

R. M. Palest  
04074



BNWL-954  
UC-32

5-

IDX,  
A ONE-DIMENSIONAL DIFFUSION CODE  
FOR GENERATING EFFECTIVE  
NUCLEAR CROSS SECTIONS

March 1969

AEC RESEARCH &  
DEVELOPMENT REPORT

PROJECT NO.	F. O. NO.	LOCATION	FILED DATE
<i>J. W. Upton</i>	33467	<i>FSB</i>	<i>MAR 25 1969</i>
	04074		

BNWL-954

## LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission:

A. Makes any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, method, or process disclosed in this report may not infringe privately owned rights; or

B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method, or process disclosed in this report.

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares, disseminates, or provides access to, any information pursuant to his employment or contract with the Commission, or his employment with such contractor.

## PACIFIC NORTHWEST LABORATORY

RICHLAND, WASHINGTON

operated by

BATTELLE MEMORIAL INSTITUTE

for the

UNITED STATES ATOMIC ENERGY COMMISSION UNDER CONTRACT AT(45-1)-1830

3 3679 00061 2228

BNWL-954

UC-32, Mathematics  
and Computers

IDX, A ONE-DIMENSIONAL DIFFUSION CODE  
FOR GENERATING  
EFFECTIVE NUCLEAR CROSS SECTIONS

By

R. W. Hardie  
W. W. Little, Jr.

Reactor Physics and Operation Department  
FFTF Project

March 1969

FIRST USE OF THIS DOCUMENT IN  
DISTRIBUTION STATE NO. 12 '69

BATTELLE MEMORIAL INSTITUTE  
PACIFIC NORTHWEST LABORATORY  
RICHLAND, WASHINGTON 99352

Printed in the United States of America  
Available from  
Clearinghouse for Federal Scientific and Technical Information  
National Bureau of Standards, U.S. Department of Commerce  
Springfield, Virginia 22151  
Price: Printed Copy \$3.00; Microfiche \$0.65

1DX, A ONE-DIMENSIONAL DIFFUSION CODE  
FOR GENERATING  
EFFECTIVE NUCLEAR CROSS SECTIONS

R. W. Hardie  
W. W. Little, Jr.

ABSTRACT

1DX is a multipurpose, one-dimensional (plane, cylinder, sphere) diffusion theory code for use in fast reactor analysis. The code is designed to:

- Compute  $k_{\text{eff}}$  and perform criticality searches on time absorption ( $\alpha$ ), reactor composition, reactor dimensions, and buckling by means of either a flux or an adjoint model
- Compute and punch collapsed microscopic and macroscopic cross sections averaged over the spectrum in any specified zone
- Compute and punch resonance shielded cross sections using data in the "Russian" format.

All programming is in FORTRAN-IV. Since variable dimensioning is employed, no simple restrictions on problem complexity can be stated. In a 65K memory, 100-group problems are feasible for a moderate number of mesh intervals ( $\sim 30$ ). A representative 26-group  $k_{\text{eff}}$  calculation with 30 spatial intervals using data in the "Russian" format requires about 40 seconds on a UNIVAC 1108. If the cross section data is in the "DTF" format, the same problem would require about 5 seconds.

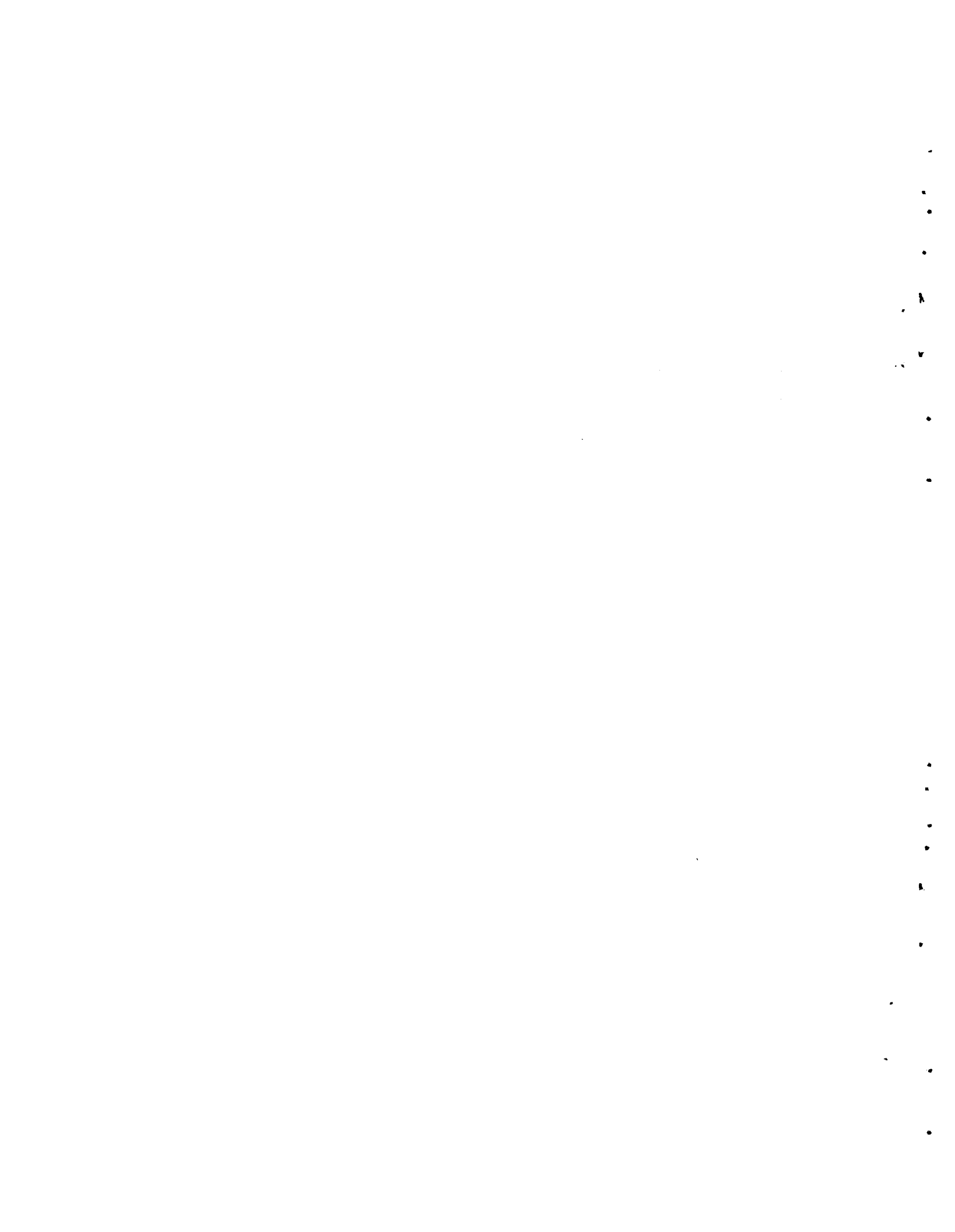


TABLE OF CONTENTS

ABSTRACT . . . . .	iii
I. INTRODUCTION . . . . .	1
II. CALCULATION OF FLUX AND EFFECTIVE MULTIPLICATION CONSTANT . . . . .	3
Formulation of Difference Equations . . . . .	3
Discussion of Boundary Conditions . . . . .	5
Solution of Difference Equations . . . . .	7
III. DISCUSSION OF SEARCH OPTIONS . . . . .	9
Time Absorption ( $\alpha$ calculation) . . . . .	9
Material Concentration C calculation) . . . . .	10
Zone Dimensions ( $\delta$ calculation) . . . . .	11
Buckling ( $B^2$ calculation) . . . . .	11
IV. CALCULATION OF COLLAPSED CROSS SECTIONS . . . . .	13
V. CALCULATION OF RESONANCE SHIELDED CROSS SECTIONS USING DATA IN THE RUSSIAN FORMAT . . . . .	15
Infinite Dilution Cross Sections and Inelastic Scattering Matrices . . . . .	15
Resonance Self-Shielding Factors . . . . .	16
Temperature Interpolation Equation . . . . .	17
$\sigma_0$ Interpolation Equation . . . . .	17
Iteration to Obtain Total Cross Sections . . . . .	18
Heterogeneity Correction to $\sigma_0$ . . . . .	19
Correction to Elastic Removal Using Current Flux Spectrum . . . . .	20
Equations for Multigroup Cross Sections . . . . .	21
REFERENCES . . . . .	23
APPENDIX A: SIMPLIFIED LOGICAL FLOW DIAGRAM . . . . .	A-1
APPENDIX B: INPUT INSTRUCTIONS . . . . .	B-1
APPENDIX C: STORAGE REQUIREMENTS . . . . .	C-1
APPENDIX D: SAMPLE PROBLEM . . . . .	D-1
APPENDIX E: SOURCE DECK LISTING . . . . .	E-1





1DX, A ONE-DIMENSIONAL DIFFUSION CODE  
FOR GENERATING  
EFFECTIVE NUCLEAR CROSS SECTIONS

R. W. Hardie  
W. W. Little, Jr.

I. INTRODUCTION

1DX is a one-dimensional diffusion theory program designed to compute resonance shielded and collapsed group cross sections. Resonance shielded cross sections are calculated using data (infinite dilution cross sections and resonance shielding factors) in the "Russian" format.<sup>(1)</sup> Interpolation schemes are used to compute shielding factors applicable to specific compositions. Collapsed group cross sections by reactor zone are calculated using flux-weighting.

The one-dimensional flux profiles and eigenvalue are computed by standard source-iteration techniques. Convergence is accelerated using fission source over-relaxation. Adjoint calculations are performed by transposing the scattering matrix and fission source and then inverting the group order of the input data.

Criticality searches can be performed on time absorption ( $\alpha$ ), material concentrations, region dimensions, and buckling. Alpha and  $k_{\text{eff}}$  can be used as parametric eigenvalues.

The format of the input data (e.g., cross sections, flux dumps, geometry, and composition specifications) is compatible with the Los Alamos one- and two-dimensional transport codes DTF-IV<sup>(2)</sup> and 2DF,<sup>(3)</sup> the BNW two-dimensional diffusion theory code 2DB,<sup>(4)</sup> and the BNW perturbation code PERT-IV.<sup>(5)</sup> All five codes use the same input module.

All programming is in Fortran-IV. Variable dimensioning is employed to make maximum use of the available fast memory.

A general description of the mathematical models used in 1DX is given in Chapters II-V. The appendices are devoted to details of code operation.

## II. CALCULATION OF FLUX AND EFFECTIVE MULTIPLICATION CONSTANT

### Formulation of Difference Equations

The multigroup diffusion equations can be written in the form

$$D_i \nabla^2 \phi_i - \Sigma_r^i \phi_i + S_i = 0, \quad i = 1, 2, \dots, \text{IGM} \quad (2.1)$$

where

$$S_i = \frac{\chi_i}{k_{\text{eff}}} \sum_{j=1}^{\text{IGM}} (\nu \Sigma_f)^j \phi_j + \sum_{j=1}^{i-1} \Sigma(j \rightarrow i) \phi_j \quad (2.2)$$

and

IGM = number of energy groups,

$i$  = energy group index,

$\phi_i$  = flux in group  $i$ ,

$S_i$  = source in group  $i$ ,

$D_i$  = diffusion constant for group  $i$  ( $= 1/3 \Sigma_{tr}^i$ ),

$(\nu \Sigma_f)^i$  = fission source cross section for group  $i$ ,

$\Sigma(j \rightarrow i)$  = downscattering cross section from group  $j$  to  $i$ ,

$\Sigma_T^i$  = removal cross section for group  $i$

$$\left[ = \Sigma_a^i + \sum_{j=i+1}^{\text{IGM}} \Sigma(i \rightarrow j) \right],$$

$\chi_i$  = fission source fraction in group  $i$ ,

$k_{\text{eff}}$  = effective multiplication constant.

The spatial difference equations in 1DX are set up such that the mesh point is placed in the center of the homogeneous mesh interval. Equations (2.1) and (2.2) are then integrated over the volume associated with each mesh point. For example, for mesh point  $k$ , the integration would be from  $[r_k - (\delta r_k/2)]$  to  $[r_k + (\delta r_k/2)]$ .

The leakage terms are obtained by transforming the volume integral over the Laplacian to a surface integral using Green's theorem,

$$\int_{D\nabla^2} \phi dV = \int_{D\nabla} \vec{\phi} \cdot d\vec{A} \quad (2.3)$$

The flux gradients at the mesh boundary are obtained by interpolating the two contiguous flux values. Thus, volume integration of Equation (2.1) for mesh point k leads to the expression

$$\begin{aligned} & \bar{D}_{k,k-1} A_{k,k-1} \frac{\phi_{k-1} - \phi_k}{r_k - r_{k-1}} + \bar{D}_{k+1,k} A_{k+1,k} \frac{\phi_{k+1} - \phi_k}{r_{k+1} - r_k} \\ & - \Sigma_r^k \phi_k V_k + S_k V_k = 0, \quad k = 1, 2, \dots, IM \end{aligned} \quad (2.4)$$

where, for simplicity, the group indices have been omitted, and

- IM = number of mesh intervals,
- $\phi_k$  = flux associated with mesh point k,
- $r_k$  = radial position of mesh point k,
- $V_k$  = volume associated with mesh point k,
- $S_k$  = source rate associated with mesh point k,
- $\Sigma_r^k$  = removal cross section associated with mesh point k,
- $A_{k,k-1}$  = area of boundary between mesh point k and mesh point k-1,
- $\bar{D}_{k,k-1}$  = effective diffusion constant between mesh point k and mesh point k-1
 
$$\left[ \bar{D}_{k,k-1} = \frac{D_k D_{k-1} (\delta r_k + \delta r_{k-1})}{D_k \delta r_{k-1} + D_{k-1} \delta r_k} \right].$$

Note that if the mesh boundary is not a zone boundary, the expression for  $\bar{D}_{k,k-1}$  reduces to  $D_k$ .

If we let

$$\alpha_k = \frac{D_{k,k-1} A_{k,k-1}}{r_k - r_{k-1}} \quad (2.5)$$

and

$$\beta_k = \alpha_k + \alpha_{k+1} + \sum_r^k V_k \quad , \quad (2.6)$$

Equation (2.4) can be recast into the convenient form

$$\begin{aligned} -\alpha_k \varphi_{k-1} + \beta_k \varphi_k - \alpha_{k+1} \varphi_{k+1} &= S_k V_k, \\ k &= 1, 2, \dots, IM \quad . \end{aligned} \quad (2.7)$$

### Discussion of Boundary Conditions

Three boundary conditions are available in 1DX: reflective ( $\vec{\nabla}\varphi = 0$ ), vacuum ( $\varphi = 0$  at  $0.71 \lambda_{tr}$ ), and periodic.

#### Reflective

Imagine that a pseudo mesh interval (interval 0) with the same composition and thickness of Interval 1 is added on the left side of the left boundary. See Figure 1. If  $\vec{\nabla}\varphi = 0$  at the boundary, then  $\varphi_0 = \varphi_1$ . Since  $(\varphi_0 - \varphi_1)$  vanishes, the coefficient of  $\varphi_0 - \varphi_1$  [see Equation (2.4)] is immaterial, and thus  $\alpha_1$  can be set equal to zero.

#### Vacuum

Again, imagine that a pseudo mesh interval (interval IM+1) with the same composition as interval IM is added to the right side of the right boundary. Since the flux vanishes at  $0.71 \lambda_{tr}$ , from Equation (2.5) we see that the appropriate coefficient of  $\varphi_{IM} - \varphi_{IM+1}$  (where  $\varphi_{IM+1} = 0$ ) is

$$\alpha_{IM+1} = \frac{D_{IM} A_{IM+1,IM}}{0.5 \delta r_{IM} + 0.71 \lambda_{tr}} \quad . \quad (2.8)$$

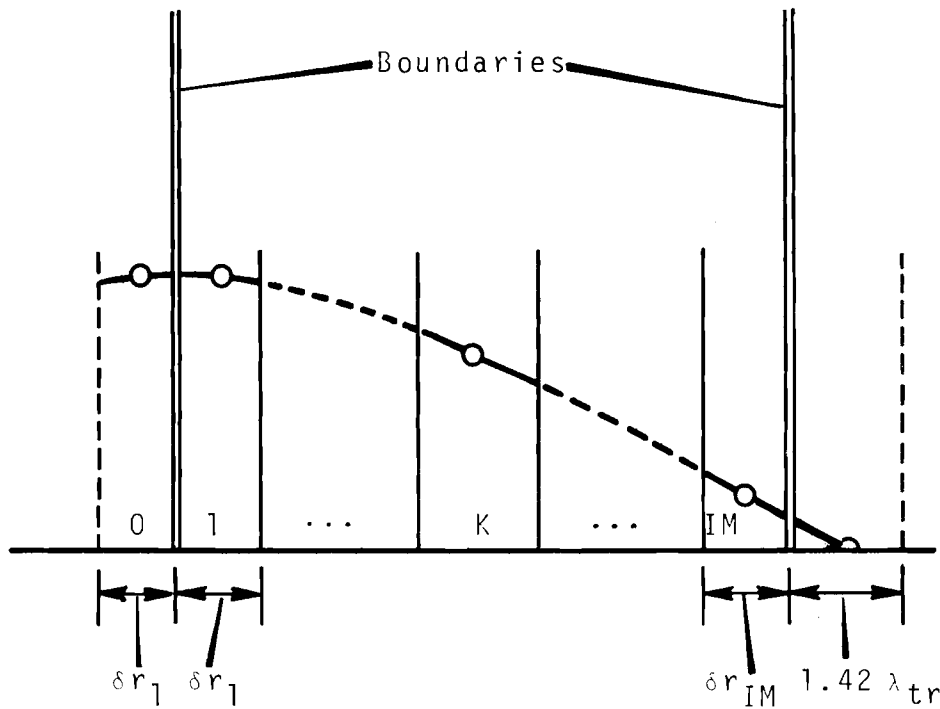


FIGURE 1. Schematic Diagram of Reactor with Reflective Left Boundary and Vacuum Right Boundary

Note that, as in the reflective case, there is no contribution of the pseudo flux in Equation (2.4); i.e., for  $\vec{\nabla} \phi = 0$ ,  $\alpha_1 = 0$ , and for  $\phi = 0$  at  $0.71 \lambda_{tr}$ ,  $\phi_{IM+1} = 0$ .

#### Periodic Flux

In the periodic boundary condition option,

$$\phi_0 \equiv \phi_{IM}$$

and

$$\phi_{IM+1} \equiv \phi_1 \quad .$$

From Equation (2.5), we see that

$$\alpha_1 = \alpha_{IM+1} = \frac{\bar{D}_{IM,1} A_{IM+1,IM}}{0.5(\delta r_1 + \delta r_{IM})} \quad . \quad (2.9)$$

It should be stressed that the pseudo mesh intervals discussed above are not in any way a part of the code. They are mentioned here only for heuristic purposes.

### Solution of Difference Equations

The eigenvalue and spatial flux profiles are computed by standard source-iteration techniques; i.e., by using an initial fission source distribution, the flux profiles in each group are sequentially calculated beginning in the top (highest energy) group. The flux profiles in each group are computed by directly inverting Equation (2.7). After the new fluxes in all groups have been calculated, a new fission source distribution is computed. The multiplication ratio,  $\lambda$ , is then obtained by taking the ratio of the new fission source rate to the old (previous iteration) fission source rate. The above sequence of events is called an outer iteration.

Before each new outer iteration, the fission spectrum is multiplied by  $1/\lambda$ , so that  $\lambda$  approaches unity as the iteration proceeds. The effective multiplication constant is simply the product of the successive  $\lambda$ 's. Convergence is assumed when  $|1 - \lambda| < \epsilon$ , where  $\epsilon$ , the eigenvalue convergence criterion, is an input parameter.

Fission source over-relaxation is employed in 1DX to accelerate convergence. The procedure is as follows: After the new fission source rate profile,  $F_1^{v+1}$ , is calculated, a second "new" value,  $F_2^{v+1}$ , is computed by magnifying the difference between the new fission source rate and the old fission source rate,  $F^v$ , by a factor of  $\beta$ , the over-relaxation factor. Thus,

$$F_2^{v+1} = F^v + \beta(F_1^{v+1} - F^v) \quad (2.10)$$

$F_2^{v+1}$  is then normalized to give the same total source as  $F_1^{v+1}$ .

The adjoint form of Equation (2.1) is solved by transposing (in energy) the scattering matrix, interchanging the role of  $\chi_i$  and  $(v\Sigma_f)^i$ , and inverting the group order of the cross sections, fission spectrum, and velocities. The calculation then proceeds as in a flux calculation. Thus, in an adjoint calculation, the group indices in the output are inverted, and the balance tables lack a direct physical significance.



### III. DISCUSSION OF SEARCH OPTIONS

The 1DX code computes implicit eigenvalue searches on time absorption, material composition, zone thickness, and material buckling. In contrast to a  $k_{\text{eff}}$  calculation, the fission spectrum is not multiplied by  $1/\lambda$  after each outer iteration. Instead, after a converged  $\lambda$  has been obtained ( $|\lambda^{v+1} - \lambda^v| < \epsilon'$ , where  $\epsilon'$  is the parametric eigenvalue convergence criterion) by a sequence of outer iterations, the desired parameter is perturbed to make  $\lambda$  approach unity. That is, first a converged  $\lambda$  is calculated for the initial system. The system is then altered by an amount specified in the input (the eigenvalue modifier) and a second converged  $\lambda$  is calculated. Subsequent parameter changes are determined using either linear or parabolic interpolation procedures. The iteration is continued until  $|1 - \lambda| < \epsilon$ .

#### Time Absorption ( $\alpha$ calculation)

For simplicity, let us consider the one-group, time dependent diffusion equation

$$\frac{1}{v} \frac{\partial \phi(\vec{r}, t)}{\partial t} = D \nabla^2 \phi(\vec{r}, t) - \Sigma_a \phi(\vec{r}, t) + v \Sigma_f \phi(\vec{r}, t) \quad (3.1)$$

If we now assume that

$$\phi(\vec{r}, t) = \phi(\vec{r}) e^{\alpha t} \quad (3.2)$$

we can obviously rewrite Equation (3.1) in the form

$$D \nabla^2 \phi(\vec{r}) - \left( \Sigma_a + \frac{\alpha}{v} \right) \phi(\vec{r}) + v \Sigma_f \phi(\vec{r}) = 0 \quad (3.3)$$

In a time absorption calculation, the parameter,  $\alpha$ , as defined and used in Equations (3.2) and (3.3), is computed as the eigenvalue. Note that  $\alpha/v$  is effectively an absorption cross section--hence the name "time absorption."

Material Concentration (C calculation)

1DX can perform a flexible and comprehensive criticality search on material composition. Any number of materials can simultaneously be added, depleted, or interchanged in any number of zones.

The format for specifying concentration searches can best be described by a simple example. Let us suppose that a zone mixture, say Mix 10, is to be composed of two materials mixed at full density, Materials 8 and 9. Let us further assume that Materials 8 and 9 are to be simultaneously interchanged such that they occupy a fixed volume fraction,  $\beta$ , of the zone mixture. The I0, I1, and I2 vectors could then be set up as shown in the following tabulation.

<u>Mix Number (I0)</u>	<u>Material Number (I1)</u>	<u>Material Density (I2)</u>
10	0	0
10	8	1.0
10	9	-1.0
10	10	0
10	8	$\alpha - 1.0$
10	9	$\beta - \alpha + 1.0$

The first row (10,0,0) instructs the code to clear the storage area for Mix 10. The second row (10,8,1.0) and third row (10,9,-1.0) cause Material 8 and Material 9 to be added to Mix 10 with densities of 1.0 and -1.0, respectively. The fourth row (10,10,0) causes the current contents of Mix 10 to be multiplied by the eigenvalue. Finally, rows five (10,8, $\alpha - 1.0$ ) and six (10,9, $\beta - \alpha + 1.0$ ) instruct the code to add Materials 8 and 9 to Mix 10 with densities of  $\alpha - 1.0$  and  $\beta - \alpha + 1.0$ , respectively.

All of the foregoing can be summarized by the expression

$$\begin{aligned} \Sigma_{10} = & 1.0 \cdot \Sigma_8 \cdot EV - 1.0 \cdot \Sigma_9 \cdot EV + (\alpha - 1.0) \Sigma_8 \\ & + (\beta - \alpha + 1.0) \Sigma_9 \end{aligned} \quad (3.4)$$

where

- $\Sigma_{10}$  = macroscopic cross section for Mix 10,
- $\Sigma_8$  = full density cross section for Material 8,
- $\Sigma_9$  = full density cross section for Material 9,
- EV = eigenvalue.

Note that for an initial eigenvalue guess of 1.0, Equation (3.4) reduces to  $\Sigma_{10} = \alpha \cdot \Sigma_8 + (\beta - \alpha) \Sigma_9$ . Therefore,  $\alpha$  and  $\beta - \alpha$  are simply the initial volume fractions of Materials 8 and 9, respectively.

#### Zone Dimensions ( $\delta$ calculation)

1DX searches on reactor dimensions by varying the dimensions of each mesh interval. Each mesh width,  $\delta r_k$ , is computed from the expression

$$\delta r_k = \delta r_k^0 [1 + (\text{dimensional modifier})_k \text{EV}] \quad , \quad (3.5)$$

where  $\delta r_k^0$  is the initial mesh spacing and EV is the eigenvalue. Different dimensional modifiers can be specified for each zone.

#### Buckling ( $B^2$ calculation)

In a buckling search, the quantity  $D_i \gamma B^2$  (where  $D_i$  is the zone dependent diffusion constant for group  $i$  and  $\gamma$  is the zone dependent buckling modifier) is added to the  $i^{\text{th}}$  group absorption cross section. The in-group scattering cross section,  $\sigma_{gg}^i$ , is reduced by the same amount so that the calculated total cross section remains equal to the input total cross section. The buckling is then computed as the eigenvalue.



#### IV. CALCULATION OF COLLAPSED CROSS SECTIONS

1DX will collapse any IGM group cross section set (microscopic or macroscopic) to a NCR group set ( $IGM > NCR \geq 1$ ) using the fluxes from any specified zone. The regrouping is determined by simply specifying the number of fine groups in each collapsed group. Thus, the energy bounds of the collapsed groups correspond to those in the original group structure. The collapsing scheme is as follows:

Let

$I$  = collapsed energy group index,

$\phi_I$  = total flux in collapsed group  $I$ ,

$\bar{\sigma}^I$  = average microscopic cross section for the  $I^{\text{th}}$  collapsed group,

$\bar{\sigma}(I \rightarrow J)$  = scattering cross section from group  $I$  to group  $J$ .

For simplicity, the zone indices have been omitted. We now define  $\phi_I$  by the equation

$$\phi_I = \sum^I \phi_i \quad , \quad (4.1)$$

where the symbol  $\sum^I$  denotes a sum over the initial energy groups comprising the  $I^{\text{th}}$  collapsed group.

The fission, absorption, and neutron emission cross sections are calculated such that reaction rates are unaltered by the averaging procedure. Therefore, for  $\bar{\sigma}_f$ ,  $\bar{\sigma}_a$ , and  $\bar{\nu}\bar{\sigma}_f$ ,

$$\bar{\sigma}^I = \frac{\sum \sigma^i \phi_i}{\phi_I} \quad . \quad (4.2)$$

Since the leakage rate in group  $I$  is given by  $\sum^I \frac{\nabla^2 \phi_i}{3\Sigma_{tr}^i}$ ,

the formulation for calculating the transport cross section is not as obvious. Two options, normalized weighting and

reciprocal weighting, are available in 1DX to calculate the transport cross section. Normalized weighting is given by

$$\bar{\sigma}_{\text{tr}}^I = \frac{\sum_I \sigma_{\text{tr}}^i \frac{\phi_i}{\sum_{\text{tr}}^i}}{\sum_I \frac{\phi_i}{\sum_{\text{tr}}^i}} \quad (4.3)$$

and reciprocal weighting by

$$\bar{\sigma}_{\text{tr}}^I = \frac{\phi_I}{\sum_I \frac{\phi_i}{\sigma_{\text{tr}}^i}} \quad (4.4)$$

where  $\sum_{\text{tr}}^i$  is the macroscopic cross section for the specified zone.

The collapsed downscattering cross sections are calculated by matching the slowing down rates. Thus,

$$\bar{\sigma}(I \rightarrow J) = \frac{\sum_I \sum_J \sigma(i \rightarrow j) \phi_i}{\phi_I} \quad (4.5)$$

Finally, the in-group scattering cross section is calculated from the identity

$$\bar{\sigma}_{\text{gg}}^I = \bar{\sigma}_{\text{tr}}^I - \bar{\sigma}_a^I - \sum_{J=I+1}^{\text{NCR}} \bar{\sigma}(I \rightarrow J) \quad (4.6)$$

V. CALCULATION OF RESONANCE SHIELDED CROSS SECTIONS  
USING DATA IN THE RUSSIAN FORMAT

Cross section data in the "Russian" format is given in three sections: infinite dilution cross sections, inelastic scattering matrices, and resonance self-shielding factors. A brief description of the format and treatment of this data is given in the following paragraphs. For a more complete description, the reader is referred to the Russian text. (1)

Infinite Dilution Cross Sections and Inelastic Scattering Matrices

For each energy group of each isotope, nine parameters are given. These are defined below.

- $\sigma_t$  = total cross section ( $\sigma_f + \sigma_c + \sigma_e + \sigma_{in}$ ),
- $\sigma_f$  = fission cross section,
- $\nu$  = neutrons released per fission,
- $\sigma_c$  = capture cross section,
- $\sigma_{in}$  = total inelastic scattering cross section (including  $\sigma_{n,2n}$  reaction),
- $\sigma_e$  = total elastic scattering cross section,
- $\mu_e$  = average cosine of elastic scattering angle,
- $\xi$  = average lethargy change by elastic scattering,
- $\sigma_{d,e}$  = elastic slowing-down cross section (in 1DX, elastic removal is always to next energy group).

The second data block contains the inelastic scattering matrix,  $\sigma_{in}(i \rightarrow j)$ . This matrix contains NXCM +1 terms (where NXCM is the number of downscattering terms) for each energy group--one for in-group scattering and NXCM for scattering to the first NXCM lower groups.

The code cross-checks a substantial portion of the above input data using the equations

$$\sigma_t = \sigma_f + \sigma_c + \sigma_e + \sigma_{in} \quad , \quad (5.1)$$

$$\sigma_{in, i} = \sum_j \sigma_{in(i \rightarrow j)} \quad (\text{if } \sigma_{n, 2n} = 0) \quad , \quad (5.2)$$

and

$$\sigma_{d,e} = \frac{\xi \sigma_e}{\delta u} \quad , \quad (5.3)$$

where  $\delta u$  is the lethargy width of the group in question.

### Resonance Self-Shielding Factors

Self-shielding factors are used to account for flux depression in the vicinity of large resonances. For each energy group of each isotope, self-shielding factors are listed for fission, capture, total, and elastic scattering. In crude terms,

$$\bar{\sigma} = f(T, \sigma_0) \sigma \quad (5.4)$$

where

- $\bar{\sigma}$  = resonance shielded cross section,
- $\sigma$  = infinite dilution cross section,
- $f$  = resonance self-shielding factor,
- $T$  = temperature,
- $\sigma_0$  = sum of total cross sections of all other elements in the medium per atom of the isotope under discussion.

The resonance self-shielding factors are compiled for discrete values of  $\sigma_0$  and temperature. The format of the input data (for a temperature dependent isotope) is pictured in Table 1. Although three rows of shielding factors are always given (if the  $f$  factors are temperature dependent), the number of columns (discrete  $\sigma_0$  values) can vary from 1 to 6.

The next two sections describe the interpolation equations employed to compute shielding factors for any temperature and total cross section.



TABLE 1. *Format for Resonance Self-Shielding Factors*

<u>Temperature,</u> <u>°K</u>	<u><math>\sigma_o^1</math></u>	<u><math>\sigma_o^2</math></u>	<u><math>\sigma_o^3</math></u>
$T_a^*$	${}^1f_a$	${}^2f_a$	${}^3f_a$
$T_b$	${}^1f_b$	${}^2f_b$	${}^3f_b$
$T_c$	${}^1f_c$	${}^2f_c$	${}^3f_c$

\*  $T_a < T_b < T_c$

### Temperature Interpolation Equation

For each  $\sigma_o$  column, the shielding factor corresponding to the current temperature is computed by the equation<sup>(6)</sup>

$$f(T) = \alpha + \beta \ln T \quad , \quad (5.5)$$

where

$\beta$  is determined using the two end points,  $f_a$  and  $f_c$ ,  
 $\alpha$  is chosen so that  $F(T_b) = f_b$ .

Although other interpolation schemes may be preferable in some instances, Equation (5.5) appears to be satisfactory for most FBR analyses.

