



UNITED STATES ATOMIC ENERGY COMMISSION CONTRACT W-7405-ENG. 36 This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the United States Atomic Energy Commission, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

Printed in the United States of America. Available from National Technical Information Service U. S. Department of Commerce 5285 Port Royal Road Springfield, Virginia 22151 Price: Printed Copy \$3.00; Microfiche \$0.95

LA-4941 UC-80 ISSUED: August 1972



Linkage of Reactor Physics Codes through Standard Interfaces

by

B. M. Carmichael J. C. Vigil

NOTICE This report was prepared as an account of work sponsored by the United States Government, Neither the United States nor the United States Atomic Energy Commission, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefunces of any information, appartus, product or process disclosed, or represents that its use would not infringe privately owned rights.

Work Supported by Division of Reactor Development and Technology

LINKAGE OF REACTOR PHYSICS CODES THROUGH STANDARD INTERFACES

Ъy

B. M. Carmichael and J. C. Vigil

ABSTRACT

Selected reactor physics codes have been linked by a set of interface files. Access to codes, free-format card input for creating or modifying the files, system-restart capability, and file-printing options are all available at any point in whatever calculation path the user chooses to prescribe.

1. INTRODUCTION AND SUMMARY

The linked set of codes described in this report is an outgrowth of the Los Alamos Scientific Laboratory's (LASL) participation in the work of the de facto Committee on Computer Code Coordination (CCCC). This Committee consisted of representatives from several laboratories selected by the AEC to study the problems of adapting reactor codes to various types of computers and the problems of interfacing reactor codes from different organizations in linked calculations. The set of prototypical codes, adopted by the Committee for exchange and interface testing, was adapted to the LASL CDC 6600 computers in a linked system under the control of a driver program. Under the driver program, the codes may be executed in any logical sequence, and the interface files may be stored at any point in a run and may be retrieved in a subsequent run. Also, any one of the interface files may be created or modified by user input at any point in a sequence of calculations.

The set of codes adopted by the CCCC, and the organizations assigned to the work of adapting the codes to the standard interfaces are:

MC² code¹ (cross-section processing) Argonne National Laboratory (ANL) ETOX code² and 1DX code³ (cross-section processing and one-dimensionel diffusion) Hanford Engineering Development Laboratory (HEDL) ANISN code⁴ (one-dimensional S_n or diffusion) Brookhaven National Laboratory (BNL)

DOT2DB code⁵ (two-dimensional S_n and diffusion) General Electric - Sunnyvile, California (GE-Sunnyvale)

DAC code^{6,7} (one-, two-, or three-dimensional perturbation) Los Alamos Scientific Laboratory (LASL) CLUB code⁸ (fuel depletion)

Oak Ridge National Laboratory (ORNL)

The standard interface files (Version I) adopted

- by the Committee for linking the above codes are: ADMNSTR (administration and control file containing dimension and option control data) GEO DIST (geometry and material distributions) SN CONS (S_n constants) INTQUANT (integral quantities, such as zone volumes and zone-averaged fluxes) MIX DATA (mixture data) MULTIGRP (multigroup cross-section data) GRP FLXS (group fluxes and currents)
 - ZONEDENS (material density by zone)

The codes cited above are well known and well documented. Readers are referred to the cited references for discussions of the algorithms solved and the detailed description of the options available. This report covers primarily the modifications related to the interfacing of the codes. These modifications include the creation of a set of code-dependent interface files and a set of service modules that provide card input, file printing, and restart capabilities. The linked system was developed to achieve a thorough verification and test of the interface files. However, the system, or components of the system, should provide capabilities useful to others. The individual codes are completely interfaced; consequently, they are readily separable from the system. Particular fixed calculation paths involving code subsets can be readily provided.

The present system of codes represents only an interim development. A new set of interface files (Version II) of greater efficiency and broader capabilities has been developed. The system is presently being adapted to these new interfaces. Other more advanced codes are also being adapted to the new interfaces and will be tested as components of the system.

11. SYS'_EM-LINKAGE PROCEDURES

Linking of the codes is accomplished through the segmentation facilities available on the CDC 6600 computer.⁹ The set of codes can be thought of as one large FORTRAN-IV program consisting of a main program (DRIVER) and many subroutines. The segmentation procedure provides a means of loading and executing the parts or segments of the large program as needed. SEGMENT calls are inserted into the program to direct the loading of segments, and SEGMENT specification statements are placed before the main program to specify the set of subroutines comprising each segment. Conversely, the SEGMENT specification statements and SEGMENT calls would be removed as a first step in converting the system of codes to the overlay systems available on other types of computers. These conversion procedures are discussed in detail in Appendix A.

The format of SEGMENT specification statements is

SEGMENT (SEG1, SUB1, SUB2, ...)

where SEG1 is the name of the segment, and SUB1, SUB2, ... are the names of the subroutines included in the segment. SEG1 is loaded using a statement of the form

CALL SEGMENT (4HFILE, LEVEL, 4HSEG1, LIB, MAP)

where

FILE is the name of the disk or tape file containing the binary object codes for the subroutines of the segment, LEVEL is the level at which the segment is loaded into core (levels 0 to 63 available), SEG1 is the name of the segment, and

LIB and MAP are flags controlling the acquisition of system library routines and printing of the loader map.

After a segment is called, control is returned to the calling routine, which may then call any of the subroutines in the called segment. Segmentation is otherwise similar to other overlay systems, except that labeled common arrays are not transmitted between segments.

The codes in the system are accessed by a series of level 1 segment calls in the DRIVER program, which resides at level 0. These calls are at the complete control of the user through card input. The codes and service modules are listed in Table I.

