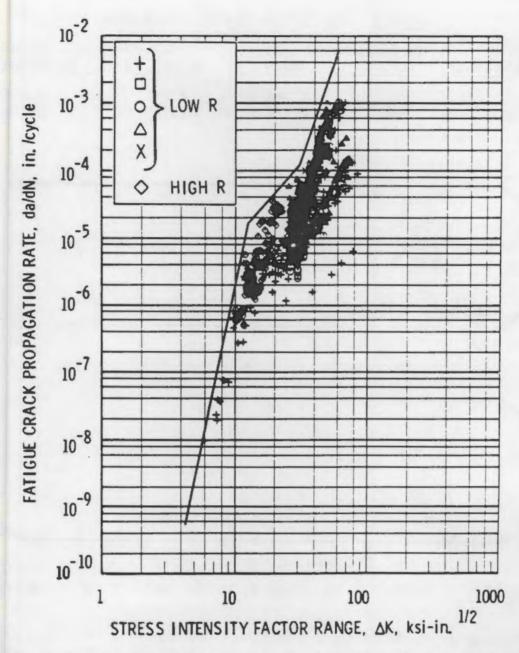
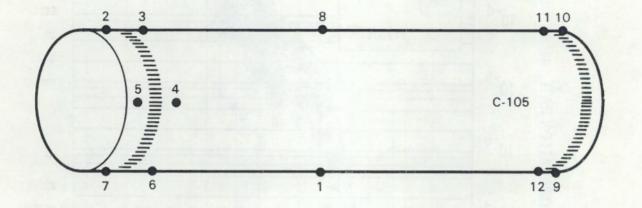


FIGURE 36. Fatigue Crack Growth Rate Data for Stainless Steels - Conditions of Low and High Stress Ratio (R), Laboratory Air, and Simulated PWR Environment (taken from Simonen et al. 1983)

TABLE 8. Description of WESF Cesium Capsules

Capsule Identification	Cs Content, KCi	Heat Output, W	Salt Weight, kg	Inner Capsule Wall Thickness, in.
C-105	55.3	265.6	2.904	0.095
C-903	54.7	262.7	2.737	0.136



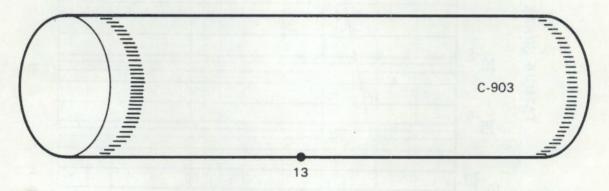


FIGURE 37. Location of Thermocouples on the Thermocycled Cesium Capsules

to characterize the temperature gradients. The remaining thermocouple was spot welded to the bottom of Capsule C-903 at its axial center.

The thermocouples were Type K with Type 304 SS grounded sheaths. They were 12 in. long and 0.062 in. in diameter with 0.100-in. caps for welded installation and had standard miniplugs that were fastened to the capsule-holding rack to minimize stress on the spot weld during in-cell movement of the capsule rack. Each couple was x-ray inspected by the vendor, calibrated against standard thermocouples, and insulated with a thin coating of

self-curing silicone rubber to prevent erroneous readings due to cooling of the sheath during the quench cycle.

The two capsules were positioned about 1.5 in. apart in a wire-framed rack. The rack was placed into an open-top tank plumbed to cycle cooled water over the capsules hourly to provide the thermal cycling. The tank was placed above a refrigerated reservoir to facilitate rapid gravity release of the water from the tank. An automatic timing circuit was set to activate a pump that circulated cooled (20°C) water over the capsules at a rate of 28 gal/min and to close a normally open solenoid drain valve. At this flow rate, the capsules were totally immersed in 5 s, but the pump continued to circulate the water for the entire 90-s quench period. The timer then deactivated the circulating pump and solenoid valve, and the water drained by gravity into the reservoir through a 1-1/2-in. overflow line and 1-1/2-in. drain line.

During the cycling operation, the temperature was monitored using either a twelve-point recorder, a rapid response two-pen recorder, or a digital volt meter.

### Temperature Measurements

Prior to initiation of the thermal cycling, the capsule temperatures were monitored to obtain equilibrium temperatures under a variety of conditions.

Table 9 lists the temperatures observed for each thermocouple and capsule condition examined.

When the capsule was suspended horizontally in the cell, the capsule centerline on the bottom side (the hottest point) was 167°C. The top center was slightly cooler (158°C) because of the larger gap at the top. The capsule ends were considerably cooler, as expected, because of lower heat generation rates. The end of the capsule with Thermocouples 2 through 7 was apparently the fill end of the capsule with the substantial void volume and the smaller heat generation rate yielding the lower temperatures observed.

Positioning the capsules vertically produced some minor changes in the equilibrium temperature because of the change in the annulus between the free-floating inner capsule and the outer capsule.

TABLE 9. Capsule Temperatures

	Temperature at Each Thermocouple Location, (a) °C												
Capsule Location	_1	_2	3	4	5		_7	8	9	_10	_11	_12	_13
Horizontal in 28°C air	167	88	91	96	91	101	93	158	132	118	124	136	
Vertical in 28°C air	156	86	85	89	86	84	85	151	126	124	131	128	149
Equilibrium in air in tank	200	101	105	112	108	120	111	182	159	147	152	162	190
Maximum during cycling (air)	192	102	102	105	102	108	104	170	125	124	127	132	174
Minimum during cycling (water quench)	28	23	26	24	26	25	26	24	26	26	28	25	25

<sup>(</sup>a) See Figure 37 for location of thermocouples.