### $\sigma_o$ Interpolation Equation

If  $\sigma_o$ , the total cross section per atom of all other isotopes, lies between any two tabulated total cross sections, say  $\sigma_o^1$  and  $\sigma_o^2$ , the shielding factor is computed from the expression

$$f(\sigma_o) = {}^1f + \frac{{}^2f - {}^1f}{\ln(\sigma_o^2/\sigma_o^1)} \ln(\sigma_o/\sigma_o^1) \quad . \quad (5.6)$$

If  $\sigma_o$  is greater than the largest tabulated total cross section,  $\sigma_o^L$ , or less than the smallest cross section,  $\sigma_o^S$ , the following approximations are employed:

$$1) \quad \sigma_o \geq 10^7, \quad f = 1.0, \quad (5.7)$$

$$2) \quad 10^7 > \sigma_o > \sigma_o^L, \quad f = 1.0 + \frac{L_f - 1.0}{\ln(\sigma_o^L/10^7)} \ln(\sigma_o/10^7), \quad (5.8)$$

$$3) \quad \sigma_o < \sigma_o^S, \quad f = S_f. \quad (5.9)$$

### Iteration to Obtain Total Cross Sections

Since  $\sigma_o$  of each isotope cannot be obtained until the total cross section of all other isotopes is known, one must iterate to obtain both the shielding factors and the total cross sections.

An initial estimate of  $\sigma_o$  for each energy group of the  $k^{\text{th}}$  isotope is

$$\sigma_{o,k}^1 = \frac{1}{N_k} \sum_{j \neq k} N_j \sigma_{t,j}, \quad (5.10)$$

where:

$\sigma_{o,k}$  = total cross section (excluding itself) per atom of the  $k^{\text{th}}$  isotope,

$\sigma_{t,j}$  = total cross section of  $j^{\text{th}}$  isotope,

$N_k$  = atom density of  $k^{\text{th}}$  isotope.

For simplicity, the group index is suppressed.

Subsequent approximations to the total cross section are obtained using the algorithm

$$\sigma_{o,k}^{v+1} = \frac{1}{N_k} \sum_{j \neq k} N_j \left( f_{c,j}^v \sigma_{c,j} + f_{f,j}^v \sigma_{f,j} + f_{e,j}^v \sigma_{e,j} + \sigma_{in,j} \right) \quad (5.11)$$

where  $v$  is the iteration index. The symbol  $f^v$  indicates that  $f$  has been evaluated using the  $v^{\text{th}}$  value of  $\sigma_o$ . It has been found that typical problems converge very well in four or five iterations; hence the number of iterations on  $\sigma_{o,k}$  has been set equal to five.

#### Heterogeneity Correction to $\sigma_o$

If desired, 1DX will compute the total cross section for each fuel isotope using the Bell modification<sup>(7)</sup> to the rational approximation. That is,

$$\sigma_{o,k} = \frac{\Sigma_t^{(f)}}{N_k} + \frac{\Sigma_t^{(m)} V_m}{N_k V_f} \cdot \frac{1}{1 + \frac{4V_m}{S_f} \Sigma_t^{(m)}}, \quad (5.12)$$

or

$$\sigma_{o,k} = \frac{\Sigma_t^{(f)}}{N_k} + \frac{\Sigma_t^{(m)} V_m}{N_k V_f} \cdot \frac{1}{1 + X \Sigma_t^{(m)'}} \quad (5.13)$$

where

$\Sigma_t^{(f)}$  = macroscopic total cross section in fuel excluding isotope  $k$ ,

$\Sigma_t^{(m)}$  = macroscopic total cross section in moderator region,

$N_k$  = atom density of isotope  $k$  in fuel region,

$V_f$  = volume of fuel region,

$V_m$  = volume of moderator region,

$S_f$  = area of fuel region,

$X = 4(V_f + V_m)/S_f$ ,

$\Sigma_t^{(m)'} = V_m \Sigma_t^{(m)} / (V_f + V_m)$ .

The heterogeneity constant,  $X$  (in units of centimeters), is the only input parameter. Note that  $X = 0$  corresponds to a homogeneous mixture of fuel and moderator.

### Correction to Elastic Removal Using Current Flux Spectrum

By necessity, the infinite dilution cross sections and shielding factors were formulated without reference to a particular reactor system. Consequently, group cross sections were calculated using an assumed in-group flux spectrum. The Bondarenko<sup>(1)</sup> cross sections, for example, were computed using a fission spectrum weighting in the top 3 groups, and a constant flux [ $\phi(u) = C$ ] weighting in the remaining groups.

For most cross sections, the above procedure is probably adequate. For elastic moderation, however, appreciable errors can be encountered if the flux that contributes to elastic moderation departs significantly from the group averaged flux.

Since elastic moderation (group transfer) usually originates near the lower group boundary ( $\xi \ll \delta u$ ), it is clearly more logical to use the lower boundary flux than the group averaged flux for computing elastic moderation.

As an option, after calculating a converged eigenvalue and fluxes, 1DX will recompute the elastic moderation cross section,  $\bar{\sigma}_{d,e}$ , using the multigroup fluxes from any specified zone. For group  $i$  (except for  $i = 1$ ), the elastic moderation rate for each isotope is computed using the flux at lethargy point  $u_i'$  where

$$u_i' = u_i - 0.66 \xi \quad . \quad (5.14)$$

Using linear interpolation on  $\xi \bar{\sigma}_e \phi(u)$ ,  $\bar{\sigma}_{d,e}$  is then computed using the expression

$$\bar{\sigma}_{d,e}^i = \frac{1}{\delta u_i} \left\{ (\xi \bar{\sigma}_e)^i + \frac{u_i' - \bar{u}_i}{\bar{u}_{i+1} - \bar{u}_i} \left[ \frac{\phi(\bar{u}_{i+1})}{\phi(\bar{u}_i)} (\xi \bar{\sigma}_e)^{i+1} - (\xi \bar{\sigma}_e)^i \right] \right\} \quad (5.15)$$

for  $i = 2, \dots$ , IGM (see Figure 2). Although any number of iterations on  $\bar{\sigma}_{d,e}$  (i.e., spectrum recalculations) can be made, it has been found that  $\bar{\sigma}_{d,e}$  converges well in a few iterations ( $\leq 5$ ).

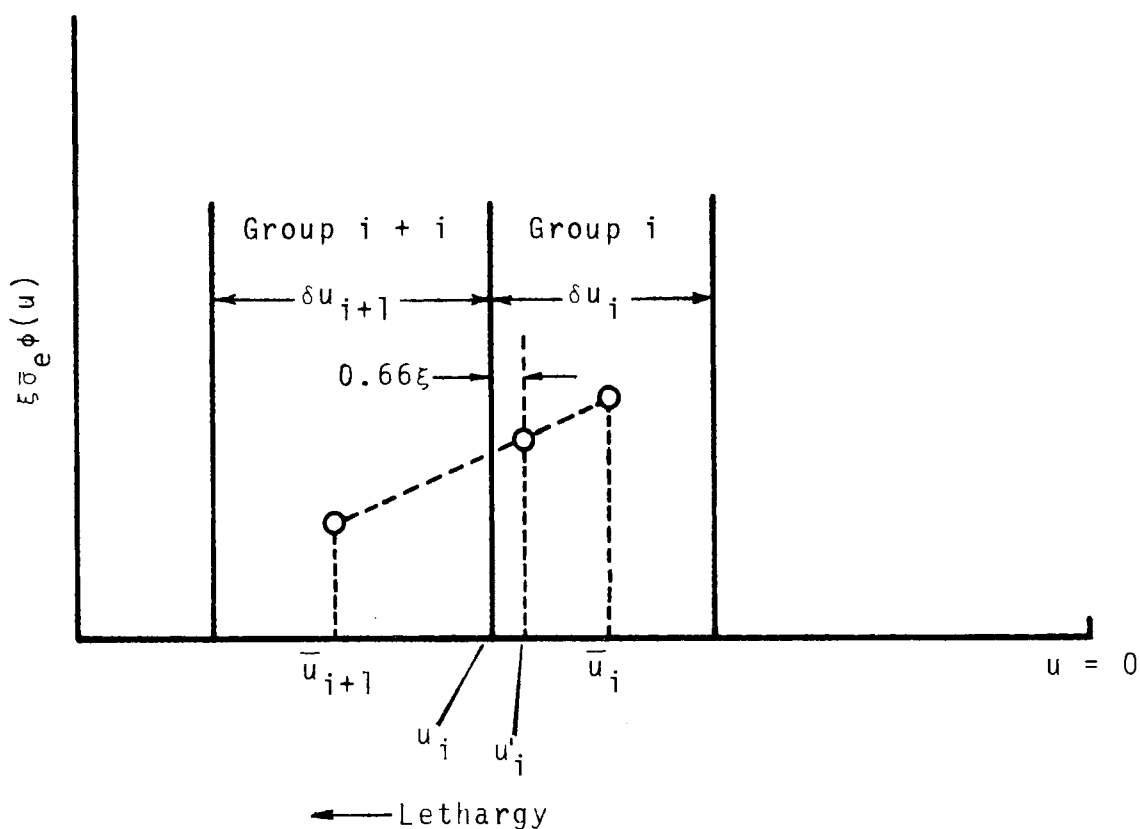


FIGURE 2. Schematic Diagram of Elastic Removal Model

#### Equations for Multigroup Cross Sections

The 1DX code uses and punches cross sections in the "DTF" format:

$$\bar{\sigma}_f^i \quad \bar{\sigma}_a^i \quad \bar{\nu}\bar{\sigma}_f^i \quad \bar{\sigma}_{tr}^i \quad \bar{\sigma}(i \rightarrow i) \quad \bar{\sigma}(i-1 \rightarrow i) \quad \dots$$

The bar indicates that the cross sections include resonance self-shielding.

The above cross sections are computed from data in the "Russian" format using the following equations:

$$\bar{\sigma}_f^i = f_f^i \sigma_f^i \quad , \quad (5.16)$$

$$\bar{\sigma}_a^i = f_f^i \sigma_f^i + f_c^i \sigma_c^i - \left[ \sum_j \sigma_{in}(i \rightarrow j) - \sigma_{in}^i \right] \quad , \quad (5.17)$$

$$(\bar{v}\sigma_f)^i = v^i f_f^i \sigma_f^i \quad , \quad (5.18)$$

$$\begin{aligned} \bar{\sigma}_{tr}^i = & \left[ f_t^i \sigma_t^i - \left( f_f^i \sigma_f^i + f_c^i \sigma_c^i + \sigma_{in}^i \right) \right] (1 - \mu_e^i) \\ & + f_f^i \sigma_f^i + f_c^i \sigma_c^i + \sigma_{in}^i \quad , \end{aligned} \quad (5.19)$$

$$\bar{\sigma}(i-1 \rightarrow i) = \bar{\sigma}_{d,e}^{i-1} + \sigma_{in}(i-1 \rightarrow i) \quad , \quad (5.20)$$

$$\bar{\sigma}(i-2 \rightarrow i) = \sigma_{in}(i-2 \rightarrow i) \quad , \quad (5.21)$$

$$\bar{\sigma}(i-3 \rightarrow i) = \sigma_{in}(i-3 \rightarrow i) \quad , \quad (5.22)$$

...

$$\bar{\sigma}(i \rightarrow i) = \bar{\sigma}_{tr}^i - \bar{\sigma}_a^i - \sum_{j \neq i} \bar{\sigma}(i \rightarrow j) \quad . \quad (5.23)$$

A few words should be given to the treatment of  $\sigma_{n,2n}$ . Since most codes do not handle  $\sigma_{n,2n}$  directly, it must be incorporated in the above cross section format. This is accomplished by adding  $2 \times \sigma_{n,2n}$  to the inelastic scattering matrix and subtracting  $\sigma_{n,2n}$  from the capture cross section.

REFERENCES

1. L. P. Abagyan et al. Group Constant for Nuclear Reactor Calculations, Consultants Bureau, New York, 1964.
2. K. D. Lathrop. DTF-IV, a FORTRAN-IV Program for Solving the Multigroup Transport Equation with Anisotropic Scattering, LA-3373. Los Alamos Scientific Laboratory, Los Alamos, New Mexico, 1965.
3. Unpublished Data. 2DF, A Two-Dimensional Transport Code from the Los Alamos Scientific Laboratory, Los Alamos, New Mexico.
4. W. W. Little, Jr. and R. W. Hardie. 2DB User's Manual, BNWL-831. Pacific Northwest Laboratory, Richland, Washington, 1968.
5. R. W. Hardie and W. W. Little, Jr. PERT-IV, A Two-Dimensional Perturbation Code in FORTRAN-IV, BNWL-409. Pacific Northwest Laboratory, Richland, Washington, 1967.
6. R. E. Schenter. Unpublished Data. Pacific Northwest Laboratory, Richland, Washington. (Personal Communication with Theoretical Physics Section)
7. G. I. Bell. "A Simple Treatment for Effective Resonance Absorption Cross Sections in Dense Lattices," Nucl. Sci. and Eng., vol. 5, p. 138. 1959.



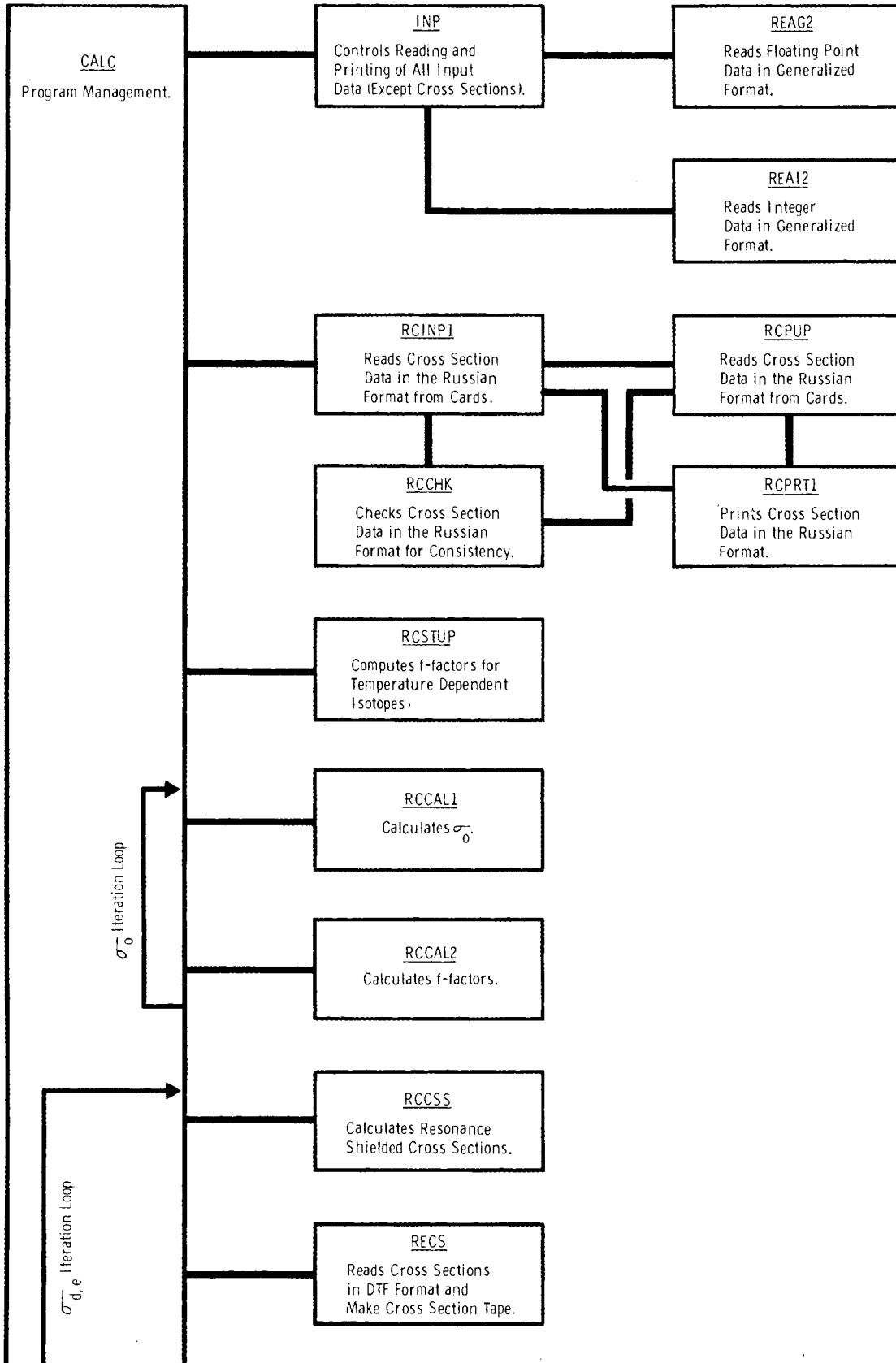


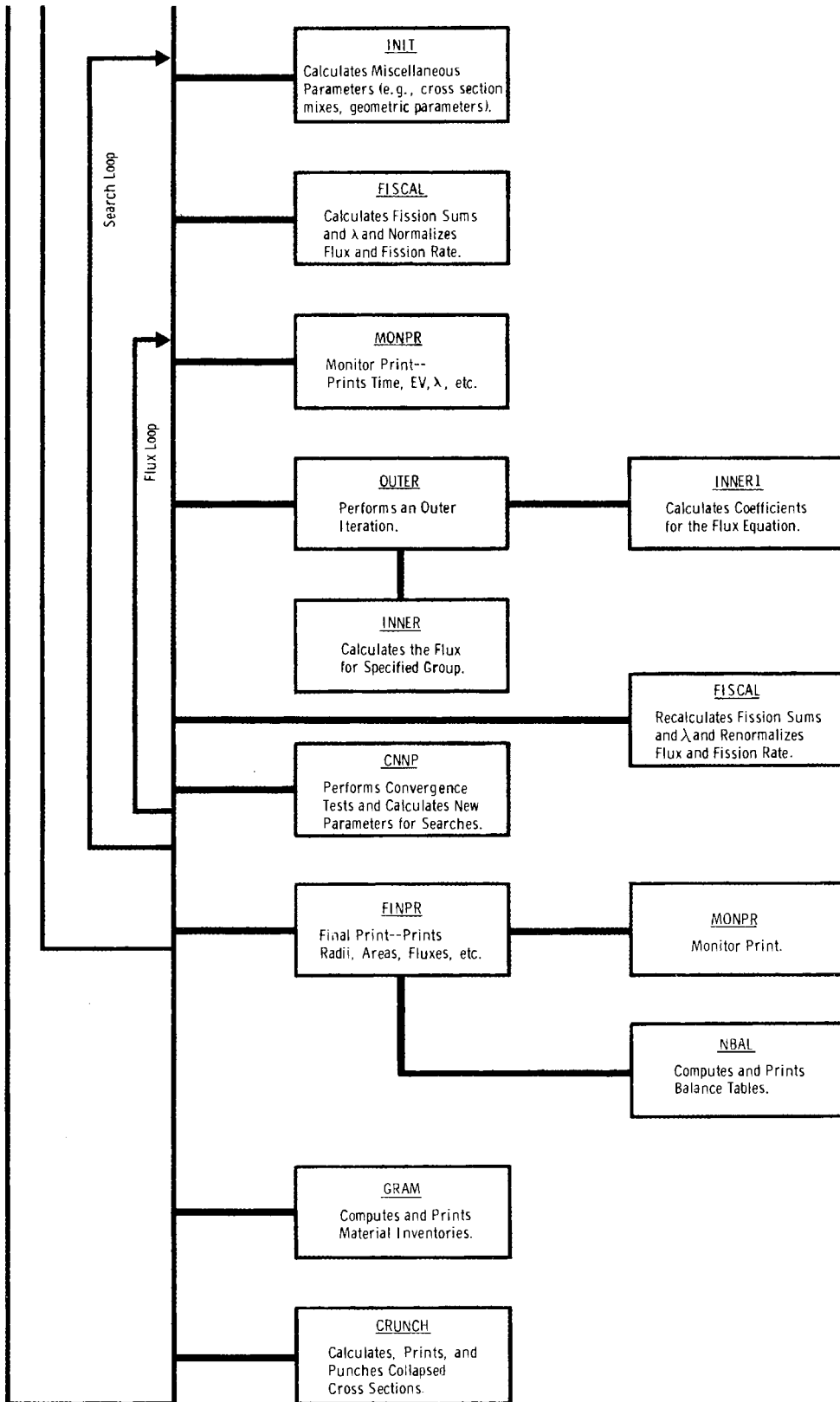
APPENDIX A  
SIMPLIFIED LOGICAL FLOW DIAGRAM

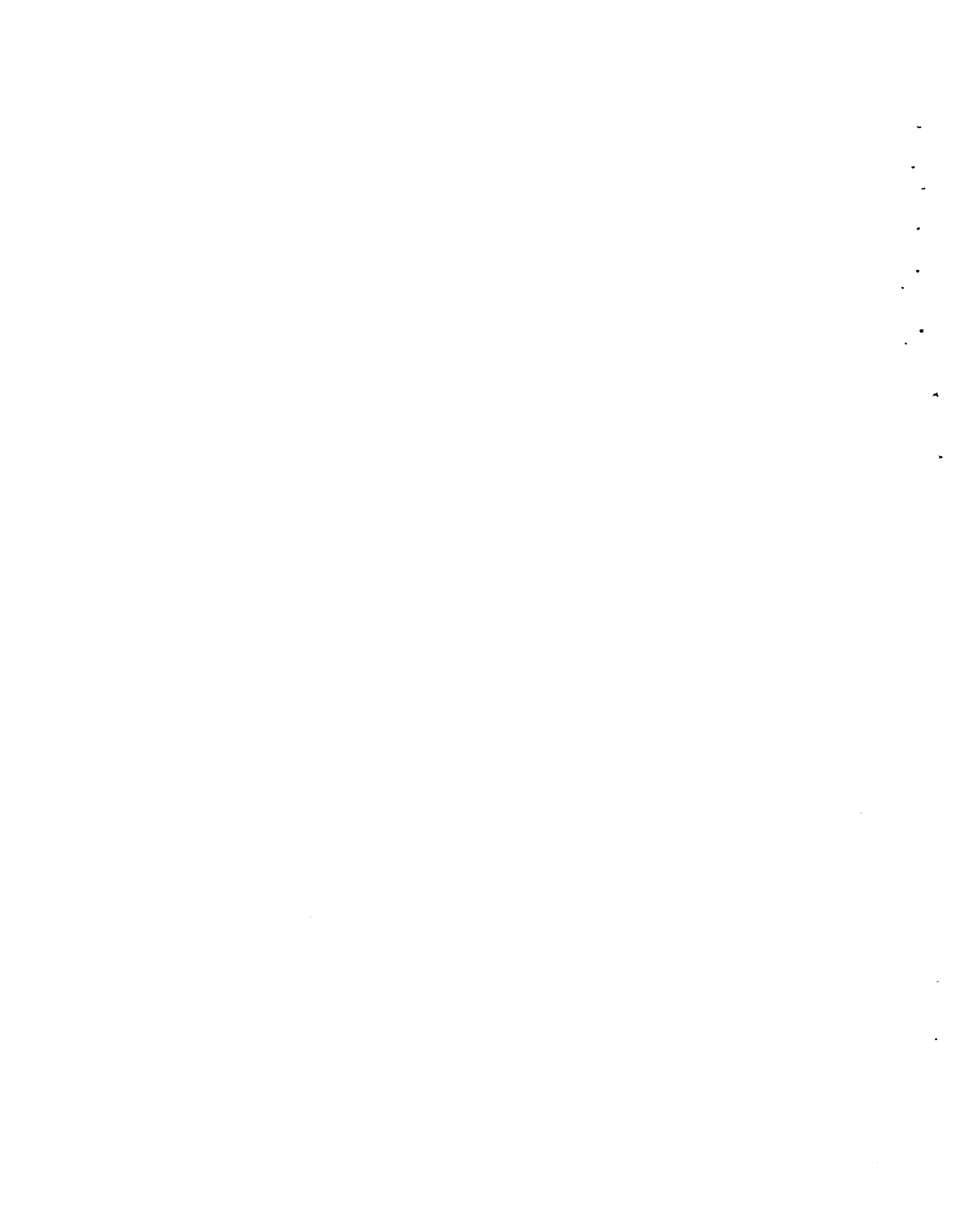
## APPENDIX A

SIMPLIFIED LOGICAL FLOW DIAGRAM

A simplified logical flow chart is given on the following two pages. With the exception of two minor subroutines, CLEAR (sets an array equal to a specified constant) and ERRO2 (prints error messages), all subroutines and their functions are shown in the flow diagram.







APPENDIX B  
INPUT INSTRUCTIONS

APPENDIX B  
INPUT INSTRUCTIONS

The following pages describe the input data for 1DX. Most input is read in via generalized input subroutines. The format for data read in through the generalized input subroutines must adhere to the following form: All cards must contain six data fields of 12 columns each, either 6(I1,I2,I9) for integer data or 6(I1,I2,E9.4) for floating point data. The last nine columns of each field contain the data associated with the particular field; columns 2-3 contain an integer, N, from 0 to 99. The first column of each field must contain:

- 0 - no effect (N = 0),
- 1 - repeat associated entry N times,
- 2 - do N linear interpolations between associated data entry and succeeding data entry,
- 3 - terminate reading of this array with previous data entry.

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
-----------------	----------------	--------------------

CARD 1: FORMAT (11A6,I6)

*To run a series of cases, repeat from this card.*

ID(11)	1-66	Identification card.
MAXT	67-72	Maximum running time (minutes). Not used if zero.

CARD 2: FORMAT (12I6)

A02	1-6	Problem type: = 0, regular calculation, = 1, adjoint calculation.
I04	7-12	Eigenvalue type: = 1, $k_{eff}$ , = 2, time absorption ( $\alpha$ ), = 3, concentration (C), = 4, zone thickness ( $\delta$ ), = 5, buckling ( $B^2$ ).

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
S02	13-18	Parametric eigenvalue type: = 0, none, = 1, $k_{eff}$ , = 2, $\alpha$ .
IGM	19-24	Number of energy groups.
NXCM	25-30	Number of downscattering terms.
ML	31-36	= -N, N input cross section materials from tape on logical unit 15, = +N, N input cross section materials from cards.
M07	37-42	Initial flux guess option: = 0, none, = 1, fluxes from cards.
NPRT	43-48	Print option: = 0, delete printing of cross section data and fluxes (mini), = 1, delete printing of input cross sections in the Russian format (midi), = 2, full print (maxi).
NPUN	49-54	Flux dump option: = 0, no punch, = 1, punch fluxes.
NRCF	55-60	Number of mixes used in generating reso- nance shielded cross sections. If NRCF = 0, cross section data is in DTF format.
NIFF	61-66	Number of spectrum recalculation itera- tions in the calculation of $\bar{\sigma}_{d,e}$ . (See Equation 5.15.) Not used if NRCF = 0.
MM01	67-72	Number of mixture specifications for generating resonance shielded cross sec- tions. (See Appendix D for sample problem.)

CARD 3: FORMAT (12I6)

IGE	1-6	Geometry: = 0, plane, = 1, cylinder, = 2, sphere.
IM	7-12	Number of space intervals ( $\geq 3$ ).
IZM	13-18	Number of material zones.
MT	19-24	Total number of materials, including mixes.
M01	25-30	Number of mixture specifications.



<u>Variable</u>	<u>Columns</u>	<u>Description</u>
B01	31-36	Left boundary condition: = 0, vacuum, = 1, reflective, = 2, periodic.
B02	37-42	Right boundary condition.
NCR	43-48	Number of collapsed groups. If $NCR \geq IGM$ , no group collapsing calculation is done.
NXINP	49-54	Number of collapsed downscattering terms. 1DX also calculates this quantity, NXCMP. If $NXINP = 0$ , NXCMP is used. If $0 < NXINP < NXCMP$ ,
		$\bar{\sigma}(I \rightarrow I + NXINP) \equiv \sum_{J=NXINP}^{NXCMP} \bar{\sigma}(I \rightarrow J) \quad .$
NTR	55-60	Type weighting for collapsed $\sigma_{tr}$ : = 0, normalized [Equation (4.3)], = 1, reciprocal [Equation (4.4)].
NFGM	61-66	Number of collapsed materials.
IPUN	67-72	= 0, print collapsed cross section data, = 1, punch collapsed cross section data, = 2, write collapsed cross section data to tape on logical unit 16, put end of file (EOF) mark after last material, and rewind tape. = 3, write collapsed cross section data to tape on logical unit 16 (an EOF mark is not put on tape and tape is not rewound). If IGM group data is desired as output, set NCR equal to IGM and continue as in a group collapsing calculation.

CARD 4: FORMAT (6E12.6)

EV	1-12	Initial eigenvalue guess. (Used only in search calculations.)
----	------	---

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
EVM	12-24	Initial eigenvalue modifier. This value should decrease $k_{eff}$ --i.e., $EV + EVM$ should produce a lower $k_{eff}$ than $EV$ . Since $EV$ and $EVM$ are completely problem dependent, no representative values can be given. However, these parameters are important so some thought should be given to estimating a reasonable value. (Used only in search calculations.)
S03	25-36	Parametric eigenvalue (see third word on Card 2).
BUCK	37-48	Buckling ( $cm^{-2}$ ). Caution--search calculations that include a buckling term cannot be performed using input cross sections (mixes) directly in zones. Furthermore, a given input mix cannot be used directly in two or more zones in $k_{eff}$ or search problems that have a buckling term. These problems can be avoided by mixing with a density of 1.0. If searching on buckling, BUCK should be zero.
LAL	49-60	Lower limit on $ \lambda - 1 $ , where $\lambda - 1$ is, in essence, the predicted change in the current reactivity. After LAL is reached, the eigenvalue slope is no longer altered. LAL is used only in search calculations. Recommended value $\approx 0.005$ .
LAH	61-72	Upper limit on $ \lambda - 1 $ . If $ \lambda - 1 $ is greater than LAH, LAH rather than $ \lambda - 1 $ is used in predicting the new eigenvalue. LAH is used only in search calculations. Recommended value $\approx 0.5$ .
<u>CARD 5: FORMAT (6E12.6)</u>		
EPS	1-12	Convergence criterion on the total fission source rate. The problem is automatically terminated after 100 iterations.
EPSA	13-24	Parametric eigenvalue convergence criterion. The eigenvalue is recalculated when $ \lambda^{v+1} - \lambda^v $ is less than EPSA. EPSA is only used in search calculations. Recommended value $\approx 10 \times EPS$ .

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
POD	25-36	Parameter oscillation damper. Ratio of the computed eigenvalue change to the predicted eigenvalue change. POD can be used to accelerate convergence or damp out oscillations. (Used only in search calculations.)
ORF	37-48	Over-relaxation factor. If instabilities arise, reduce ORF. Recommended value $\approx 1.4$ .
S01	49-60	If X is negative, the total power is normalized to  X  Mwt using the conversion factor of 215 MeV/fission. If positive, X = total source/ $k_{eff}$ .

CARD 6: FORMAT [6(I1,I2,E9.4)]\*

*Optional--required if M07=1.*

NO(IM,IGM)	1-12	Initial flux guess for first mesh point in first group.
NO(IM,IGM)	13-24	Initial flux guess for second mesh point in first group. Continue for all mesh points and all energy groups. The flux guess for each group does <u>not</u> start on a new card.
. . .		

CARD 7: FORMAT [6(I1,I2,E9.4)]

RO(IM+1)	1-12	Spatial position (cm) of first mesh boundary (0.0).
RO(IM+1)	13-24	Spatial position of second mesh boundary.
. . .		

CARD 8: FORMAT [6(I1,I2,I9)]

MO(IM)	1-12	Zone number for first mesh interval.
MO(IM)	13-24	Zone number for second mesh interval.
. . .		

---

\* Generalized input format (see page B-1).

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
<u>CARD 9: FORMAT [6(I1,I2,I9)]</u>		
M2(IZM)	1-12	Material number for first zone.
M2(IZM)	13-24	Material number for second zone.
. . .		
<u>CARD 10: FORMAT [6(I1,I2,E9.4)]</u>		
<i>Optional--required if BUCK≠0 or if I04=5.</i>		
GAM(IZM)	1-12	Buckling modifier for first zone.
GAM(IZN)	13-24	Buckling modifier for second zone.
. . .		
<u>CARD 11: FORMAT [6(I1,I2,E9.4)]</u>		
K7(IGM)	1-12	Fission fraction (spectrum) in first energy group.
K7(IGM)	13-24	Fission fraction in second energy group.
. . .		
<u>CARD 12: FORMAT [6(I1,I2,E9.4)]</u>		
V7(IGM)	1-12	Neutron velocity for first energy group (cm/sec). Not used if I04≠2 and S02≠2.
V7(IGM)	13-24	Neutron velocity for second energy group.
. . .		
<u>CARD 13: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if M01&gt;0. For instructions in setting up the I0, I1, and I2 cards for a "C" search and for a sample problem using data in the Russian format, see Chapter III and Appendix D, respectively.</i>		
I0(M01)	1-12	Material number of Mix 1.
I0(M01)	13-24	Material number of Mix 1.
. . .		
I0(M01)	N-(N+12)	Material number of Mix 2.
. . .		
<u>CARD 14: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if M01&gt;0.</i>		
I1(M01)	1-12	0.
I1(M01)	13-24	Material number of first material in Mix 1.
I1(M01)	25-36	Material number of second material in Mix 1.
. . .		
I1(M01)	N-(N+12)	0.