TABLE I REACTOR CODE SET

Call Name	<u>File</u>	Code_or_Mcdule
ETX	ETX	ETOX code for processing cross sections.
ODX	NDX	IDX code one-dimensional dif- fusion code used primarily to convert ETOX cross sections into standard multigroup file.
ANI	ANI	ANISN code one-dimensions'. S _n or diffusion code.
DOT	DOT	DOT2DB code two-dimensionfl S _n or diffusion code.
CLB	CLB	CLUB code burnup code, which may use fluxes from 1DX, ANISN, or DOT2DB.
DAC	DAC	One-, two-, or three-dimensional perturbation code, which may use total fluxes and currents or S_n angular fluxes from IDX, ANISN, or DOT2DB.
INP	SER	Service module for processing card input.
PRN	SER	Service module for printing in- terface files.
FLO	SER	Service module for retrieving files from tape for vestart.
FL1	SER	Service module for storing files on tape for restart.

Communication between codes and modules is accomplished by means of binary interface data files. The set of standard files in the system is given in Table II.

Input data other than those contained in the standard files are required by each code. Such data

TABLE II

VERSION I STANDARD FILES

<u>File Name</u>	General Contents
ADMNSTR	Administration file containing con- trol flags and dimension parameters.
SN CONS	S _n constants file.
INTQUANT	Integral quantities including derived constants, zone-averaged fluxes, and zone volumes.
GEO DIST	Geometry and material distributions.
MIX DATA	Mixture data.
GRP F',XS	Regular fluxes, currents, and angular fluxes.
ADJ FLXS	Adjoint fluxes, currents, and angular fluxes.
MULTIGRP	Reference multigroup cross sections.
PERTMTGP	Perturbed multigroup cross sections.
ZONEDENS	Zone atomic densities.

are incorporated in special files called codedependent files. These files are listed in Table III.

TABLE III

CODE-DEPENDENT FILES

File Name	Contents
ETX INFT	ETOX input
ODX INPT	1DX input
ANI INPT	ANISN input
DGT INPT	DOT2DB input
CLB INPT	CLUB input
DAC INPT	DAC input

Initially, the operation of the system may start with the assignment of three magnetic tapes. The binary code files ETX, ODX, ANI, DOT, CLB, DAC, and SER plus the DRIVER file are stacked onto one tape (CODEB). The code files are copied onto individual disk files using control cards such as

COPYBF(CODEB, ETX) REWIND(ETX) .

The second tape (tape 49) contains standard and code-dependent data files that were generated in previous runs. The third tape (tape 50) is used for storing data files at the end of the run.

At the beginning of execution of the main program, or DRIVER, the execution path is read from cards. The path must be so arranged that the input data files to a code are configured properly for execution of the code in the path. These input files may be created from cards using INP, may be retrieved from tape 49 using FLO, or may be output by the execution of another code. In the two latter cases, where files already exist, INP may be called to alter the existing files selectively. The print module PRN is usually called to print files that ware created or extensively altered by execution of INP and to obtain prints of code output files.

A simple example of a path is INP ANI PRN .

INP is used to create the input files ADMNSTR, SN CONS, GEO DIST, MIX DATA, MULTIGRP, and ANI INPUT, which define the problem to be solved by the ANISN code. PRN provides a detailed print of all the files, including the GRP FLXS file generated by ANISN.

In executing this path, the three level 1 segment calls

CALL SEGMENT (3H3ER, 1, 4HINPS, LIB, MAP)

CALL SEGMENT (3HANI, 1, 4HANIS, LIB, MAP)

CALL SEGMENT (3HSER, 1, 4UPRNS, LIB, MAP)

are performed in sequence by DRIVER. An S is appended to the segment names because the segmentation system requires the file name (ANI) and segment name (ANIS) to be different.

The CLUB code is a special case because it involves a closed-loop calculation. CLUB calls one of the flux codes periodically while performing fuel-cycle calculations. The user selects the desired flux code by card input to the CLB INPT file. All the codes called by DRIVER are loaded at level 1. Consequently, a flux code called from CLUB is loaded at a higher level. Because the flux codes are individually segmented, variable level parameters are required. The MAIN subroutine of the flux code is loaded at level I and higher level segments are referenced with respect to I. The variable I is transmitted between segments by blank common.

Many different calculation paths are conceivable A few examples are listed below:

- (1) INP ETX ODX INP DOT PRN FL1
- (2) FLO INP DOT INP DOT PRN
- (3) FLO INP ANI INP ANI DAC
- (4) INP PRN CLB .

In path (1), ETX and ODX provide a multigroup cross-section file for use in a DOT2DB problem. In

path (2), a two-dimensional diffusion calculation is performed in the first DOT call. The diffusion fluxes provide a good flux guess for a twodimensional S calculation performed under the second DOT call. INP is called between the two DOT calls to alter the ADMNSTR file from the diffusion option to the S_n option. Because of the overlay facilities in the input processor, only the input for changing the one flag is required. In path (3), regular and adjoint ANISN calculations are performed to obtain fluxes for use in DAC perturbation calculations. Path (4) provides a CLUB burnup calculation. As mentioned before, ODX, ANI, or DOT may be used to provide the fluxes for the burnup calculations. Any path can be interrupted with an FL1 call to save the files created to the given point. The path can then be resumed in a later run by using FLO to retrieve the files.

Some routine procedures do not require the flexibility provided. For such procedures, the user could be provided with a deck that already contains the input cards for the desired path. Alternatively, a DATA specification card containing a desired fixed path could replace the path input statement in DRIVER. The procedures required for converting a code or subset of codes into a freestanding fixed-calculation tocility are similarly straightforward.

Any free-standing interfaced code can be added to the system by converting it to the segmentation form without changing the existing system. However, it would ultimately be desirable to add any codedependent input required by the new code to the service modules.

An interfaced version of the MC^2 cross-section processing code is available for use with the system; however, it has not been converted for direct access by a path call. Because of the computation times involved in the MC^2 calculations, users would normally prefer to operate it as a free-standing code. The output MULTIGRP file from MC^2 , however, is written on a tape that can be accessed by the system.