When the capsules were permitted to equilibrate for 16 h in the air of the tank, they achieved a significantly higher temperature than when suspended in the cell. This is a result of the restricted free air flow around the capsules. This effect led to a somewhat higher temperature during the thermocycling than would be expected in circulating or free flowing air in an irradiator, slightly exaggerating the thermal stress during the test. The minimum temperatures achieved were also slightly lower than expected in an operating irradiator because cooled (20°C) water was used for the quench cycle.

# Thermal Cycle Test

During thermal cycling, the quench cycle was set for 90 s, followed by a self-heating cycle of 60 min in air. The temperature change with time for a point on each side of the weld during cycling is shown in Figure 38.

The system was automatically cycled on this schedule during a period of six months, giving a total of 3219 cycles over the temperature range shown in Figure 38. After nearly 2000 cycles, the refrigeration unit located in the high radiation field failed, and the lower quench temperature increased with time. With the refrigeration unit operating, the temperatures in the vicinity of the weld (Thermocouples 11 and 12) cycled from about 24 to  $136^{\circ}$ C ( $\Delta T = 112^{\circ}$ C); without a functioning refrigeration unit, the temperatures cycled from 43 to  $138^{\circ}$ C ( $\Delta T = 95^{\circ}$ C). Therefore, an additional 626 cycles were achieved with a slightly smaller temperature difference between the maximum and minimum cycle temperature. At this point, the test was interrupted, and a new

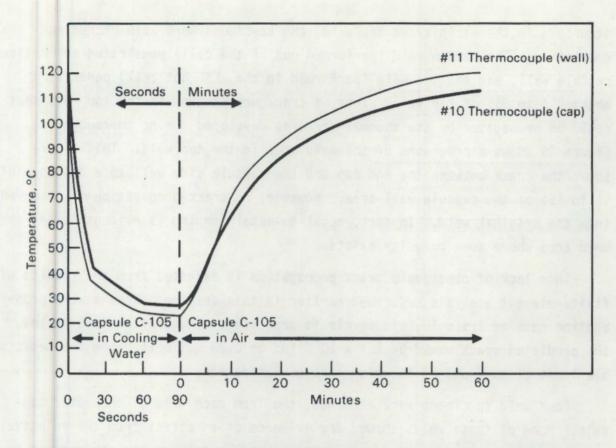


FIGURE 38. Temperature Cycles of Thermocycled Capsules at Two Points,
One on Each Side of Weld

refrigeration unit was installed with the compressor located outside of the cell. The total number of cycles achieved was 3845.

### Weld Examination

Following the test, Capsule C-903 was removed from the rack to obtain samples from the outer capsule welds for metallographic examination. An air-powered grinder with a 1/16-in. thick abrasive wheel was used to cut the end caps from the capsule. A guide cap attachment aided in removing the end welds and caps without damaging the inner capsule. The caps were washed with water and removed from the hot cell. Two metallographic specimens were taken from each weld (180° apart) to permit a thorough examination of the weld. Each of these samples was potted and polished in preparation for the microscopic examination. The micrographs from the as-polished samples showed no structural features even at high magnification. To more clearly observe the

structure in the vicinity of the weld, the specimens were acid etched and reexamined. The bottom weld (performed out of the cell) penetrated the entire capsule wall, and the top weld (performed in the WESF hot cell) penetrated about 85% to 95% of the wall. Thus, a crack was present in the top weld that could be propagated by the thermal stresses developed during thermocycling. Figure 39 shows micrographs of the weld area in the top weld. This figure shows the crack between the end cap and the capsule side wall to a distance of 5% to 15% of the capsule wall area. However, no crack propagation is observed into the original weld. In fact, metal-to-metal bonding is evident beyond the weld area where some porosity exists.

This lack of observable crack propagation is expected from the results of finite-element analysis described earlier in this section, where a weld propagation rate of 2.9 x  $10^{-9}$  in./cycle is predicted. Thus, after 3845 cycles, the predicted crack would be 1.1 x  $10^{-5}$  in. or 0.28  $\mu$ m, well below the detectable limit (1 to 2  $\mu$ m) of the microscopic examination.

Four weld specimens were examined, two from each weld of the outer capsule. None of these welds showed any evidence of an effect from the relatively minor thermal stress generated by cycling capsules from an air environment to water. This test is expected to develop somewhat more stress than would be present in an actual irradiator because the water used was maintained at 20°C, significantly cooler than expected in an irradiator. Furthermore, the air flow around the capsule was restricted by the water bath and the capsules appear to equilibrate at about 20°C higher than in free flowing air.

### DISCUSSION

The results of this study indicate that the outer capsule welds, which receive the maximum thermal stress for the thermal cycle described, will maintain integrity over the 30-year service life. This period is equivalent to approximately  $1.1 \times 10^4$  cycles if cycled at the rate of one cycle/day.