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
I1(M01)	(N+13) -(N+24)	Number of first material in Mix 2.
. . .		
<u>CARD 15: FORMAT [6(I1,I2,E9.4)]</u>		
<i>Optional--required if M01&gt;0.</i>		
I2(M01)	1-12	0 (to clear storage area for Mix 1).
I2(M01)	13-24	Concentration of first material in Mix 1 (atoms/barn-cm).
I2(M01)	25-36	Concentration of second material in Mix 1.
. . .		
I2(M01)	N-(N+12)	0 (to clear storage area for Mix 2).
I2(M01)	(N+13) -(N+24)	Concentration of first material in Mix 2.
. . .		
<u>CARD 16: FORMAT [6(I1,I2,E9.4)]</u>		
<i>Optional--required if I04=4.</i>		
R3(IZM)	1-12	Dimensional modifier for first zone (if zero, zone width is held constant).
R3(IZM)	13-24	Dimensional modifier for second zone.
. . .		
<u>CARD 17: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if NCR≤IGM.</i>		
NPN(NCR)	1-12	Number of original groups in first collapsed group.
NPN(NCR)	13-24	Number of original groups in second collapsed group.
. . .		
<u>CARD 18: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if NCR≤IGM.</i>		
NFP(NFGM)	1-12	Material number of first material to be collapsed.
NFP(NFGM)	13-24	Material number of second material to be collapsed.
. . .		

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
<u>CARD 19: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if <math>NCR \leq IGM</math>.</i>		
NZN(NFGM)	1-12	Zone number of fluxes used for first collapsed material.
NZN(NFGM)	13-24	Zone number of fluxes used for second collapsed material.
. . .		
<u>CARD 20: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if <math>NRCF &gt; 0</math>.</i>		
ZN(NRCF)	1-12	Zone number of fluxes used in $\sigma_{d,e}$ iteration for first cross section mix.
ZN(NRCF)	13-24	Zone number of fluxes used in $\sigma_{d,e}$ iteration for second cross section mix.
. . .		
<u>CARD 21: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required <math>NRCF &gt; 0</math>.</i>		
HETC(NRCF)	1-12	Heterogeneity constant (cm) for first cross section mix. See Equation (5.13).
HETC(NRCF)	13-24	Heterogeneity constant for second cross section mix.
. . .		
<u>CARD 22: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if <math>NRCF &gt; 0</math>.</i>		
J1(MM01)	1-12	Not used.
J1(MM01)	13-24	Number of first input isotope in first cross section mix ( $\leq ML$ ).
J1(MM01)	25-36	Number of second input isotope in second cross section mix.
. . .		
J1(MM01)	N-(N+12)	Not used.
J1(MM01)	(N+13) -(N+24)	Number of first input isotope in second cross section mix ( $\leq ML$ ).
. . .		

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
<u>CARD 23: FORMAT [6(I1,I2,E9.4)]</u>		
<i>Optional--required if NRCF&gt;0.</i>		
AEM(MM01)	1-12	Not used.
AEM(MM01)	13-24	Temperature of first input isotope in first cross section mix.
AEM(MM01)	25-36	Temperature of second input isotope in first cross section mix.
. . .		
<u>CARD 24: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if NRCF&gt;0.</i>		
MF(MM01)	1-12	Not used.
MF(MM01)	13-24	Fuel or moderator designation (0/1 = fuel/moderator) of first input isotope in first cross section mix.
MF(MM01)	25-36	Fuel or moderator designation of second input isotope in first cross section mix.
. . .		
<u>CARD 25: FORMAT [6(I1,I2,E9.4)]</u>		
<i>Optional--required if NRCF&gt;0.</i>		
U7(IGM)	1-12	Lethargy width ( $\delta u$ ) for first energy group.
U7(IGM)	13-24	Lethargy width for second energy group.
. . .		
<u>CARD 26: FORMAT [6(I1,I2,I9)]</u>		
<i>Optional--required if NRCF&gt;0 and ML&lt;0.</i>		
NUT( ML )	1-12	Sequence number on cross section tape (generated by PUPX) of first input isotope.
NUT( ML )	13-24	Sequence number on cross section tape of second input isotope.
. . .		
<u>CARD 27: FORMAT (A6,E6.2,7I6,3E6.0)</u>		
<i>Optional--required if NRCF&gt;0 and ML&gt;0.</i>		
HOLN(ML)	1-6	Name (of first input isotope).

<u>Variable</u>	<u>Columns</u>	<u>Description</u>
ATW(ML)	7-12	Atomic weight (used in calculating material inventories). Caution--if NRCF>0 and the same input isotope is in more than one mix used in generating resonance shielded cross sections, the material inventory table is frequently in error.
NT(ML)	13-18	= 0, isotope is not temperature dependent, = 1, isotope is temperature dependent.
NGB(ML)	19-24	Group number where f-factors start.
NGE(ML)	25-30	Group number where f-factors end.
NPFF(4,ML)	31-36	Number of discrete $\sigma_0$ values for fission f-factors.
NPFF(4,ML)	37-42	Number of discrete $\sigma_0$ values for capture f-factors.
NPFF(4,ML)	43-48	Number of discrete $\sigma_0$ values for total f-factors.
NPFF(4,ML)	49-54	Number of discrete $\sigma_0$ values for elastic scattering f-factors.
TEM(3,ML)	55-60	First temperature f-factors are given for.
TEM(3,ML)	61-66	Second temperature f-factors are given for.
TEM(3,ML)	67-72	Third temperature f-factors are given for.

CARD 28: FORMAT (12E6.3)

*Optional--required if NRCF>0, ML>0, and NPFF(1,ML)>0.*

SF(4,ML,6)	1-6	First discrete $\sigma_0$ value for fission f-factors.
SF(4,ML,6)	7-12	Second discrete $\sigma_0$ value for fission f-factors.

. . .

CARD 29: FORMAT (12E6.3)

*Optional--required if NRCF>0, ML>0, NPFF(1,ML)>0, and NT(ML) = 0.*

FF(4, 3xIGM,6)	1-6	First fission f-factor for group NGB(ML).
FF(4, 3xIGM,6)	7-12	Second fission f-factor for group NGB(ML).

. . .



<u>Variable</u>	<u>Columns</u>	<u>Description</u>
FF(4,3xIGM,6)	N-(N+6)	First fission f-factor for group NGB(ML)+1.

. . .

*Continue through group NGB(ML); then repeat cards 28 and 29 for capture, total, and elastic scattering if NFFF(2,ML), NFFF(3,ML) and NFFF(4,ML) are not equal to zero respectively.*

CARD 30: FORMAT (12E6.3)

*Optional--required if NRCF>0, ML>0, NFFF(1,ML)>0, and NT(ML) = 1.*

FF(4,3xIGM,6)	1-6	First fission f-factor for group NGB(ML) at temperature TEM(1,ML).
FF(4,3xIGM,6)	7-12	Second fission f-factor for group NGB(ML) at temperature TEM(1,ML).

. . .

FF(4,3xIGM,6)	N-(N+6)	First fission f-factor for group NGB(ML) at temperature TEM(2,ML).
FF(4,3xIGM,6)	(N+7) -(N+12)	Second fission f-factor for group NGB(ML) at temperature TEM(2,ML).

. . .

*Continue for TEM(3,ML) and through group NGB(ML); then repeat cards 28 and 30 for capture, total, and elastic scattering if NFFF(2,ML), NFFF(3,ML), and NFFF(4,ML) are not equal to zero respectively.*

CARD 31: FORMAT(I3,F10.4,F9.4,F5.3,F10.4,F6.4,F9.4,F6.4,F6.4,F8.4)

*Optional--required if NRCF>0 and ML>0.*

Group index	1-3	Group number.
SR(9,IGM)	4-13	$\sigma_t$ (for first group).
SR(9,IGM)	14-22	$\sigma_f$ .
SR(9,IGM)	23-27	$v$ .
SR(9,IGM)	28-37	$\sigma_c$ .
SR(9,IGM)	38-43	$\sigma_{in}$ .
SR(9,IGM)	44-52	$\sigma_e$ .
SR(9,IGM)	53-58	$\mu_e$ .

Variable	Columns	Description
SR(9,IGM)	59-64	$\xi$ .
SR(9,IGM)	65-72	$\sigma_{d,e}$ .

Repeat through group IGM.

CARD 32: FORMAT [I3,E7.4,10E6.4/(12E6.4)]

Optional--required if  $NRCF > 0$  and  $ML > 0$ .

Group index	1-3	Group number.
SM(IGM,NXCM+1)	4-10	$\sigma_{in}(i+i)$ --for first group.
SM(IGM,NXCM+1)	11-16	$\sigma_{in}(i+i+1)$ .
SM(IGM,NXCM+1)	17-22	$\sigma_{in}(i+i+2)$ .

. . .

Continue through  $\sigma_{in}(i+i+NXCM)$ . Repeat through group IGM.

Repeat from card 27 for ML isotopes.

CARD 33: FORMAT (A6,E6.2,10A6)

Optional--required if  $NRCF = 0$ .

HOLN(ML)	1-6	Name (of first input isotope).
ATW(ML)	7-12	Atomic weight.
AA(10)	13-72	Miscellaneous information.

CARD 34: FORMAT (6E12.5)

Optional--required if  $NRCF = 0$  and  $ML > 0$ . Punched cross section data from 1DX is also in this format.

C(ITL,IGM,ML)	1-12	$\sigma_f$ (barns)--for first group of first isotope.
C(ITL,IGM,ML)	13-24	$\sigma_a$ .
C(ITL,IGM,ML)	25-36	$v\sigma_f$ .
C(ITL,IGM,ML)	37-48	$\sigma_{tr}$ .
C(ITL,IGM,ML)	49-60	$\sigma(i+i)$ .
C(ITL,IGM,ML)	61-72	$\sigma(i-1+i)$ .

. . .

Continue through  $\sigma(i-NXCM+i)$ . Repeat through group IGM.

Repeat from card 33 for ML isotopes (if  $ML < 0$ , simply repeat card 33 for each input isotope).

# 1DX

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
1D(11) Identification Card																																																																	MAXT Max. Running Time (min)														
<u>A02</u> Problem Type: = 0, Regular = 1, Adjoint	<u>I04</u> Eigenvalue Type: = 1, $k_{eff}$ = 2, Absorption ( $\kappa$ ) = 3, Concentration (C) = 4, Zone Thickness ( $\delta$ ) = 5, Buckling ( $B^2$ )	<u>S O 2</u> Parametric Eigenvalue Type: = 0, None = 1, $k_{eff}$ = 2, $\kappa$	<u>IGM</u> No. of Energy Groups	<u>NXGM</u> No. of Down- Scattering Terms	<u>ML</u> = -N, N Input Materials From Tape = +N, N Input Materials From Cards	<u>M07</u> Initial Flux Guess Option: = 0, None = 1, Fluxes From Cards	<u>NPRT</u> Print Option: = 0, Mini = 1, Midi = 2, Maxi	<u>NPUN</u> Flux Punch Option: = 0, No Punch = 1, Punch Fluxes on Cards	<u>NRCF</u> No. of Mixes Used in Genera- ting Resonance Shielded Cross Sections (If NRCF = 0, XS Data is in DTF Format)	<u>NIFF</u> No. of Spectrum Recalculat- ions in the Calculation of $\sigma_s, e$	<u>MMO1</u> No. of Mix Specifica- tions for Generating Shielded Cross Sections																																																																				
<u>IGE</u> Geometry: = 0, Plane = 1, Cylinder = 2, Sphere	<u>IM</u> No. of Space Intervals	<u>I ZM</u> No. of Material Zones	<u>MT</u> Total No. of Materials Including Mixes	<u>MO1</u> No. of Mixture Specifica- tions	<u>BO1</u> Left Boundary Condition: = 0, Vacuum = 1, Reflec- tive = 2, Periodic	<u>BO2</u> Right Boundary Condition	<u>NCR</u> No. of Collapsed Groups (If NCR $\neq$ IGM No Group Collapsing)	<u>NXINP</u> No. of Collapsed Down- scattering terms (If NXINP = 0, Calculated by Code)	<u>NTR</u> Type Weighting for Collapsed $\sigma_{tr}$ = 0, Normalized = 1, Reciprocal	<u>NFGM</u> No. of Collapsed Materials	<u>IPUN</u> = 0, Print Collapsed Cross Section Data = 1, Punch Collapsed Data = 2, Write Collapsed Data on Tape, EOF, and Rewind Tape = 3, Write Collapsed Data on Tape																																																																				
1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80

B-13

BNWL-954

	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
●	EV Initial Eigenvalue Guess		EVM Initial Eigenvalue Modifier		S03 Parametric Eigenvalue		BUCK Buckling (cm <sup>-2</sup> )		LAL Lambda Lower		LAH Lambda Upper																																																																					
●	EPS Eigenvalue Convergence Criterion		EPSA Parametric Eigenvalue Convergence Criterion		POD Parameter Oscillation Damper		ORF Over-relaxation Factor		SQ1 • Negative, Power (Mwt) • Positive, Neutron Source Rate																																																																							
IF (MOD) = 1 { ●	Flux Guess for First Mesh Point in First Group		Flux Guess for Second Mesh Point in First Group		● ● ●		Flux Guess for 1Mth Mesh Point in First Group		Flux Guess for First Mesh Point in Second Group		● ● ● NO(1M, 1GM)																																																																					
●	Spatial Position of First Mesh Boundary (0, 0)		Spatial Position of Second Mesh Boundary (cm)		● ● ●							RO(1M+1)																																																																				
●	Zone No. for First Mesh Interval		Zone No. for Second Mesh Interval		● ● ●							MO(1M)																																																																				
●	Material No. for First Zone		Material No. for Second Zone		● ● ●							M2(1ZM)																																																																				
IF (BUCK) = 0 or IF (MOD) = 5 { ●	Buckling Modifier for First Zone		Buckling Modifier for Second Zone		● ● ●							GAM(1ZM)																																																																				
	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80

B-14

BNWL-954

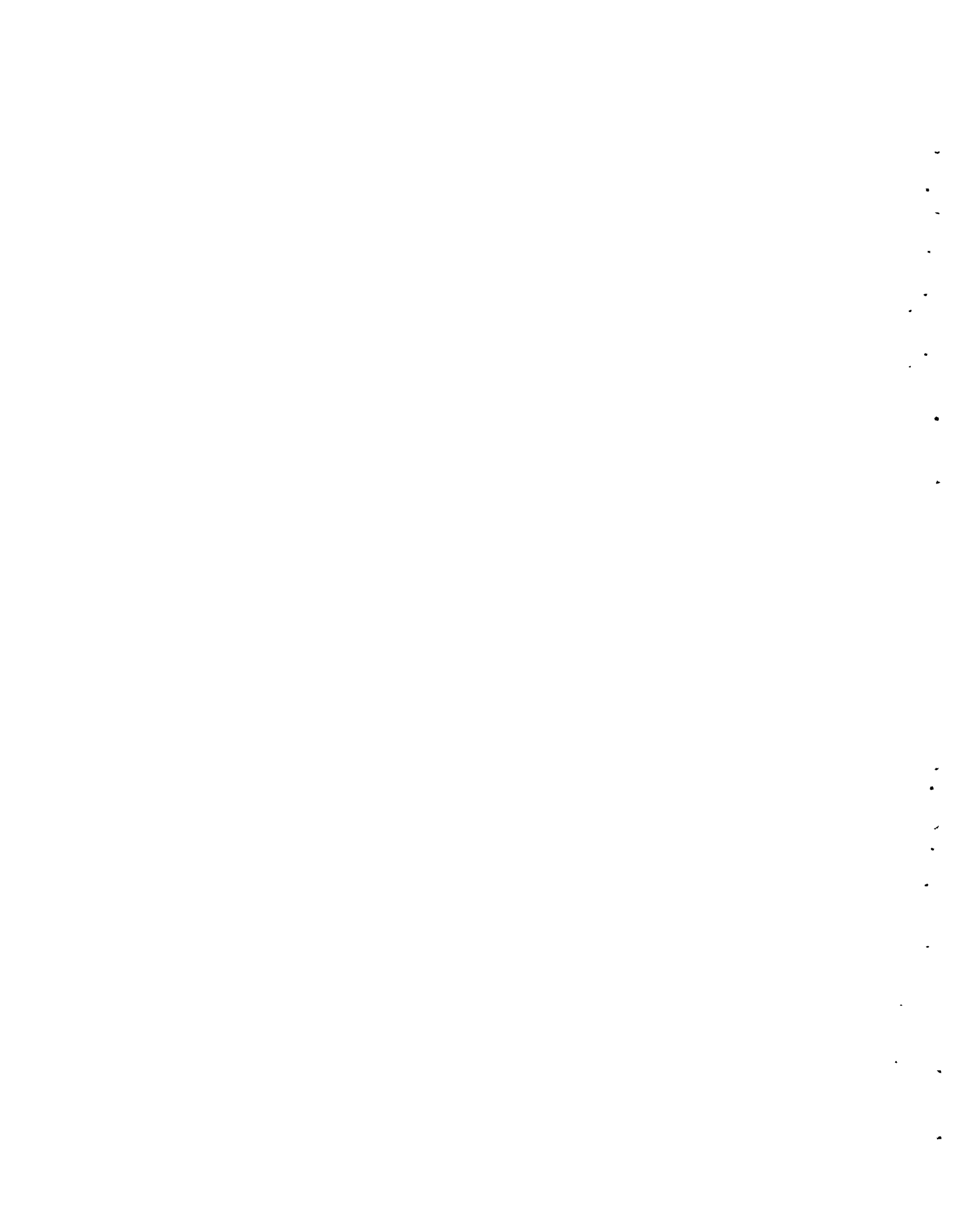
		1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
IF(MO1) > 0	Fission Fraction in First Energy Group	Fission Fraction in Second Energy Group		● ● ●																																	K7(IGM)																																												
	Neutron Velocity for First Energy Group	Neutron Velocity for Second Energy Group		● ● ●																																	V7(IGM)																																												
	Material No. of Mix 1	Material No. of Mix 1		● ● ●			Material No. of Mix 2										Material No. of Mix 2										● ● ●										I0(MO1)																																												
	0	Material No. of First Material in Mix 1		Material No. of Second Material in Mix 1			● ● ●										0										● ● ●										I1(MO1)																																												
	0	Concentration of First Material in Mix 1		Concentration of Second Material in Mix 1			● ● ●										0										● ● ●										I2(MO1)																																												
	IF(OA) = 4	Dimensional Modifier for First Zone	Dimensional Modifier for Second Zone		● ● ●																																	R3(IZM)																																											
No. of Original Groups in First Collapsed Group		No. of Original Groups in Second Collapsed Group		● ● ●																																	NPN(NCR)																																												

IF INCR) ≤ IGM

IF INRCF > 0

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
Material No. of First Material to be Collapsed		Material No. of Second Material to be Collapsed		●			●			●																			NFP(NFGM)																																																		
Zone No. of Fluxes Used for First Collapsed Material		Zone No. of Fluxes Used for Second Collapsed Material		●			●			●																			NZN(NFGM)																																																		
Zone No. of Fluxes Used in $\sigma_{d,e}$ Iteration for First Cross Section Mix		Zone No. of Fluxes Used in $\sigma_{d,e}$ Iteration for Second Cross Section Mix		●			●			●																			ZN(INRCF)																																																		
Heterogeneity Constant for First Cross Section Mix		Heterogeneity Constant for Second Cross Section Mix		●			●			●																			HETC(INRCF)																																																		
0		Number of First Input Isotope in First Cross Section Mix		Number of Second Input Isotope in First Cross Section Mix			●			●			●																			0		●			●			J1(MMO1)																																							
0		Temperature of First Input Isotope in First Cross Section Mix		Temperature of Second Input Isotope in First Cross Section Mix			●			●			●																			0		●			●			ATEM(MMO1)																																							

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
													0	Fuel or Moderator (0/1 - Fuel/Moderator) Designation of First Isotope in First Cross Section Mix													Fuel or Moderator Designation of Second Isotope in First Cross Section Mix													● ● ●			0	● ● ●			MF(MMO1)																																
Lethargy Width for First Energy Group													Lethargy Width for Second Energy Group													● ● ●																U7(IGM)																																					
Cross Section Data																																																																															
1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80





APPENDIX C  
STORAGE REQUIREMENTS

## APPENDIX C

STORAGE REQUIREMENTS

1DX uses variable dimensioning by storing the subscripted variables in one array, A(35000). The variable dimensioned arrays require N storage locations ( $N \leq 35000$ ), where

$$N = \text{MAX}(N_1, N_2)$$

$$\begin{aligned} N_1 = & \text{IGM}(13 + 5 \cdot \text{MM01}^\dagger - 4 \cdot \text{NRCF}^\dagger + \text{NXCM}^\dagger + 83^\dagger + 1^\times) \\ & + \text{IZM}(\text{ML} + \text{IGM} + 4 + 2 \cdot \text{NCR}^\times + 2 \cdot \text{IGM}^\times + 1^*) \\ & + \text{IM}(4 \cdot \text{IGM} + 12) \\ & + \text{ML}(4 + 34^\dagger) \\ & + 4 \cdot \text{M01} \\ & + 3 \cdot \text{MM01}^\dagger \\ & + 3 \cdot \text{NRCF}^\dagger \\ & + \text{NCR}^\times \\ & + \text{MT}(\text{NXCM} + 5) \\ & + 2 \cdot \text{NFGM}^\times \\ & + 12 \end{aligned}$$

and

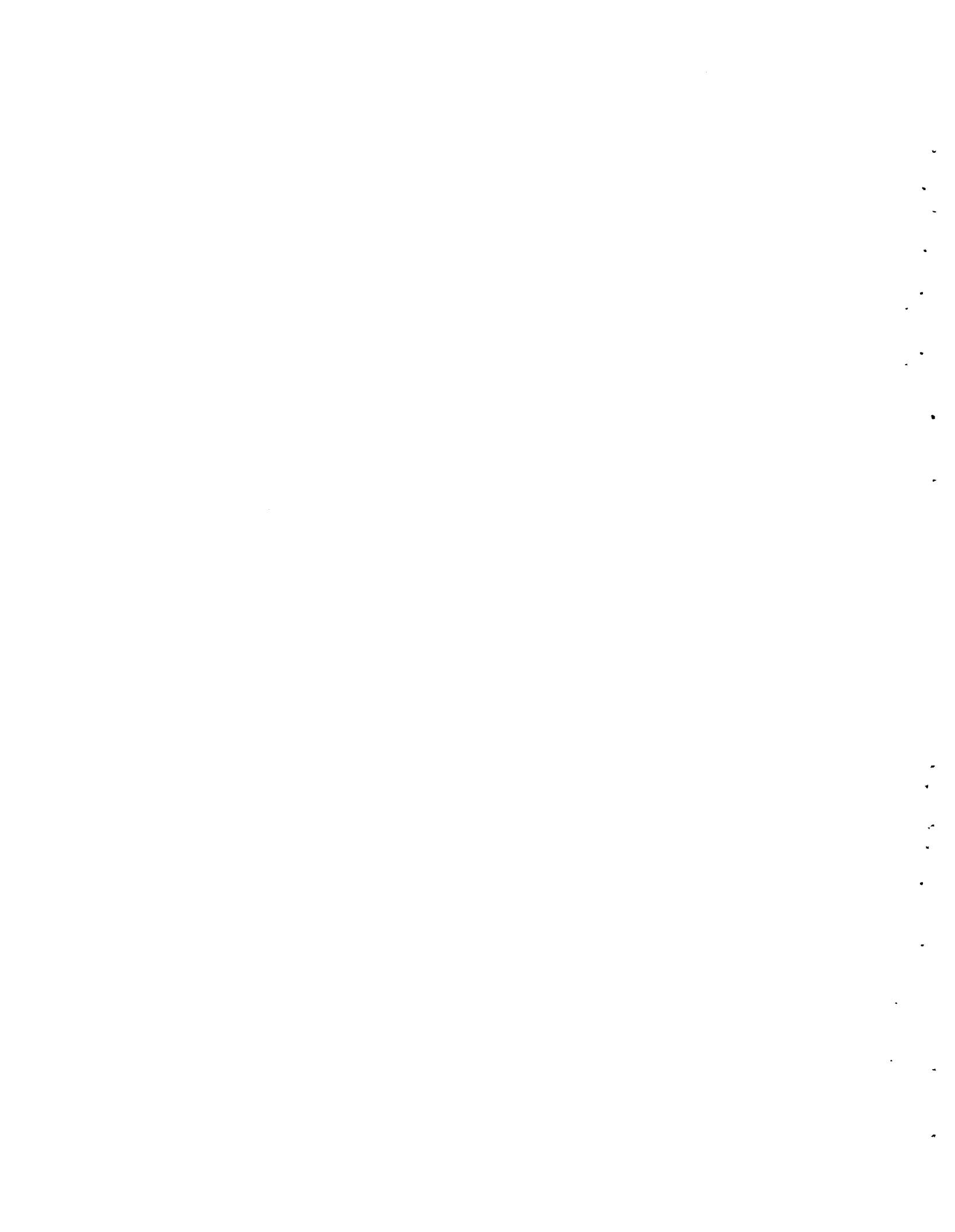
$$\begin{aligned} N_2 = & \text{IGM}(2 + 5 \cdot \text{MM01}^\dagger - 4 \cdot \text{NRCF}^\dagger + \text{NXCM}^\dagger + 83^\dagger + 1^\times) \\ & + \text{IZM}(\text{IGM} + 3 + 2 \cdot \text{NCR}^\times + 2 \cdot \text{IGM}^\times + 1^*) \\ & + \text{MT}(\text{NXCM} + 5)(\text{IGM} + 1) \leftarrow \\ & + \text{IM}(\text{IGM} + 7) \\ & + \text{ML}(4 + 34^\dagger) \\ & + 3 \cdot \text{M01} \\ & + 3 \cdot \text{MM01}^\dagger \\ & + 3 \cdot \text{NRCF}^\dagger \\ & + \text{NCR}^\times \\ & + 2 \cdot \text{NFGM}^\times \\ & + 4 \end{aligned}$$

All of the above variables are input to 1DX on cards 1 and 2.

<sup>†</sup> resonance shielding calculation

<sup>×</sup> group collapsing calculation

<sup>\*</sup> delta ( $\delta$ ) eigenvalue calculation



APPENDIX D  
SAMPLE PROBLEM

APPENDIX D  
SAMPLE PROBLEM

This section demonstrates the procedure used in setting up a 1DX input deck for a problem using cross section data in the Russian format. The problem is a 26 group  $k_{\text{eff}}$  calculation for a two-zone spherical reactor (see Figure D.1). The number of mesh intervals (IM) is 30 (20 for core and 10 for blanket); each interval is equal to 2 cm. The number of mixes used in generating resonance shielded cross sections (NRCF) is two, one mix for the core and one for the reflector.

Isotope	Atom Densities	
	Core	Blanket
1 Pu <sup>239</sup>	0.0020	0.0000
2 U <sup>238</sup>	0.0080	0.0150
3 O	0.0192	0.0300
4 Fe	0.0120	0.0200
ML=5 Na	0.0110	0.0050

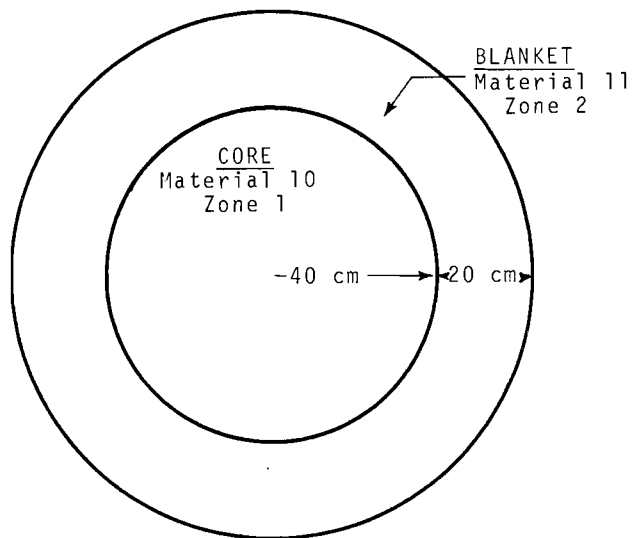


FIGURE D-1. Schematic Diagram of Reactor Considered in Sample Problem

The input is self-explanatory, except perhaps, for the I0, I1, I2, and J1 arrays (see Table D.1). If the cross section data is in the DTF format, the I1 array gives the material number (macroscopic or microscopic) of the cross section to be added to material I0 at density I2. However, for cross section data in the Russian format, the array J1 is the number of the input isotope and I1 is the resulting material

number of the shielded cross section set. In the sample problem, for example,  $U^{238}$ , the second input isotope, is designated material 2 (in the core) and material 6 (in the reflector). Because of resonance shielding, it is necessary to give the same isotope a different material number whenever it is used in a cross section mix. From MM01 (the number of cross section mix specifications) + 1 to M01, the I1 vector has the same meaning as when the data is in the DTF format. Thus, in our example, rows 12, 13, and 14 result in a macroscopic  $U^{238}O_2$  cross section set calculated in the reflector mix environment.