III. SERVICE MODULES

There are three service modules in the code system that provide various file handling services, including creation of files from card input, printing of files, and storage and retrieval of files. These functions are discussed below.

A. Input Processor (INP)

All card input for the system except for input to MC² and the print processor is processed by INP. The detailed definition and description of the card input are given in Appendix B. The input is organized in terms of the data files listed in Tables II and III. A set of flags is read first by INP to control the handling of input for each file. The options available are:

- (1) Read entire file from cards.
- (2) Read existing file using no card input. This option normally applies only to the ADMNSTE file, which contains parameters required in the reading of other files.
- (3) Read existing file and overlay with card input to create a new file. This option is most frequently used to alter the options requested from calculational modules or to correct erroneous data. In the multigroup cross-section file, data for selected isotopes on the existing file may be altered or replaced and additional isotopes may be added.
- (4) Skip file. This option is used when an existing file requires no changes, or when the given file is not required for the selected calculation path.

When Option (3) is requested for a given file, the first record read from cards for that file contains a set of record read control flags. These flags enable the user to overlay or skip over each record as desired. Further, in the reading of an individual record from cards, words in the record may be omitted in the reading by using the skipwords option. An order on the input card of the type Sn is punched to indicate skip n words in overlaying the record.

All card input is processed through subroutine RGEN, which is a free-format card-input processor written completely in standard FORTRAN.¹⁰ This subroutine is described in a separate report.¹¹ Mumeric entries are delimited by blanks, and numerics are interpreted as integers if they contain no decimal point or exponent. Otherwise, numerics are taken to be floating-point numbers. Hollerith data are delimited by asterisks. The maximum size of a Hollerith word is six characters; however, a Hollerith string containing more than six characters is allowed, as in

INPbbbprn .

In this example, INPbbb is loaded as the first word, and PRN is loaded left-adjusted as a second word. A record must be terminated by T. A slash (/) may be used to signify an end to the data to be stored from a given card. Comments may be inserted after T or / on a card or otherwise may be set off by dollar signs (\$). Card images are printed as they are read.

In addition to the skip option cited above, repeat, nested repeat, and interpolate options are also available. The repeat option is of the form Rn(a,b,c,...) ,

where n is an integer signifying that n copies of the array contained in the parontheses are to be stored. The a,b,c,... arguments of the repeat operation may be numerics, nested repeats, and/or interpolation operations. The interpolation operation is of the form

In(a),

which directs that n interpolates, between the last word read prior to the I operation and a, be calculated and stored. The R and I specifications must contain no imbedded blanks, and commas must be used to separate items in an argument list. Several error diagnostics and associated printed comments are provided.

B. Print Processor (PRN)

This processor provides the option to print files or selected records of files at any point in the path. As in INP, a set of flags that govern the printing of files is read first. The three options available are:

- (1) Skip file.
- (2) Print entire file.
- (3) Print selected records in file.

If Option (3) is used, additional flags are read to identify which records are to be printed in the given file.

Most of the data output by the codes is contained in the 16 interface-data files. Consequently, it is planned that eventually individual codes will print only information that indicates the progress of calculations such as iteration monitor lines.

C. Storage and Retrieval of Files (FLO and FL1)

Both scorage and retrieval of files are processed by subroutine FILER. An FLO call in the path directs FILER to read the files that are stacked on tape 49 and to store them on disk as individual files. An FL1 call requests FILER to perform the inverse operation of copying the files from disk to tape 50.

A common array NFILES is used for storing the logical unit number of the files. NFILES is initialized to zero by DRIVER, and the associated logical unit numbers are stored in NFILES as files are created. NFILES is also written as the first record on tape 50 when an FL1 call is made. When files are read under FL0, a zero in NFILES signals to FILER that the associated file does not exist and is to be omitted in the transfer operation. The NFILES record on tape is also read into the NFILES array under FL0 to restore the configuration of logical units associated with the disk files. Methods used to assign logical unit numbers and buffers to the interface files are discussed in Appendix C.

D. Segmentation Structure of Service Modules

The segmentation structure of the service modules is given in Table IV. As indicated in Table I,

TABLE IV

SERVICE MODULE SEGMENT STRUCTURE

Call Name	Segment Name	Routines Included
	DRIVE1 (SEGZERC)	DRIVER, BUFOPEN, STOW, REED, RITE
INP	INPS	CDINP, SNIFF, RGEN, IFDGIT, ITGR, FLOT, SETFMT
PRN	PRNS	FPRINT, RGEN, IFDGIT, ITGR, FLOT, SETFMT, SNIFF, WOT, WOT8, WOTI
FLO,FL1	FILR	FILER

these segments, except for DRIVE1, are stored in file SER. DRIVE1 is the main segment of the system, which is loaded at level 0. This segment is prescribed in a special specification statement

SEGZERO(DRIVEJ, DRIVER, BUFOPEN, STOW, REED, RITE) . The service modules each consist of a single level 1 segment. These segments are loaded by DRIVER. Subroutines in the service modules are described in Table V.