The highest thermal loading occurs in the bottom weld of the outer capsule where an approximate fatigue crack propagation rate of 2.9 x  $10^{-9}$  in./cycle is predicted. This means that 3.5 x  $10^5$  cycles are required for the crack to

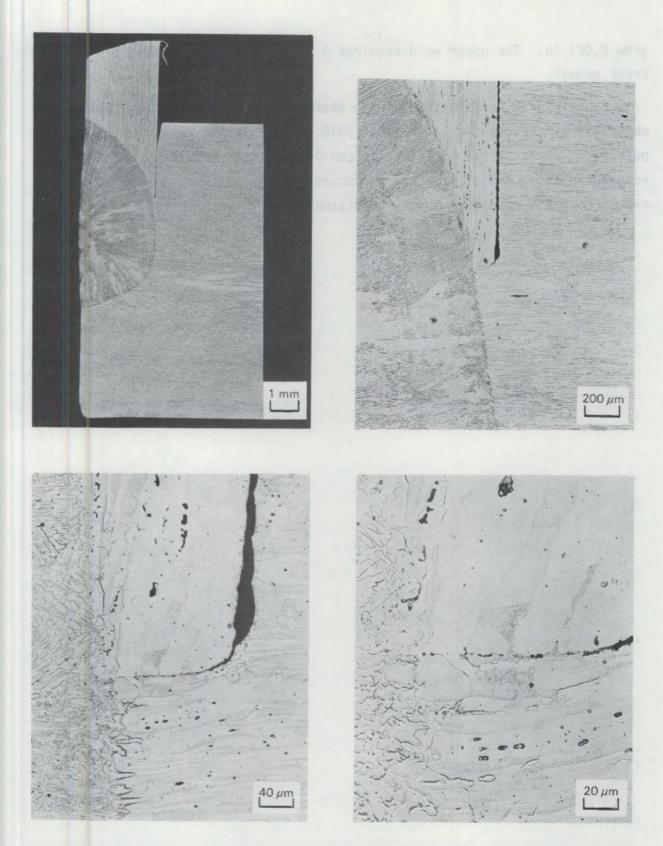


FIGURE 39. Micrographs of the Etched Weld Section of a WESF Cesium Capsule After Thermal Cycling for 3845 Cycles

grow 0.001 in. The upper weld requires  $1 \times 10^7$  cycles to obtain the same crack growth.

Experimental verification of the thermal stress calculations showed no observable effect on the welds after 3845 cycles. Although, for this limited period, no quantitative verification can be made, the results do verify that no unanticipated rapid crack growth occurs. This test exceeds the stress expected in the operation of an irradiator for a 10-year period.

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#### APPENDIX A

CALCULATIONS SUPPORTING ASSUMPTION REGARDING HEAT TRANSFER ACROSS THE ANNULUS BETWEEN THE INNER AND OUTER CAPSULE WALLS

#### FREE CONVECTION IN ENCLOSED SPACES

From "Heat Transfer" by J. P. Holman, 4th ed., McGraw-Hill, 1976, pp. 255-256.

Low Grashof numbers = no convection  $Gr_{\delta} \leq 1000$  $\beta = \left(\frac{1}{1 + T_2}\right)$  $\gamma_{air} = 26.26 \times 10^{-5} \text{ m}^2/\text{sec}$ kinematic viscosity g = gravity acceleration  $\gamma_{N_2}$  200°K (-100°F) = 7.568 x 10<sup>-6</sup> m<sup>2</sup>/sec 1200°K (1700°F) = 156.1 x  $10^{-6}$  m<sup>2</sup>/sec  $\gamma = 51.19 \times 10^{-6} \frac{m^2}{s}$  1.55 x  $10^3 \frac{in.^2/s}{m^2/s} = 0.07934 \frac{in.^2}{s}$  $Gr_{\delta} = \frac{386 \frac{\text{in.}}{\text{sec}^2} \frac{2}{(T_1 + T_2)} (T_1 - T_2)(0.0515)^3}{(0.07934)^2} = 16.75 \left(\frac{T_1 - T_2}{T_1 + T_2}\right)$ 16.75  $\left(\frac{T_1 - T_2}{T_1 + T_2}\right) = 1000$  Minimum Grashof for free convection to occur  $\frac{1}{1} - \frac{1}{2} = 60$  $T_1 - T_2 = 60T_1 + 60T_2$  $(1 - 60)T_1 = 61T_2$  $-59T_{1} = 61T_{2}$ 

 $\frac{\Gamma_1}{\Gamma_2} = \frac{61}{59} = -1.0334$ ,  $Gr_{\delta} = 1000$ 

 $T_1 \simeq -T_2$  to satisfy Grashof requirement Therefore, no convection heat transfer in annulus.

# APPENDIX B

REAL CONSTANT INPUT FOR ANSYS RADIATION LINKS

# TABLE OF ELEMENT REAL CONSTANTS

No.	Area	Shape Factor	Emissivity	Stephen- Boltzman Constant
1	0.99000-001	0.34500+000	0.50000+000	0.33060-014
2	0.99000-001	0.31600+000	0.50000+000	0.33060-014
3	0.99000-001	0.12200+000	0.50000+000	0.33060-014
4	0.16800+000	0.10000+001	0.50000+000	0.33060-014
5	0.99000-001	0.10800+000	0.50000+000	0.33060-014
6	0.99000-001	100-00088.0	0.50000+000	0.33060-014
7	0.72000-001	0.13700+000	0.50000+000	0.33060-014
8	0.72000-001	0.15600+000	0.50000+000	0.33060-014
9	0.72000-001	0.17700+000	0.50000+000	0.33060-014
10	0.72000-001	0.49000+000	0.50000+000	0.33060-014
11	0.11800+000	0.10000+001	0.50000+000	0.33060-014
12	0.14100+000	0.10000+001	0.50000+000	0.33060-014
13	0.19700+000	0.64700+000	0.50000+000	0.33060-014
14	0.19700+000	0.34500+000	0.50000+000	0.33060-014
15	0.22500+000	0.26800+000	0.50000+000	0.33060-014
16	0.22500+000	0.49000+000	0.50000+000	0.33060-014
17	0.15300+000	0.34500+000	0.50000+000	0.33060-014
18	0.15300+000	0.64700+000	0.50000+000	0.33060-014
19	0.15300+000	0.10800+000	0.50000+000	0.33060-014
20	0.15300+000	0.12200+000	0.50000+000	0.33060-014
21	0.15300+000	0.88000-001	0.50000+000	0.33060-014
22	0.15300+000	0.55000-001	0.50000+000	0.33060-014
23	0.15300+000	0.64000-001	0.50000+000	0.33060-014
24	0.15300+000	0.45000-001	0.50000+000	0.33060-014
25	0.15300+000	0.11000-001	0.50000+000	0.33060-014
26	0.15300+000	0,20000-002	0,50000+000	0.33060-014
27	0.15300+000	0.18000-001	0.50000+000	0.33060-014
28	0.15300+000	0.17700+000	0.50000+000	0.33060-014
29	0.11800+000	0.10800+000	0,50000+000	0.33060-014
30	0.11800+000	0.88000-001	0.50000+000	0.33060-014
31	0.11800+000	0.64000-001	0.50000+000	0.33060-014
32	0.11800+000	0.55000-001	0.50000+000	0.33060-014