TABLE D-1. *Material Specifications for Sample Problem*

	<u>I0</u>	<u>I1</u>	<u>I2</u>	<u>J1</u>
1	10	0	0	0
2	10	1	0.0020	1
3	10	2	0.0080	2
4	10	3	0.0192	3
5	10	4	0.0120	4
6	10	5	0.0110	5
7	11	0	0	0
8	11	6	0.0150	2
9	11	7	0.0300	3
10	11	8	0.0200	4
11 (MM01)	11	9	0.0050	5
12	12	0	0	
13	12	6	0.0244	
14 (M01)	12	7	0.0488	

1DX	SAMPLE PROBLEM												
	0	1	0	26	10	-5	0	1	0	2	5	2	
	2	30	2	12	14	1	0	4	0	1	11	11	
		0.0		0.0		.0		0.0		.001		0	
		1.0-6		1.0-5		1.0		1.3		-400.0		.5	
219		0.02 9		40.0		60.03							
120		1110		23									
		10		113									
		0.020		0.098		0.190		0.268		0.196		0.135	
		0.058		0.022		0.009		0.003		0.001115		0.0	
3													
126		1.03											
1 6		101 5		111 3		123							
		0		1		2		3		4		5	
		0		6		7		8		9		0	
		6		73									
		0		.0020		. 80		.0192		.0120		.0110	
		0		.0150		. 300		.0200		.0050		0	
		.0244		.04883									
		4		4		6		123					
		1		2		3		4		5		10	
		6		7		8		9		113			
1 6		11 5		23									
		1		23									
1 2		1.03											
		0		1		2		3		4		5	
				2		3		4		53			
1 6		900.01 5		800.03									
1 4		1 2		11 3		01 2		13					
1 3		.481 2		.571 3		.69117		.77		1.03			
		1		4		6		8		103			

R0  
 M0  
 M2  
 K7  
 K7  
 K7  
 V7  
 I0  
 I1  
 I1  
 I1  
 I2  
 I2  
 I2  
 NPN  
 NFP  
 NFP  
 NZN  
 ZN  
 HETC  
 J1  
 J1  
 ATEM  
 MF  
 U7  
 NUT

\* \* \* \* I D X \* \* \* \*

1DX SAMPLE PROBLEM

2

A02	0/1=REGULAR CALCULATION/ADJOINT CALCULATION	0
I04	EIGENVALUE TYPE (1/2/3/4/5=KEFF/ALPHA/CONCENTRATION/DELTA/BUCKLING)	1
S02	PARAMETRIC EIGENVALUE TYPE (0/1/2=NONE/KEFF/ALPHA)	0
IGM	NUMBER OF ENERGY GROUPS	26
NXCM	NUMBER OF DOWNSCATTERING TERMS	10
ML	NEGATIVE/POSITIVE=NUMBER OF MATERIALS FROM TAPE/CARDS	-5
M07	FLUX GUESS (0/1=NONE/CARDS)	0
NPRT	PRINT OPTION (0/1/2=MINI/MIDI/MAXI)	0
NPUN	FLUX DUMP (0/1=NO/YES)	0
NRCF	NO. OF MIXES USED IN GENERATING XS DATA (IF 0, DATA IS IN DTF FORMAT)	2
NIFF	NUMBER OF ITERATIONS IN CALCULATION OF SIGE	5
MM01	NUMBER OF MIX SPECIF. FOR GENERATING SHIELDED XS	11
IGE	GEOMETRY (0/1/2=PLANE/CYLINDER/SPHERE)	2
IM	NUMBER OF SPACE INTERVALS	30
IZM	NUMBER OF MATERIAL ZONES	2
MT	TOTAL NUMBER OF MATERIALS INCLUDING MIXES	12
M01	NUMBER OF MIXTURE SPECIFICATIONS	14
B01	LEFT BOUNDARY CONDITION (0/1/2=VACUUM/REFLECTIVE/PERIODIC)	1
B02	RIGHT BOUNDARY CONDITION (0/1/2=VACUUM/REFLECTIVE/PERIODIC)	0
NCR	NUMBER OF COLLAPSED GROUPS (IF NCR>IGM, NO EFFECT)	4
NXINP	NUMBER OF COLLAPSED DOWNSCATTERING TERMS (IF 0, CALCULATED BY 1DX)	0
NTR	TYPE WEIGHTING FOR COLLAPSED SIGTR (0/1=NORMALIZED/RECIPROCAL)	1
NFGM	NUMBER OF COLLAPSED MATERIALS	5
IPUN	COLLAPSED CROSS SECTION OUTPUT (0/1/2/3=PRINT/PUNCH/TAPE+EOF/TAPE)	0
EV	FIRST EIGENVALUE GUESS	0.0000
EVM	EIGENVALUE MODIFIER	0.0000
S03	PARAMETRIC EIGENVALUE	0.0000
BUCK	BUCKLING (CM-2)	0.0000
LAL	LAMBDA LOWER	1.0000-03
LAH	LAMBDA UPPER	5.0000-01
EPS	EIGENVALUE CONVERGENCE CRITERION	1.0000-06
EPSA	PARAMETER CONVERGENCE CRITERION	1.0000-05
POD	PARAMETER OSCILLATION DAMPER	1.0000+00
ORF	OVER-RELAXATION FACTOR	1.3000+00
S01	NEGATIVE/POSITIVE=POWER (MWT)/NEUTRON SOURCE RATE	-4.0000+02

LAST 10035



MESH POINTS

R0	31									
.00000	.20000+01	.40000+01	.60000+01	.80000+01	.10000+02	.12000+02	.14000+02	.16000+02	.18000+02	
.20000+02	.22000+02	.24000+02	.26000+02	.28000+02	.30000+02	.32000+02	.34000+02	.36000+02	.38000+02	
.40000+02	.42000+02	.44000+02	.46000+02	.48000+02	.50000+02	.52000+02	.54000+02	.56000+02	.58000+02	
.60000+02										

ZONE NUMBERS BY MESH INTERVAL

M0	30									
1	1	1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1	1	1
2	2	2	2	2	2	2	2	2	2	2

MATERIAL NUMBERS BY ZONE

M2	2	
10	11	

FISSION FRACTIONS

K7	26									
.20000-01	.98000-01	.19000-00	.26800-00	.19600-00	.13500-00	.58000-01	.22000-01	.90000-02	.30000-02	
.10000-02	.00000	.00000	.00000	.00000	.00000	.00000	.00000	.00000	.00000	
.00000	.00000	.00000	.00000	.00000	.00000					

NEUTRON VELOCITY

V7	26									
.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01
.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01
.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01	.10000+01

MIXTURE SPECIFICATIONS (I0/I1/I2=MIX NUMBER/MAT. NUMBER FOR MIX/MATERIAL DENSITY)

I0	14									
10	10	10	10	10	10	10	11	11	11	11
11	12	12	12	12						
I1	14									
0	1	2	3	4	5	0	6	7	8	
9	0	6	7							
I2	14									
.00000	.20000-02	.80000-02	.19200-01	.12000-01	.11000-01	.00000	.15000-01	.30000-01	.20000-01	
.50000-02	.00000	.24400-01	.48800-01							

NUMBER OF GROUPS PER CRUNCHED GROUP

NPN	4			
4	4	4	6	12

NUMBERS OF MATERIALS TO BE CRUNCHED

NFP	5			
1	2	10	11	12

ZONE NUMBERS OF FLUXES USED FOR CRUNCH CALCULATION

NZN 5 1 1 2 2

ZONE NUMBERS FOR EACH CROSS SECTION MIX

ZN 2 2  
1 2

HETEROGENEITY CONSTANT FOR EACH CROSS SECTION MIX

HETC 2  
.10000+01 .10000+01

ISOTOPE NUMBER FOR EACH TERM IN XS MIX VECTOR

J1 11 0 1 2 3 4 5 0 2 3 4  
5

TEMPERATURES FOR EACH TERM IN XS MIX VECTOR

ATEM 11  
.90000+03 .90000+03 .90000+03 .90000+03 .90000+03 .90000+03 .80000+03 .80000+03 .80000+03 .80000+03  
.80000+03

FUEL OR MODERATOR DESIGNATION FOR EACH TERM IN XS MIX VECTOR (0/1=FUEL/MODERATOR)

MF 11 0 0 0 0 1 1 0 0 0 1  
1

LETHARGY WIDTHS

U7 26  
.48000-00 .48000-00 .48000-00 .57000-00 .57000-00 .69000-00 .69000-00 .69000-00 .77000-00 .77000-00  
.77000-00 .77000-00 .77000-00 .77000-00 .77000-00 .77000-00 .77000-00 .77000-00 .77000-00 .77000-00  
.77000-00 .77000-00 .77000-00 .77000-00 .77000-00 .10000+01

SEQUENCE NUMBERS ON TAPE FOR CROSS SECTION DATA IN THE RUSSIAN FORMAT

NUT 5 4 6 8 10

- 1 CHECK M3PU49 GROUP 1 PRESUMABLY SIGMA(N,2N) = 4.200000-01
- 2 CHECK M1U238 GROUP 1 PRESUMABLY SIGMA(N,2N) = 8.019999-01

CROSS SECTION DATA FOR THE FOLLOWING ISOTOPES WAS GIVEN IN THE RUSSIAN FORMAT

ISOTOPE NUMBER	ISOTOPE	NUMBER ON TAPE
1	M3PU49	1
2	M1U238	4
3	0	6
4	M2 FE	8
5	HA	10

MIXTURE SPECIFICATIONS FOR CALCULATING SHIELDED CROSS SECTIONS

MIXTURE NUMBER	MIX COMMAND	MATERIAL DENSITY	ISOTOPE NUMBER	TEMPERATURE (DEG K)	0/1=FUEL/MOD
10	0	.00000000	0	.90000000+03	0
10	1	.20000000-02	1	.90000000+03	0
10	2	.79999999-02	2	.90000000+03	0
10	3	.19200000-01	3	.90000000+03	0
10	4	.12000000-01	4	.90000000+03	1
10	5	.11000000-01	5	.90000000+03	1
11	0	.00000000	0	.80000000+03	0
11	6	.15000000-01	2	.80000000+03	0
11	7	.30000000-01	3	.80000000+03	0
11	8	.20000000-01	4	.80000000+03	1
11	9	.49999999-02	5	.80000000+03	1

	CROSS SECTION MIX NUMBER	ZONE FLUXES FOR XS MIX	HETC. CONST. FOR XS MIX
1	10	1	.10000000+01
2	11	2	.10000000+01

MIXTURE SPECIFICATIONS

MIXTURE NUMBER	MIX COMMAND	MATERIAL DENSITY
1	10	0
2	10	1
3	10	2
4	10	3
5	10	4
6	10	5
7	11	0
8	11	6
9	11	7
10	11	8
11	11	9
12	12	0
13	12	6
14	12	7

.00000000
.20000000-02
.79999999-02
.19200000-01
.12000000-01
.11000000-01
.00000000
.15000000-01
.30000000-01
.20000000-01
.49999999-02
.00000000
.24400000-01
.48800000-01

10X SAMPLE PROBLEM

TIME (MINUTES)	SIGE ITERATIONS	OUTER ITERATIONS	EIGENVALUE SLOPE	EIGENVALUE	LAMBDA
.16	0	0	.00000000	.00000000	.00000000
.18	0	1	.00000000	.98287991-00	.98287988-00
.18	0	2	.00000000	.10072591+01	.10248039+01
.19	0	3	.00000000	.10163560+01	.10090313+01
.20	0	4	.00000000	.10179630+01	.10015812+01
.21	0	5	.00000000	.10183523+01	.10003824+01
.22	0	6	.00000000	.10184363+01	.10000826+01
.23	0	7	.00000000	.10184537+01	.10000170+01
.24	0	8	.00000000	.10184575+01	.10000038+01
.24	0	9	.00000000	.10184584+01	.10000008+01
.27	1	10	.00000000	.10184584+01	.00000000
.28	1	11	.00000000	.10097408+01	.10097408+01
.29	1	12	.00000000	.10124162+01	.10026496+01
.30	1	13	.00000000	.10123253+01	.99991015-00
.31	1	14	.00000000	.10123755+01	.10000496+01
.32	1	15	.00000000	.10123785+01	.10000029+01
.33	1	16	.00000000	.10123799+01	.10000014+01
.33	1	17	.00000000	.10123802+01	.10000002+01
.36	2	18	.00000000	.10123802+01	.00000000
.37	2	19	.00000000	.10116640+01	.10116640+01
.38	2	20	.00000000	.10115890+01	.99992584-00
.39	2	21	.00000000	.10116042+01	.10000150+01
.39	2	22	.00000000	.10116036+01	.99999944-00
.42	3	23	.00000000	.10116036+01	.00000000
.43	3	24	.00000000	.10115381+01	.10115381+01
.44	3	25	.00000000	.10114366+01	.99989972-00
.45	3	26	.00000000	.10114443+01	.10000076+01
.45	3	27	.00000000	.10114427+01	.99999841-00
.46	3	28	.00000000	.10114427+01	.10000000+01
.49	4	29	.00000000	.10114427+01	.00000000
.50	4	30	.00000000	.10114083+01	.10114082+01
.51	4	31	.00000000	.10113741+01	.99996619-00
.51	4	32	.00000000	.10113764+01	.10000023+01
.52	4	33	.00000000	.10113758+01	.99999938-00
.55	5	34	.00000000	.10113758+01	.00000000
.56	5	35	.00000000	.10113455+01	.10113455+01
.56	5	36	.00000000	.10113393+01	.99999379-00
.57	5	37	.00000000	.10113397+01	.10000004+01

FINAL NEUTRON BALANCE TABLE

GROUP	FISS. SOURCE	FISSIONS	IN-SCATTER	OUT-SCATTER	ABSORPTION	LEFT LEAK.	RIGHT LEAK.	TOTAL LEAKAGE
1	6.718+17	1.141+17	-8.590+09	5.716+17	9.391+16	0.000	6.274+15	6.274+15
2	3.292+18	3.877+17	9.134+16	2.875+18	4.820+17	0.000	2.650+16	2.650+16
3	6.382+18	9.575+17	9.794+17	6.293+18	1.005+18	0.000	6.341+16	6.341+16
4	9.002+18	1.449+18	3.475+18	1.081+19	1.569+18	0.000	1.027+17	1.027+17
5	6.584+18	6.913+17	7.720+18	1.318+19	1.002+18	0.000	1.191+17	1.191+17
6	4.535+18	1.306+18	1.540+19	1.728+19	2.119+18	0.000	5.348+17	5.348+17
7	1.948+18	1.525+18	2.079+19	1.907+19	2.736+18	0.000	9.315+17	9.315+17
8	7.390+17	1.462+18	2.068+19	1.736+19	2.950+18	0.000	1.106+18	1.106+18
9	3.023+17	1.278+18	1.807+19	1.386+19	3.343+18	0.000	1.164+18	1.164+18
10	1.008+17	9.130+17	1.374+19	9.035+18	3.820+18	0.000	9.842+17	9.842+17
11	3.359+16	5.648+17	9.321+18	5.448+18	3.258+18	0.000	6.485+17	6.485+17
12	0.000	3.173+17	5.512+18	3.621+18	1.610+18	0.000	2.815+17	2.815+17
13	0.000	1.440+17	3.634+18	2.720+18	8.122+17	0.000	1.021+17	1.021+17
14	0.000	2.514+17	2.720+18	1.280+18	1.292+18	0.000	1.480+17	1.480+17
15	0.000	1.520+17	1.280+18	5.370+17	6.742+17	0.000	6.923+16	6.923+16
16	0.000	6.950+16	5.370+17	2.272+17	2.698+17	0.000	3.995+16	3.995+16
17	0.000	2.188+16	2.272+17	8.702+16	1.242+17	0.000	1.604+16	1.604+16
18	0.000	7.721+15	8.702+16	3.612+16	4.207+16	0.000	8.831+15	8.831+15
19	0.000	1.078+15	3.612+16	1.158+16	2.154+16	0.000	2.999+15	2.999+15
20	0.000	6.092+14	1.158+16	2.578+15	8.200+15	0.000	7.972+14	7.972+14
21	0.000	3.274+13	3.274+13	3.614+14	2.134+15	0.000	8.262+13	8.262+13
22	0.000	1.982+13	3.614+14	1.933+14	1.201+14	0.000	4.803+13	4.803+13
23	0.000	1.748+13	1.933+14	9.332+13	7.492+13	0.000	2.502+13	2.502+13
24	0.000	7.836+12	9.332+13	3.834+13	4.375+13	0.000	1.123+13	1.123+13
25	0.000	1.552+12	3.834+13	1.272+13	2.123+13	0.000	4.386+12	4.386+12
26	0.000	2.703+11	1.272+13	-8.033+05	1.195+13	0.000	7.668+11	7.668+11
27	3.359+19	1.161+19	1.243+20	1.243+20	2.723+19	0.000	6.356+18	6.356+18

ZONE		RADIUS (CM)	AVG RADIUS (CM)	AREA (CM2)	VOLUME (CM3)	TOTAL FLUX (N/CM2-SEC)	POWER (MWT/LITER)	FISSION SOURCE
1	1	-0.00000	1.00000+00	0.00000	3.35103+01	1.74422+16	2.45974+00	2.09526+14
2	1	2.00000+00	3.00000+00	5.02654+01	2.34572+02	1.73668+16	2.44919+00	2.08627+14
3	1	4.00000+00	5.00000+00	2.01062+02	6.36696+02	1.72167+16	2.42816+00	2.06835+14
4	1	6.00000+00	7.00000+00	4.52389+02	1.23988+03	1.69932+16	2.39685+00	2.04165+14
5	1	8.00000+00	9.00000+00	8.04247+02	2.04413+03	1.66980+16	2.35550+00	2.00641+14
6	1	1.00000+01	1.10000+01	1.25664+03	3.04944+03	1.63336+16	2.30446+00	1.96290+14
7	1	1.20000+01	1.30000+01	1.80956+03	4.25581+03	1.59032+16	2.24415+00	1.91149+14
8	1	1.40000+01	1.50000+01	2.46301+03	5.66324+03	1.54103+16	2.17508+00	1.85260+14
9	1	1.60000+01	1.70000+01	3.21699+03	7.27173+03	1.48591+16	2.09781+00	1.78671+14
10	1	1.80000+01	1.90000+01	4.07150+03	9.08129+03	1.42542+16	2.01299+00	1.71438+14
11	1	2.00000+01	2.10000+01	5.02654+03	1.10919+04	1.36009+16	1.92132+00	1.63619+14
12	1	2.20000+01	2.30000+01	6.08212+03	1.33036+04	1.29047+16	1.82357+00	1.55280+14
13	1	2.40000+01	2.50000+01	7.23822+03	1.57163+04	1.21718+16	1.72054+00	1.46489+14
14	1	2.60000+01	2.70000+01	8.49486+03	1.83301+04	1.14085+16	1.61312+00	1.37319+14
15	1	2.80000+01	2.90000+01	9.85203+03	2.11450+04	1.06218+16	1.50221+00	1.27849+14
16	1	3.00000+01	3.10000+01	1.13097+04	2.41609+04	9.81890+15	1.38879+00	1.18158+14
17	1	3.20000+01	3.30000+01	1.28680+04	2.73779+04	9.00742+15	1.27387+00	1.08334+14
18	1	3.40000+01	3.50000+01	1.45267+04	3.07960+04	8.19547+15	1.15857+00	9.84687+13
19	1	3.60000+01	3.70000+01	1.62860+04	3.44151+04	7.39169+15	1.04415+00	8.86670+13
20	1	3.80000+01	3.90000+01	1.81458+04	3.82352+04	6.60545+15	9.32118-01	7.90539+13
21	2	4.00000+01	4.10000+01	2.01062+04	4.22565+04	5.71045+15	1.16288-01	9.54698+12
22	2	4.20000+01	4.30000+01	2.21671+04	4.64788+04	4.78247+15	7.86473-02	6.45536+12
23	2	4.40000+01	4.50000+01	2.43285+04	5.09021+04	3.95988+15	5.32829-02	4.37264+12
24	2	4.60000+01	4.70000+01	2.65904+04	5.55266+04	3.23770+15	3.61502-02	2.96619+12
25	2	4.80000+01	4.90000+01	2.89529+04	6.03520+04	2.60730+15	2.45502-02	2.01414+12
26	2	5.00000+01	5.10000+01	3.14159+04	6.53736+04	2.05818+15	1.66757-02	1.36802+12
27	2	5.20000+01	5.30000+01	3.39794+04	7.06062+04	1.57913+15	1.13128-02	9.28116+11
28	2	5.40000+01	5.50000+01	3.66435+04	7.60349+04	1.15891+15	7.64313-03	6.27210+11
29	2	5.60000+01	5.70000+01	3.94081+04	8.16646+04	7.86697+14	5.11190-03	4.19787+11
30	2	5.80000+01	5.90000+01	4.22732+04	8.74954+04	4.52293+14	3.34048-03	2.74791+11
31		6.00000+01		4.52389+04				



10X SAMPLE PROBLEM

MATERIAL INVENTORY (KILOGRAMS) FOR EACH ZONE

MATERIAL	ATOMIC WT.	ZONE 1 2.681+02 LITERS	ZONE 2 6.367+02 LITERS
1 M3PU49	239.050	2.128+02	0.000
2 M1U238	238.050	8.476+02	3.775+03
3 O	16.000	1.307+02	5.074+02
4 M2 FE	55.850	2.983+02	1.181+03
5 NA	22.990	1.126+02	1.215+02

ZONE AVERAGED FLUXES

ZONE	FLUX (N/CM2-SEC)	VOLUME (LITERS)
1	1.00857+16	2.68082+02
2	2.26212+15	6.36096+02

COLLAPSED FISSION FRACTIONS AND ZONE AVERAGED FLUXES BY GROUP

GROUP	FISS. FRACT.	ZONE 1	ZONE 2
1	5.76000-01	1.07719+15	9.79983+13
2	4.11000-01	5.54458+15	1.00094+15
3	1.30000-02	3.40588+15	1.11941+15
4	0.00000	5.80837+13	4.37702+13

COLLAPSED CROSS SECTIONS

GROUP		SIGF	SIGA	NUSIGF	SIGTR	GXG	G-1XG	G-2XG	. . .
MAT 1	ZONE 1	.19824+01	.20060+01	.64166+01	.47172+01	.16430+01	.00000	.00000	.00000
MAT 2	ZONE 1	.54311-00	.56218-00	.15391+01	.48499+01	.17153+01	.00000	.00000	.00000
MAT 10	ZONE 1	.83097-02	.89441-02	.25146-01	.12166+00	.71173-01	.00000	.00000	.00000
MAT 11	ZONE 2	.81467-02	.90855-02	.23081-01	.16666-00	.93433-01	.00000	.00000	.00000
MAT 12	ZONE 2	.13252-01	.14480-01	.37545-01	.17977-00	.86639-01	.00000	.00000	.00000
GROUP		SIGF	SIGA	NUSIGF	SIGTR	GXG	G-1XG	G-2XG	. . .
MAT 1	ZONE 1	.16623+01	.18210+01	.48668+01	.75511+01	.56343+01	.10229+01	.00000	.00000
MAT 2	ZONE 1	.24943-02	.14847-00	.64354-02	.73530+01	.70309+01	.24990+01	.00000	.00000
MAT 10	ZONE 1	.33445-02	.49037-02	.97851-02	.21271-00	.20101-00	.40850-01	.00000	.00000
MAT 11	ZONE 2	.20165-04	.23834-02	.52024-04	.29508-00	.28025-00	.63009-01	.00000	.00000
MAT 12	ZONE 2	.32801-04	.36821-02	.84626-04	.36255-00	.34253-00	.76853-01	.00000	.00000
GROUP		SIGF	SIGA	NUSIGF	SIGTR	GXG	G-1XG	G-2XG	. . .
MAT 1	ZONE 1	.18994+01	.25344+01	.54582+01	.12894+02	.10356+02	.95812-01	.45270-01	.00000
MAT 2	ZONE 1	.00000	.49021-00	.00000	.12751+02	.12258+02	.17361-00	.73426-01	.00000
MAT 10	ZONE 1	.37987-02	.92252-02	.10916-01	.30036-00	.29054-00	.67968-02	.69665-03	.00000
MAT 11	ZONE 2	.00000	.80143-02	.00000	.41401-00	.40496-00	.12447-01	.11342-02	.00000
MAT 12	ZONE 2	.00000	.12405-01	.00000	.48102-00	.46751-00	.16337-01	.17952-02	.00000
GROUP		SIGF	SIGA	NUSIGF	SIGTR	GXG	G-1XG	G-2XG	. . .
MAT 1	ZONE 1	.81188+01	.13437+02	.23301+02	.23952+02	.10516+02	.29637-02	.00000	.00000
MAT 2	ZONE 1	.00000	.14359+01	.00000	.12756+02	.11321+02	.33372-02	.00000	.00000
MAT 10	ZONE 1	.16238-01	.38757-01	.46602-01	.37768-00	.33892-00	.59797-03	.00000	.00000
MAT 11	ZONE 2	.00000	.19338-01	.00000	.51202-00	.49269-00	.10305-02	.00000	.00000
MAT 12	ZONE 2	.00000	.30407-01	.00000	.46603-00	.43563-00	.11066-02	.00000	.00000

APPENDIX E  
SOURCE DECK LISTING

## E-1

```

-IL PDP INCL                                1DX 0001
ABC* FCOPY                                  1DX 0002
COMMON                                       1DX 0003
1      NCR1,  NFF,  NINP,  NOUT,  NSCRAT,  NSCR1,  NSCR2,  1DX 0004
      NSM,   NSR,   ALA,   BC7,   CNT,   CVT,   E01,  1DX 0005
2      EC2,   EC3,   EQ,    EVP,   EVPP,  FEF,   GBAR,  1DX 0006
3      GLH,  ICARD,  IGEP,  IGM3,  IGP,   IGW,   IHS,   1DX 0007
4      IHT,  IP,    IRED,  ITEMP, ITEMP1, ITEMP2, ITL,  1DX 0008
5      ITLP, KPAGE,  LAP,   LAPP,  LAR,   ME,   MMT,   1DX 0009
6      NB,   NCRUN,  NE,    NGOTO,  NP02,  NRED,  NSIGO,  1DX 0010
7      NXCMP, NXCMR,  P02,   PBAR,  P12,   R02,   SBAR,  1DX 0011
8      SK7,  TC6,   T7,    T11,  TEMP,  TEMP1,  TEMP2,  1DX 0012
9      TEMP3, TEMP4,  TI,    TSD,   V11,                   1DX 0013
COMMON   ID(11), MAXT,  A02,   I04,   S02,   IGM,   NXCM,  1DX 0014
1      ML,   M07,  NPRT,  NPUN,  NRCF,  NIFF,  MM01,  1DX 0015
2      IGE,  IM,   IZM,   MT,   M01,   B01,   B02,  1DX 0016
3      NCR,  NXINP, NTR,   NFGM,  IPUN,  EV,    EVM,   1DX 0017
4      SC3,  BUCK,  LAL,   LAH,   EPS,   EPSA,  PUB,   1DX 0018
5      ORF,  S01,                   1DX 0019
COMMON   LRC,   LMO,   LM2,   LGAM,  LK7,   LV7,   LI0,  1DX 0020
1      LI1,  LI2,  LR3,   LNO,   LATW,  LHOLN,  LNUT,  1DX 0021
2      LMPUP, LNT,  LNGB,  LNGE,  LNPF,  LFF,   LSF,   1DX 0022
3      LSR,  LSM,  LZN,   LHETC,  LJ1,  LATEM,  LMF,   1DX 0023
4      LU7,  LTEM,  LAG,   LNXS,  LFSS,  LSIGO,  LNP,   1DX 0024
5      LNFP, LNZN,  LNV,   LC2,   LALPH,  LALPS,  LPHJ,  1DX 0025
6      LPHI, LCO,  LA0,   LR1,   LR4,   LR5,   LV0,  1DX 0026
7      LN2,  LFO,  LF2,   LI3,  LF6,   LS2,   LCXS,  1DX 0027
8      LVOL, LMASS, LHA,   LPA,   LEB,   LAST,                   1DX 0028
INTEGER  A02,  B01,  B02,  B07,  CNT,   CVT,   P02,  1DX 0029
1      R02,  S02,  T06,  ZN,                   1DX 0030
REAL     I2,  I3,  K6,   K7,   LAH,   LAL,   LAP,  1DX 0031
1      LAPP, LAR,  MASS,  NC,   N2,                   1DX 0032
END

```

-ITC FOR CALC,CALC

E-2

C		1DX 0033
C		1DX 0034
C	***** DESCRIPTION OF SUBROUTINES *****	1DX 0035
C		1DX 0036
C	CALC MAIN PROGRAM.	1DX 0037
C		1DX 0038
C	INP CONTROLS READING AND PRINTING OF ALL INPUT DATA (EXCEPT	1DX 0039
C	CROSS SECTION DATA).	1DX 0040
C		1DX 0041
C	ERRO2 PRINTS ERROR MESSAGES.	1DX 0042
C		1DX 0043
C	CLEAR SETS AN ARRAY OF A GIVEN LENGTH EQUAL TO A SPECIFIED	1DX 0044
C	CONSTANT.	1DX 0045
C	REAG2 READS FLOATING POINT DATA IN GENERALIZED FORMAT.	1DX 0046
C		1DX 0047
C	REAI2 READS INTEGER DATA IN GENERALIZED FORMAT.	1DX 0048
C		1DX 0049
C	RCINP1 READS CROSS SECTION DATA IN THE RUSSIAN FORMAT FROM	1DX 0050
C	CARDS.	1DX 0051
C		1DX 0052
C	RCPUP READS CROSS SECTION DATA IN THE RUSSIAN FORMAT FROM TAPE.	1DX 0053
C		1DX 0054
C	RCPRT1 PRINTS CROSS SECTION DATA IN THE RUSSIAN FORMAT.	1DX 0055
C		1DX 0056
C	RCCHK CHECKS CROSS SECTION DATA IN THE RUSSIAN FORMAT FOR	1DX 0057
C	CONSISTENCY.	1DX 0058
C		1DX 0059
C	RCSTUP COMPUTES F-FACTORS FOR TEMPERATURE DEPENDENT ISOTOPES	1DX 0060
C	AND WRITES THE CROSS SECTION DATA ON TAPE IN THE PROPER	1DX 0061
C	ORDER.	1DX 0062
C		1DX 0063
C	RCCAL1 CALCULATES SIGO.	1DX 0064
C		1DX 0065
C	RCCAL2 CALCULATES F-FACTORS.	1DX 0066
C		1DX 0067
C	RCCSS CALCULATES RESONANCE SHIELDED CROSS SECTIONS.	1DX 0068
C		1DX 0069
C	RECS READS CROSS SECTIONS, PERFORMS ADJOINT REVERSALS IF	1DX 0070
C	REQUIRED, AND WRITES CROSS SECTION TAPE.	1DX 0071
C		1DX 0072
C	INIT PERFORMS ADJOINT REVERSALS ON V7 AND K7, MIXES AND PRINTS	1DX 0073
C	CROSS SECTIONS, MODIFIES GEOMETRY, AND CALCULATES AREAS,	1DX 0074
C	VOLUMES, EFFECTIVE FISSION SPECTRUM AND FISSION RATE.	1DX 0075
C		1DX 0076
C	FISCAL CALCULATES FISSION SUMS AND LAMBDA AND NORMALIZES FLUX	1DX 0077
C	AND FISSION SOURCE RATE.	1DX 0078
C		1DX 0079
C	MONPR MONITOR PRINT--PRINTS TIME, ITERATION NUMBER, EIGENVALUE	1DX 0080
C	SLOPE, EIGENVALUE, AND LAMBDA AFTER EACH OUTER ITERATION.	1DX 0081
C		1DX 0082
C	OUTER PERFORMS AN OUTER ITERATION--CALCULATES THE SOURCE INTO	1DX 0083
C	EACH GROUP AND THE NEW FISSION AND FISSION SOURCE RATE.	1DX 0084
C		1DX 0085
C	INNER1 CALCULATES COEFFICIENTS FOR THE FLUX EQUATION.	1DX 0086
C		1DX 0087
C	INNER CALCULATES THE FLUX FOR SPECIFIED GROUP.	1DX 0088
C		1DX 0089
C	CNNP PERFORMS CONVERGENCE TESTS AND CALCULATES NEW PARAMETERS	1DX 0090
C	FOR SEARCH OPTIONS.	1DX 0091

C	FINPR	FINAL PRINT--PRINTS RADII, AREAS, VOLUMES, FLUXES,	1DX 0092
C		POWER, AND FISSION SOURCE RATE.	1DX 0093
C			1DX 0094
C	NBAL	COMPUTES AND PRINTS BALANCE TABLES.	1DX 0095
C			1DX 0096
C	GRAM	CALCULATES AND PRINTS MATERIAL INVENTORIES FOR EACH	1DX 0097
C		ZONE.	1DX 0098
C			1DX 0099
C			1DX 0100
C	CRUNCH	CALCULATES, PRINTS, AND PUNCHES COLLAPSED CROSS SECTIONS.	1DX 0101
C			1DX 0102
C			1DX 0103
C		***** INTERNAL VARIABLES *****	1DX 0104
C			1DX 0105
C	NCR1	CROSS SECTION TAPE	1DX 0106
C	NFF	RESONANCE SHIELDING FACTOR TAPE	1DX 0107
C	NINP	INPUT TAPE	1DX 0108
C	NOUT	OUTPUT TAPE	1DX 0109
C	NSCRAT	SCRATCH TAPE	1DX 0110
C	NSCR1	SCRATCH TAPE	1DX 0111
C	NSCR2	SCRATCH TAPE	1DX 0112
C	NSM	INELASTIC SCATTERING MATRIX TAPE	1DX 0113
C	NSR	INFINITE DILUTION CROSS SECTION TAPE	1DX 0114
C	ALA	LAMBDA	1DX 0115
C	B07	USED FOR INTERNAL COMPUTATION IN FISCAL AND INIT	1DX 0116
C	CNT	CONVERGENCE TRIGGER FOR LAMBDA	1DX 0117
C	CVT	CONVERGENCE TRIGGER	1DX 0118
C	E01	TEMPORARY	1DX 0119
C	E02	TEMPORARY	1DX 0120
C	E03	TEMPORARY	1DX 0121
C	EQ	EIGENVALUE SLOPE	1DX 0122
C	EVP	PREVIOUS EIGENVALUE	1DX 0123
C	EVPP	EIGENVALUE FOR TWO ITERATIONS BACK	1DX 0124
C	FEF	ENERGY RELEASED PER FISSION (=215 MEV)	1DX 0125
C	GBAR	GROUP INDICATOR FOR TAPE MOTION IN OUTER	1DX 0126
C	GLH	MAXIMUM TIME IN SECONDS	1DX 0127
C	ICARD	NEGATIVE/POSITIVE=XS DATA FROM TAPE/CARDS	1DX 0128
C	IGEP	IGE + 1	1DX 0129
C	IGM3	IGM*3	1DX 0130
C	IGP	IGM + 1	1DX 0131
C	IGV	GROUP INDICATOR FOR INNER AND OUTER	1DX 0132
C	IHS	POSITION OF SIGMA SELF SCATTER	1DX 0133
C	IHT	POSITION OF SIGMA TRANSPORT	1DX 0134
C	IP	IM + 1	1DX 0135
C	IRED	FIRST PASS THROUGH RCCAL1 (1/2=YES/NO)	1DX 0136
C	ITEMP	TEMPORARY	1DX 0137
C	ITEMP1	TEMPORARY	1DX 0138
C	ITEMP2	TEMPORARY	1DX 0139
C	ITL	CROSS SECTION TABLE LENGTH	1DX 0140
C	ITLP	CRUNCHED CROSS SECTION TABLE LENGTH	1DX 0141
C	KPAGE	PAGE COUNTER FOR MONITOR PRINT	1DX 0142
C	LAP	LAMBDA FOR PREVIOUS EIGENVALUE	1DX 0143
C	LAPP	LAMBDA FOR TWO ITERATIONS BACK	1DX 0144
C	LAR	LAMBDA FOR PREVIOUS ITERATION	1DX 0145
C	ME	CROSS SECTION DATA INCONSISTENCY COUNT	1DX 0146
C	MMT	MM01 - NRCF	1DX 0147
C	NR	TEMPORARY	1DX 0148
C	NCRUN	CRUNCH CALCULATION (0/1=NO/YES)	1DX 0149
C	NF	TEMPORARY	1DX 0150