TABLE V

SERVICE MODULE SUBROUTINES

Routine	Description		
DRIVER	Main program of system. Calls codes and service modules as directed by path pre- scribed by user input.		
BI FOPEN	Written in COMPASS assembly language. ¹² Called by DRIVER and SNIFF routines to assign buffers to files.		
STOW	Small routine for transferring array from one location in core to another. Called		

- R):ED Reads an array from a file. May be replaced to accommodate local environment
- where cther than FORTRAN I/O is required. Called by CDINP, FPRINT, and FILER.
- RITE Writes an array on a file. May also be replaced as required. Called by CDINP, FPRINT, and FILER.
- SNIFF Assigns a file number corresponding to a given file name and assigns buffers. Calls BUFOPEN and is called by CDINP. FILER, and FPRINT. For machines other than CDC 6600, SNIFF should be altered or replaced to suit local file environment.
- CDINP Main subroutine of card input module INP. Called by DRIVER and calls REED, FITE, STOW, SNIFF, and RGEN.
- NGEN Reads a free-format array from cards. Called by DRIVER, CDINP, and FPRINT and calls IFDGIT, ITGR, FLOT, and SETFMT.
- IFDGIT Identifies integer characters. Called by RGEN.
- ITGR Decodes a string of Hollerith characters to form a signed integer constant. Called by RGEN.
- FLOT Decodes a string of Hollerith characters to form a signed <u>floating point</u> constant. <u>Called by KGEN</u>.
- SETFMT Converts a positive integer less than 100 into two consecutive Hollerit' characters. Called by RGEN.
- FPRINT Main subroutine of print module PRN. Called by DRIVER and calls REED, RITE, STOW, SNIFF, RGEN, WOT, WOT8, and WOT1.
- WOT Prints one-, two-, or three-dimensional floating-point arrays. Called by FPRINT.
- WOT8 Prints up to eight one-dimensional arrays. Called by FPRINT.
- WOTI Prints one-, two-, or three-dimensional integer arrays. Called by FPRINT.
- FILER Stores and retrieves files from tape in response to FLO and FL1 path calls. Called by DRIVER and calls SNIFF, REED, RITE, and BUFOPEN.

IV. CODE DESCRIPTIONS

The seven reactor physics codes in the system are described in this section. Organization of the codes under the linked system is stressed, and the code capabilities are summarized.

A. ETOX Code

The ETOX (ENDF/B TO 1DX) code processes the Evaluated Nuclear Data File¹³ (ENDF/B) to produce multigroup constants in the Bondarenko format¹⁴ that can be used as input to the 1DX code. Both Version-I and -II ENDF/B data can be processed, but the data must be in standard binary (Mode 1) form. Petails of the computational methods used in ETOX are given in Ref. 2.

ETOX is currently restricted to a maximum of 99 energy groups, 250 resonances, and single-level Breit-Wigner representation of resolved resonances. The dimension of the common array A in the DRIVER module must be set at 35,000 to provide sufficient storage for the code. The output must be processed by the 1DX code to produce a standard MULTIGRP file.

Output from ETOX includes infinite-dilution group cross sections, inelastic-transfer matrices (P_0 with downscatter only), and resonance-shielding factors by group for specified values of temperature and σ_0 (total cross section per atom). The principal advantage of this form of output is that calculations can be made for fast reactors of various compositions and temperatures using the same set of group constants. This is done by interpolating on the resonance-shielding factors.

A segment in the code is available for creating or updating a cross-section library. The new library is written on tape 46. If an old library is to be updated, it must be assigned to tape 47. Also, the ENDF/B data file must be assigned to tape 48.

Input specifications for the ETOX code are given in Appendix B (ETX INPT data file). Input parameters must be within the ranges specified in the ETX INPT file description; otherwise, a comment is printed and the run is terminated.

The segmentation structure of the ETOX code is shown in Table VI. ETXS, the level 1 segment, is loaded by DRIVER. All level 2 segments are loaded by subroutine MAIN, level 3 segments FIL2 and FIL3 are loaded by subroutine ENDFB, and level 3 segments OVLAY21 and OVLAY22 are loaded by subroutine RESON.

TABLE VI

SEGMENTATION STRUCTURE OF ETOX CODE

Segment	Segment	
Level	Name	Subroutines Included in Segment
1	ETXS	MAIN, ERRF, ERROR, FFLUX TERP1, ZERO, RETREV, CHI, AVER, AVSEC, SNIFF
2	WLIBR	WLIB, W
2	INPTR	INPT
2	OVLAY10	ENDFB, FILE1, LOCISO, HEADR
2	OVLAY20	RESON, AVSECR, AJK, NA23, QK, QUICKW, RELIB
2	FFAC	FFACNR
2	OVLAY 30	INELAS, FILE5, INTERP, INTORP
2	OVLAY40	OUTPDX
2	PUPXS	PUPX, UPDATE, PRINT, 1NPUT, OUTPUT, NTRAN
З	FIL2	FILE2
3	FIL3	FILE3, SETUP1, TSUM, MISS, REMT, VER2, CAPSUM, WCP, TERP2
3	OVLAY21	RRES, SR, SETUFG, ROMB
3	OVLAY22	URES, SETUPJ

A simplified flow diagram for the ETOX code is shown in Fig. 1 (system library routines are not shown), and a brief description of each subroutine is given in Table VII. Two formerly free-standing codes, WLIB and PUPX, are incorporated as segments in the ETOX code. WLIB computes a table of psi and chi functions for the complex probability integral and is called at the beginning of a calculation before cross-section processing begins. PUPX creates or updates a library of cross sections and is called after all of the isotopes in the problem have been processed,

ETOX contains a number of programmed error stops; the reason for some of the error stops is stated explicitly in the output. However, many of the stops are identified only by the comment ERROR STOP N, where N may have any of the values given in Table VIII. The subroutine in which this type of error stop occurs and the condition causing the stop are given in Table VIII.