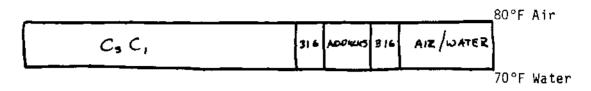
33	0.11800+000	0.33000-001	0.50000+000	0.33060-014
34	0.11800+000	0.24000-001	0.50000+000	0.33060-014
<b>35</b>	0.11800+000	0.12200+000	0.50000+000	0.33060-014
36	0.16800+000	0.55000-001	0.50000+000	0.33060-014
37	0.16800+000	0.45000-001	0.50000+000	0.33060-014
38	0.16800+000	0.24000-001	0.50000+000	0.33060-014
39	0.16800+000	0.14000-001	0.50000+000	0.33060-014
40	0.16800+000	0.13700+000	0.50000+000	0.33060-014
41	0.15300+000	0.14000-001	0.50000+000	0.33060-014
42	0.15300+000	0.60000-002	0.50000+000	0.33060-014
43	0.15300+000	0.15600+000	0.50000+000	0.33060-014
44	0.15300+000	0.13700+000	0.50000+000	0.33060-014
45	0.99000-001	0.30000-002	0.50000+000	0.33060-014
46	0.99000-001	0.24000-001	0.50000+000	0.33060-014
47	0.99000-001	0.64000-001	0.50000+000	0.33060-014
48	0.99000-001	0.15600+000	0.50000+000	0.33060-014
49	0.99000-001	0.20800+000	0.50000+000	0.33060-014
50	0.99000-001	0.33000-001	0.50000+000	0.33060-014
51	0.57000-001	0.24000-001	0.50000+000	0.33060-014
52	0.57000-001	0.64000-001	0.50000+000	0.33060-014
53	0.57000-001	0.15600+000	0.50000+000	0.33060-014
54	0.57000-001	0.34500+000	0.50000+000	0.33060-014
55	0.57000-001	0.75000-001	0.50000+000	0.33060-014
56	0.60000-002	0.75000-001	0.50000+000	0.33060-014
57	0.60000-002	0.20800+000	0.50000+000	0.33060-014
58	0.60000-002	0.49000+000	0.50000+000	0.33060-014
59	0.60000-002	0.13700+000	0.50000+000	0.33060-014
60	0.15300+000	0.75000-001	0.90000+000	0.33060-014
61	0.19100+000	0.26800+000	0.90000+000	0.33060-014
62	0.19100+000	0.20800+000	0.90000+000	0.33060-014
63	0.19100+000	0.15600+000	0.90000+000	0.33060-014
64	0.19100+000	0.18000-001	0.90000+000	0.33060-014
65	0.19100+000	0.12200+000	0.90000+000	0.33060-014
66	0.19100+000	0.88000-001	0.90000+000	0.33060-014
67	0.19100+000	0.60000-002	0.90000+000	0.33060-014

68	0.11800+000	0.20800+000	0.90000+000	0.33060-014
69	0.11800+000	0.17700+000	0.90000+000	0.33060-014
70	0.11800+000	0.15600+000	0.90000+000	0.33060-014
71	0.11800+000	0.33000-001	0.90000+000	0.33060-014
72	0.]1800+000	0.13700+000	0.90000+000	0.33060-014
73	0.11800+000	0.12800+000	0.90000+000	0.33060-014
74	0.11800+000	0.90000-002	0.90000+000	0.33060-014
75	0.57000-001	0.13700+000	0.90000+000	0.33060-014
76	0.57000-001	0.45000-001	0.90000+000	0.33060-014
77	0.57000-001	0.20800+000	0.90000+000	0.33060-014
78	0.57000-001	0.17700+000	0.90000+000	0.33060-014
79	0.57000-001	0.90000-002	0.90000+000	0.33060-014
80	0.70000-002	0.88000-001	0.90000+000	0.33060-014
81	0.70000-002	0.64000-001	0.90000+000	0.33060-014
82	0.70000-002	0.17700+000	0.90000+000	0.33060-014
83	0.70000-002	0.20800+000	0.90000+000	0.33060-014
84	0.70000-002	0.33000-001	0.90000+000	0.33060-014
85	0.57000-001	0.11000-001	0.50000+000	0.33060-014
86	0.57000-001	0.55000-001	0.50000+000	0.33060-014
87	0.57000-001	0.64000-001	0.50000+000	0.33060-014
88	0.57000-001	0.88000-001	0.50000+000	0.33060-014
89	0.57000-001	0.10600+000	0.50000+000	0.33060-014
90	0.57000-001	0.12200+000	0,50000+000	0.33060-014
91	0.35000-001	0.11000-001	0.50000+000	0.33060-014
92	0.35000-001	0.45000-001	0.50000+000	0.33060-014
93	0.35000-001	0.55000-001	0.50000+000	0.33060-014
94	0.35000-001	0.64000-001	0.50000+000	0.33060-014
95	0.35000-001	0.75000-001	0.50000+000	0.33060-014
96	0.35000-001	0.88000-001	0.50000+000	0.33060-014
97	0.35000-001	0.10800+000	0.50000+000	0.33060-014
98	0.35000-001	0.13700+000	0.50000+000	0.33060-014
99	0.35000-001	0.17700+000	0.50000+000	0.33060-014
100	0.35000-001	0.29800+000	0.50000+000	0.33060-014
101	0.19000-001	0.10000+001	0.65000+000	0.33060-014
102	0.60000-002	0.10000+001	0.65000+000	0.33060-014