## E-4

C	NGOTO	FLAG SET EQUAL TO ONE IF PROBLEM IS FINISHED	1DX 0151
C	NP02	TOTAL ITERATION COUNT	1DX 0152
C	NRED	PERFORM ANOTHER ITERATION ON SIGE (0/1=NO/YES)	1DX 0153
C	NSIGO	SIGO ITERATION COUNT	1DX 0154
C	NXCMP	NUMBER OF DOWNSCATTERING TERMS FOR CRUNCHED GROUPS	1DX 0155
C	NXCMR	NXCM + 1	1DX 0156
C	P02	OUTER ITERATION COUNT	1DX 0157
C	PBAR	TEMPORARY	1DX 0158
C	PI2	6.28318	1DX 0159
C	R02	SIGE ITERATION COUNT	1DX 0160
C	SBAR	TEMPORARY	1DX 0161
C	SK7	SUM OF K7 OVER ALL GROUPS	1DX 0162
C	T06	0/1=NOT DELTA/DELTA CALCULATION	1DX 0163
C	T7	ALPHA/VELOCITY	1DX 0164
C	T11	PREVIOUS FISSION TOTAL	1DX 0165
C	TEMP	TEMPORARY	1DX 0166
C	TEMP1	TEMPORARY	1DX 0167
C	TEMP2	TEMPORARY	1DX 0168
C	TFMP3	TEMPORARY	1DX 0169
C	TEMP4	TEMPORARY	1DX 0170
C	TI	TIME	1DX 0171
C	TSD	(MW-SEC)/(FISSIONS)	1DX 0172
C	V11	TOTAL SOURCE FOR THE GROUP	1DX 0173
C			1DX 0174
C	*****	INPUT VARIABLES (CARDS 1-5) *****	1DX 0175
C			1DX 0176
C	ID(11)	IDENTIFICATION CARD	1DX 0177
C	MAXT	MAXIMUM TIME IN MINUTES	1DX 0178
C	A02	0/1=FLUX CALCULATION/ADJOINT CALCULATION	1DX 0179
C	I04	EIGENVALUE TYPE (1/2/3/4/5=KEFF/ALPHA/CONCENTRATION/ DELTA/BUCKLING)	1DX 0180
C	S02	PARAMETRIC EIGENVALUE TYPE (0/1/2=NONE/KEFF/ALPHA)	1DX 0182
C	IGM	NUMBER OF GROUPS	1DX 0183
C	NXCM	NUMBER OF DOWNSCATTERING TERMS	1DX 0184
C	ML	NEG/POS=NUMBER OF MATERIALS FROM TAPE/CARDS	1DX 0185
C	M07	FLUX GUESS (0/1=NONE/CARDS)	1DX 0186
C	NPRT	PRINT OPTION (0/1/2=MINI/MIDI/MAXI)	1DX 0187
C	NPUN	FLUX DUMP (0/1=NO/YES)	1DX 0188
C	NRCF	NO. MIXES USED IN GENERATING SHIELDED XS (IF 0, DATA IS IN DTF FORMAT)	1DX 0189
C	NIFF	NUMBER OF ITERATIONS IN CALCULATION OF SIGE	1DX 0191
C	MM01	NO. MIX SPECIF. FOR GENERATING SHIELDED XS	1DX 0192
C	IGE	GEOMETRY (0/1/2=PLANE/CYLINDER/SPHERE)	1DX 0193
C	IM	NUMBER OF SPACE INTERVALS	1DX 0194
C	I2M	NUMBER OF MATERIAL ZONES	1DX 0195
C	MT	TOTAL NUMBER OF MATERIALS INCLUDING MIXES	1DX 0196
C	M01	NUMBER OF MIXTURE SPECIFICATIONS	1DX 0197
C	B01	LEFT BOUNDARY CONDITION (0/1/2=VACUUM/REFL/PER)	1DX 0198
C	B02	RIGHT BOUNDARY CONDITION (0/1/2=VACUUM/REFL/PER)	1DX 0199
C	NCR	NUMBER OF CRUNCHED GROUPS	1DX 0200
C	NXINP	NO. COLLAPS. DOWNSCAT. TERMS (IF 0, CALC. BY 1UX)	1DX 0201
C	NTR	TYPE WEIGHTING FOR SIGMA TRANSPORT (0/1=NW/RW)	1DX 0202
C	NFGM	NO. COLLAPSED MATERIALS	1DX 0203
C	IPUN	COLLAPS. XS OUTPUT (0/1/2/3=PRINT/PUNCH/TAPE+EOF/ TAPE)	1DX 0204
C			1DX 0205
C	EV	FIRST EIGENVALUE GUESS	1DX 0206
C	EVM	EIGENVALUE MODIFIER	1DX 0207
C	S03	PARAMETRIC EIGENVALUE	1DX 0208
C	BUCK	BUCKLING	1DX 0209

C	LAL	LAMBDA LOWER	1DX	0210
C	LAH	LAMBDA UPPER	1DX	0211
C	EPS	EIGENVALUE CONVERGENCE CRITERION	1DX	0212
C	EPSA	PARAMETRIC EIGENVALUE CONVERGENCE CRITERION	1DX	0213
C	POD	PARAMETER OSCILLATION DAMPER	1DX	0214
C	ORF	FISSION SOURCE OVER-RELAXATION FACTOR	1DX	0215
C	S01	NEG/POS=POWER (MWT)/NEUTRON SOURCE RATE	1DX	0216
C			1DX	0217
C	*****	SUBSCRIPTED VARIABLES *****	1DX	0218
C			1DX	0219
C	R0(IP)	INITIAL RADII	1DX	0220
C	M0(IM)	ZONE NUMBERS	1DX	0221
C	M2(IZM)	MATERIAL NUMBERS BY ZONE	1DX	0222
C	K7(IGM)	FISSION SPECTRUM (INPUT)	1DX	0223
C	V7(IGM)	NEUTRON VELOCITIES	1DX	0224
C	I0(M01)	MIX NUMBER	1DX	0225
C	I1(M01)	MATERIAL NUMBER FOR MIX	1DX	0226
C	I2(M01)	MATERIAL DENSITY	1DX	0227
C	R3(IZM)	RADIAL ZONE NUMBERS (DELTA CALCULATION ONLY)	1DX	0228
C	N0(IM,IGM)	FLUX (OLD)	1DX	0229
C	ATW(ML)	MATERIAL ATOMIC WEIGHT	1DX	0230
C	HOLN(ML)	MATERIAL NAME	1DX	0231
C	NUT(ML)	SEQUENCE NUMBER ON TAPE FOR DATA IN RUSSIAN FORMAT	1DX	0232
C	MPUP(ML)	ISOTOPE NOS. OF MATERIALS IN ORDER READ FROM TAPE	1DX	0233
C	NT(ML)	ISOTOPE IS TEMPERATURE DEPENDENT (0/1=NO/YES)	1DX	0234
C	NGB(ML)	GROUP NUMBER F-FACTORS BEGIN AT	1DX	0235
C	NGE(ML)	GROUP NUMBER F-FACTORS END AT	1DX	0236
C	NPF(4,ML)	NO. OF SIGO COLUMNS FOR 1/2/3/4=FISSION/CAPTURE/ TOTAL/ELASTIC F-FACTORS	1DX	0237
C	FF(4,IGM3,6)	INPUT SELF-SHIELDING FACTORS	1DX	0239
C	SF(4,ML,6)	DISCRETE SIGO VALUES FOR INPUT F-FACTORS	1DX	0240
C	SR(9,IGM)	INFINITE DILUTION CROSS SECTIONS	1DX	0241
C	SM(IGM,NXC MR)	INELASTIC SCATTERING MATRIX	1DX	0242
C	ZN(NRCF)	ZONE NOS. OF FLUXES TO BE USED IN SIGE CALCULATION	1DX	0243
C	HETC(NRCF)	HETEROGENEITY CONSTANTS	1DX	0244
C	J1(MM01)	ISOTOPE NUMBER IN XS MIX VECTOR	1DX	0245
C	ATEM(MM01)	ISOTOPE TEMPERATURE IN XS MIX VECTOR	1DX	0246
C	MF(MM01)	ISOTOPE IS 0/1=FUEL/MODERATOR	1DX	0247
C	U7(IGM)	LETHARGY WIDTHS	1DX	0248
C	TEM(3,ML)	DISCRETE TEMPERATURE VALUES FOR INPUT F-FACTORS	1DX	0249
C	AG(NRCF,IGM)	USED IN HETEROGENEITY CALCULATION	1DX	0250
C	NXS(NRCF)	NO. OF MIX SPECIFICATIONS IN EACH XS MIX	1DX	0251
C	FSS(4,MMT,IGM)	CALCULATED SELF-SHIELDING FACTORS	1DX	0252
C	SIGO(MMT,IGM)	CALCULATED SIGO VALUES	1DX	0253
C	NPN(NCR)	NUMBER OF GROUPS PER CRUNCHED GROUP	1DX	0254
C	NFP(NFGM)	MATERIAL NUMBERS TO BE CRUNCHED	1DX	0255
C	NZN(NFGM)	ZONE NUMBERS OF FLUXES USED FOR CRUNCH CALCULATION	1DX	0256
C	NV(IGM)	COLLAPSED GROUP NO. FOR EACH FINE GROUP	1DX	0257
C	C2(IZM,IGM)	ZONE MACROSCOPIC TRANSPORT CROSS SECTIONS	1DX	0258
C	ALPH(IZM,IGM)	FINE-GROUP TRANSPORT CROSS SECTION WEIGHTING COEFF.	1DX	0259
C	ALPS(IZM,NCR)	ALPH(IZM,IGM) SUMMED OVER A COLLAPSED GROUP	1DX	0260
C	PHJ(IZM,NCR)	ZONE FLUXES FOR EACH COLLAPSED GROUP	1DX	0261
C	PHI(IZM,IGP)	ZONE FLUXES FOR EACH FINE GROUP (AND TOTAL)	1DX	0262
C	C0(IITL,MT)	CROSS SECTION ARRAY FOR CURRENT GROUP	1DX	0263
C	A0(IP)	AREA ELEMENTS	1DX	0264
C	R1(IP)	CURRENT RADII	1DX	0265
C	R4(IM)	AVERAGE RADII	1DX	0266
C	R5(IM)	DELTA-R	1DX	0267
C	V0(IM)	VOLUME ELEMENTS	1DX	0268



```

C      N2(IM,IGM)   FLUX (NEW)                               1DX 0269
C      F0(IM)      FISSON SOURCE (OLD)                     1DX 0270
C      F2(IM)      FISSON SOURCE (NEW)                     1DX 0271
C      I3(M01)     MATERIAL DENSITIES USED IN GRAM         1DX 0272
C      K6(IGM)     FISSON SPECTRUM (EFFECTIVE)             1DX 0273
C      S2(IM)      SOURCE                                   1DX 0274
C      CXS(IP,IGM,2)  CONSTANTS INVOLVING CROSS SECTIONS FOR FLUX CALC. 1DX 0275
C      VOL(IZM)    ZONE VOLUME (LITERS)                    1DX 0276
C      MASS(ML,IZM) MATERIAL INVENTORY IN EACH ZONE       1DX 0277
C      HA(IM)      TEMP STORAGE FOR FLUX CALCULATION      1DX 0278
C      PA(IM)      TEMP STORAGE FOR FLUX CALCULATION      1DX 0279
C      EB(8,IGP)   1/2/3/4/5/6/7/8=FISSION SOURCE/IN-SCATTER/ 1DX 0280
C                OUT-SCATTER/ABSORPTIONS/LEFT LEAK./RIGHT LEAK./ 1DX 0281
C                TOTAL LEAK./FISSION RATE                 1DX 0282
C      GAM(IZM)    BUCKLING MODIFIERS FOR EACH ZONE       1DX 0283
C
C      INCLUDE ABC                                         1DX 0285
C      COMMON A(35000)                                     1DX 0286
1      CALL ETIME                                          1DX 0287
C      READ INPUT DATA                                    1DX 0288
C      CALL INP                                           1DX 0289
C      IF(NRCF)      20,20,10                              1DX 0290
C      CROSS SECTION DATA IS IN RUSSIAN FORMAT           1DX 0291
10     CALL RCINP1(A(LHOLN),A(LATW),A(LNT),A(LNGB),A(LNGE),A(LNPPF),
1         A(LFF),IGM3,ML,A(LSF),A(LSR),A(LSM),IGM,A(LU7), 1DX 0293
2         A(LTEM),A(LATEM),A(LIC),A(LI1),A(LI2),A(LJ1), 1DX 0294
3         A(LMF),A(LZN),A(LHETC),A(LNUT),A(LMPUP),A(LNXS)) 1DX 0295
C      CALL RCSTUP(A(LNPPF),A(LNT),A(LFF),A(LTEM),A(LATEM),IGM3,A(LNGB),
1         A(LNGE),A(LJ1),A(LSR),A(LSM),IGM,A(LMPUP))      1DX 0297
C      DO 15 NSIGO = 1,5                                   1DX 0298
C      CALL RCCAL1(A(LAG),A(LNXS),A(LI1),A(LSR),A(LMF),A(LI2),A(LFSS),
1         A(LSIGO),A(LHETC),NRCF,MMT)                       1DX 0300
15     CALL RCCAL2(A(LNXS), A(LI1), A(LJ1), A(LNGB), A(LNGE), A(LNPPF),
1         A(LFF), A(LSIGO),A(LSF), A(LFSS),IGM3,MMT,ML,A(LHOLN),
2         A(LI2))                                           1DX 0303
16     R02 = R02 + 1                                       1DX 0304
C      IF(R02 - NIFF) 18,17,17                             1DX 0305
17     NRED = 0                                             1DX 0306
18     CALL RCCSS(A(LZN),A(LNXS),A(LI1),A(LSR),A(LSM),A(LN2),A(LFSS),
1         A(LU7),A(LPHI),IGM,ITL,MMT,IZM)                   1DX 0308
C      READ DATA IN DTF FORMAT, PERFORM ADJ REVERSALS, AND MAKE XS TAPE 1DX 0309
20     CALL RECS (A(LN2),ITL,IGM,MT,A(LATW),A(LHOLN))      1DX 0310
C      DO 25 I=LN2, LAST                                   1DX 0311
25     A(I) = 0.0                                           1DX 0312
102    CALL INIT(A(LK6), A(LK7), A(LI0), A(LI1), A(LI2), A(LM0), A(LM2),
1         A(LNO), A(LR0), A(LR1), A(LR3), A(LR4), A(LR5), A(LA0),
2         A(LF0), A(LC0), A(LV0),ITL,IM, A(LV7),MT,A(LER),
3         A(LGAM))                                           1DX 0316
C      PERFORM FISSION CALCULATION                         1DX 0317
C      CALL FISCAL (A(LNO),A(LF0), A(LV0), A(LC0), A(LK6),
2         A(LM0), A(LM2),ITL,MT, IM, A(LER))               1DX 0319
C      CALL MONITOR PRINT                                  1DX 0320
101    CALL MONPR                                          1DX 0321
C      GO TO (100, 107, 107, 107), NGOTO                 1DX 0322
C      PERFORM AN OUTER ITERATION                         1DX 0323
107    CALL OUTER( A(LA0), A(LC0), A(LF0), A(LK6), A(LM0), A(LM2),
1         A(LNO), A(LN2), A(LS2), A(LV0), A(LV7), A(LF2), ITL,
2         MT, A(LCX), IM, A(LR5), A(LR4), A(LHA), A(LPA),
3         A(LER), IP, IGM)                                   1DX 0327

```

C	PERFORM FISSION CALCULATION	1DX 0328
	CALL FISCAL (A(LNU),A(LFC),A(LVU),A(LCU),A(LK6),	1DX 0329
2	A(LM0),A(LM2),ITL,MT,IM,A(LEB))	1DX 0330
C	PERFORM CONVERGENCE AND NEW PARAMETER CALCULATIONS	1DX 0331
	CALL CNNP(A(LF2),A(LK6),A(LEB))	1DX 0332
	GO TO (100,101,102),NGOTO	1DX 0333
C	100/101/102=FINAL PRINT/MONITOR PRINT/SEARCH CALCULATION	1DX 0334
C	FINAL PRINT	1DX 0335
100	CALL FINPR(A(LF2),A(LPA),A(LCU),A(LM0),A(LM2),A(LN2),A(LR1),	1DX 0336
1	A(LR4),A(LA0),A(LVU),A(LFU),ITL,IM,A(LEB),A(LCXS),	1DX 0337
2	IP,IGM,A(LPHI),IZM,A(LC2),IGM,A(LGAM))	1DX 0338
	IF(NRED) 150,150,16	1DX 0339
C	CALCULATE MATERIAL INVENTORY	1DX 0340
150	CALL GRAM(A(LMASS),A(LVOL),A(LATW),A(LHOLN),IM,A(LM0),A(LM2),	1DX 0341
1	A(LVU),A(LIU),A(LI1),A(LI2),ML,A(LI3),A(LJ1))	1DX 0342
	IF(NCRUN) 240,240,210	1DX 0343
C	COLLAPSE CROSS SECTIONS	1DX 0344
210	CALL CRUNCH(A(LPHI),A(LN2),A(LVU),A(LV7),A(LPHJ),A(LNPN),A(LK7),	1DX 0345
1	A(LN2),A(LCU),A(LNFP),A(LNZN),A(LM0),A(LHOLN),	1DX 0346
2	A(LATW),IZM,ITLP,A(LNV),NCR,ITL,IM,A(LVOL),A(LC2),	1DX 0347
3	IGM,A(LK6),A(LALPH),A(LALPS))	1DX 0348
240	GO TO 1	1DX 0349
	END	1DX 0350

```

-ITC FOR INP,INP                                1DX 0351
  SUBROUTINE INP                                1DX 0352
  INCLUDE ABC                                  1DX 0353
  COMMON A(35000)                              1DX 0354
C THIS SUBROUTINE CONTROLS THE READING OF INPUT DATA EXCEPT XS DATA 1DX 0355
C SET UP DRUM UNITS                            1DX 0356
  DIMENSION JLPTAB(49)                         1DX 0357
  CALL SETDR(3,150000,200000,JLPTAB)          1DX 0358
  CALL SETDR(4,350000,200000,JLPTAB(8))      1DX 0359
  CALL SETDR(8,550000,100000,JLPTAB(15))     1DX 0360
  CALL SETDR(9,650000,100000,JLPTAB(22))     1DX 0361
  CALL SETDR(10,750000,350000,JLPTAB(29))    1DX 0362
  CALL SETDR(11,1100000,100000,JLPTAB(36))   1DX 0363
  CALL SETDR(12,1200000,100000,JLPTAB(43))   1DX 0364
  NCR1 = 3                                      1DX 0365
  NSCRAT = 4                                    1DX 0366
  NINP = 5                                      1DX 0367
  NOUT = 6                                      1DX 0368
  NFF = 8                                       1DX 0369
  NSR = 9                                       1DX 0370
  NSM = 10                                      1DX 0371
  NSCR1 = 11                                    1DX 0372
  NSCR2 = 12                                    1DX 0373
  REWIND NCR1                                    1DX 0374
  REWIND NSCRAT                                  1DX 0375
  REWIND NFF                                    1DX 0376
  REWIND NSR                                    1DX 0377
  REWIND NSM                                    1DX 0378
  REWIND NSCR1                                  1DX 0379
  REWIND NSCR2                                  1DX 0380
  WRITE(NOUT,10)                                1DX 0381
10  FORMAT(1H1,41X,35H * * * * 1 D X * * * * //) 1DX 0382
  READ(NINP,20) (ID(I),I=1,11), MAXT          1DX 0383
20  FORMAT(11A6,I6)                              1DX 0384
  WRITE(NOUT,30) (ID(I),I=1,11), MAXT         1DX 0385
30  FORMAT(10X,1H ,11A6,I6/)                   1DX 0386
  READ(NINP,40) AU2, I04, S02, IGM, NXCM, ML, MO7, NPRT, NPUN, NRCF, 1DX 0387
  1 NIFF, MMU1, IGE, IM, IZM, MT, MO1, BU1, BU2, NCR, NXINP, NTR, 1DX 0388
  2 NFGM, IPUN                                  1DX 0389
40  FORMAT(12I6)                                1DX 0390
  READ(NINP,50) EV, EVM, S03, BUCK, LAL, LAH, EPS, EPSA, POD, ORF, 1DX 0391
  1 S01                                          1DX 0392
50  FORMAT(6E12.6)                              1DX 0393
  WRITE(NOUT,60) AU2, I04, S02, IGM, NXCM, ML  1DX 0394
60  FORMAT(                                     1DX 0395
192H  AU2          0/1=REGULAR CALCULATION/ADJOINT CALCULATION 1DX 0396
2          I9/                                     1DX 0397
392H  I04          EIGENVALUE TYPE (1/2/3/4/5=KEFF/ALPHA/CONCENTRATION 1DX 0398
4N/DELTA/BUCKLING) I9/                             1DX 0399
592H  S02          PARAMETRIC EIGENVALUE TYPE (0/1/2=NONE/KEFF/ALPHA) 1DX 0400
6          I9/                                     1DX 0401
992H  IGM          NUMBER OF ENERGY GROUPS        1DX 0402
1          I9/                                     1DX 0403
292H  NXCM         NUMBER OF DOWNSCATTERING TERMS  1DX 0404
3          I9/                                     1DX 0405
192H  ML           NEGATIVE/POSITIVE=NUMBER OF MATERIALS FROM TAPE/CAR 1DX 0406
2DS          I9)                                  1DX 0407
  WRITE(NOUT,70) MU7, NPRT, NPUN, NRCF, NIFF, MMU1 1DX 0408
70  FORMAT(                                     1DX 0409

```

	392H	M07	FLUX GUESS (0/1=NONE/CARDS)	1DX	0410
	4		19/	1DX	0411
	592H	NPRT	PRINT OPTION (0/1/2=MINI/MIDI/MAXI)	1DX	0412
	6		19/	1DX	0413
	792H	NPUN	FLUX DUMP (0/1=NO/YES)	1DX	0414
	8		19/	1DX	0415
	792H	NRCF	NO. OF MIXES USED IN GENERATING XS DATA (IF 0, DATA	1DX	0416
	8A IS	IN DTF FORMAT)	19/	1DX	0417
	992H	NIFF	NUMBER OF ITERATIONS IN CALCULATION OF SIGE	1DX	0418
	1		19/	1DX	0419
	292H	MM01	NUMBER OF MIX SPECIF. FOR GENERATING SHIELDED XS	1DX	0420
	3		19/)	1DX	0421
		WRITE(NOUT,80)	IGE, IM, IZM, MT, M01, B01	1DX	0422
80		FORMAT(		1DX	0423
	192H	IGE	GEOMETRY (0/1/2=PLANE/CYLINDER/SPHERE)	1DX	0424
	2		19/	1DX	0425
	392H	IM	NUMBER OF SPACE INTERVALS	1DX	0426
	4		19/	1DX	0427
	592H	IZM	NUMBER OF MATERIAL ZONES	1DX	0428
	6		19/	1DX	0429
	792H	MT	TOTAL NUMBER OF MATERIALS INCLUDING MIXES	1DX	0430
	8		19/	1DX	0431
	992H	M01	NUMBER OF MIXTURE SPECIFICATIONS	1DX	0432
	1		19/	1DX	0433
	292H	B01	LEFT BOUNDARY CONDITION (0/1/2=VACUUM/REFLECTIVE/	1DX	0434
	3PERIODIC)		19/)	1DX	0435
		WRITE(NOUT,90)	B02, NCR, NXINP, NTR, NFGM, IPUN	1DX	0436
90		FORMAT(		1DX	0437
	192H	B02	RIGHT BOUNDARY CONDITION (0/1/2=VACUUM/REFLECTIVE/	1DX	0438
	2PERIODIC)		19/	1DX	0439
	392H	NCR	NUMBER OF COLLAPSED GROUPS (IF NCR=IGM, NO EFFECT)	1DX	0440
	4		19/	1DX	0441
	592H	NXINP	NUMBER OF COLLAPSED DOWNSCATTERING TERMS (IF 0, CALD	1DX	0442
	6LCULATED BY	1DX)	19/	1DX	0443
	592H	NTR	TYPE WEIGHTING FOR COLLAPSED SIGTR (0/1=NORMALIZED	1DX	0444
	6/RECIPROCAL)		19/	1DX	0445
	792H	NFGM	NUMBER OF COLLAPSED MATERIALS	1DX	0446
	8		19/	1DX	0447
	992H	IPUN	COLLAPSED CROSS SECTION OUTPUT (0/1/2/3=PRINT/PUNCI	1DX	0448
	1H/TAPE+EOF/TAPE)		19/)	1DX	0449
		WRITE(NOUT,100)	EV, EVM, S03, BUCK, LAL, LAH	1DX	0450
100		FORMAT(		1DX	0451
	191H	EV	FIRST EIGENVALUE GUESS	1DX	0452
	2		1PE10.4/	1DX	0453
	391H	EVM	EIGENVALUE MODIFIER	1DX	0454
	4		1PE10.4/	1DX	0455
	591H	S03	PARAMETRIC EIGENVALUE	1DX	0456
	6		1PE10.4/	1DX	0457
	791H	BUCK	BUCKLING (CM-2)	1DX	0458
	8		1PE10.4/	1DX	0459
	991H	LAL	LAMBDA LOWER	1DX	0460
	1		1PE10.4/	1DX	0461
	291H	LAH	LAMBDA UPPER	1DX	0462
	3		1PE10.4/)	1DX	0463
		WRITE(NOUT,120)	EPS, EPSA, POD, ORF, S01	1DX	0464
120		FORMAT(		1DX	0465
	191H	EPS	EIGENVALUE CONVERGENCE CRITERION	1DX	0466
	2		1PE10.4/	1DX	0467
	391H	EPSA	PARAMETER CONVERGENCE CRITERION	1DX	0468

## E-10

4		1PE10.4/	1DX 0469
591H	POD	PARAMETER OSCILLATION DAMPER	1DX 0470
6		1PE10.4/	1DX 0471
791H	ORF	OVER-RELAXATION FACTOR	1DX 0472
8		1PE10.4/	1DX 0473
991H	S01	NEGATIVE/POSITIVE=POWER (MWT)/NEUTRON SOURCE RATE	1DX 0474
1		1PE10.4/	1DX 0475
291H			1DX 0476
3		1PE10.4/)	1DX 0477
	IF(A02)	230,230,210	1DX 0478
210	IF(NIFF)	230,230,220	1DX 0479
220	CALL ERRO2(6H	INP,220,1)	1DX 0480
230	IF(S02)	240, 260, 240	1DX 0481
240	IF(S03)	260, 250, 260	1DX 0482
250	CALL ERRO2(6H	INP,250,1)	1DX 0483
260	FEF =	215.0	1DX 0484
	TSD =	FEF*1.602*10.**(-19)	1DX 0485
	GLH =	MAXT*60	1DX 0486
	IGMIM =	IGM*IM	1DX 0487
	KPAGE =	100	1DX 0488
	MF =	0	1DX 0489
	IHS =	5	1DX 0490
	ITL =	NXCM + 5	1DX 0491
	IHT =	4	1DX 0492
	IP =	IM + 1	1DX 0493
	IGP =	IGM + 1	1DX 0494
	IGEP =	IGE + 1	1DX 0495
	EQ =	.0	1DX 0496
	LAP =	.0	1DX 0497
	LAPP =	.0	1DX 0498
	LAR =	.0	1DX 0499
	ALA =	.0	1DX 0500
	P02 =	0	1DX 0501
	NP02 =	0	1DX 0502
	CVT =	0	1DX 0503
	CNT =	0	1DX 0504
	NXCMP =	NXCM	1DX 0505
	IF(NXCM -	NCR + 1) 275,275,270	1DX 0506
270	NXCMP =	NCR - 1	1DX 0507
275	ITLP =	NXCMP + 5	1DX 0508
	NCRUN =	0	1DX 0509
	IF(NCR-IGM)	280,280,290	1DX 0510
280	NCRUN =	1	1DX 0511
290	T06 =	0	1DX 0512
	IF(I04-4)	310, 300, 310	1DX 0513
300	T06 =	1	1DX 0514
310	R02 =	0	1DX 0515
	MMT =	MM01 - NRCF	1DX 0516
	ICARD =	ML	1DX 0517
	ML =	IABS(ML)	1DX 0518
	IRED =	1	1DX 0519
	NRED =	0	1DX 0520
	IF(NRCF)	316, 316, 314	1DX 0521
314	NRED =	1	1DX 0522
	R02 =	-1	1DX 0523
316	IGM3 =	IGM*3	1DX 0524
	NXCMR =	NXCM + 1	1DX 0525
C	COMPUTE	DIMENSION POINTERS	1DX 0526
	LRO =	1	1DX 0527

LM0 = LR0 + IP	1DX 0528
LM2 = LM0 + IM	1DX 0529
LGAM = LM2 + IZM	1DX 0530
LK7 = LGAM + IZM	1DX 0531
LV7 = LK7 + IGM	1DX 0532
LIO = LV7 + IGM	1DX 0533
LI1 = LIO + M01	1DX 0534
LI2 = LI1 + M01	1DX 0535
LR3 = LI2 + M01	1DX 0536
LNO = LR3 + IZM*TO6	1DX 0537
LATW = LNO + IM*IGM	1DX 0538
LHOLN = LATW + ML	1DX 0539
LNUT = LHOLN + ML	1DX 0540
LMPUP = LNUT + ML	1DX 0541
LNT = LMPUP + ML	1DX 0542
LNGB = LNT + ML*NRED	1DX 0543
LNGE = LNGB + ML*NRED	1DX 0544
LNPFF = LNGE + ML*NRED	1DX 0545
LFF = LNPFF + 4*ML*NRED	1DX 0546
LSF = LFF + 4*IGM3*6*NRED	1DX 0547
LSR = LSF + 4*ML*6*NRED	1DX 0548
LSM = LSR + 9*IGM*NRED	1DX 0549
LZN = LSM + IGM*NXC MR*NRED	1DX 0550
LHETC = LZN + NRCF*NRED	1DX 0551
LJ1 = LHETC + NRCF*NRED	1DX 0552
LATEM = LJ1 + MM01*NRED	1DX 0553
LMF = LATEM + MM01*NRED	1DX 0554
LU7 = LMF + MM01*NRED	1DX 0555
LTEM = LU7 + IGM*NRED	1DX 0556
LAG = LTEM + 3*ML*NRED	1DX 0557
LNXS = LAG + NRCF*IGM*NRED	1DX 0558
LFSS = LNXS + NRCF*NRED	1DX 0559
LSIGO = LFSS + 4*MMT*IGM*NRED	1DX 0560
LNPN = LSIGO + MMT*IGM*NRED	1DX 0561
LNFP = LNPN + NCR*NCRUN	1DX 0562
LNZN = LNFP + NFGM*NCRUN	1DX 0563
LNV = LNZN + NFGM*NCRUN	1DX 0564
LC2 = LNV + IGM*NCRUN	1DX 0565
LALPH = LC2 + IZM*IGM*NCRUN	1DX 0566
LALPS = LALPH + IZM*IGM*NCRUN	1DX 0567
LPHJ = LALPS + IZM*NCR*NCRUN	1DX 0568
LPHI = LPHJ + IZM*NCR*NCRUN	1DX 0569
LCO = LPHI + IZM*IGP	1DX 0570
LA0 = LCO + ITL*VT	1DX 0571
LR1 = LA0 + IP	1DX 0572
LR4 = LR1 + IP	1DX 0573
LR5 = LR4 + IM	1DX 0574
LV0 = LR5 + IM	1DX 0575
LN2 = LV0 + IM	1DX 0576
LFO = LN2 + IM*IGM	1DX 0577
LF2 = LFO + IM	1DX 0578
LI3 = LF2 + IM	1DX 0579
LK6 = LI3 + M01	1DX 0580
LS2 = LK6 + IGM	1DX 0581
LCXS = LS2 + IM	1DX 0582
LVOL = LCXS + IP*IGM*2	1DX 0583
LMASS = LVOL + IZM	1DX 0584
LHA = LMASS + ML*IZM	1DX 0585
LPA = LHA + IM	1DX 0586

```

LFB = LPA + IM                                1DX 0587
LAST = LEB + 8*IGP                             1DX 0588
ITEMP = LN2 + ITL*IGM*MT                       1DX 0589
LAST = MAX0(LAST,ITEMP)                       1DX 0590
WRITE(NOUT, 320) LAST                          1DX 0591
320  FORMAT(5H LAST,I6///)                     1DX 0592
      DO 325 I=1,LAST                          1DX 0593
325  A(I) = 0.0                                1DX 0594
C    READ FLUXES                               1DX 0595
      IF(M07-1) 330, 350, 350                 1DX 0596
330  ITEMP = LN0 + IGMIM - 1                   1DX 0597
      DO 340 I=LN0,ITEMP                       1DX 0598
340  A(I) = 1.0                                1DX 0599
      GO TO 370                                1DX 0600
350  WRITE(NOUT,360)                           1DX 0601
360  FORMAT(11H0FLUX GUESS)                   1DX 0602
      CALL REAG2(6H  NO,A(LN0),IGMIM)          1DX 0603
C    READ MESH POINTS                          1DX 0604
370  WRITE(NOUT,380)                           1DX 0605
380  FORMAT(12H0MESH POINTS)                  1DX 0606
      CALL REAG2(6H  R0,A(LR0),IP)             1DX 0607
C    READ ZONE NUMBERS                        1DX 0608
      WRITE(NOUT,390)                          1DX 0609
390  FORMAT(30H0ZONE NUMBERS BY MESH INTERVAL) 1DX 0610
      CALL REA12(6H  M0,A(LM0),IM)             1DX 0611
C    READ MATERIAL NUMBERS                    1DX 0612
      WRITE(NOUT,400)                          1DX 0613
400  FORMAT(25H0MATERIAL NUMBERS BY ZONE)     1DX 0614
      CALL REA12(6H  M2,A(LM2),IZM)            1DX 0615
C    READ BUCKLING MODIFIERS                 1DX 0616
      IF(I04 - 5) 401,402,401                 1DX 0617
401  IF(BUCK) 402,408,402                     1DX 0618
402  WRITE(NOUT,404)                           1DX 0619
404  FORMAT(27H0BUCKLING MODIFIERS BY ZONE)   1DX 0620
      CALL REAG2(6H  GAM,A(LGAM),IZM)          1DX 0621
C    READ FISSION FRACTIONS                  1DX 0622
408  WRITE(NOUT,410)                           1DX 0623
410  FORMAT(18H0FISSION FRACTIONS)           1DX 0624
      CALL REAG2(6H  K7,A(LK7),IGM)           1DX 0625
C    READ VELOCITIES                         1DX 0626
      WRITE(NOUT,420)                          1DX 0627
420  FORMAT(17H0NEUTRON VELOCITY)            1DX 0628
      CALL REAG2(6H  V7,A(LV7),IGM)           1DX 0629
C    READ MIXTURE SPECIFICATIONS              1DX 0630
      IF(M01) 440,440,430                     1DX 0631
430  WRITE(NOUT,445)                           1DX 0632
445  FORMAT(82H0MIXTURE SPECIFICATIONS (I0/I1/I2=MIX NUMBER/MAT. NUMBER)
1  FOR MIX/MATERIAL DENSITY))                 1DX 0633
      CALL REA12(6H  I0,A(LI0),M01)            1DX 0635
      CALL REA12(6H  I1,A(LI1),M01)            1DX 0636
      CALL REAG2(6H  I2,A(LI2),M01)            1DX 0637
C    READ ZONE MODIFIERS IF DELTA CALCULATION 1DX 0638
440  IF(I04-4) 500,450,500                   1DX 0639
450  WRITE(NOUT,460)                           1DX 0640
460  FORMAT(32H0ZONE MODIFIERS FOR DELTA OPTION) 1DX 0641
      CALL REAG2(6H  R3,A(LR3),IZM)            1DX 0642
C    READ NUMBERS OF MATERIALS TO BE PUNCHED 1DX 0643
C    READ NUMBER OF GROUPS FOR EACH CRUNCHED GROUP 1DX 0644
500  IF(NCRUN) 575,575,540                   1DX 0645