TABLE VII

DESCRIPTION OF SUBROUTINES IN ETOX CODE

Subroutine	Description	Subroutine	Description				
MAIN	Controls the overall flow of the ETOX calculation and reads the first part of the ETX INPT data file. Also loads	HEADR	Reads ENDF/B tape ID record. Called by ENDFB and calls ERROR.				
	all level 2 segments. Called by DRIVER and calls subroutines BUFOPEN, WLIB. ZERO. SNIFF. INPT. ENDFB. RESON.	LOCISO	Locates isotope on ENDF/B library tape. Called by ENDFB and calls ERROR.				
	FFACNR, INELAS, OUTPDX, CHI, and PUPX. BUFOPEN is part of the DRIVER module.	FILE1	Read's File 1 data (general informa- tion) from ENDF/B tape. Called by				
SNIFF	Assigns a file number corresponding to		ENDFB and calls ERROR.				
	a given file name and assigns a parti- cular buffer to the file. Calls BUFOPEN (see DRIVER module) and is called by MAIN FILE? FILE? FETDEV	FILE2	Reads File 2 data (resonance param- eters) from ENDF/B tape. Called by ENDFB and calls ERROR and SNIFF.				
WLIB	WCP, TSUM, RELIB, URES, INELAS, and OUTPDX. Controls calculation of psi and chi functions (W table) for the complex probability integral. Called by MAIN and calls W.	FILE3 REMT	Reads File 3 data (smooth cross sec- tions) from ENDF/B tape. Called by ENDFB and calls REMT, SNIFF, ERROR, SETUP1, TERP2, ZRRO, and TSUM. Locates a particular cross-section type on the ENDF/B tape. Called by				
				W	Computes W table. Called by WLIB.		FILES and calls CAPSUM, MISS, ERROR, VER2, and WCP.
				ZERO	Zeros out arrays in common. Called by MAIN, FILE3, TSUM, CAPSUM, RESON, SR, URES, and OUTPDX.	CAPSUM	Calculates the total capture cross section at fine-group points by sum- ming all capture components. Called
INPT	Reads remainder of ETX INPT data file and edits all the input data. Called by MAIN.		by REMT and calls TERP2 and ZERO.				
		TERP2	Interpolates a series of points accord- ing to ENDF/B specifications. Called				
ENDFB	Controls reading of ENDF/B library . tape. Also loads level 3 segments FILE2 and FILE3. Called by MAIN and calls HEADR, LOCISO, FILE1, FILE2, and FILE3.		by FILE3 and CAPSUM and calls ERROR and TERP1.				



Fig. 1. Simplified flow diagram for ETOX code.

TABLE VII (cont.)

Subroutine	Description	Subi
TERP 1	Interpolates one point according to ENDF/B specifications. Called by TERP2, VER2, and URES and calls ERROR.	SETU
MISS	Prints out any missing cross-section types in File 3 of ENDF/B library tape. Called by REMT.	
VER2	Computes inelastic-scattering cross sections from File 3 data. Called by REMT and calls TERP1, FFLUX, and ERRF.	ROME
FFLUX	Computes fission flux spectrum. Called by SETUP1, VER2, INELAS, and OUTPDX.	SR
ERRF	Calculates the error function. Called by CHI, VER2, and INELAS.	
WCP	Writes capture cross sections on scratch file. Called by REMT and calls SNIFF and ERROR.	URES
SETUP1	Sets up the fine-group energy mesh. Called by FILE3 and calls ERROR and FFLUX.	SETU
TSUM	Sums cross sections over fine groups. Called by FILE3 and calls ZERO, SNIFF, RETREV, and ERKOR.	AJK
RETREV	Reads File 3 smooth cross sections previously written on a scratch file by FILE3. Called by TSUM, AVSEC, AVSECR, SETUFG, RRES, SETUPU, URES,	FFAC
	FFACNR, and INELAS and cails SNIFF and ERROR.	1120
RESON	Controls resonance region calculations. Also loads level 3 segments OVLAY21 and OVLAY22. Called by MAIN and calls NA23, ZERO, AVSEC, RELIB, AVSECR, RRES, and URES.	דד די
NA23	Calculates resonance cross sections for 23 Na only. Called from RESON and calls RELIB and QK.	FILL.
RELIB	Reads W table previously computed by subroutine W. Called by NA23 and RESON and calls SNIFF.	INTE
QK	Calls QUICKW and is called by NA23 and SR.	INTO
QUICKW	Generates psi and chi line shape func- tions from W table. Called by QK and AJK.	OUTPI
AVSEC	Controls calculation of infinite- dilution cross sections in the re- solved resonance region. Called by RESON and calls AVER and RETREV.	ERROF
AVER	Calculates infinite-dilution cross sections from fine-group data. Called by AVSEC, AVSECR, and URES.	
AVSECR	Controls calculation of infinite- dilution cross sections in the non- resonance region. Called by RESON and calls RETREV and AVER.	СНІ
RRES	Performs and controls resolved reso- nance calculations. Called by RESON and calls SETUFG, ERROR, ROMB, and RETREV.	PUPX

(,	
Subroutine	Description
SETUFG	Sets up energy mesh for the ultrafine groups in the resolved region, deter- mines which resonances contribute to a given ultrafine group, and determines the average "floor" cross sections for each ultrafine group. Called by RRES and calls RETREV.
ROMB	Calculates resolved-resonance integrals over each ultrafine group using the Romberg method (order=7). Called by RRES and calls SR.
SR	Generates integrands used in the ROMB calculations. Called by ROMB and calls ZERO and QK.
URES	Performs and controls unresolved- resonance calculations. Called by RESON and calls SNIFF, ERROR, ZERO, SETUPU, TERP1, AJK, RETREV, and AVER.
SETUPU	Sets up energy mesh for the unresolved groups. Called by URES and calls RETREV.
AJK	Calculates J and K integrals. Called by URES and calls QUICKW.
FFACNR	Calculates self-shielding factors for resonance regions not described by resonance parameters. Called by MAIN.
INELAS	Performs and controls inelastic- scattering calculations [including (n,2n)] using File 5 data (partial en- ergy distributions) on the ENDF/B tape. Called by MAIN and calls RETREV, FILE5, SNIFF, ERROR, INTERP, INTORP, FFLUX, and ERRF.
FILE5	Reads and labels the File 5 inelastic (n,n) or (n,2n) partial energy dis- tributions from the ENDF/B library. Called by INELAS and calls EFROR.
INTERP	Interpolates the inelastic (n,n) or (n,2n) cross sections. Called by INELAS.
INTORP	Interpolates the inelastic (n,n) or (n,2n) probabilities. Called by INELAS.
DUTPDX	Prints and writes (on disk file TAPE1) infinite-dilution group cross sections and self-shielding factors. Called by MAIN and calls SNIFF, ERROR, FFLUX, and ZERO.
ERROR	Prints error message when calculation is beyond limitations of the code. Called by HEADR, LOCISO, FILE1, FILE2, FILE3, SETUP1, TERP1, TERP2, RETREV, TSUM, REMT, WCP, RRES, URES, INELAS, FILE5, and OUTPDX.
СНІ	Computes group fission spectrum. Called by MAIN and calls ERRF.

