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FLUX DISTRIBUTIONS AND LEAKAGE CURRENTS FOR SRE, P-16
By
R. Balent
A. Two-Group Flux Distribution

Using the lattice constants given in NaA-SR-MEMO-700, p. 22,
TWO-GROUP, TWO-REGION CRITICALITY CALCULATIONS WERE MADE FOR 10 FT. AND

11 ft. diameter tanks. The constants bsed in these calculations are as FOLLOWS:

```
Core Constants
\(K_{0 .}=1.264\)
\(\tau=353 \mathrm{~cm}^{2}\)
\(L^{2}=212 \mathrm{~cm}^{2}\)
    Coefficient \(=D_{F}=1.113 \mathrm{~cm}\)
```

FASt Diffuspon $\quad D_{\text {TH }}=0.901$
Thermal Diffusion $\mathrm{P}=1$
COEFFICIENT $=D_{\text {Th }}=0.901 \mathrm{~cm}$
$p=0.82$

A4O CENTIMETER AXIAL REFLECTOR SAVINGS WAS ASSUMED SO THE UNREFLECTED height was taken to be 263 cm . The axial buckling term, $(\mathbb{\pi} / \mathrm{H})^{2}=1.43 \times 10^{-4}$ $\mathrm{Cm}^{-2}$, was accounted for in the radial components of the two-group equations. The 10 ft. tank required a core radius of 102 cm for criticality (l.e., diameter $=6.7 \mathrm{Ft}$ ), which correspongs to a 1.65 ft. radial reflector and a reflector savings of 42 cm . The 11 ft . tank required a core radius of 95 CM FOR CRIticality (I.E, DIAMETER $=6.2 \mathrm{Ft}$ ), WHich CORRESponds to a 2.4 ft. radial reflector and a reflector savings of 49 cm .

In the two-group, two-region criticality calculations the fluxes are assumed to go to zero at the edge of the graphite reeeector. Reference to Fig. SRDP-9-20033 dated December 18, 1953 shows that the graphite has an equivalent diameter only slightly greater than 10 feet. However, since the

CORE TANK, THERMAL AND CONCRETE SHIELDS WILL HAVE SOME REFLECTING PROPERTIES, CALCULATIONS OF THE NEUTRON LEAKAGES ARE BASED ON THE 11 FOOT DIAMETER CASE, AlSo, the calculations are for a fully loaded core. Since a higher $U^{235}$ ENRICHMENT THAN USED IN THESE CALCULATIONS WILL BE LOADED INTO THE CORE, THE REACTOR SHOULD GO CRITICAL BEFORE REACHING THE FULLY LOADED CONDITION AND THUS THE RESULTING NEUTRON LEAKAGE WILL BE REDUCED SOMEWHAT FROM THE CALCULATED VALUES.

The equations giving the flux distributions are as follows:
In CORE:

$$
\begin{aligned}
& \operatorname{NV}(\text { FASt })=A J_{0}(C G R)+C I_{0}(\beta R) \\
& N V\left(\text { THERMAL }=S_{1} A J_{0}(\alpha R)+S_{2} C I_{0}(\beta R)\right.
\end{aligned}
$$

In REFLECTOR:

$$
\begin{aligned}
& N V(F A S T)=F Z\left(\alpha_{1 R}, R\right) \\
& N V\left(T H E R M A L=G Z\left(\alpha_{2 R}, R\right)+S_{3} F Z\left(\alpha_{1 R}, R\right)\right.
\end{aligned}
$$

Where

$$
Z=1_{0}(\alpha R)-\frac{l_{0}\left(\alpha R_{0}\right)}{K_{0}\left(\alpha R_{0}\right)} \quad K_{0}(\alpha R)
$$

The numerical values for the $11^{\prime}$ tank are

$$
\begin{array}{lll}
\alpha=0.0164 & \alpha_{1 R}=0.0548 & A=1 \\
\mathcal{B}=0.0900 & \alpha_{2 R}=0.0195 & C=-0.193 \times 10^{-3} \\
S_{1}=0.5594 & & F=-0.362 \times 10^{-5} \\
S_{2}=-0.8841 & R_{0}=168 \mathrm{~cm} & G=-0.0267 \\
S_{3}=-1.313 & &
\end{array}
$$

## B. Four-group Flux Distribution

The fast group of the two-group calculation was broken down into 3 fast groups as shown in the following table along with the slowing down AREA (AGE) AND SLOWING DOWN CROSS SECTION FOR EACH GROUP:
(THIS IS A COPY)

| Group | Energy Range (ELECTRON VOLTS) | $\boldsymbol{\tau}=\underset{\left(\mathrm{cm}^{2}\right)}{\text { slowing area }}$ $\qquad$ | $\sum_{\left(\mathrm{CM}^{-1}\right)}$ |
| :---: | :---: | :---: | :---: |
| 1 | $2 \times 10^{6}$ то $3 \times 10^{4}$ | $85 \mathrm{~cm}^{2}$ | $1.312 \times 10^{-2}$ |
| 2 | $3 \times 10^{4}$ то $10^{4}$ | $22.4 \mathrm{~cm}^{2}$ | $4.9 \times 10^{-2}$ |
| 3 | $10^{4}$ то 0.058 | $245 \mathrm{~cm}^{2}$ | $0.455 \times 10^{-2}$ |

THE FLUX FOR EACH OF THE THREE FAST GROUPS CAN BE TAKEN AS A SIMPLE RATIO OF THE SLOWING DOWN AREA OF THAT GROUP TO THE TOTAL AGE TIMES THE FAST

FLUX OF THE TWO-GROUP SOLUTION. THUS:


It is assumed that all resonance capture takes place between the 3rd and THERMAL GROUPS.
C. LEAKAGE

The leakage out of the core and reflector for the 4 energy groups is given in Following table. The first column gives the leadage for the CASE WHERE THE THERMAL FLUX HAS BEEN NORMALIZED TO UNITY AT R $=0$. E. WEISNER HAS ESTIMATED THE MAXIMUM THERMAL FLUX IN THE GRAPHITE FOR A POWER LEVEL OF 20 MW to be $6 \times 10^{13}$; thUS, the SECOND COLUMN GIVES THE LEAKAGE FOR THIS POWER LEVEL.


IT IS HOPED THE ABOVE WILL BE USEFUL IN SHIELDING CALCULATIONS.
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