```

540	WRITE(NOUT,550)	1DX	0646
550	FORMAT(36H0NUMBER OF GROUPS PER CRUNCHED GROUP)	1DX	0647
	CALL REAI2(6H NPN,A(LNPN),NCR)	1DX	0648
C	READ NUMBERS OF MATERIALS TO BE CRUNCHED	1DX	0649
	WRITE(NOUT,560)	1DX	0650
560	FORMAT(36H0NUMBERS OF MATERIALS TO BE CRUNCHED)	1DX	0651
	CALL REAI2(6H NFP,A(LNFP),NFGM)	1DX	0652
C	READ ZONE NUMBERS OF FLUXES USED FOR CRUNCH CALCULATION	1DX	0653
	WRITE(NOUT,570)	1DX	0654
570	FORMAT(51H0ZONE NUMBERS OF FLUXES USED FOR CRUNCH CALCULATION)	1DX	0655
	CALL REAI2(6H NZN,A(LNZN),NFGM)	1DX	0656
575	IF(NRCF) 670, 670, 580	1DX	0657
C	READ ZONE NUMBERS FOR EACH CROSS SECTION MIX	1DX	0658
580	WRITE(NOUT,590)	1DX	0659
590	FORMAT(40H0ZONE NUMBERS FOR EACH CROSS SECTION MIX)	1DX	0660
	CALL REAI2(6H ZN,A(LZN),NRCF)	1DX	0661
C	READ HETEROGENEITY CONSTANT FOR EACH CROSS SECTION MIX	1DX	0662
	WRITE(NOUT,600)	1DX	0663
600	FORMAT(50H0HETEROGENEITY CONSTANT FOR EACH CROSS SECTION MIX)	1DX	0664
	CALL REAG2(6H HETC,A(LHETC),NRCF)	1DX	0665
C	READ ISOTOPE NUMBER FOR EACH TERM IN XS MIX VECTOR	1DX	0666
	WRITE(NOUT,610)	1DX	0667
610	FORMAT(46H0ISOTOPE NUMBER FOR EACH TERM IN XS MIX VECTOR)	1DX	0668
	CALL REAI2(6H J1,A(LJ1),MMQ1)	1DX	0669
C	READ TEMPERATURE FOR EACH TERM IN XS MIX VECTOR	1DX	0670
	WRITE(NOUT,620)	1DX	0671
620	FORMAT(44H0TEMPERATURES FOR EACH TERM IN XS MIX VECTOR)	1DX	0672
	CALL REAG2(6H ATEM,A(LATEM),MMQ1)	1DX	0673
C	READ FUEL OR MODERATOR DESIGNATION FOR EACH TERM IN XS MIX VECTOR	1DX	0674
	WRITE(NOUT,630)	1DX	0675
630	FORMAT(83H0FUEL OR MODERATOR DESIGNATION FOR EACH TERM IN XS MIX V	1DX	0676
	ECTOR (0/1=FUEL/MODERATOR)	1DX	0677
	CALL REAI2(6H MF,A(LMF),MMQ1)	1DX	0678
C	READ LETHARGY WIDTHS	1DX	0679
	WRITE(NOUT,640)	1DX	0680
640	FORMAT(16H0LETHARGY WIDTHS)	1DX	0681
	CALL REAG2(6H U7,A(LU7),IGM)	1DX	0682
	IF(ICARD) 650,670,670	1DX	0683
C	READ SEQUENCE NUMBERS ON TAPE FOR INPUT CROSS SECTIONS	1DX	0684
650	WRITE(NOUT,660)	1DX	0685
660	FORMAT(70H0SEQUENCE NUMBERS ON TAPE FOR CROSS SECTION DATA IN THE	1DX	0686
	RUSSIAN FORMAT)	1DX	0687
	CALL REAI2(6H NUT,A(LNUT),ML)	1DX	0688
C	END OF INPUT DATA (EXCEPT CROSS SECTION DATA)	1DX	0689
670	IF(LAST - 35000) 700, 700, 680	1DX	0690
680	WRITE(NOUT,690)	1DX	0691
690	FORMAT(26H PROGRAM CAPACITY EXCEEDED)	1DX	0692
	STOP	1DX	0693
700	RETURN	1DX	0694
	END	1DX	0695





```
-IT FOR CLEAR,CLEAR
  SUBROUTINE CLEAR (X,Y,N)
  DIMENSION Y(1)
  DO 1 I=1,N
1  Y(I)=X
  RETURN
  END
```

```
1DX 0705
1DX 0706
1DX 0707
1DX 0708
1DX 0709
1DX 0710
1DX 0711
-
```



-IT	FOR REAI2,REAI2	1DX 0759
	SUBROUTINE REAI2(HOLL,IARRAY,NCOUNT)	1DX 0760
	DIMENSION IARRAY(1),IV(6),K(6),IN(6)	1DX 0761
	COMMON       NCR1,     NFF,     NINP,     NOUT, NSCRAT,   NSCR1,   NSCR2	1DX 0762
	J=1	1DX 0763
10	READ(NINP,20)               (K(I),IN(I),IV(I),I=1,6)	1DX 0764
20	FORMAT(6(I1,I2,I9))	1DX 0765
	DO 70I=1,6	1DX 0766
	L=K(I)+1	1DX 0767
	GO TO (30,40,60,80 ),L	1DX 0768
C	NO MODIFICATION	1DX 0769
30	IARRAY(J)=IV(I)	1DX 0770
	J=J+1	1DX 0771
	GO TO 70	1DX 0772
C	RFPEAT	1DX 0773
40	L=IN(I)	1DX 0774
	DO 50 M=1,L	1DX 0775
	IARRAY(J)=IV(I)	1DX 0776
50	J=J+1	1DX 0777
	GO TO 70	1DX 0778
C	INTERPOLATE	1DX 0779
60	CALL ERRO2(6H REAI,60,1)	1DX 0780
70	CONTINUE	1DX 0781
	GO TO 10	1DX 0782
C	TERMINATE	1DX 0783
80	J=J-1	1DX 0784
	WRITE (NOUT,90)           HOLL,J       ,(IARRAY(I),I=1,J)	1DX 0785
	IF(J .-NCOUNT)100,110,100	1DX 0786
90	FORMAT(6X,A6,I6/(10I12))	1DX 0787
100	CALL ERRO2( 6H REAI2,100,1)	1DX 0788
110	RETURN	1DX 0789
	END	1DX 0790

```

-ITC FOR RCINP1,RCINP1                                1DX 0791
SUBROUTINE RCINP1(HOLN,ATW,NT,NGR,NGF,NPFF,FF,JGM3,JML,SF, 1DX 0792
1 SR, SM, JGM, U7, TEM, ATEM, IO, I1, I2, J1, MF, 1DX 0793
2 ZN, HETC,NUT,MPUP,NXS)                                1DX 0794
DIMENSION HOLN(1), ATW(1), NT(1), NGB(1), NGE(1), NPFF(4,1), 1DX 0795
1 FF(4,JGM3,1), SF(4,JML,1), SR( 9,1), SM( JGM,1), 1DX 0796
2 U7(1), TEM(3,1), ATEM(1), IO(1), I1(1), I2(1), J1(1), 1DX 0797
3 MF(1), ZN(1), HETC(1), NUT(1), MPUP(1), NXS(1) 1DX 0798
INCLUDE ABC                                            1DX 0799
C READ CROSS SECTION DATA IN THE RUSSIAN FORMAT 1DX 0800
ITFMP = 0                                             1DX 0801
ITEMP1 = 1                                           1DX 0802
DO 7 J=2,MM01                                        1DX 0803
IF(I0(J) - I0(J-1)) 3, 7, 3                          1DX 0804
3 ITEMP = ITEMP + 1                                  1DX 0805
ITEMP2 = J                                           1DX 0806
NXS(ITEMP) = ITEMP2 - ITEMP1                         1DX 0807
ITEMP1 = ITEMP2                                     1DX 0808
7 CONTINUE                                           1DX 0809
NXS(NRCF) = MM01 + 1 - ITEMP1                       1DX 0810
IF(ICARD) 10,20,20                                   1DX 0811
10 CALL RCPUP(HOLN,ATW,NT,NGB,NGE,NPFF,FF,JGM3,JML,SF, 1DX 0812
1 SR, SM, JGM, U7, TEM, ATEM, IO, I1, I2, J1, MF, 1DX 0813
2 ZN, HETC,NUT,MPUP)                                1DX 0814
GO TO 640                                            1DX 0815
20 DO 600 M=1,ML                                     1DX 0816
READ(NINP,30) HOLN(M), ATW(M), NT(M), NGB(M), NGE(M), 1DX 0817
1 (NPFF(K,M), K=1,4), (TEM(K,M),K=1,3) 1DX 0818
30 FORMAT(A6,E6.2,7I6,3E6.0)                         1DX 0819
IF(NT(M)) 40, 40, 50                                1DX 0820
40 NB = NGB(M)                                       1DX 0821
NE = NGE(M)                                          1DX 0822
GO TO 60                                             1DX 0823
50 NB = 3*NGB(M) - 2                                 1DX 0824
NF = 3*NGE(M)                                       1DX 0825
60 DO 100 K=1,4                                      1DX 0826
NF = NPFF(K,M)                                       1DX 0827
IF(NF) 100,100,62                                    1DX 0828
62 READ(NINP,80) (SF(K,M,N),N=1,NF) 1DX 0829
READ(NINP,80) ((FF(K,J,N), N=1,NF), J=NB,NE) 1DX 0830
WRITE(NFF) ((FF(K,J,N), N=1,NF), J=NB,NE) 1DX 0831
80 FORMAT(12E6.3)                                     1DX 0832
100 CONTINUE                                         1DX 0833
DO 200 IIG=1,IGM                                     1DX 0834
READ(NINP,160) L, (SR( K,IIG), K=1,9) 1DX 0835
160 FORMAT(I3, F10.4, F9.4, F5.3, F10.4, F6.4, F9.4, F6.4, F6.4, F8.4) 1DX 0836
IF(L-IIG) 170, 200, 170                              1DX 0837
170 WRITE(NOUT,180) HOLN(M),IIG 1DX 0838
180 FORMAT(32H CHECK CROSS SECTION SEQUENCE ,A6,6H GROUP I3///) 1DX 0839
200 CONTINUE                                         1DX 0840
WRITE(NSR) ((SR(K,IIG), K=1,9), IIG=1,IGM) 1DX 0841
DO 300 IIG=1,IGM                                     1DX 0842
IF(NXCNR - 11) 230,230,250 1DX 0843
230 READ(NINP,240) L, (SM( IIG,K),K=1,NXCNR) 1DX 0844
240 FORMAT(I3, E7.4, 10E6.4) 1DX 0845
GO TO 265                                            1DX 0846
250 READ(NINP,260) L, (SM( IIG,K),K=1,NXCNR) 1DX 0847
260 FORMAT(I3, E7.4, 10E6.4/(12E6.4)) 1DX 0848
265 IF(L-IIG) 270, 300, 270 1DX 0849

```

```

270 WRITE(NOUT,280) HOLN(M),IIG 1DX 0850
280 FORMAT(36H CHECK SCATTERING MATRIX SEQUENCE ,A6,6H GROUP I3//) 1DX 0851
300 CONTINUE 1DX 0852
WRITE(NSM) ((SM(IIG,K), K=1,NXCMR), IIG=1,IGM) 1DX 0853
NB = NGB(M) 1DX 0854
NE = NGE(M) 1DX 0855
IF(NPRT - 1) 600,600,320 1DX 0856
320 CALL RCPRT1(HOLN, SR, SM, NPFF, SF, FF, NT, M, JML, JGM, JGM3, 1DX 0857
1 NGB, NGE, TEM) 1DX 0858
600 CALL RCCHK(M, SR, HOLN, U7, SM, NT, NPFF, FF, NGB, NGE, JML, 1DX 0859
1 JGM, JGM3) 1DX 0860
IF(NPRT - 1) 610,610,640 1DX 0861
610 WRITE(NOUT,620) (M,HOLN(M), M=1,ML) 1DX 0862
620 FORMAT(79H1 CROSS SECTION DATA FOR THE FOLLOWING ISOTOPES WAS GIVE1DX 0863
1N IN THE RUSSIAN FORMAT// 30H ISOTOPE ISOTOPE / 1DX 0864
3 8H NUMBER// (I6,17X,A6)) 1DX 0865
640 WRITE(NOUT,650) (I0(J),I1(J),I2(J),J1(J),ATEM(J),MF(J),J=1,MM01) 1DX 0866
650 FORMAT(63H1MIXTURE SPECIFICATIONS FOR CALCULATING SHIELDED CROSS S1DX 0867
1ECTIONS// 116H MIXTURE MIX MATERI1DX 0868
2AL ISOTOPE TEMPERATURE 0/1=FUEL/MOD 1DX 0869
3/ 96H NUMBER COMMAND DENS1DX 0870
4TY NUMBER (DEG K) //(I10, I21, E25.8, 1DX 0871
5 I15, E25.8, I14)) 1DX 0872
WRITE(NOUT,660) 1DX 0873
660 FORMAT(//////8X, 1DX 0874
156H CROSS SECTION ZONE FLUXES HETC. CONST./ 8X, 1DX 0875
256H MIX NUMBER FOR XS MIX FOR XS MIX /) 1DX 0876
ITEMP1 = 1 1DX 0877
DO 670 J=1,NRCF 1DX 0878
ITEMP = I0(ITEMP1) 1DX 0879
ITEMP1 = ITEMPI + NXS(J) 1DX 0880
670 WRITE(NOUT,680) J, ITEMPI, ZN(J), HETC(J) 1DX 0881
680 FORMAT( I5, I14, I20, E25.8) 1DX 0882
REWIND NFF 1DX 0883
REWIND NSR 1DX 0884
REWIND NSM 1DX 0885
RETURN 1DX 0886
END 1DX 0887

```







```

-ITC FOR RCCHK,RCCHK                                1DX 1002
  SUBROUTINE RCCHK(M, SR, HOLN, U7, SM, NT, NPFF, FF, NGB, NGE, 1DX 1003
1      JML, JGM, JGM3)                                1DX 1004
  DIMENSION SR( 9,1), HOLN(1), U7(1), SM( JGM,1), NT(1), 1DX 1005
1      NPFF(4,1), FF(4,JGM3,1), NGB(1), NGE(1)        1DX 1006
  INCLUDE ABC                                          1DX 1007
C  THIS SUBROUTINE CHECKS THE RUSSIAN DATA FOR CONSISTENCY 1DX 1008
  DO 300 IIG = 1,IGM                                  1DX 1009
  TEMP = (SR( 1,IIG) - (SR( 2,IIG) + SR( 4,IIG) +SR( 5,IIG) + 1DX 1010
1      SR( 6,IIG)))/SR( 1,IIG)                        1DX 1011
  IF(ABS(TEMP) - .005) 70, 40, 40                    1DX 1012
40  ME = ME + 1                                        1DX 1013
  WRITE(NOUT,50) ME, HOLN(M), IIG                     1DX 1014
50  FORMAT(/14,7H CHECK A6,7H GROUP I3)              1DX 1015
70  IF(IIG - 3) 120, 120, 80                          1DX 1016
80  IF(IIG - IGM) 90, 120, 120                        1DX 1017
90  TEMP = (SR( 9,IIG) - (SR( 8,IIG)*SR( 6,IIG)/U7(IIG)))/ 1DX 1018
1      SR( 9,IIG)                                      1DX 1019
  IF(ABS(TEMP) - .03) 120, 100, 100                  1DX 1020
100 ME = ME + 1                                       1DX 1021
  WRITE(NOUT,110) ME, HOLN(M), IIG                   1DX 1022
110 FORMAT(/14,7H CHECK A6,7H GROUP I3,19H SIGMADEL LOOKS BAD) 1DX 1023
120 TEMP = .0                                          1DX 1024
  DO 130 K=1,NXCMR                                    1DX 1025
130 TEMP = TEMP + SM( IIG,K)                          1DX 1026
  IF(ABS(TEMP - SR( 5,IIG)) - .005) 300, 140, 140   1DX 1027
140 TEMP = TEMP - SR( 5,IIG)                          1DX 1028
  MF = ME + 1                                         1DX 1029
  WRITE(NOUT,150) ME, HOLN(M), IIG, TEMP             1DX 1030
150 FORMAT(/14,7H CHECK A6,7H GROUP I3,5X,26H PRESUMABLY SIGMA(N,2N) 1DX 1031
1= 1PE12.6)                                          1DX 1032
300 CONTINUE                                          1DX 1033
  IF(NT(M)) 310, 310, 320                             1DX 1034
310 ITEMP1 = NGB(M)                                   1DX 1035
  ITEMP2 = NGE(M)                                     1DX 1036
  GO TO 330                                           1DX 1037
320 ITEMP1 = 3*NGB(M) - 2                             1DX 1038
  ITEMP2 = 3*NGE(M)                                   1DX 1039
330 DO 505 K=1,4                                       1DX 1040
  NF = NPFF(K,M)                                       1DX 1041
  IF(NF) 505,505,340                                   1DX 1042
340 DO 500 N=1,NF                                       1DX 1043
  DO 500 J=ITEMP1,ITEMP2                               1DX 1044
  IF(N-1) 430, 430, 370                               1DX 1045
370 IF(FF(K,J,N) - FF(K,J,N-1) - .00001) 430, 410, 410 1DX 1046
410 ME = ME + 1                                       1DX 1047
  WRITE(NOUT,420) ME, HOLN(M)                         1DX 1048
420 FORMAT(/14, 7H CHECK ,A6,10H F FACTORS)          1DX 1049
430 IF(FF(K,J,N) - 1.00001) 500, 480, 480           1DX 1050
480 ME = ME + 1                                       1DX 1051
  WRITE(NOUT,490) ME, HOLN(M)                         1DX 1052
490 FORMAT(/14,14H F FACTORS IN ,A6,23H ARE GREATER THAN UNITY) 1DX 1053
500 CONTINUE                                          1DX 1054
505 CONTINUE                                          1DX 1055
  IF(NT(M)) 900, 900, 510                             1DX 1056
510 DO 560 K=1,4                                       1DX 1057
  NF = NPFF(K,M)                                       1DX 1058
  IF(NF) 560,560,515                                   1DX 1059
515 DO 540 N=1,NF                                       1DX 1060

```

	DO 540 IIG=NB,NE	1DX 1061
	J = 3*IIG - 2	1DX 1062
	IF(FF(K,J,N) - FF(K,J+1,N) - .00001) 520, 525, 525	1DX 1063
520	IF(FF(K,J+1,N) - FF(K,J+2,N) - .00001) 540, 525, 525	1DX 1064
525	ME = ME + 1	1DX 1065
	WRITE(NOUT,420) ME, HOLN(M)	1DX 1066
540	CONTINUE	1DX 1067
560	CONTINUE	1DX 1068
900	RETURN	1DX 1069
	END	1DX 1070

```

-ITC FOR RCSTUP,RCSTUP                                1DX 1071
  SUBROUTINE RCSTUP(NPFF,NT,FF,TEM,ATEM,JGM3,NGB,NGE,J1,SR,SM,JGM, 1DX 1072
  1 MPUP)                                              1DX 1073
  DIMENSION NPFF(4,1), NT(1), FF(4,JGM3,1), TEM(3,1), ATEM(1), 1DX 1074
  1 NGB(1), NGE(1), J1(1), SR(9,1), SM(JGM,1), MPUP(1) 1DX 1075
  INCLUDE ABC                                          1DX 1076
C THIS SUBROUTINE COMPUTES F-FACTORS FOR TEMP. DEPENDENT MATERIALS 1DX 1077
C AND PUTS THE CROSS SECTION DATA ON TAPE IN THE PROPER ORDER 1DX 1078
  DO 200 J=1,MM01                                     1DX 1079
  IF(J1(J)) 200, 200, 30                               1DX 1080
30 MM = J1(J)                                          1DX 1081
  DO 85 MK=1,ML                                       1DX 1082
  IF(ICARD) 34,36,36                                   1DX 1083
34 M = MPUP(MK)                                       1DX 1084
  GO TO 38                                             1DX 1085
36 M = MK                                             1DX 1086
38 DO 80 K=1,4                                         1DX 1087
  NF = NPFF(K,M)                                       1DX 1088
  IF(NF) 80, 80, 40                                    1DX 1089
40 IF(NT(M)) 50, 50, 60                               1DX 1090
50 ITEMP1 = NGB(M)                                     1DX 1091
  ITEMP2 = NGE(M)                                     1DX 1092
  GO TO 70                                             1DX 1093
60 ITEMP1 = 3*NGB(M) - 2                              1DX 1094
  ITEMP2 = 3*NGE(M)                                  1DX 1095
70 READ(NFF) ((FF(K,JJ,N), N=1,NF), JJ = ITEMP1,ITEMP2) 1DX 1096
80 CONTINUE                                           1DX 1097
  READ(NSR) ((SR(K,IIG), K=1,9), IIG=1,IGM)          1DX 1098
  READ(NSM) ((SM(IIG,K), K=1,NXCMR), IIG=1,IGM)      1DX 1099
  IF(M-MM) 85, 90, 85                                 1DX 1100
85 CONTINUE                                           1DX 1101
90 NR = NGB(M)                                         1DX 1102
  NF = NGE(M)                                         1DX 1103
  DO 150 K=1,4                                         1DX 1104
  NF = NPFF(K,M)                                       1DX 1105
  IF(NF) 150, 150, 95                                  1DX 1106
95 IF(NT(M)) 120,120,98                               1DX 1107
98 DO 100 IIG=NB,NE                                    1DX 1108
  JJ = 3*IIG - 2                                       1DX 1109
  DO 100 N=1,NF                                       1DX 1110
  AA = FF(K,JJ,N)                                       1DX 1111
  BB = FF(K,JJ+1,N)                                     1DX 1112
  CC = FF(K,JJ+2,N)                                     1DX 1113
  DD = TEM(3,M)/TEM(1,M)                               1DX 1114
  EE = ATEM(J)/TEM(2,M)                               1DX 1115
  Y = (CC-AA)/ALOG(DD)                                  1DX 1116
100 FF(K,IIG,N) = BB + Y*ALOG(EE)                    1DX 1117
120 WRITE(NSCRAT) ((FF(K,IIG,N), N=1,NF), IIG=NB,NE) 1DX 1118
150 CONTINUE                                           1DX 1119
  WRITE(NSCR1) ((SR(K,IIG), K=1,9), IIG=1,IGM)       1DX 1120
  WRITE(NSCR2) ((SM(IIG,K), K=1,NXCMR), IIG=1,IGM)   1DX 1121
  REWIND NFF                                           1DX 1122
  REWIND NSR                                           1DX 1123
  REWIND NSM                                           1DX 1124
200 CONTINUE                                           1DX 1125
  REWIND NSCRAT                                       1DX 1126
  REWIND NSCR1                                       1DX 1127
  REWIND NSCR2                                       1DX 1128
C SWITCH TAPE DESIGNATIONS                            1DX 1129

```

```
ITEMP = NFF  
ITEMP1 = NSR  
ITEMP2 = NSM  
NFF = NSCRAT  
NSR = NSCR1  
NSM = NSCR2  
NSCRAT = ITEMP  
NSCR1 = ITEMP1  
NSCR2 = ITEMP2  
RETURN  
END
```

```
1DX 1130  
1DX 1131  
1DX 1132  
1DX 1133  
1DX 1134  
1DX 1135  
1DX 1136  
1DX 1137  
1DX 1138  
1DX 1139  
1DX 1140  
-
```

```

-ITC FOR RCCAL1,RCCAL1                                1DX 1141
SUBROUTINE RCCAL1(AG,NXS,I1,SR,MF,I2,FSS,SIGO,HETC,MRCF,JMT) 1DX 1142
DIMENSION AG(MRCF,1), NXS(1), I1(1), SR(9,1), MF(1), I2(1), 1DX 1143
1 FSS(4,JMT,1), SIGO(JMT,1), HETC(1)                    1DX 1144
INCLUDE ABC                                            1DX 1145
C THIS SUBROUTINE CALCULATES SIGO                      1DX 1146
ITEMP = 0                                             1DX 1147
DO 6 L=1,NRCF                                         1DX 1148
IF(HETC(L)) 4,6,4                                     1DX 1149
4 ITEMP = 1                                           1DX 1150
GO TO 8                                               1DX 1151
6 CONTINUE                                           1DX 1152
8 IF(ITEMP) 9,60,9                                    1DX 1153
9 ITEMP2 = 0                                          1DX 1154
DO 50 L=1,NRCF                                        1DX 1155
DO 10 IIG=1,IGM                                       1DX 1156
10 AG(L,IIG) = 0.0                                     1DX 1157
ITEMP1 = ITEMP2 + 2                                   1DX 1158
ITEMP2 = ITEMP2 + NXS(L)                             1DX 1159
DO 50 J=ITEMP1,ITEMP2                                 1DX 1160
M = I1(J)                                             1DX 1161
READ(NSR) ((SR(K,IIG), K=1,9), IIG=1,IGM)           1DX 1162
IF(MF(J)) 50, 50, 20                                  1DX 1163
20 GO TO (30,40), IRED                                1DX 1164
30 DO 35 IIG=1,IGM                                    1DX 1165
35 AG(L,IIG) = AG(L,IIG) + I2(J)*SR(1,IIG)          1DX 1166
GO TO 50                                              1DX 1167
40 DO 45 IIG=1,IGM                                    1DX 1168
45 AG(L,IIG) = AG(L,IIG) + I2(J)*SR(1,IIG)*FSS(3,M,IIG) 1DX 1169
50 CONTINUE                                           1DX 1170
REWIND NSR                                           1DX 1171
60 DO 70 M=1,MMT                                       1DX 1172
DO 70 IIG=1,IGM                                       1DX 1173
70 SIGO(M,IIG) = 0.0                                  1DX 1174
ITEMP2 = 0                                           1DX 1175
DO 180 L=1,NRCF                                       1DX 1176
ITEMP1 = ITEMP2 + 2                                   1DX 1177
ITEMP2 = ITEMP2 + NXS(L)                             1DX 1178
DO 180 J=ITEMP1,ITEMP2                                 1DX 1179
M = I1(J)                                             1DX 1180
READ(NSR) ((SR(K,IIG), K=1,9), IIG=1,IGM)           1DX 1181
100 DO 180 JJ=ITEMP1,ITEMP2                            1DX 1182
MM = I1(JJ)                                           1DX 1183
IF(M-MM) 105, 180, 105                                1DX 1184
105 DO 170 IIG=1,IGM                                    1DX 1185
IF(MF(J)) 140,140,110                                  1DX 1186
IF(MF(JJ)) 120,120,140                                  1DX 1187
120 GO TO (130,135), IRED                              1DX 1188
130 SIGO(MM,IIG) = SIGO(MM,IIG) + I2(J)*SR(1,IIG)/(I2(JJ)* 1DX 1189
1 (1.+HETC(L)*AG(L,IIG)))                             1DX 1190
GO TO 170                                             1DX 1191
135 SIGO(MM,IIG) = SIGO(MM,IIG) + I2(J)*(FSS(2,M,IIG)*SR(4,IIG) + 1DX 1192
1 FSS(1,M,IIG)*SR(2,IIG) + FSS(4,M,IIG)*SR(6,IIG) + SR(5,IIG))/ 1DX 1193
2 (I2(JJ)*(1.+HETC(L)*AG(L,IIG)))                    1DX 1194
GO TO 170                                             1DX 1195
140 GO TO (150,155), IRED                              1DX 1196
150 SIGO(MM,IIG) = SIGO(MM,IIG) + I2(J)*SR(1,IIG)/I2(JJ) 1DX 1197
GO TO 170                                             1DX 1198
155 SIGO(MM,IIG) = SIGO(MM,IIG) + I2(J)*(FSS(2,M,IIG)*SR(4,IIG) + 1DX 1199