X Control subroutine for creating, updating, or printing a library of ETOX cross sections. Reads last record of ETX INPT data file. Called by MAIN and calls BUFOPEN, UPDATE, and PRINT. BUFOPEN is part of the DRIVER program.

TABLE VII (cont.)

Subroutine	Description	Subroutine	Description
UPDATE	Creates or updates a library of ETOX cross sections. Called by PUPX and calls INPUT, OUTPUT, and NTRAN.	OUTPUT	Writes cross-section data on the out- put library tape. Called by UPDATE and calls NTRAN.
PRINT	Prints cross-section data from the output library. Called by PUPX and calls NTRAN and INPUT.	NTRAN	Either skips over a specified number of files or writes an end-of-file on the ETOX library tape. Called by UP-
INPUT	Reads cross-section data from the in- put library tape. Called by UPDATE and PRINT and calls NTRAN.		DATE, INPUT, OUTPUT, and PRINT.

TABLE VIII

_

SUMMARY OF "ERROR STOP N" OCCURRENCES IN ETOX CODE

<u>N</u>	Routine	Condition Causing Error Stop	<u>N</u>	Routine	Condition Causing Error Stop
1	URES	K3.GT.6. Dimension of a subscript- ed variable exceeded.	`16	FILE2	NER.NE.1.AND.NER.NE.2. This ENDF/B parameter indicates the number of
2	HEADR	(MF+MT).NE.O. File number and in- teraction type should be zero in			energy regions in the resonance range.
3	LOCISO	ENDF/B header record. MAT.EQ1. End of data on ENDF/B tape.	18		LRU.NE.1.AND.LRU.NE.2. This ENDF/B parameter indicates whether there are resolved (LRU=1) or unresolved
4	ĺ	MAT.GT.MATS. Could not find the		[(LRU=2) resonances in the energy region in question.
		tape. This error stop occurs if the materials are not processed in the order in which they reside on the ENDF/B tape because the tapc is not rewound after each material is processed.	19		LRF.NE.1. This ENDF/B parameter indicates the form in which the re- solved resonance data are given. Only single-level Breit-Wigner data (LRF=1) can be handled by the cur- rent version of ETOX.
5	FILE1	MAT.NE.MATN. Not positioned to correct material on ENDF/B tape.	20		NLS1.GT.5. Limitation on the num- ber of ℓ states allowed in the resolved resonance region.
6		MF.NE.1. Not positioned to correct file on ENDF/B tape.	21		INDEX.GT.250. Total number of re-
7		MT.NE.451. Not positioned to cor- rect interaction type on ENDF/B tape.			solved resonances exceeds the dimension of several subscripted variables.
8		MAT.EQ.O. Material number read from ENDF/B tape should not be equal to zero at this point.	23		LFW.NE.O.OR.LFW.NE.1. ENDF/B pa- rameter indicating whether average fission widths for the unresolved region are given (LFW=1) or not
9		LNU.EQ.O.OR.LNU.GE.3. Type representation for $\overline{\nu}(E)$ from ENDF/B	34		(LFW=0).
		should be equal to either one or two.	24		resolved resonances exceeds the dimension of several subscripted
10		MT.NE.452. See Error Stop 7.			variables.
11		NC.GT.10. Number of terms used in polynomial representation (LNU=1) of $\overline{V}(E)$ exceeds the dimension of a	25		LFW.NE.1. LFW (see Error Stop 23) should be equal to one at this point in the code.
		subscripted variable.	26		NLS1.GT.3. Restriction on the num-
12		N18.GT.250. Number of points at which $\overline{v}(E)$ is given on the ENDF/B			ber of l states in the unresolved region.
	ŧ	<pre>tape (LNU=2) exceeds the dimension of a subscripted variable.</pre>	27		NJS1.GT.3. Restriction on the num- ber of j states in the unresolved
13	FILE2	MAT.NE.MATN. See Error Stop 5.			region.
14		MF.NE.2. See Error Stop 6.	28		NEU1.GT.100. Restriction on the
15	ł	MT.NE.151. See Error Stop 7.			energy-dependent widths are tabu- lated for the unresolved region.

TABLE VIII (cont.)

