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FLUX DISTRIBUTIONS AND LEAKAGE CURRENTS FOR SRE, P-16

BY R. BALENT

A. Two-Group FLux Distribution

Using the Lattice constants given in Naa-SR-MEMO-700, p. 22, two-group, two-region criticality calculations were made for 10 ft. and 11 ft. diameter tanks. The constants gged in these calculations are as follows:

CORE CONSTANTS	REFLECTOR	CONSTANTS
к = 1.264	T =	350 см ²
T = 353 cm ²	L ² =	4177 cm ²
L ² = 212 cm ²	D _F =	1.113
Fast Diffuspon Coefficient = D _F = 1.113 cm	D _{TH} =	0.901
THERMAL DIFFUSION COEFFICIENT= D _{RH} = 0.901 CM	Р 💻	1
P = 0.82		

A40 CENTIMETER AXIAL REFLECTOR SAVINGS WAS ASSUMED SO THE UNREFLECTED HEIGHT WAS TAKEN TO BE 263 CM. THE AXIAL BUCKLING TERM, $(\pi/H)^2 = 1.43 \times 10^{-4}$ CM⁻², WAS ACCOUNTED FOR IN THE RADIAL COMPONENTS OF THE TWO-GROUP EQUATIONS. THE 10 FT. TANK REQUIRED A CORE RADIUS OF 102 CM FOR CRITICALITY (i.e., DIAMETER = 6.7 FT), WHICH CORRESPONDS TO A 1.65 FT. RADIAL REFLECTOR AND A REFLECTOR SAVINGS OF 42 CM. THE 11 FT. TANK REQUIRED A CORE RADIUS OF 95 CM FOR CRITICALITY (i.e., DIAMETER = 6.2 FT), WHICH CORRESPONDS TO A 2.4 FT. RADIAL REFLECTOR AND A REFLECTOR SAVINGS OF 49 CM.

In the two-group, two-region criticality calculations the fluxes are assumed to go to zero at the edge of the graphite reflector. Reference to Fig. SRDP-9-20033 dated December 18, 1953 shows that the graphite has an equivalent diameter only slightly greater than 10 feet. However, since the

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CORE TANK, THERMAL AND CONCRETE SHIELDS WILL HAVE SOME REFLECTING PROPERTIES, CALCULATIONS OF THE NEUTRON LEAKAGES ARE BASED ON THE 11 FOOT DIAMETER CASE, ALSO, THE CALCULATIONS ARE FOR A FULLY LOADED CORE. SINCE A HIGHER U235 ENRICHMENT THAN USBD IN THESE CALCULATIONS WILL BE LOADED INTO THE CORE, THE REACTOR SHOULD GO CRITICAL BEFORE REACHING THE FULLY LOADED CONDITION AND THUS THE RESULTING NEUTRON LEAKAGE WILL BE REDUCED SOMEWHAT FROM THE CALCULATED VALUES.

THE EQUATIONS GIVING THE FLUX DISTRIBUTIONS ARE AS FOLLOWS:

IN CORE:

Nv(fast) =
$$AJ_0$$
 (cc R) + C I_0 (β R)
Nv(thermal = S_1 A J_0 (α R) + S_2 C I_0 (β R)

IN REFLECTOR:

$$NV(FAST) = F Z(\mathcal{A}_{1R},R)$$

$$NV(THERMAL = G Z(\mathcal{A}_{2R},R) + S_3 F Z (\mathcal{A}_{1R},R)$$

WHERE

$$Z = I_o (x_R) - I_o (x_R)$$
 $K_o (x_R)$

THE NUMERICAL VALUES FOR THE 11' TANK ARE

≪ : 0.0164	~ _{1 R} = 0.0548	А	=	1
B = 0.0900		С	=	-0.193 x 10 ⁻³
S ₁ = 0.5594		F	=	-0.362 x 10 ⁻⁵
S ₂ = -0.8841	R _o = 168 cm	G	=	-0.0267
S ₃ : -1.313				

B. FOUR-GROUP FLUX DISTRIBUTION

THE FAST GROUP OF THE TWO-GROUP CALCULATION WAS BROKEN DOWN INTO 3 FAST GROUPS AS SHOWN IN THE FOLLOWING TABLE ALONG WITH THE SLOWING DOWN AREA (AGE) AND SLOWING DOWN CROSS SECTION FOR EACH GROUP:

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GROUP	Energy Range (electron volts)	C: SLOWING DOWN AREA	Page 3 \(\sum_{\cup (cm^{-1})}\)
1	2 x 10 ⁶ то 3 x 10 ⁴	85 cm ²	1.312 x 10 ²
2	3×10^{4} to 10^{4}	22.4 cm ²	4.9 x 10 ²
3	10 ^Ц то 0.058	245 см ²	0.455 x 10 ⁻²

THE FLUX FOR EACH OF THE THREE FAST GROUPS CAN BE TAKEN AS A SIMPLE RATIO OF THE SLOWING DOWN AREA OF THAT GROUP TO THE TOTAL AGE TIMES THE FAST FLUX OF THE TWO-GROUP SOLUTION. THUS:

$${}^{NV}_{1} = \frac{z_{1}}{\overline{z}} \qquad {}^{NV}_{F}$$
(Similarly for NV2 and NV3)

IT IS ASSUMED THAT ALL RESONANCE CAPTURE TAKES PLACE BETWEEN THE 3RD AND THERMAL GROUPS.

C. LEAKAGE

THE LEAKAGE OUT OF THE CORE AND REFLECTOR FOR THE 4 ENERGY GROUPS

IS GIVEN IN FOLLOWING TABLE. THE FIRST COLUMN GIVES THE LEADAGE FOR THE

CASE WHERE THE THERMAL FLUX HAS BEEN NORMALIZED TO UNITY AT R = 0.

E. WEISNER HAS ESTIMATED THE MAXIMUM THERMAL FLUX IN THE GRAPHITE FOR A

POWER LEVEL OF 20 MW TO BE 6 x 10¹³; THUS, THE SECOND COLUMN GIVES THE LEAKAGE

FOR THIS POWER LEVEL.

Energy Group	LEAKAGE OU (I.E., R = NEUTRONS/C	of core 95 cm) m ² /sec. 2	LEAKAGE OUT O (I.E., R = 16 NEUTRONS/CM ² /	
1[2 x 10 ⁶ то 3 x 10 ⁴ EV]	-9.85 x 10 ⁻³	-5.9 x 10 ¹¹	-2.74 x 10 ⁻⁴	-1.65 x 10 ¹⁰
2[3 x 10 ¹ TO 10 ¹ EV]	-2.6 x 10 ⁻³	-1.56 x 10 ¹¹	721 x 10 ⁻¹⁴	433 x 10 ¹⁰
3[10 ¹ TO 0.058 EV]	-28.4 x 10 ⁻³	-1.71 x 10 ¹²	-7.9 x 10 ⁻⁴	-4.75 x 10 ¹⁰
Thermal Group	+12.5 x 10 ⁻³	+7.53 x 10 ¹¹	-9.1 x 10 ⁻³	-5.45 x 10 ¹¹

IT IS HOPED THE ABOVE WILL BE USEFUL IN SHIELDING CALCULATIONS.

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