103	0.49000-001	0.10000+001	0.65000+000	0.33060-014
104	0.10000+000	0.10000+001	0.65000+000	0.33060-014
105	0.16000+000	0.10000+001	0.65000+000	0.33060-014
106	0.71000-001	0.10000+001	0.65000+000	0.33060-014
107	0.11500+000	0.10000+001	0.65000+000	0.33060-014
108	0.81000-001	0.10000+003	0.65000+000	0.33060-014
109	0.20000+000	0.10000+001	0.65000+000	0.33060-014
110	0.13400+000	0.10000+001	0.65000+000	0.33060-014
111	0.59000-001	0.10000+001	0.65000+000	0.33060-014
112	0.23000-001	0.10000+001	0.65000+000	0.33060-014
113	0.15000-001	0.10000+001	0.65000+000	0.33060-014
114	0.12000-001	100+0001.0	0.65000+000	0.33060-014
115	0.28000-001	0.10000+001	0.65000+000	0.33060-014
116	0.40000-001	0.10000+001	0.65000+000	0.33060-014
117	0.40000-002	0.10000+001	0.65000+000	0.33060-014
118	0.30000-002	0.10000+001	0.65000+000	0.33060-014
119	0.33000-001	0.10000+001	0.65000+000	0.33060-014

# APPENDIX C

# CALCULATIONS SUPPORTING THE ONE-DIMENSIONAL STEADY-STATE ANSYS TEMPERATURE PROFILE



# RADIATION AND CONVECTION AT OUTER SURFACE

#### Convection

$$q_{c} = \frac{T_{1} - T_{\infty}}{\frac{1}{hA}} = \frac{T_{1} - T_{\infty}}{\frac{1}{3.4 \times 10^{-6}(2\pi)(1.3125)1}} = \frac{T_{1} - T_{\infty}}{35665}$$
 (Air)

$$q_{c} = \frac{T_{1} - T_{\infty}}{\frac{1}{hA}} = \frac{T_{1} - T_{\infty}}{\frac{1}{1.929 \times 10^{-4}(2\pi)(1.3125)1}} = \frac{T_{1} - T_{\infty}}{628.62}$$
 (Water)

#### Radiation Resistance

$$R_{R} = \frac{1 - \epsilon_{1}}{\epsilon_{2}A_{1}} + \frac{1}{F_{12}A_{1}} + \frac{1 - \epsilon_{2}}{\epsilon_{2}A_{2}}$$

$$= \frac{1 - 0.7}{0.7(2\pi)(1.3125)(1)} + \frac{1}{2\pi(1.3125)(1)(1)} + 0 = 1.7323 \times 10^{-1}$$

$$\frac{1.7323 \times 10^{-1}}{0.3306 \times 10^{-14}} = 5.2399 \times 10^{13}$$

 $T_1^{Air} = 295$ °F

From attached program T1 = 76.6°F

#### Conduction Through Outer Wall

$$q_{cond} = \frac{T_1 - T_2}{\frac{\Delta X}{kA}} = \frac{T_1 - T_2}{\frac{1n(r_2/r_1)}{2\pi kL}} = \frac{T_1 - T_2}{\frac{1n}{1.3125}}$$

$$R = \frac{1.741 \times 10^{-2}}{k}$$

$$R^{Air} = \frac{1.741 \times 10^{-2}}{2 \times 10^{-4}}$$

$$R^{Water} = \frac{1.741 \times 10^{-2}}{1.8071 \times 10^{-4}}$$

$$R^{A} = 87.0496$$
  $R^{W} = 96.3418$   $q = 1.06 \times 10^{-2} = \frac{T_1 - T_2}{R}$   $T_1 = R \cdot 1.06 \times 10^{-2} + T_2$   $T_1^{A} = 87.0496 \cdot (1.06 \times 10^{-2}) + 295$   $T_1^{W} = 96.3418 \cdot (1.06 \times 10^{-2}) + 76.6 \cdot T_1^{A} = 295.9$   $T_1^{W} = 77.6^{\circ}F$ 

Thermal short - no change in temperature through wall.