				A	
<u>N</u>	<u>Routine</u>	Condition Causing Error Stop	<u>N</u>	Routine	Condition Causing Error Stop
30	FILE2	INDEU.GT.250. Same as Error Stop 24 but in a different loop of the routine.	45	SETUPU	NURG.GT.99. Number of groups in the unresolved region exceeds the dimension of a subscripted vari- able. Increase DELUMX.
31	ļ	IDXGFU.GT.250. Dimension of sever- al subscripted variables exceeded in the unresolved region.	48	RRES	EH(1).GE.EG(NG+1). Upper energy bound for the resolved region
32	REMT	MF.NE.3. See Error Stop 5.			put group boundary. Increase
33	REMT	MTX.GT.108. See Error Stop 7.			EG(NG+1).
35	TERP2	X(NA).GT.X(NA+1). X should be in increasing order in the ENDF/B in- terpolation table.	50	FILE5	NK.GT.25. Number of partial dis- tributions given in File 5 of ENDF/B exceeds the dimension of exceeds a when interviewed and the
36		XP(M-1).GT.XP(M). Values of X at which Y is to be interpolated should be in increasing order.	51		INDEX.GT.2000. Total number of tabulated data points in ENDF/B fo
37	ł	K.GT.N1. Interpolation table in ENDF/B is incorrect.			the dimension of saveral subscript ed variables.
38 TERP1	II.LE.O. Interpolation code in ENDF/B is out of range.	52		NR1.GT.10, Number of energy re- gions having different interpola-	
39		XP.LE.O. Zero or negative value of X cannot be interpolated by logs.			tion schemes in File 5 exceeds the dimension of several subscripted variables.
40	ł.	XA.EQ.XB. Cannot interpolate on a discontinuity (X1=X2).	53		INDEX.GT.2000. Same as Error Stop 51 but in a different section of
41	SETUP1	NFGT.GT.2699. Total number of fine groups exceeds the dimension of several subscripted variables.	54		the routine. NR1.GT.10. Same as Error Stop 52 but in a different section of the
		ANFMPD.		}	routine.
42	8	K.GT.600. Number of fine groups in	55		INDEX.GT.2000. See Error Stop 53.
		the energy region defined by a	56	•	NR1.GT.10. See Error Stop 54.
		ables. Increase DELMAX and/or EMNLEF and/or decrease ANFMPD.	998	RETREV	JD.LT.O. Argument returned by sub- routine SNIFF indicating that a scratch file could not be located.
43		MFFL.EQ.MFFU. Test on the lowest and highest group numbers for re- sonance shielding calculations.	999	FLE2 TSUM WCP FILE3 URES INELAS OUTPDX	JD.LT.O. See Error Stop 998.

B. 1DX Code

IDX is a one-dimensional (slab, cylinder, or sphere), multigroup, diffusion-theory code designed primarily to compute reson-nce-shielded and collapsed-group cross sections. The code can also be used to compute k_{eff} , compute alpha, or perform criticality searches on material concentrations, region dimensions, and buckling for either regular or adjoint cases. Alpha or k_{eff} can be used as a parametric eigenvalue in a search calculation. Variable dimensioning is used in IDX. Reference 3 gives a description of the mathematical models used. Resonance-shielded cross sections are calculated from cross-section data generated by the ETOX code. These data are in the Bondarenko format which include infinite-dilution group cross sections, inelasticscattering matrices, and group resonance-shielding factors for specified values of temperature and σ_0 . Interpolation schemes are used to compute shielding factors and effective cross sections applicable to the specific compositions and temperatures in the reactor. After the flux calculation, 1DX computes collapsed cross sections averaged over the spectrum in any specified zone. Collapsed cross-section output is in standard MULTIGRP format. If cross-section data are to be read from an ETOX library, 1DX expects the library to be present on tape 46. A library created or updated by ETOX is written on the same logical unit. At user option, cross-section input to 1DX can be read instead from the standard MULTIGRP data file.

Standard data files that can be read by the 1DX code are ADMNSTR, GEO DIST, GRP FLXS (either regular or adjoint), MIX DATA, and MULTIGRP (either reference or perturbed). Input data not contained in the standard data files or in the ETOX library are read from the code-dependent data file, ODX INPT. Card input for creation of the ODX INPT file is described in Appendix B. Of the files mentioned above, ODX INPT, ADMNSTR, GEO DIST, and MIX DATA are always required. If the flux guess is to be read from GRP FLXS and the cross sections are to be read from MULTIGRP, these two data files must also be available.

Standard data files that can be created by 1DX are INTQUANT, GRP FLXS (either regular or adjoint), ZONEDENS, and MULTIGRP (either reference or perturbed). These files are always created except MULTIGRP, which is created only if a group collapse is specified.

1DX does not treat upscatter and cannot read a MULTIGRP file in which the number of downscatter terms is not constant for every group and isotope in the file.

The segmentation structure for the 1DX code, which can be loaded by either the DRIVER or CLUB code, is shown in Table IX. Segment ODXS is loaded at level I+1 by either the DRIVER module (I=0) or by the CLUB code (I=2). All level I+2 segments are loaded by subroutine MAIN. Level I+3 segment MULTI is loaded by subroutines INP, RECS, and CRUNCH. Level I+3 segment DATFS is loaded by subroutines INP and FINPR.

TABLE IX

SEGMENTATION STRUCTURE OF 1DX CODE

Segment Level ^a	Segment Name	Subroutines Included in Segment
I+1	ODXS	MAIN, ERRO2, CLEAR, RCSTUP, SNIFF, MIXDAT, NTRAN
1+2	INPR	FXINP, ADMNST, INP
1+2	RCXS	RCINP1, RCPUP, RCPRT1, RCCHK
I+2	RCCAL	RCCAL1, RCCAL2
I+2	FLXCAL	RCCSS, RECS, INIT, FISCAL, MONPR, OUTER, INNER1, INNER, CNNP, FINPR, NBAL
I+2	XSCOL	GRAM, CRUNCH, INTQUA, ZONEDE
I+3	MULTI	MULTIG
I+3	DATS	GRPFLX, GEODIS

^a 1=0 if 1DX is loaded by DRIVER, and I=2 if loaded by CLUB.

A simplified flow diagram for the 1DX code is shown in Fig. 2, and a brief description of each subroutine in the code is given in Table X. The causes of a few error stops are explicitly identified in the output. Several other error stops are identified only by the subroutine and statement number where the error occurred. Conditions leading to the latter error stops are given in Table XI.