```

	1	FSS(1,M,IIG)*SR(2,IIG) + FSS(4,M,IIG)*SR(6,IIG) + SR(5,IIG))/	1DX 1200
	2	I2(JJ)	1DX 1201
170		CONTINUE	1DX 1202
180		CONTINUE	1DX 1203
		IRED = 2	1DX 1204
		REWIND NSR	1DX 1205
		RETURN	1DX 1206
		END	1DX 1207

-



```

-ITC FOR  RCCSS,RCCSS                                1DX 1268
  SUBROUTINE RCCSS(ZN,NXS,I1,SR,SM,C,FSS,U7,PHI,JGM,JTL,JMT,JZM) 1DX 1269
  DIMENSION ZN(1), NXS(1), I1(1), SR(9,1), SM(JGM,1), C(JTL,JGM,1), 1DX 1270
  1 FSS(4,JMT,1), U7(1), PHI(JZM,1)                    1DX 1271
  INCLUDE ABC                                          1DX 1272
C  THIS SUBROUTINE CALCULATES RESONANCE SHIELDED CROSS SECTIONS 1DX 1273
  LAP = .0                                           1DX 1274
  LAPP = .0                                          1DX 1275
  LAR = .0                                           1DX 1276
  ALA = .0                                           1DX 1277
  P02 = 0                                           1DX 1278
  CVT = 0                                           1DX 1279
  CNT = 0                                           1DX 1280
  IF(RO2) 20,20,5                                     1DX 1281
  DO 10  IIG=1,IGM                                    1DX 1282
  10  READ(NCR1) ((C(L,IIG,M), L=1,ITL), M=1,MT)      1DX 1283
  REWIND NCR1                                         1DX 1284
  20  ITEMP2 = 0                                       1DX 1285
  DO 400 L=1,NRCF                                     1DX 1286
  KZ = ZN(L)                                          1DX 1287
  ITEMP1 = ITEMP2 + 2                                1DX 1288
  ITEMP2 = ITEMP2 + NXS(L)                           1DX 1289
  DO 400 J=ITEMP1,ITEMP2                             1DX 1290
  M = I1(J)                                           1DX 1291
  READ(NSR) ((SR(K,IIG), K=1,9), IIG=1,IGM)         1DX 1292
  READ(NSM) ((SM(IIG,K), K=1,NXCMR), IIG=1,IGM)     1DX 1293
  U0 = 0.0                                           1DX 1294
  DO 400 IIG=1,IGM                                    1DX 1295
  TEMP = 0.0                                          1DX 1296
  DO 30 K=1,NXCMR                                     1DX 1297
  30  TEMP = TEMP + SM(IIG,K)                         1DX 1298
  IF(RO2) 40, 40, 140                                1DX 1299
  40  C(1,IIG,M) = FSS(1,M,IIG)*SR(2,IIG)            1DX 1300
  C(2,IIG,M) = C(1,IIG,M) + FSS(2,M,IIG)*SR(4,IIG) - 1DX 1301
  1  (TEMP - SR(5,IIG))                               1DX 1302
  C(3,IIG,M) = SR(3,IIG)*C(1,IIG,M)                 1DX 1303
  C(4,IIG,M) = (FSS(3,M,IIG)*SR(1,IIG) - C(2,IIG,M) - TEMP) 1DX 1304
  1  *(1. - SR(7,IIG)) + C(2,IIG,M) + TEMP           1DX 1305
  C(5,IIG,M) = C(4,IIG,M) - C(2,IIG,M) - TEMP + SM(IIG,1) 1DX 1306
  1  - FSS(4,M,IIG)*SR(9,IIG)                       1DX 1307
  IF(IIG - 1) 50, 50, 60                             1DX 1308
  50  C(6,IIG,M) = .0                                  1DX 1309
  GO TO 80                                            1DX 1310
  60  C(6,IIG,M) = FSS(4,M,IIG-1)*SR(9,IIG-1) + SM(IIG-1,2) 1DX 1311
  80  DO 100 KK=7,ITL                                 1DX 1312
  K = KK-5                                           1DX 1313
  C(KK,IIG,M) = 0.0                                  1DX 1314
  JJG = IIG - K                                       1DX 1315
  IF(JJG) 100, 100, 90                               1DX 1316
  90  C(KK,IIG,M) = SM(JJG,K+1)                     1DX 1317
  100 CONTINUE                                       1DX 1318
  GO TO 400                                          1DX 1319
C  ITERATE ON ELASTIC DOWNSCATTERING CROSS SECTION 1DX 1320
  140 IF(IIG - IGM) 150, 145, 145                   1DX 1321
  145 SCDP = SCD                                      1DX 1322
  SCD = 0.0                                          1DX 1323
  GO TO 210                                          1DX 1324
  150 U0 = U0 + U7(IIG)                              1DX 1325
  U1 = U0 - .5*U7(IIG)                              1DX 1326

```



	U2 = U0 + .5*U7(IIG+1)	1DX 1327
	UX = U0 - .66*SR(8,IIG)	1DX 1328
	TEMP1 = PHI(KZ,IIG+1)*U7(IIG)/(PHI(KZ,IIG)*U7(IIG+1))	1DX 1329
	TEMP2 = (UX - U1)/(U2 - U1)	1DX 1330
	TEMP3 = SR(8,IIG)*FSS(4,M,IIG)*SR(6,IIG)	1DX 1331
	TEMP4 = SR(8,IIG+1)*FSS(4,M,IIG+1)*SR(6,IIG+1)	1DX 1332
200	SCDP = SCD	1DX 1333
	SCD = (TEMP3 + TEMP2*(TEMP4*TEMP1 - TEMP3))/U7(IIG)	1DX 1334
	IF(IIG - 1) 205,205,210	1DX 1335
205	SCD = SR(9,IIG)*FSS(4,M,IIG)	1DX 1336
	GO TO 400	1DX 1337
210	C(6,IIG,M) = SCDP + SM(IIG-1,2)	1DX 1338
	C(5,IIG,M) = C(4,IIG,M) - C(2,IIG,M) - TEMP + SM(IIG,1) - SCD	1DX 1339
400	CONTINUE	1DX 1340
	REWIND NSR	1DX 1341
	REWIND NSM	1DX 1342
	RETURN	1DX 1343
	END	1DX 1344

```

-ITC FOR RECS,RECS                                1DX 1345
  SUBROUTINE RECS (C,JTL,JGM,JMT,ATW,HOLN)          1DX 1346
  INCLUDE ABC                                       1DX 1347
  DIMENSION C(JTL,JGM,JMT), ATW(1), HOLN(1)        1DX 1348
C  C(ITL,IGM,MT) CROSS SECTION ARRAY--STARTS AT A(LN2) 1DX 1349
  DIMENSION AA(10)                                  1DX 1350
C  READ AND CHECK CROSS SECTIONS, PERFORM ADJOINT REVERSALS IF 1DX 1351
C  REQUIRED, AND WRITE CROSS SECTION TAPE           1DX 1352
  IF(NRCF) 1,1,160                                  1DX 1353
  1  WRITE(NOUT,5) (ID(I), I=1,11)                 1DX 1354
  5  FORMAT(1H1,11A6,///)                           1DX 1355
  WRITE(NOUT,10)                                     1DX 1356
  10 FORMAT(45H THE FOLLOWING NUCLIDES ARE IN THE DTF FORMAT/) 1DX 1357
  DO 50 I=1,ML                                       1DX 1358
  READ(NINP, 20) HOLN(I), ATW(I), (AA(J), J=1,10)  1DX 1359
  20 FORMAT(A6, E6.2, 10A6)                          1DX 1360
  IF(ICARD) 25,25,30                                 1DX 1361
  25 READ(15) ((C(L,IIG,I), L=1,ITL), IIG=1,IGM)  1DX 1362
  GO TO 50                                           1DX 1363
  30 DO 35 IIG=1,IGM                                  1DX 1364
  35 READ(NINP,40) (C(L,IIG,I), L=1,ITL)           1DX 1365
  40 FORMAT(6E12.5)                                   1DX 1366
  50 WRITE(NOUT, 60) I, HOLN(I), (AA(J), J=2,10)  1DX 1367
  60 FORMAT(I3, 6X, A6, 6X, 10A6)                   1DX 1368
C  CHECK ON CROSS SECTION CONSISTENCY AND ORDER    1DX 1369
  ITEMP = 0                                          1DX 1370
  DO 140 J=1,ML                                       1DX 1371
  DO 140 I=1,IGM                                       1DX 1372
  G = C(2,I,J) + C(5,I,J)                            1DX 1373
  DO 110 K = 1, NXCM                                  1DX 1374
  KK = I + K                                          1DX 1375
  M = 5 + K                                          1DX 1376
  IF(KK - IGM) 100, 100, 110                         1DX 1377
  G = G + C(M,KK,J)                                   1DX 1378
  100 CONTINUE                                       1DX 1379
  IF(ABS((G - C(4,I,J))/C(4,I,J)) - .01) 135, 120, 120 1DX 1380
  120 ITEMP = 1                                       1DX 1381
  130 FORMAT(1H /,16H CHECK MATERIAL 12,5X, 7H GROUP 12) 1DX 1382
  135 IF(ABS((G - C(4,I,J))/C(4,I,J)) - .0001) 140, 138, 138 1DX 1383
  138 WRITE(NOUT,130) J, I                            1DX 1384
  140 CONTINUE                                       1DX 1385
  IF (ITEMP) 160,160,150                             1DX 1386
  150 CALL EXIT                                       1DX 1387
C  A02=0/1=FLUX CALCULATION/ADJOINT CALCULATION    1DX 1388
  160 IF(A02) 170, 280, 170                          1DX 1389
  170 DO 190 IIG=1,IGM                                1DX 1390
  IGBAR=IGM-IIG+1                                    1DX 1391
  DO 180 M=1,MT                                       1DX 1392
  DO 180 L = 1,IHS                                    1DX 1393
  TEMP=C(L,IIG,M)                                    1DX 1394
  C(L,IIG,M)=C(L,IGBAR,M)                            1DX 1395
  180 C(L,IGBAR,M)=TEMP                               1DX 1396
  IF (IGBAR - IIG -1) 200, 200, 190                 1DX 1397
  190 CONTINUE                                       1DX 1398
  200 CONTINUE                                       1DX 1399
  KK = ITL - IHS                                     1DX 1400
  IF (KK) 280, 280, 210                              1DX 1401
  210 DO 240 M = 1,MT                                 1DX 1402
  DO 240 IIG = 1,IGM                                 1DX 1403

```

	IGBAR = IGM - IIG + 1	1DX 1404
	DO 240 L = 1, KK	1DX 1405
	IF (L - IIG) 220, 240, 240	1DX 1406
220	I = L + IHS	1DX 1407
	ITEMP = IGBAR + L	1DX 1408
	IF (IIG - ITEMP) 230, 230, 240	1DX 1409
230	TEMP = C(I, IIG, M)	1DX 1410
	C(I, IIG, M) = C(I, ITEMP, M)	1DX 1411
	C(I, ITEMP, M) = TEMP	1DX 1412
240	CONTINUE	1DX 1413
C	WRITE CROSS SECTION TAPE	1DX 1414
280	DO 300 IIG=1, IGM	1DX 1415
300	WRITE(NCR1) ((C(L, IIG, M), L=1, ITL), M=1, MT)	1DX 1416
	REWIND NCR1	1DX 1417
	RETURN	1DX 1418
	END	1DX 1419

-

```

-ITC FOR INIT,INIT                                1DX 1420
  SUBROUTINE INIT (K6, K7, I0, I1, I2, MU, M2, NO, R0, R1, R3, R4, 1DX 1421
  1          R5, A0, F0, C0, V0, JTL, JIM, V7, JMT, EB, GAM) 1DX 1422
    INCLUDE ABC                                    1DX 1423
    DIMENSION K6(1), K7(1), I0(1), I1(1), I2(1), R0(1), R1(1), 1DX 1424
  1          R3(1), R4(1), R5(1), A0(1), C0(JTL,JMT), 1DX 1425
  2          V0(1), MU(1), M2(1), NO(JIM,1), F0(1), V7(1), EB(8,1), 1DX 1426
  3          GAM(1)                                  1DX 1427
C    PERFORM ADJOINT REVERSALS ON V7 AND K7        1DX 1428
  IF(A02) 10, 40, 10                                1DX 1429
10  IF(P02) 40, 20, 40                                1DX 1430
20  IF(R02) 25,25,40                                  1DX 1431
25  IIG = 1                                           1DX 1432
    IGBAR=IGM                                         1DX 1433
30  TEMP=K7(IIG)                                       1DX 1434
    K7(IIG)=K7(IGBAR)                                 1DX 1435
    K7(IGBAR)=TEMP                                    1DX 1436
    TEMP=V7(IIG)                                       1DX 1437
    V7(IIG)=V7(IGBAR)                                 1DX 1438
    V7(IGBAR)=TEMP                                    1DX 1439
    IIG=IIG+1                                          1DX 1440
    IGBAR=IGBAR-1                                      1DX 1441
    IF(IIG-IGBAR) 30, 40, 40                          1DX 1442
C                                                    1DX 1443
C    MIX CROSS SECTIONS                               1DX 1444
40  B07=1                                             1DX 1445
    IF(P02) 50, 60, 50                                1DX 1446
50  GO TO (430,430,120,430,320), 104                 1DX 1447
60  IF(M01) 90, 90, 70                                1DX 1448
70  IF(R02) 75,75,90                                  1DX 1449
75  WRITE(NOUT, 80 ) (J, I0(J), I1(J), I2(J), J = 1, M01) 1DX 1450
80  FORMAT(28H1MIXTURE SPECIFICATIONS //             1DX 1451
  1          3X, 16H MIXTURE NUMBER ,18H MIX COMMAND , 1DX 1452
124H MATERIAL DENSITY //(14,1X,18,8X,18,8X,E20.8)) 1DX 1453
90  IF(NPRT) 120, 120, 100                            1DX 1454
100 IF(NRED) 105,105,120                              1DX 1455
105 KPAGE = 100                                        1DX 1456
    WRITE(NOUT,110)                                    1DX 1457
110 FORMAT(24H1SHIELDED CROSS SECTIONS)             1DX 1458
120 REWIND NCR1                                       1DX 1459
    DO 310 IIG=1,IGM                                   1DX 1460
    READ (NCR1) ((C0(I,J),I=1,ITL),J=1,MT)          1DX 1461
    IF(M01) 130, 240, 130                             1DX 1462
130 DO 230 M=1,M01                                    1DX 1463
    IF(I0(M)-MT) 150, 150, 140                        1DX 1464
140 CALL ERRO2(6H INIT,140,1)                         1DX 1465
150 IF(I1(M)-MT) 160, 160, 140                       1DX 1466
160 N=I0(M)                                           1DX 1467
    L=I1(M)                                           1DX 1468
    E01=I2(M)                                          1DX 1469
    IF(L) 200, 200, 170                               1DX 1470
170 IF(E01) 200, 180, 200                             1DX 1471
180 IF (N-L) 200, 190, 200                            1DX 1472
190 E01 = EV                                           1DX 1473
    L = 0                                              1DX 1474
200 DO 230 I=1,ITL                                    1DX 1475
    IF (L) 210, 220, 210                              1DX 1476
210 C0(I,N)=C0(I,N)+C0(I,L)*E01                    1DX 1477
    GO TO 230                                          1DX 1478

```

```

220 CO(I,N)=CO(I,N)*E01 1DX 1479
230 CONTINUE 1DX 1480
240 IF(P02) 300, 250, 300 1DX 1481
250 IF(NPRT) 300, 300, 260 1DX 1482
260 IF(NRED) 265,265,300 1DX 1483
265 WRITE (NOUT, 270) IIG 1DX 1484
270 FORMAT(6HUGROUP,13, 84H SIGF SIGA NUSIGF SIGTR 1DX 1485
1 GXG G-1XG G-2XG . . .) 1DX 1486
DO 280 N=1,MT 1DX 1487
280 WRITE (NOUT, 290) N,(CO(I,N),I=1,ITL) 1DX 1488
290 FORMAT(4H MAT,13,10E11.5/(7X,10E11.5)) 1DX 1489
300 WRITE (NSCRAT) ((CO(I,J),I=1,ITL),J=1,MT) 1DX 1490
310 CONTINUE 1DX 1491
REWIND NCR1 1DX 1492
REWIND NSCRAT 1DX 1493
C SWITCH TAPE DESIGNATIONS 1DX 1494
ITEMP=NSCRAT 1DX 1495
NSCRAT=NCR1 1DX 1496
NCR1=ITEMP 1DX 1497
320 IF(I04-5) 330, 350, 330 1DX 1498
330 IF(BUCK) 340, 430, 340 1DX 1499
340 TEMP = BUCK 1DX 1500
GO TO 380 1DX 1501
350 IF(P02) 360, 360, 370 1DX 1502
360 BUCK = 0. 1DX 1503
370 TEMP = EV - BUCK 1DX 1504
BUCK = EV 1DX 1505
380 DO 420 IIG=1,IGM 1DX 1506
READ(NCR1) ((CO(I,J), I=1,ITL),J=1,MT) 1DX 1507
DO 410 MTZ = 1,MT 1DX 1508
DO 400 KZ=1,IZM 1DX 1509
IF(M2(KZ) - MTZ) 400, 390, 400 1DX 1510
390 TEMP1 = (TEMP*GAM(KZ))/(3.*CO(4,MTZ)) 1DX 1511
CO(2,MTZ) = CO(2,MTZ) + TEMP1 1DX 1512
CO(5,MTZ) = CO(5,MTZ) - TEMP1 1DX 1513
GO TO 410 1DX 1514
400 CONTINUE 1DX 1515
410 CONTINUE 1DX 1516
WRITE(NSCRAT) ((CO(I,J), I=1,ITL),J=1,MT) 1DX 1517
420 CONTINUE 1DX 1518
REWIND NCR1 1DX 1519
REWIND NSCRAT 1DX 1520
C SWITCH TAPE DESIGNATIONS 1DX 1521
ITEMP = NSCRAT 1DX 1522
NSCRAT = NCR1 1DX 1523
NCR1 = ITEMp 1DX 1524
C 1DX 1525
C MODIFY GEOMETRY 1DX 1526
430 IF(P02) 460, 440, 460 1DX 1527
440 IF(R02) 445, 445, 620 1DX 1528
445 DO 450 I=1,IP 1DX 1529
450 R1(I)=R0(I) 1DX 1530
460 IF(I04-4) 490, 470, 490 1DX 1531
470 DO 480 I=1,IM 1DX 1532
K=MO(I) 1DX 1533
480 R1(I+1)=R1(I)+(R0(I+1)-R0(I))*(1.0+ EV*R3(K)) 1DX 1534
C 1DX 1535
C CALCULATE ARFAS AND VOLUMES 1DX 1536
490 PI2=6.28318 1DX 1537

```

500	IF(P02) 500, 510, 500	1DX 1538
	IF(I04 - 4) 620, 510, 620	1DX 1539
510	DO 570 I=1,IM	1DX 1540
	R4(I)=(R1(I+1)+R1(I))*0.5	1DX 1541
	R5(I)=R1(I+1)-R1(I)	1DX 1542
	IF( R5(I) ) 520, 520, 530	1DX 1543
520	CALL ERRO2 (6H INIT,520,1)	1DX 1544
530	GO TO (540,550,560),IGEP	1DX 1545
540	A0(I)=1.0	1DX 1546
	A0(IP)=1.0	1DX 1547
	V0(I) = R5(I)	1DX 1548
	GO TO 570	1DX 1549
550	A0(I)=PI2*R1(I)	1DX 1550
	A0(IP)=PI2*R1(IP)	1DX 1551
	V0(I) = PI2*R5(I)*R4(I)	1DX 1552
	GO TO 570	1DX 1553
560	A0(I) = 2.*PI2*R1(I)*R1(I)	1DX 1554
	A0(IP) = 2.*PI2*R1(IP)*R1(IP)	1DX 1555
	V0(I) = (2.0*PI2*(R1(I+1)**3 - R1(I)**3))/3.0	1DX 1556
570	CONTINUE	1DX 1557
C		1DX 1558
C	CALCULATE EFFECTIVE FISSION SPECTRUM	1DX 1559
620	IF(P02) 680, 640, 680	1DX 1560
640	SK7=0.	1DX 1561
	DO 670 IIG=1,IGM	1DX 1562
	IF(S02-1) 660, 650, 660	1DX 1563
650	K6(IIG)=K7(IIG)/S03	1DX 1564
	GO TO 670	1DX 1565
660	K6(IIG)=K7(IIG)	1DX 1566
670	SK7=SK7+K7(IIG)	1DX 1567
680	CONTINUE	1DX 1568
C		1DX 1569
C	CALCULATE FISSION RATE	1DX 1570
	T11 = EB(1,IGP)	1DX 1571
690	CALL CLEAR(U.0,F0,IM)	1DX 1572
	DO 720 IIG=1,IGM	1DX 1573
	EB(8,IIG) = .0	1DX 1574
	READ (NCR1) ((C0(I,J),I=1,ITL),J=1,MT)	1DX 1575
	DO 720 I = 1, IM	1DX 1576
	ITEMP=M0(I)	1DX 1577
	ITEMP=M2(ITEMP)	1DX 1578
	EB(8,IIG) = EB(8,IIG) + V0(I)*NU(I,IIG)*C0(1,ITEMP)	1DX 1579
	IF(A02) 700, 710, 700	1DX 1580
700	F0(I)=F0(I)+K7(IIG)*NU(I,IIG)	1DX 1581
	GO TO 720	1DX 1582
710	F0(I)=F0(I)+C0(IHT-1,ITEMP)*NU(I,IIG)	1DX 1583
720	CONTINUE	1DX 1584
	REWIND NCR1	1DX 1585
	RETURN	1DX 1586
	END	1DX 1587



-ITC FOR MONPR,MONPR						1DX 1641
SUBROUTINE MONPR						1DX 1642
INCLUDE ABC						1DX 1643
C  MONITOR PRINT						1DX 1644
CALL ETIMEF(TI)						1DX 1645
TI = TI/60.						1DX 1646
KPAGE = KPAGE + 1						1DX 1647
IF(KPAGE - 40) 40, 10, 10						1DX 1648
10  KPAGE = 0						1DX 1649
WRITE(NOUT,20 ) (ID(1), I=1,11)						1DX 1650
20  FORMAT(1H1,11A6//)						1DX 1651
WRITE(NOUT, 30 )						1DX 1652
30  FORMAT(5X,117H                  TIME                                  SIGE                                  OUTER						1DX 1653
1                  EIGENVALUE                                  EIGENVALUE                                  LAMBDA                                  /						1DX 1654
2                  5X, 73H                  (MINUTES)                                  ITERATIONS                                  ITERATI						1DX 1655
3ONS                  SLOPE /)						1DX 1656
40  WRITE(NOUT,50) TI, R02, NP02, EQ, EV, ALA						1DX 1657
50  FORMAT(F17.2, I18, I20, E26.8, 2E20.8)						1DX 1658
60  P02=P02+1						1DX 1659
NP02 = NP02 + 1						1DX 1660
IF(NP02-100) 80, 80, 70						1DX 1661
70  NGOTO = 1						1DX 1662
NRED = 0						1DX 1663
GO TO 90						1DX 1664
80  NGOTO = 4						1DX 1665
90  RETURN						1DX 1666
END						1DX 1667



```

-ITC FOR OUTER,OUTER                                1DX 1668
  SUBROUTINE OUTER( A0, C0, F0, K6, M0, M2, N0, N2, S2, V0, V7, F2, 1DX 1669
1      JTL,JMT,CXS,JIM, R5, R4, HA, PA, EB,JIP,JIGM) 1DX 1670
  DIMENSION AU(1), F0(1), K6(1), M0(1), M2(1), N0(JIM,1), N2(JIM,1),1DX 1671
1      S2(1), V0(1), V7(1), F2(1), CO(JTL,JMT), HA(1), PA(1), 1DX 1672
2      CXS(JIP,JIGM,1),R5(1), R4(1), EB(8,1)        1DX 1673
  INTEGER      GBAR, PBAR, SBAR                      1DX 1674
  INCLUDE ABC                                       1DX 1675
  IGV=1                                             1DX 1676
C SOURCE CALCULATION                               1DX 1677
20 READ(NCR1)      ((CU(I,M),I=1,ITL),M=1,MT)      1DX 1678
  DO 30 I=1,IM                                       1DX 1679
30  S2(I)=0.                                          1DX 1680
40  IF(A02) 70, 50, 70                               1DX 1681
50  DO 60 I=1,IM                                       1DX 1682
60  S2(I)=S2(I)+K6(IGV)*F0(I)                       1DX 1683
  GO TO 90                                           1DX 1684
70  DO 80 I=1,IM                                       1DX 1685
  ITEMP1=M0(I)                                       1DX 1686
  ITEMP1=M2(ITEMP1)                                  1DX 1687
80  S2(I)=S2(I)+CO(IHT-1,ITEMP1)*F0(I)             1DX 1688
90  GBAR=IGV+IHS-ITL                                  1DX 1689
  IF(GBAR-1) 100, 110, 110                          1DX 1690
100 GBAR=1                                           1DX 1691
110 PBAR = IHS + IGV - 1                             1DX 1692
  IF(PBAR - ITL) 115, 115, 112                      1DX 1693
112 PBAR = ITL                                       1DX 1694
115 IF(GBAR - IGV) 120, 140, 140                   1DX 1695
120 DO 130 I=1,IM                                       1DX 1696
  ITEMP1=M0(I)                                       1DX 1697
  ITEMP1=M2(ITEMP1)                                  1DX 1698
  ITEMP=ITEMP1                                       1DX 1699
  TEMP=CU(PBAR,ITEMP)                                1DX 1700
130 S2(I)=S2(I)+N2(I,GBAR)*TEMP                    1DX 1701
140 GBAR=GBAR+1                                       1DX 1702
  PBAR=PBAR-1                                       1DX 1703
  IF(GBAR - IGV) 120, 140, 150                     1DX 1704
150 IF(IGV - IGM) 170, 160, 170                   1DX 1705
160 REWIND NCR1                                       1DX 1706
170 V11 = 0.                                          1DX 1707
  DO 180 I=1,IM                                       1DX 1708
  S2(I)=S2(I)*V0(I)                                  1DX 1709
180 V11=V11+S2(I)                                    1DX 1710
  EB(2,IGV) = V11 - EB(1,IGV)                       1DX 1711
C SOURCE ALPHA                                     1DX 1712
190 IF(I04 - 2) 200, 230, 200                       1DX 1713
200 IF(S02 - 2) 220, 210, 220                       1DX 1714
210 T7 = S03/V7(IGV)                                 1DX 1715
  GO TO 240                                           1DX 1716
220 T7 = 0.0                                          1DX 1717
  GO TO 260                                           1DX 1718
230 T7 = EV/V7(IGV)                                  1DX 1719
240 DO 250 K = 1, IZM                                 1DX 1720
  ITEMP1 = M2(K)                                     1DX 1721
250 CO(IHS, ITEMP1) = CO(IHS,ITEMP1) - T7          1DX 1722
260 CONTINUE                                         1DX 1723
C GROUP FLUX CALCULATION                          1DX 1724
270 IF(V11) 280, 300, 280                            1DX 1725
280 IF(P02 - 1) 284, 284, 282                      1DX 1726

```

```

282 IF(I04-1) 284, 286, 284 1DX 1727
284 CALL INNER1(MU,M2,CXS,VU,CO,A0,R5,R4,JIM,JTL,JIP,JIGM) 1DX 1728
286 CALL INNER(NU,N2,CXS,S2,HA,PA,JIP,JIGM,JTL,JIM,EB,CO,VO,M0,M2) 1DX 1729
DO 290 K = 1,IZM 1DX 1730
ITEMP1 = M2(K) 1DX 1731
290 CO(IHS,ITEMP1) = CO(IHS,ITEMP1) + T7 1DX 1732
GO TO 320 1DX 1733
300 DO 310 I=1,IM 1DX 1734
N2(I,IGV) = .0 1DX 1735
310 N0(I,IGV) = .0 1DX 1736
C CALCULATE FISSION RATE AND FISSION SOURCE RATE 1DX 1737
320 IF(V11) 330, 380, 330 1DX 1738
330 IF(A02) 340, 360, 340 1DX 1739
340 EB(8,IGV) = .0 1DX 1740
DO 350 I=1,IM 1DX 1741
ITEMP1=M0(I) 1DX 1742
ITEMP1=M2(ITEMP1) 1DX 1743
EB(8,IGV) = EB(8,IGV) + CO(1,ITEMP1)*N2(I,IGV)*VO(I) 1DX 1744
350 F2(I)=F2(I)+K6(IGV)*N2(I,IGV) 1DX 1745
GO TO 380 1DX 1746
360 EB(8,IGV) = .0 1DX 1747
DO 370 I=1,IM 1DX 1748
ITEMP1=M0(I) 1DX 1749
ITEMP1=M2(ITEMP1) 1DX 1750
EB(8,IGV) = EB(8,IGV) + CO(1,ITEMP1)*N2(I,IGV)*VO(I) 1DX 1751
370 F2(I)=F2(I)+CO(IHT-1,ITEMP1)*N2(I,IGV) 1DX 1752
380 IGV=IGV+1 1DX 1753
IF(IGV-IGM) 20, 20, 390 1DX 1754
390 T11 = EB(1,IGP) 1DX 1755
REWIND NCR1 1DX 1756
C 1DX 1757
C OVER-RELAX FISSION SOURCE 1DX 1758
E01 = .0 1DX 1759
E02 = .0 1DX 1760
EB(1,IGP) = .0 1DX 1761
IF(A02) 520, 580, 520 1DX 1762
C FOR ADJOINT CALCULATION, S2(I) WILL STORE ORFED F2(I) 1DX 1763
520 DO 522 I=1,IM 1DX 1764
522 S2(I) = F0(I) + ORF*(F2(I) - F0(I)) 1DX 1765
DO 540 IIG=1,IGM 1DX 1766
READ(NCR1) ((CO(I,J),I=1,ITL), J=1,MT) 1DX 1767
EB(1,IIG) = .0 1DX 1768
DO 530 I=1,IM 1DX 1769
ITEMP = M0(I) 1DX 1770
ITEMP = M2(ITEMP) 1DX 1771
EB(1,IIG) = EB(1,IIG) + CO(IHT-1,ITEMP)*F2(I)*VO(I) 1DX 1772
530 E02 = E02 + CO(IHT-1,ITEMP)*S2(I)*VO(I) 1DX 1773
540 EB(1,IGP) = EB(1,IGP) + EB(1,IIG) 1DX 1774
TEMP1 = EB(1,IGP)/E02 1DX 1775
DO 550 I=1,IM 1DX 1776
550 F0(I) = TEMP1*S2(I) 1DX 1777
REWIND NCR1 1DX 1778
GO TO 620 1DX 1779
580 DO 590 I=1,IM 1DX 1780
E01 = E01 + VO(I)*F2(I) 1DX 1781
F2(I) = F0(I) + ORF*(F2(I) - F0(I)) 1DX 1782
590 E02 = E02 + VO(I)*F2(I) 1DX 1783
TEMP1 = E01/E02 1DX 1784
DO 600 I=1,IM 1DX 1785

```

600	FO(I) = TEMP1*F2(I)	1DX 1786
	DO 610 IIG=1,IGM	1DX 1787
610	EB(1,IIG) = K6(IIG)*E01	1DX 1788
620	EB(1,IGP) = .0	1DX 1789
	EB(8,IGP) = .0	1DX 1790
	DO 640 IIG=1,IGM	1DX 1791
	EB(8,IGP) = EB(8,IGP) + EB(8,IIG)	1DX 1792
640	EB(1,IGP) = EB(1,IGP) + EB(1,IIG)	1DX 1793
	RETURN	1DX 1794
	END	1DX 1795

-





```

-ITC FOR CNNP,CNNP                                1DX 1889
SUBROUTINE CNNP (F2,K6,EB)                        1DX 1890
DIMENSION F2(1), K6(1), EB(8,1)                 1DX 1891
INCLUDE ABC                                       1DX 1892
C CONVERGENCE TESTS                               1DX 1893
IF(MAXT) 25, 25, 10                              1DX 1894
10 CALL ETIMEF(TEMP)                               1DX 1895
IF(TEMP - GLH) 25, 15, 15                        1DX 1896
15 NGOTO = 1                                       1DX 1897
NRED = 0                                           1DX 1898
WRITE(NOUT,20)                                     1DX 1899
20 FORMAT(53H1 * * RUNNING TIME EXCEEDED--FORCED CONVERGENCE * *//) 1DX 1900
GO TO 135                                          1DX 1901
25 E01=1.0-ALA                                     1DX 1902
E02=ABS(E01)                                       1DX 1903
50 IF(EB(1,IGP)) 55,130,55                        1DX 1904
55 IF (E02 - EPS) 60, 60, 70                      1DX 1905
60 CVT=1                                           1DX 1906
70 CALL CLEAR (0.0, F2, IM)                       1DX 1907
GO TO 105                                          1DX 1908
80 EV=EV+POD*EQ*E01                                1DX 1909
GO TO 170                                          1DX 1910
C FINAL PRINT                                      1DX 1911
90 NGOTO=1                                          1DX 1912
IF (I04 - 1) 95, 95, 80                          1DX 1913
95 EV=0.0                                          1DX 1914
DO 100 I=1,IGM                                    1DX 1915
100 EV=EV+K6(I)                                    1DX 1916
EV=SK7/EV                                          1DX 1917
GO TO 135                                          1DX 1918
105 IF(CVT-1) 110, 90, 110                        1DX 1919
110 IF(I04-1) 115, 120, 140                       1DX 1920
C MONITOR PRINT                                    1DX 1921
115 NGOTO=2                                        1DX 1922
GO TO 135                                          1DX 1923
120 EV=0.                                          1DX 1924
DO 125 I=1,IGM                                    1DX 1925
125 EV=EV+K6(I)                                    1DX 1926
EV=SK7/EV                                          1DX 1927
GO TO 115                                          1DX 1928
130 CALL ERRO2(6H CNNP,130,1)                     1DX 1929
135 RETURN                                         1DX 1930
140 CONTINUE                                       1DX 1931
C                                                   1DX 1932
C CALCULATE NEW PARAMETERS FOR SEARCH CALCULATIONS 1DX 1933
145 E03=ABS (ALA-LAR)                              1DX 1934
IF (LAPP) 270, 150, 270                          1DX 1935
150 IF (LAP) 230, 155, 230                        1DX 1936
155 IF (EQ) 200, 160, 200                         1DX 1937
160 IF (E03-EP5A) 175, 175, 165                  1DX 1938
C MONITOR PRINT.                                  1DX 1939
165 NGOTO=2                                        1DX 1940
RETURN                                             1DX 1941
C FINAL PRINT EXIT.                               1DX 1942
170 NGOTO=1                                        1DX 1943
RETURN                                             1DX 1944
175 LAP=ALA                                         1DX 1945
EVP=EV                                             1DX 1946
IF (E01) 185,185,180                              1DX 1947