#### INNER GAP

#### Conduction

$$R_{cond} = \frac{\ln \frac{r_0}{r_1}}{2\pi kL} = \frac{\ln \frac{1.1765}{1.125}}{2\pi kL} = \frac{7.162 \times 10^{-3}}{k}$$

$$R_{cond}^{A} = \frac{7.162 \times 10^{-3}}{4.83 \times 10^{-7}} = 14828 \qquad R_{cond}^{W} = \frac{7.162 \times 10^{-3}}{3.74 \times 10^{-7}} = 19133$$

#### Radiation Resistance

$$R_{AD} = \frac{1 - 0.5}{(0.5)(2\pi)(1.125)1} + \frac{1}{2\pi(1.125)1(1)} + \frac{1 - 0.5}{(0.5)2\pi(1.1765)1} = 0.41822$$

$$\frac{0.41822}{0.3306 \times 10^{-14}} \approx 1.265 \times 10^{14}$$

$$T_2 = 300^{\circ}F$$

$$T_2 = 76.6^{\circ}F$$

 $T_1 = 425^{\circ}F$  From attached program  $T_1 = 253^{\circ}F$ 

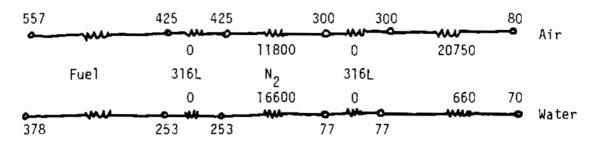
#### <u>Profile in Fuel</u>

Water Storage

$$T - 425 = \frac{3.4466 \times 10^{-3}}{4(6.3 \times 10^{-6})} (0.989^{2} - r^{2}) \qquad T - 253 = \frac{3.4466 \times 10^{-3}}{4(6.75 \times 10^{-6})} (0.989^{2} - r^{2})$$

$$T - 425 = 135(0.989^{2} - r^{2}) \qquad T - 253 = 127.65(0.989^{2} - r^{2})$$

$$T_{\bullet} = 557^{\circ}F \qquad T_{\bullet} = 378^{\circ}F$$



HP Program for Combined Radiation and Convection or Conduction

<nown temperature ... Store 2</pre>

Convection or conduction resistance ... Store 9

Radiation resistance ... Store 7

Key in unknown temperature guess.

001	*LBLE	21 15	021 + <b>-</b> 55
002	ENT↑	-21	022 RCL5 36 05
003	3T01	35 01	023 x -35
004	RCL2	36 02	024 RCL7 36 07
005	-	<b>-</b> 45	025 X <b>‡</b> Y -41
006	ST03	35 03	026 ÷ -24
007	RCL1	36 01	027 ST08 35 08
800	RCL4	36 04	028 RCL9 36 09
009	+	-55	029 x -35
010	<b>X</b> 2	53	030 RCL9 36 09
011	RCL2	36 02	031 RCL8 36 08
012	RCL4	36 04	<b>03</b> 2 + <b>-55</b>
013	+	<b>-</b> 55	03324
014	χ2	53	. 034 STOC 35 13
015	+	-55	035 RCL3 36 03
<b>01</b> 6	ST05	35 05	036 X <b>Հ</b> Y -41
017	RCL1	36 01	037 ÷ -24
018	RCL2	36 02	038 STOB 35 12
019	+	-55	039 RCLA 36 11
020	RCL 6	36 06	040 - <b>-</b> 45
			041 R/S 51

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# APPENDIX D

# STRESS INTENSITY FACTOR CALCULATIONS USING THE DISPLACEMENT METHOD

#### From Tada Handbook

$$u = \frac{K_{I}}{G} \left[ r/(2\pi) \right]^{1/2} \cos \frac{\theta}{2} \left[ 1 - 2\nu + \sin^{2} \frac{\theta}{2} \right]$$

$$\frac{\delta}{2} = \nu = \frac{K_{I}}{G} \left[ r/(2\pi) \right]^{1/2} \sin \frac{\theta}{2} \left[ 2 - 2\nu + \cos^{2} \frac{\theta}{2} \right]$$

$$v = \text{crack opening}$$

$$v = \left\{ \frac{\delta_{I} - \delta_{2}}{2} \right\} \text{ computer output}$$

$$\theta = 180^{\circ}$$

$$\frac{\delta}{2} = v = \frac{K_{I}}{G} \left( r/(2\pi) \right)^{1/2} (2 - 2v) \quad \text{(plane strain)}$$

$$K_{I} = VG \left( \frac{2\pi}{r} \right)^{1/2} \frac{1}{2(1 - v)} , G = \frac{E}{2(1 + v)}$$

$$K_{I} = V \frac{E}{2(1 + v)} \frac{\sqrt{2\pi}}{\sqrt{r}} \cdot \frac{1}{2(1 - v)}$$

$$= \frac{v}{\sqrt{r}} \frac{E\sqrt{2\pi}}{4(1 - v^{2})} = \frac{\sqrt{2\pi}}{4} \left( \frac{E}{1 - v^{2}} \right) \frac{v}{\sqrt{r}}$$

$$K_{I} = 0.627 \left( \frac{E}{1 - v^{2}} \right) \frac{v}{\sqrt{r}} = 0.313 \left( \frac{E}{1 - v^{2}} \right) \frac{\delta}{\sqrt{r}}$$

v = vertical displacement component from  $\theta$  = 180° at nearest node to crack tip in opening

 $\delta$  = total vertical travel of two nodes nearest crack tip

E = temperature dependent elastic modulus

ν = temperature dependent Poisson's ratio

r = distance (radial) from crack tip to nodes of interest

# Extremes for Upper and Lower Weld

# Upper Weld

Quench 20 sec 
$$v = -8.4 \times 10^{-6}$$
 Temp  $\approx 133^{\circ}F$   
Air Heatup 400 sec  $v = 1.75 \times 10^{-6}$  Temp  $\approx 96^{\circ}F$ 

Temp = 
$$96^{\circ}F$$
  $v = 0.26654$   $E = 28 \times 10^6$   $v = 0.26813$   $E = 27.85 \times 10^6$ 

$$r = (x_1 - x_2) = (1.2507 - 1.2566) = |0.0059| \quad \Delta \text{ node coords.}$$

$$K_I^1 = 0.627 \frac{28 \times 10^6}{1 - (0.26654)^2} \frac{-8.4 \times 10^{-6}}{\sqrt{0.0059}}$$

$$K_I^1 = -2067$$

$$K_I^2 = 0.627 \frac{27.85 \times 10^6}{1 - (0.26813)^2} \frac{1.75 \times 10^{-6}}{\sqrt{0.0059}}$$

$$K_I^2 = 430 \qquad \Delta K = 430 - (-2067) \approx 2500 \text{ psi } \sqrt{\text{in.}}$$

$$\Delta K = 2.5 \text{ ksi } \sqrt{\text{in.}}$$

#### Lower Weld

Quench 30 sec 
$$v = -2.5402 \times 10^{-5}$$
 Temp  $\approx 117^{\circ}F$ 

Air Heatup 400 sec  $v = -1.432 \times 10^{-6}$  Temp  $\approx 96^{\circ}F$ 

Temp  $\approx 117^{\circ}F$   $v = 0.26744$   $E = 27.92 \times 10^{6}$   $r = (1.2249 - 1.2326) = 0.0077$ 

$$K_{I}^{1} = 0.627 \frac{27.92 \times 10^{6}}{1 - (0.26744)^{2}} \frac{-2.5402 \times 10^{-5}}{\sqrt{0.0077}} = -5458$$

$$K_{I}^{2} = 0.627 \frac{27.85 \times 10^{6}}{1 - (0.26813)^{2}} \frac{-1.432 \times 10^{-6}}{\sqrt{0.0077}} = -307$$

$$\Delta K = |K_{I}^{1}| - |K_{I}^{2}| = 5151 \text{ psi } \sqrt{\text{in.}}$$

$$\Delta K = 5.15 \text{ ksi } \sqrt{\text{in.}}$$

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# APPENDIX E

# STRESS INTENSITY FACTOR CALCULATIONS USING THE CRACK CLOSURE WORK METHOD

# K ENERGY METHOD

# ORIGINAL CRACK TIP DISP

# 20 sec Quench

#### Upper weld

<u>Node</u>	Displacement				
2033	U U V	=	0.852861 0.149924	x x	10 <sup>-3</sup>
2209	U V U V	= =	0.849709 0.150088	X X	10 <sup>-3</sup> 10 <sup>-1</sup>

# 400 sec Heatup

#### Upper weld

Node	Displacement				
2033	$U_v = 0.255778 \times 10^{-3}$ $U_v^x = 0.128926 \times 10^{-1}$				
	J				
2209	$U_v = 0.256196 \times 10^{-3}$ $U_v^X = 0.128892 \times 10^{-3}$				
	$U_y^* = 0.128892 \times 10^{-3}$				

#### CORRESPONDING NODAL FORCES

# 20 Sec Quench

Upper weld

Node 2033 Element	2401
74.4593	F <sub>y</sub> -70.0324

Element	2404
Fx	Fy
-41.2338	-55.2343

# Node 2209

Element	2322	Element	2323
F <sub>x</sub>	F <sub>y</sub>	F <sub>x</sub>	F <sub>y</sub>
18.8921	52.3352	-52.1176	72.9315

#### 30 sec Quench

Lower weld

#### Node 2403

Element 431		Element 432	
f x 50.8553	F <sub>y</sub> -156.972	F <sub>x</sub> -153.769	F <sub>y</sub> -220.278

# 30 sec Quench

#### Lower weld

Node	Displacement
2403	$U_y = 0.134615 \times 10^{-2}$ $U_y^x = -0.402575 \times 10^{-3}$
2445	$U_{y} = 0.135574 \times 10^{-2}$ $U_{y}^{x} = -0.350449 \times 10^{-3}$

# 400 sec Heatup

#### Lower weld

<u>Node</u>	Displacement			
2403	U = 0.544617 U <sub>v</sub> = 0.10404	x 10 <sup>-3</sup> x 10 <sup>-3</sup>		
2445	U = 0.545147 U = 0.106978	x 10 <sup>-3</sup> x 10 <sup>-3</sup>		

Node 2445

#### 400 sec Heatup

Upper weld

Node 2033

Element	2401	Element	2404
F <sub>x</sub>	F <sub>y</sub>	F <sub>x</sub>	F <sub>y</sub>
-13,524	13.8875	8.96348	12.0141

Node 2209

#### 400 sec Heatup

Lower weld

Node 2403

Node 2445

Elemen	t 223	Element	262
F <sub>X</sub>	F <sub>y</sub>	F <sub>x</sub>	Fy
12.9	11.9718	-7.07385	9,41248

#### CRACK CLOSURE WORK METHOD

#### Plane Strain

$$E_T = \frac{1 - v}{2G} K_I^2$$
  $E_T = \frac{dw}{da} = \frac{\Delta w}{\Delta a}$ 