```

180	EV=EV-EVM	1DX 1948
	GO TO 190	1DX 1949
185	EV=EV+EVM	1DX 1950
190	IF (I04-2) 195, 165, 195	1DX 1951
C	MIX X-SECS.	1DX 1952
195	NGOTO=3	1DX 1953
	RETURN	1DX 1954
200	IF (CVT) 170, 205, 170	1DX 1955
205	EV=EV+POD*EQ*E01	1DX 1956
210	IF ((LAPP-1.0)/(LAP-1.0)) 215, 190, 190	1DX 1957
215	TEMP1=AMIN1(EVP, EVPP)	1DX 1958
	IF (EV-TEMP1) 220, 225, 225	1DX 1959
220	EV=(EVPP+EVP)/2.	1DX 1960
	GO TO 190	1DX 1961
225	TEMP1=AMAX1(EVP, EVPP)	1DX 1962
	IF (EV-TEMP1) 190, 220, 220	1DX 1963
230	IF (E03-EPSA) 235, 235, 165	1DX 1964
235	EQ=(EVP-EV)/(LAP-ALA)	1DX 1965
240	IF (CNT) 260, 245, 260	1DX 1966
245	IF (E02-LAL) 265, 265, 250	1DX 1967
250	IF (E02-LAH) 260, 260, 255	1DX 1968
255	E01=SIGN (LAH, E01)	1DX 1969
260	LAPP=LAP	1DX 1970
	LAP=ALA	1DX 1971
	EVPP=EVP	1DX 1972
	EVP=EV	1DX 1973
	GO TO 205	1DX 1974
265	CNT=1	1DX 1975
	LAP=0.0	1DX 1976
	LAPP=0.0	1DX 1977
	GO TO 205	1DX 1978
270	IF (E03-EPSA) 275, 275, 165	1DX 1979
C	CALCULATE QUADRATIC COEFFICIENTS.	1DX 1980
275	TEMP1=EVP-EV	1DX 1981
	TEMP2=EVPP-EV	1DX 1982
	TEMP3=EVPP-EVP	1DX 1983
	TEMP4=TEMP1*(EVP+EV)	1DX 1984
	TEMP5=-TEMP2*(EV+EVPP)	1DX 1985
	TEMP6=TEMP3*(EVPP+EVP)	1DX 1986
	DENOM=TEMP3*TEMP2*TEMP1	1DX 1987
	EQA=((LAPP-1.0)*TEMP1*EVP*EV-(LAP-1.0)*TEMP2	1DX 1988
	1*EV*EVPP+(ALA-1.0)*TEMP3*EVPP*EVP)/DENOM	1DX 1989
	EQB=-((LAPP*TEMP4+LAP*TEMP5+ALA*TEMP6)/DENOM	1DX 1990
	EQC=(LAPP*TEMP1-LAP*TEMP2+ALA*TEMP3)/DENOM	1DX 1991
	DISCR=EQB*EQB-4.0*EQA*EQC	1DX 1992
	IF (DISCR) 235, 280, 280	1DX 1993
280	IF (E02-LAL) 265, 265, 285	1DX 1994
285	TEMP1=EQC+EQB	1DX 1995
	TEMP=SQRT (DISCR)	1DX 1996
	EQ=1.0/(EQB+EV*TEMP1)	1DX 1997
	LAPP=LAP	1DX 1998
	LAP=ALA	1DX 1999
	EVPP=EVP	1DX 2000
	EVP=EV	1DX 2001
	EV1=(TEMP-EQB)/TEMP1	1DX 2002
	EV2=-((TEMP+EQB)/TEMP1	1DX 2003
	EVA=ABS (EV-EV1)	1DX 2004
	EV2=ABS (EV-EV2)	1DX 2005
	IF (EVA-EVB) 290, 290, 295	1DX 2006

290 EV=EV1  
GO TO 210  
295 EV=EV2  
GO TO 210  
END

1DX 2007  
1DX 2008  
1DX 2009  
1DX 2010  
1DX 2011  
-



```

-ITC FOR FINPR,FINPR                                1DX 2012
  SUBROUTINE FINPR(F2,PA,C0,M0,M2,N2,R1,R4,A0,V0,F0,JTL,JIM,EB,CXS, 1DX 2013
1      JIP,JIGM,PHI,JZM,C2,JGM,GAM)                  1DX 2014
  DIMENSION F2(1), PA(1), C0(JTL,1), M0(1), M2(1), N2(JIM,1), 1DX 2015
1      R1(1), R4(1), A0(1), V0(1), F0(1), EB(8,1)    1DX 2016
2      , CXS(JIP,JIGM,1), PHI(JZM,1), C2(JZM,1), GAM(1) 1DX 2017
  INCLUDE ABC                                        1DX 2018
C  FINAL PRINT                                       1DX 2019
  CALL MONPR                                         1DX 2020
C  COMPUTE ZONE FLUXES                              1DX 2021
  DO 10 IIG=1,IGM                                    1DX 2022
  DO 5  KZ=1,IZM                                      1DX 2023
5  PHI(KZ,IIG) = 0.0                                  1DX 2024
  DO 10 I=1,IM                                       1DX 2025
  KZ = M0(I)                                         1DX 2026
10  PHI(KZ,IIG) = PHI(KZ,IIG) + N2(I,IIG)*V0(I)    1DX 2027
  IF(NRED) 20,20,999                                  1DX 2028
C  BALANCE TABLES                                  1DX 2029
20  CALL NBAL(N2,C0,V0,CXS,M0,M2,EB,JTL,JIM,JIP,JIGM) 1DX 2030
  DO 21 I=1,IM                                       1DX 2031
  F2(I) = .0                                         1DX 2032
21  PA(I) = .0                                        1DX 2033
  TEMP = 0.0                                         1DX 2034
  IF(I04 - 5) 23,25,23                               1DX 2035
23  IF(BUCK) 24,26,24                                1DX 2036
24  TEMP = BUCK                                       1DX 2037
  GO TO 26                                           1DX 2038
25  TEMP = EV                                         1DX 2039
26  DO 38 IIG=1,IGM                                    1DX 2040
  READ(NCR1) ((C0(II,J), II=1,ITL), J=1,MT)        1DX 2041
  IF(NCRUN) 37,37,27                                  1DX 2042
27  IF(TEMP) 29,355,29                               1DX 2043
29  DO 35 MTZ =1,MT                                    1DX 2044
  DO 34 KZ=1,IZM                                      1DX 2045
  IF(M2(KZ) - MTZ) 34,30,34                          1DX 2046
30  TEMP1 = (TEMP*GAM(KZ))/(3.*C0(4,MTZ))           1DX 2047
  C0(2,MTZ) = C0(2,MTZ) - TEMP1                     1DX 2048
  C0(5,MTZ) = C0(5,MTZ) + TEMP1                     1DX 2049
  GO TO 35                                           1DX 2050
34  CONTINUE                                         1DX 2051
35  CONTINUE                                         1DX 2052
  WRITE(NSCRAT) ((C0(I,J), I=1,ITL), J=1,MT)        1DX 2053
355  DO 36 KZ=1,IZM                                   1DX 2054
  ITEMP = M2(KZ)                                     1DX 2055
36  C2(KZ,IIG) = C0(4,ITEMP)                         1DX 2056
37  DO 38 I=1,IM                                       1DX 2057
  ITEMP = M0(I)                                       1DX 2058
  ITEMP = M2(ITEMP)                                  1DX 2059
C  POWER                                             1DX 2060
  F2(I) = F2(I) + C0(1,ITEMP)*N2(I,IIG)*1000.*TSD  1DX 2061
C  TOTAL FLUX                                       1DX 2062
38  PA(I) = PA(I) + N2(I,IIG)                       1DX 2063
  IF(NCRUN) 42, 42, 39                               1DX 2064
39  IF(TEMP) 40, 42, 40                              1DX 2065
40  REWIND NCR1                                       1DX 2066
  REWIND NSCRAT                                       1DX 2067
  ITEMP1 = NSCRAT                                     1DX 2068
  NSCRAT = NCR1                                       1DX 2069
  NCR1 = ITEMP1                                       1DX 2070

```

## E-47

```

42  WRITE(NOUT,45)
45  FORMAT(132H1      ZONE      RADIUS      AVG RADIUS      1DX 2071
      1EA          VOLUME      TOTAL FLUX      POWER      FISSI1DX 2072
2ON SOURCE /
3    132H          (CM)          (CM)          (CM)1DX 2073
42)          (CM3)          (N/CM2-SEC)      (MWT/LITER)      1DX 2074
5          /)          1DX 2075
      DO 50 I=1,IM          1DX 2076
      ITEMP = M0(I)          1DX 2077
50  WRITE(NOUT,60) I,ITEMP,R1(I),R4(I), A0(I),V0(I),PA(I),F2(I),F0(I)1DX 2078
60  FORMAT(I4,I6,1PE16.5,1P6E17.5)          1DX 2079
      WRITE(NOUT,70) IP, R1(IP), A0(IP)          1DX 2080
70  FORMAT(I4,10X,1PE12.5,20X,1PE14.5)          1DX 2081
      IF(NPRT-1) 160, 75, 75          1DX 2082
75  WRITE(NOUT,80)          1DX 2083
80  FORMAT(30H1FLUX BY GROUP AND SPACE POINT//)          1DX 2084
      DATA GROUP/6H GROUP/          1DX 2085
      DO 150 IIG=1,IGM,6          1DX 2086
      LL = IIG + 5          1DX 2087
      IF(LL - IGM) 100,100,90          1DX 2088
90  LL = IGM          1DX 2089
100 WRITE(NOUT,110) ((GROUP,K), K=IIG,LL)          1DX 2090
110 FORMAT(26H0      ZONE      AVG RAD ,      6(7X,A6,I4))          1DX 2091
      WRITE(NOUT,125)          1DX 2092
125 FORMAT(1H )          1DX 2093
      DO 130 I=1,IM          1DX 2094
      ITEMP = M0(I)          1DX 2095
130 WRITE(NOUT,60) I, ITEMP, R4(I), (N2(I,K), K=IIG,LL)          1DX 2096
      IF(LL - IGM) 150,160,160          1DX 2097
150 CONTINUE          1DX 2098
160 IF(NPUN) 200,250,170          1DX 2099
170 PUNCH 180, ((N2(I,IIG), I=1,IM), IIG=1,IGM)          1DX 2100
180 FORMAT(6(3X,E9.4))          1DX 2101
200 REWIND NCR1          1DX 2102
999 RETURN          1DX 2103
      END          1DX 2104

```



```

-ITC FOR GRAM,GRAM                                1DX 2155
SUBROUTINE GRAM(MASS, VOL, ATW, HOLN, JIM, M0, M2, V0, 1DX 2156
1 IO, I1, I2, JML, I3, J1)                          1DX 2157
  INCLUDE ABC                                       1DX 2158
  DIMENSION MASS(JML,1), VOL(1), ATW(1), HOLN(1), M0(1), 1DX 2159
1 M2(1), V0(1), IO(1), I1(1), I2(1), I3(1), J1(1) 1DX 2160
C CALCULATE MATERIAL INVENTORIES                  1DX 2161
WRITE(NOUT,10) (ID(I), I=1,11)                    1DX 2162
10 FORMAT(1H1,11A6///)                             1DX 2163
WRITE(NOUT,20)                                      1DX 2164
20 FORMAT(45H MATERIAL INVENTORY (KILOGRAMS) FOR EACH ZONE / ) 1DX 2165
  CALL CLEAR(0.0,VOL,IZM)                          1DX 2166
  ITEMP = ML*IZM                                    1DX 2167
  CALL CLEAR(0.0,MASS,ITEMP)                       1DX 2168
  DO 30 I = 1, IM                                   1DX 2169
  K = M0(I)                                         1DX 2170
30 VOL(K) = VOL(K) + V0(I)*.001                    1DX 2171
  DO 39 M=1,M01                                     1DX 2172
  I3(M) = I2(M)                                     1DX 2173
  IF(IO(M) - I1(M)) 39,35,39                        1DX 2174
35 IF(I2(M)) 39,36,39                               1DX 2175
36 DO 38 MM=1,M                                     1DX 2176
  IF(IO(M) - IO(MM)) 38,37,38                      1DX 2177
37 I3(MM) = I2(MM)*FV                              1DX 2178
38 CONTINUE                                         1DX 2179
39 CONTINUE                                         1DX 2180
  DO 190 N = 1, IZM                                 1DX 2181
  NN = M2(N)                                        1DX 2182
  DO 190 M = 1,M01                                  1DX 2183
  IF(IO(M) - NN) 190, 40, 190                      1DX 2184
40 IF(M - MM01) 42,42,44                           1DX 2185
42 L = J1(M)                                        1DX 2186
  GO TO 48                                          1DX 2187
44 L = I1(M)                                        1DX 2188
48 IF(L - ML) 170, 170, 50                         1DX 2189
50 NNA = L                                          1DX 2190
  IF(L - IO(M)) 130,190, 130                      1DX 2191
130 DO 160 MAA = 1, M01                             1DX 2192
  IF(IO(MAA) - NNA) 160, 140, 160                 1DX 2193
140 IF(MAA - MM01) 142,142,144                    1DX 2194
142 L = J1(MAA)                                    1DX 2195
  GO TO 148                                        1DX 2196
144 L = I1(MAA)                                    1DX 2197
148 IF(L) 160, 160, 150                           1DX 2198
150 E01 = I3(MAA)*I3(M)                           1DX 2199
  MASS(L,N) = ((E01*ATW(L)*VOL(N))/ .6023) + MASS(L,N) 1DX 2200
160 CONTINUE                                       1DX 2201
  GO TO 190                                        1DX 2202
170 IF(L) 190, 190, 180                           1DX 2203
180 E01 = I3(M)                                    1DX 2204
  MASS(L,N) = ((E01*ATW(L)*VOL(N))/ .6023) + MASS(L,N) 1DX 2205
190 CONTINUE                                       1DX 2206
  DATA ZONE/6H ZONE /                             1DX 2207
  DO 270 L = 1, IZM, 5                             1DX 2208
  LL = L + 4                                       1DX 2209
  IF(LL - IZM) 210, 210, 200                      1DX 2210
200 LL = IZM                                       1DX 2211
210 WRITE(NOUT,220) ((ZONE, K), K=L, LL)          1DX 2212
220 FORMAT(/28H MATERIAL ATOMIC WT. ,3X, 5(A6,I2,12X)) 1DX 2213

```

## E-50

230	WRITE(NOUT,230) (VOL(K), K = L, LL)	1DX 2214
	FORMAT(28X, 5(1PE8.3, 7H LITERS, 5X))	1DX 2215
	WRITE(NOUT,240)	1DX 2216
240	FORMAT(1H )	1DX 2217
	DO 250 K = 1, ML	1DX 2218
250	WRITE(NOUT,260) K, HOLN(K), ATW(K), (MASS(K, I), I = L, LL)	1DX 2219
260	FORMAT( 13,3X, A6, F13.3, 1X, 1PE13.3, 1P4E20.3)	1DX 2220
	IF(LL - IZM) 270, 280, 280	1DX 2221
270	CONTINUE	1DX 2222
280	RETURN	1DX 2223
	END	1DX 2224

-

```

-ITC FOR CRUNCH,CRUNCH                                1DX 2225
SUBROUTINE CRUNCH(PHI,N2,VO,K8,PHJ,NPN,K7,C1,C0,NFP,NZN,MO,HOLN, 1DX 2226
1          ATW,JZM,JTLP,NV,KCR,JTL,JJM,VOL,C2,JGM,K6,      1DX 2227
2          ALPH,ALPS)                                     1DX 2228
  DIMENSION PHI(JZM,1), N2(JJM,1), VO(1), K8(1), PHJ(JZM,1),NPN(1),1DX 2229
1K7(1), C1(JTLP,KCR,1), C0(JTL,1), NFP(1), NZN(1), MO(1), HOLN(1), 1DX 2230
2ATW(1), NV(1), VOL(1), C2(JZM,1),      K6(1), ALPH(JZM,1),    1DX 2231
3ALPS(JZM,1)                                             1DX 2232
  REAL K8                                               1DX 2233
C  C1(ITLP,NCR,MT) COLLAPSED CROSS SECTION ARRAY--STARTS AT A(LN2) 1DX 2234
C  K8(NCR) COLLAPSED FISSION SPECTRUM--STARTS AT A(LV7)      1DX 2235
  INCLUDE ABC                                           1DX 2236
C  CRUNCH CALCULATES, PRINTS, AND PUNCHES COLLAPSED CROSS SECTIONS 1DX 2237
C  CALCULATE ZONE AVERAGED FLUXES FOR EACH FINE GROUP      1DX 2238
  DO 30 KZ=1,IZM                                        1DX 2239
  PHI(KZ,IGP) = 0.0                                     1DX 2240
  DO 30 IIG=1,IGM                                       1DX 2241
  PHI(KZ,IIG) = PHI(KZ,IIG)*.001/VOL(KZ)              1DX 2242
30  PHI(KZ,IGP) = PHI(KZ,IGP) + PHI(KZ,IIG)           1DX 2243
C  CALC. FISSION SPECTRUM AND ZONE AV. FLUXES FOR EACH COLLAPSED GROUP 1DX 2244
  DO 50 JJG=1,NCR                                       1DX 2245
  K8(JJG) = 0.0                                         1DX 2246
  DO 50 KZ=1,IZM                                       1DX 2247
50  PHJ(KZ,JJG) = 0.0                                   1DX 2248
  IIG = 0                                               1DX 2249
  DO 100 JJG=1,NCR                                      1DX 2250
  ITEMP = NPN(JJG)                                     1DX 2251
  DO 100 K=1,ITEMP                                     1DX 2252
  IIG = IIG + 1                                        1DX 2253
  K8(JJG) = K8(JJG) + K7(IIG)                        1DX 2254
  NV(IIG) = JJG                                       1DX 2255
  DO 100 KZ=1,IZM                                      1DX 2256
100  PHJ(KZ,JJG) = PHJ(KZ,JJG) + PHI(KZ,IIG)         1DX 2257
  WRITE(NOUT,110) (KZ,PHI(KZ,IGP),VOL(KZ),KZ=1,IZM)  1DX 2258
110  FORMAT(21H1ZONE AVERAGED FLUXES//
1          4OH0ZONE          FLUX          VOLUME /      1DX 2260
2          4OH              (N/CM2-SEC)      (LITERS) //(I4,1P2E18.5))1DX 2261
115  WRITE(NOUT,120)                                    1DX 2262
120  FORMAT(///62H COLLAPSED FISSION FRACTIONS AND ZONE AVERAGED FLUXE1DX 2263
1S BY GROUP//)
  DATA ZONE/6H ZONE/                                  1DX 2265
  DO 180 KZ=1,IZM,6                                    1DX 2266
  ITEMP = KZ + 5                                       1DX 2267
  IF(ITEMP - IZM) 140,140,130                          1DX 2268
130  ITEMP = IZM                                       1DX 2269
140  WRITE(NOUT,150) ((ZONE, K), K=KZ,ITEMP)           1DX 2270
150  FORMAT(20H GROUP      FISSION FRACT.      , 6(A6, I3, 9X)) 1DX 2271
  WRITE(NOUT,160)                                       1DX 2272
160  FORMAT(1H )                                         1DX 2273
170  FORMAT(I4, 1P7E18.5)                               1DX 2274
  DO 180 JJG=1,NCR                                     1DX 2275
180  WRITE(NOUT,170) JJG,K8(JJG),(PHJ(K,JJG), K=KZ,ITEMP) 1DX 2276
C  CALCULATION OF COLLAPSED SIGF, SIGA, NUSIGF, AND SIGTR    1DX 2277
190  IF(NCR - IGM) 195, 192, 192                       1DX 2278
192  DO 194 IIG=1,IGM                                   1DX 2279
  READ(NCR1) ((C0(I,J),I=1,ITL),J=1,MT)              1DX 2280
  DO 194 M=1,NFGM                                       1DX 2281
  ITEMP1 = NFP(M)                                       1DX 2282
  DO 194 I=1,ITL                                       1DX 2283

```

```

194 C1(I,IIG,M) = C0(I,ITEMP1) 1DX 2284
    GO TO 955 1DX 2285
195 DO 200 JJG=1,NCR 1DX 2286
    DO 200 KZ=1,IZM 1DX 2287
200 ALPS(KZ,JJG) = 0.0 1DX 2288
    IIG = 0 1DX 2289
    DO 235 JJG = 1, NCR 1DX 2290
    ITEMP = NPN(JJG) 1DX 2291
    DO 235 K=1,ITEMP 1DX 2292
    IIG = IIG + 1 1DX 2293
    DO 235 KZ=1,IZM 1DX 2294
    ALPH(KZ,IIG) = PHI(KZ,IIG)/C2(KZ,IIG) 1DX 2295
235 ALPS(KZ,JJG) = ALPS(KZ,JJG) + ALPH(KZ,IIG) 1DX 2296
    IIG = 0 1DX 2297
    DO 290 JJG=1,NCR 1DX 2298
    DO 240 JT=1,ITLP 1DX 2299
    DO 240 M=1,NFGM 1DX 2300
240 C1(JT,JJG,M) = 0.0 1DX 2301
    ITEMP = NPN(JJG) 1DX 2302
    DO 290 K=1,ITEMP 1DX 2303
    IIG = IIG + 1 1DX 2304
    READ(NCR1) ((C0(I,J),I=1,ITL),J=1,MT) 1DX 2305
    DO 290 M=1,NFGM 1DX 2306
    ITEMP1 = NFP(M) 1DX 2307
    KZ = NZN(M) 1DX 2308
    DO 250 JT=1,3 1DX 2309
250 C1(JT,JJG,M) = C1(JT,JJG,M) + C0(JT,ITEMP1)*PHI(KZ,IIG)/ 1DX 2310
    PHJ(KZ,JJG) 1DX 2311
1 IF(NTR) 280, 280, 260 1DX 2312
260 C1(4,JJG,M) = C1(4,JJG,M) + PHI(KZ,IIG)/C0(4,ITEMP1) 1DX 2313
    GO TO 290 1DX 2314
280 C1(4,JJG,M) = C1(4,JJG,M) + C0(4,ITEMP1)*ALPH(KZ,IIG)/ALPS(KZ,JJG) 1DX 2315
290 CONTINUE 1DX 2316
    IF(NTR) 310, 310, 295 1DX 2317
295 DO 300 JJG=1,NCR 1DX 2318
    DO 300 M=1,NFGM 1DX 2319
    KZ = NZN(M) 1DX 2320
    ITEMP = NPN(JJG) 1DX 2321
300 C1(4,JJG,M) = PHJ(KZ,JJG)/C1(4,JJG,M) 1DX 2322
310 REWIND NCR1 1DX 2323
C CALCULATION OF SCATTERING MATRIX 1DX 2324
    IF(NXCMP) 900, 900, 330 1DX 2325
330 IIG = 0 1DX 2326
    DO 500 JJG=1,NCR 1DX 2327
    ITEMP = NPN(JJG) 1DX 2328
    DO 500 K=1,ITEMP 1DX 2329
    READ(NCR1) ((C0(I,J),I=1,ITL),J=1,MT) 1DX 2330
    IIG = IIG + 1 1DX 2331
    DO 500 M=1,NFGM 1DX 2332
    ITEMP1 = NFP(M) 1DX 2333
    KZ = NZN(M) 1DX 2334
    DO 500 LT=1,NXCM 1DX 2335
    IT = LT + 5 1DX 2336
    ING = IIG - LT 1DX 2337
    JNG = NV(ING) 1DX 2338
    JT = JJG + 5 - JNG 1DX 2339
    IF(JT-5) 500, 500, 480 1DX 2340
480 IF(ING) 500, 500, 490 1DX 2341
490 C1(JT,JJG,M) = C1(JT,JJG,M) + C0(IT,ITEMP1)*PHI(KZ,ING)/ 1DX 2342

```

500	1	CONTINUE	PHJ(KZ,JNG)					1DX 2343
		IF(NXINP)	900, 900, 530					1DX 2344
530		IF(NXCMP - NXINP)	900, 900, 540					1DX 2345
540		ITEMP = NXINP + 5						1DX 2346
		ITEMP1 = NXINP + 6						1DX 2347
		DO 570	M=1,NFGM					1DX 2348
		DO 570	JJG=1,NCR					1DX 2349
		DO 570	JT=ITEMP1,ITLP					1DX 2350
		KKG = JJG + JT + 1 - ITEMP1						1DX 2351
		IF(KKG - NCR)	550, 550, 570					1DX 2352
550		C1(ITEMP,JJG,M) = C1(ITEMP,JJG,M) + C1(JT,KKG,M)						1DX 2353
570		CONTINUE						1DX 2354
		ITLP = NXINP + 5						1DX 2355
		NXCMP = NXINP						1DX 2356
C		CALCULATION OF SIGG						1DX 2357
900		DO 950	JJG=1,NCR					1DX 2358
		DO 950	M=1,NFGM					1DX 2359
		TEMP = .0						1DX 2360
		DO 930	L=1,NXCMP					1DX 2361
		JT = L + 5						1DX 2362
		J = JJG + L						1DX 2363
		IF(J - NCR)	930, 930, 950					1DX 2364
930		TEMP = TEMP + C1(JT,J,M)						1DX 2365
950		C1(5,JJG,M) = C1(4,JJG,M) - C1(2,JJG,M) - TEMP						1DX 2366
C		PRINT COLLAPSED CROSS SECTIONS						1DX 2367
955		WRITE(NOUT,960)						1DX 2368
960		FORMAT(/25H1COLLAPSED CROSS SECTIONS)						1DX 2369
		DO 980	JJG=1,NCR					1DX 2370
		WRITE(NOUT,970)	JJG					1DX 2371
970		FORMAT(6HUGROUP,I3,11X,82H SIGF	SIGA	NUSIGF	SIGTR			1DX 2372
	1	GXG	G-1XG	G-2XG	. . .)			1DX 2373
		DO 980	M=1,NFGM					1DX 2374
		ITEMP1 = NFP(M)						1DX 2375
		KZ = NZN(M)						1DX 2376
980		WRITE(NOUT,990)	ITEMP1, KZ, (C1(JT,JJG,M),JT=1,ITLP)					1DX 2377
990		FORMAT( 4H MAT,I3,1X,5H ZONE,I3,10E11.5/ (16X,10E11.5))						1DX 2378
		IF(IPUN )	1100, 1100, 1000					1DX 2379
C		PUNCH COLLAPSED CROSS SECTIONS						1DX 2380
1000		DO 1070	M=1,NFGM					1DX 2381
		Z=1.0						1DX 2382
		PUNCH 1040,NFP(M), Z, NCR						1DX 2383
1040		FORMAT(3HMAT,I3,F6.2,10X,I3,7H GROUPS)						1DX 2384
1045		IF(IPUN - 1)	1050,1050,1060					1DX 2385
1050		DO 1055	JJG=1,NCR					1DX 2386
1055		PUNCH 1080, (C1(JT,JJG,M),JT=1,ITLP)						1DX 2387
		GO TO 1070						1DX 2388
1060		WRITE(16) ((C1(JT,JJG,M), JT=1,ITLP), JJG=1,NCR)						1DX 2389
1070		CONTINUE						1DX 2390
1080		FORMAT(6E12.6)						1DX 2391
		IF(IPUN-2)	1100,1090,1100					1DX 2392
1090		CALL NTRAN(16,9,11)						1DX 2393
1100		REWIND NCR1						1DX 2394
		RETURN						1DX 2395
		END						1DX 2396



DISTRIBUTION

No. of  
Copies  
OFFSITE

- 5      AEC Division of Technical Information Extension
- 30      AEC Division of Reactor Development and Technology  
          M Shaw, Director, RDT  
          Asst Dir for Nuclear Safety  
          Analysis & Evaluation Br, RDT:NS  
          Environmental & Sanitary Engrg Br, RDT:NS  
          Research & Development Br, RDT:NS  
          Asst Dir for Plant Engrg, RDT  
          Applications & Facilities Br, RDT:PE  
          Components Br, RDT:PE  
          Instrumentation & Control Br, RDT:PE  
          Liquid Metal Systems Br, RDT:PE  
          Asst Dir for Program Analysis, RDT  
          Asst Dir for Project Mgmt, RDT  
          Liquid Metals Projects Br, RDT:PM  
          FFTF Project Manager, RDT:PM (3)  
          Asst Dir for Reactor Engrg  
          Control Mechanisms Br, RDT:RE  
          Core Design Br, RDT:RE (2)  
          Fuel Fabrication Br, RDT:RE  
          Fuel Handling Br, RDT:RE  
          Reactor Vessels Br, RDT:RE  
          Asst Dir for Reactor Tech  
          Coolant Chemistry Br, RDT  
          Fuel Recycle Br, RDT  
          Fuels & Materials Br, RDT:RT  
          Reactor Physics Br, RDT:RT  
          Special Technology Br, RDT:RT  
          Asst Dir for Engrg Standards, RDT
- 1      AEC Chicago Patent Group  
          G. H. Lee, Chief
- 1      AEC Idaho Operations Office  
          Nuclear Technology Division  
          C. W. Bills, Director
- 1      AEC San Francisco Operations Office  
          Director, Reactor Division
- 4      AEC Site Representatives  
          Argonne National Laboratory  
          Atomics International  
          Atomic Power Development Associates, Inc.  
          General Electric Company

No. of  
Copies

3     Argonne National Laboratory  
      R. A. Jaross  
      N. J. Swanson  
      LMFBR Program Office

1     Atomic Power Development Associates, Inc.  
      Document Librarian

7     Atomics International  
      D. J. Crockeram (5)  
      Liquid Metal Information Center  
      J. J. Droher (2)

2     Babcock & Wilcox Company (AEC)  
      S. H. Esleeck  
      G. B. Garton

5     Bechtel Corporation  
      J. J. Teachnor, Project Administrator, FFTF

1     BNW Representative  
      N. A. Hill (ZPR-III)

1     Combustion Engineering  
      1000 MW<sub>e</sub> Follow-On Study  
      W. P. Staker, Project Manager

5     General Electric Company  
      Advanced Products Operation  
      Karl Cohen (3)  
      Bertram Wolfe  
      Nuclear Systems Programs  
      D. H. Ahmann

2     Gulf General Atomic Inc.  
      General Atomic Division  
      D. Coburn

1     Idaho Nuclear Corporation  
      D. R. deBoisblanc

No. of  
Copies

- 1     Stanford University  
Nuclear Division  
Division of Mechanical Engineering  
R. Sher
- 1     United Nuclear Corporation  
Research and Engineering Center  
R. F. DeAngelis
- 10    Westinghouse Electric Corporation  
Atomic Power Division  
Advanced Reactor Systems  
J. C. R. Kelly

ONSITE-HANFORD

- 1     AEC Chicago Patent Group  
R. K. Sharp (Richland)
- 4     AEC RDT Site Representative - BNW  
P. G. Holsted
- 3     AEC Richland Operations Office  
  
J. M. Shivley
- 3     Battelle Memorial Institute
- 1     Bechtel Corporation  
D. H. Weiss (Richland)
- 1     Westinghouse Electric Corporation  
R. Strzelecki (Richland)
- 84    Battelle-Northwest  
S. J. Altschuler  
S. O. Arneson  
E. R. Astley  
Q. L. Baird  
J. L. Baker  
R. A. Bennett  
E. T. Boulette  
W. L. Bunch  
G. J. Busselman

Battelle-Northwest (contd)

J. L. Carter  
J. C. Cochran  
D. L. Condotta  
F. G. Dawson  
S. L. DeMyer  
E. A. Eschbach  
E. A. Evans  
R. W. Hardie (25)  
R. E. Heineman  
R. H. Holeman  
P. L. Hofmann  
R. B. Kidman  
D. D. Lanning  
R. C. Liikala  
C. W. Lindenmeier  
W. W. Little  
L. L. Maas  
D. R. Marr  
D. D. Matsumoto  
W. B. McDonald  
J. S. McMahan  
J. V. Nelson  
L. D. O'Dell  
H. C. Ripfel  
W. E. Roake  
R. F. Schenter  
L. C. Schmid  
J. R. Sheff  
R. J. Squires  
K. B. Stewart  
A. E. Waltar  
W. R. Young  
FFTF Files (10)  
Technical Publications (2)  
Technical Information Files (5)  
Legal (2)