From Tada Mode I

$$\frac{\Delta w}{\Delta a} = \frac{1/2 (F_y U_y + F_x U_x)}{R(a_2 - a_1)}$$
 Total work includes

modes I and II

 $E_T$  = total energy

v = Poisson's ratio

G = shear modulus, E/2(1 + v)

$$K_{I} = \text{stress intensity factor}$$

$$F_{y} = \text{nodal y force } (r \text{ all element contributions})$$

$$U_{y} = \text{nodal y displ-relative}$$

$$F_{x} = \text{nodal x force } (r \text{ all element contributions})$$

$$U_{x} = \text{nodal displ-relative}$$

$$A_{1} = \text{original crack length}$$

$$A_{2} = \text{first unzipping}$$

$$R = \text{radius to unzipped nodes (released this time step})$$

$$\frac{\Delta w}{\Delta a} = \frac{1 - v}{2G} \quad K_{1}^{2}$$

$$K_{I} = \sqrt{\frac{\Delta w}{\Delta a}} \quad \frac{2G}{1 - v} = K_{1}^{2}$$

$$K_{I} = \sqrt{\frac{\Delta w}{\Delta a}} \quad \frac{2G}{1 - v^{2}} = \sqrt{\frac{\Delta w}{\Delta a}} \quad \frac{2}{1 - v} \quad \frac{E}{2(1 + v)}$$

$$K_{I} = \left[\frac{1/2(F_{y}U_{y} + F_{x}U_{x})}{R(a_{2} - a_{1})} \left(\frac{E}{1 - v^{2}}\right)\right]^{1/2}$$

$$= \left[\frac{E}{2R(a_{2} - a_{1})(1 - v^{2})} \quad (F_{y}U_{y} + F_{x}U_{x})\right]^{1/2}$$

$$K_{I} = \left[\frac{E(F_{y}U_{y} + F_{x}U_{x})}{2R(a_{2} - a_{1})(1 - v^{2})} \left(F_{y}U_{y} + F_{x}U_{x}\right)\right]^{1/2}$$

$$K_{I} = \left[\frac{E(F_{y}U_{y} + F_{x}U_{x})}{2R(a_{2} - a_{1})(1 - v^{2})} \right]^{1/2}$$

so for Mode I

$$K_{I} = \left[\frac{E(F_{y}U_{y})}{2R(a_{2} - a_{1})(1 - v^{2})}\right]^{-1/2}$$

#### Upper Weld

Quench 20 sec 
$$v = 0.26813$$
  $E = 27.85 \times 10^6$   
Heatup 400 sec  $v = 0.26654$   $E = 28 \times 10^6$ 

#### Lower Weld

Quench 30 sec 
$$v = 0.26744$$
 E = 27.92 x  $10^6$  Heatup 400 sec  $v = 0.26654$  E = 28 x  $10^6$ 

#### Quench (20 sec) (Upper)

#### N2033

$$K_{\rm I} = \sqrt{\frac{27.85 \times 10^6 \; (-55.2343 \, + \, -70.0324) \; (0.149924 \times 10^{-1} \, - \, 0.150088 \times 10^{-1})}{2 \frac{(1.2566 \, + \, 1.2624)}{2} \left[ (1.2087 \, - \, 1.2566) \, - \, (1.2087 \, - \, 1.2624) \right] \; (1 \, - \, 0.26813^2)}$$

$$K_{I} = -2054 \text{ psi } \sqrt{\text{in}}.$$
 (-2067 from Displacement Method)

#### Heatup (400 sec)

$$K_{I} = \sqrt{\frac{28 \times 10^{6} (13.887 + 12.0141)(0.128925 \times 10^{-1} - 0.128892 \times 10^{-1})}{2(1.2566 + 1.2624)}[1.2624 - 1.2566](1 - 0.26654^{2})}$$

$$K_{\tau}$$
 = 426 psi  $\sqrt{\text{in.}}$  (430 from Displacement Method)

 $\Delta K_{I} = 426 - (-2054) = 2.48 \text{ ksi } \sqrt{\text{in}}$ . (2.5 from Displacement Method)

Same growth rate for upper weld.

#### Lower Weld

$$K_{I} = \frac{EF_{y}U_{y}}{2R(a_{2} - a_{1})(1 - v^{2})}$$

# Quench (30 sec)

#### N2445

$$\kappa_{\rm I} = \sqrt{\frac{27.92 \times 10^6 (211.377 + 165.872)(0.350449 \times 10^{-3} - 0.402575 \times 10^{-3})}{\frac{2(1.2326 + 1.2403)}{2} (1.2403 - 1.2326) - (1 - 0.26744^2)}}$$

$$K_{\tau}$$
 = -5573 psi  $\sqrt{\text{in.}}$  (-5458 from Displacement Method)

# Heatup (400 sec)

N2445

$$K_{I} = \sqrt{\frac{(28 \times 10^{6})(11.9718 + 9.41248)(0.10404 \times 10^{-3} - 0.106978 \times 10^{-3})}{(1.2326 + 1.2403)(1.2403 - 1.2326)(1 - 0.26744^{2})}}$$

$$K_{\gamma} = -315 \text{ psi } \sqrt{\text{in.}}$$
 (-307 from Displacement Method)

$$\Delta K = -315 - 5573 = 5258$$
 ksi  $\sqrt{in}$ . (5.15 from Displacement Method)

Virtually no difference between results of displacement and energy method.

	Displacement Method K	Energy Method K
Upper weld	-2067 psi √in. 430 ∆K = 2497	-2054 psi √in. 426 ΔK = 2480
Lower weld	-5458 psi √in. -307 ΔK = 5151	-5573 psi √in. -315 5258

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