		TABLE X				
DESCRIPTION	OF	SUBROUTINES	IN	1DX	CODE	

Subroutine	Description	<u>Subroutine</u>	Description
MAIN	Controls overall flow of 1DX calcula- tion and loads segments INPR, RCXS, RCCAL, FLXCAL, and XSCOL. Called by DRIVER and CLUB and calls subroutines BUFOPEN, INP, RCINP1, RCSTUP, RCCAL1,		ADMNST, ERRO2, GRPFLX, GEODIS, MULTIG, MIXDAT, and FXINP. In the call to subroutine MULTIG at this point, only the velocities and fission spectrum are read from the MULTIGRP file.
	RCCAL2, RCCSS, RECS, INIT, FISCAL, MONPR, OUTER, CNNP, FINPR, GRAM, and CRUNCH. BUFOPEN is part of the DRIVER	ADMNST	Reads the ADMNSTR standard file. Called by INP and calls SNIFF.
INP	module. Controls reading, writing, and print-	SNIFF	Assigns a buffer to a specified logi- cal unit. Called by ADMNST, GRPFLX, GEODIS, INTQUA, ZONEDE, MIXDAT, and
	sections. Also loads segments MULTI and DATFS. Called by MAIN and calls		MULTIG and calls BUFOPEN. BUFOPEN is in the DRIVER module.

TABLE X (cont.)

Subroutine	Description	Subroutine	Description
NTRAN	Writes an end-of-file on a specified logical unit or reads through a speci- fied number of files on a specified logical unit. Called by RCPUP.	RECS	Checks cross sections from standard file MULTIGRP, performs adjoint rever- sal on the cross sections, and writes the cross sections on a scratch file.
ERRO2	Writes an error message identifying the routine and statement number where		Also loads segment MULTI. Called by MAIN and calls MULTIG.
	CNNP, INP, and MULTIG.	INIT	Performs adjoint reversals on the ve- locities and fission spectrum, mixes
GRPFLX	Reads and writes the GRP FLXS standard file. Called by INP and FINPR and calls SNIFF.		intervals, and computes areas and vol- umes. Also calculates the effective fission spectrum and the fission rate.
GEODIS	Reads the GEO DIST standard file. Called by INP and calls SNIFF.		Called by MAIN and calls ERRO2 and CLEAR.
MULTIG	Reads and writes the MULTIGRP standard file. Called by INP, RECS, and CRUNCH and calls SNIFF and ERRO2.	CLEAR	Sets a specified number of words in a specified array to a specified value. Called by INIT, CNNP, and GRAM.
MIXDAT	Reads the MIX DATA standard file. Called by INP and calls SNIFF.	FISCAL	Calculates fission sums and lambda and normalizes the flux and fission rate.
FXINP	Assigns a flat flux guess. Called by INP.	MONPR	Prints the monitor line (time, itera-
RCINP1	Calls RCPUP and is called by MAIN.		tion counts, eigenvalue slope, eigen- value, and lambda). Called by MAIN and
RCPUP	Reads cross-section data in the Bon- darenko format from an ETOX library tape. Called by RCINP1 and calls RCPRT1, RCCHK, and NTRAN.	OUTER	FINPR. Performs an outer iteration and over- relaxes the figsion source. Called by MAIN and calls INNER and INNER
RCPRT1	Prints cross-section data in the Bon- darenko format. Called by RCPUP.	INNER1	Calculates coefficients for the flux eduation. Called by OUTER.
RCCHK	Checks cross-section data in the Bon- darenko format for consistency. Called by RCPUP.	INNER	Calculates the flux for a specified group (inner iteration). Called by OUTER.
RCSTUP	Calculates a table of σ_0 -dependent resonance-shielding factors (appro- priate to the temperature of the mix- ture) for each isotope in the mixture. This is done by interpolating the	CNNP	Performs convergence tests and calcu- lates new parameters for search calcu- lations. Called by MAIN and calls CLEAR and ERRO2.
	temperature-dependent shielding fac- tors for each σ_0 . Called by MAIN.	FINPR	Computes zone fluxes and total flux; prints area, volume, total flux, power,
RCCAL1	Calculates the appropriate σ_0 for each isotope in the mix. Since σ_0 cannot be calculated until the appropriate shielding factors are known (and vice versa), iteration is required between RCCAL1 and RCCAL2. RCCAL1 is called		val; prints flux by group and space point; and controls calculation of the belance table. Also loads segment DATFS. Called by MAIN and calls MONPR, NBAL, and GRPFLX.
PCCAL 1	by MAIN.	NBAL	Computes and prints the neutron balance tables. Called by FINPR.
RCCAL2	Calculates resonance-shielding factors for each isotope appropriate to the value of σ_0 computed in RCCAL1. This is done by interpolating on the table of σ_0 -dependent values computed in RCSTUP. RCCAL2 is called by MAIN.	GRAM	Calculates and prints material inven- tories. Called by MAIN and calls CLEAR and ZONEDE.
		ZONEDE	Writes the ZONEDENS standard file. Called by GRAM and calls SNIFF.
RCCSS	Calculates resonance-shielded cross sections by applying the appropriate shielding factors to the infinite- dilution cross sections. Called by	CRUNCH	Calculates and prints collapsed cross sections and fission spectrum. Also loads segment MULTI. Called by MAIN and calls INTQUA and MULTIC.
	MAIN.	INTQUA	Writes the INTQUANT standard file. Called by CRUNCH and calls SNIFF.



Fig. 2. Simplified flow diagram for 1DX code.

THOOD ALL	TA	BL	É	х	I
-----------	----	----	---	---	---

ERROR STOPS IN 1DX CODE

Routine	Statement Number	Condition Causing Error Stop
MULTIG	51	Incorrect number of energy groups in the input MULTIGRP file.
MULTIG	53	1DX cannot read Bondarenko data from the MULTIGRP file.
MULTIG	82	1DX cannot read a MULTIGRP file that contains upscatter.
MULTIG	84	Incorrect number of downscat- ter terms in the MULTIGRP file.
CNNP	130	Zero fission source.
INIT	140	Error in the IO or Il tables, i.e., IO(M)>MT or Il(M)>MT for some 1≤M≤MO1.
INP	250	Input parameter SO3 cannot equal zero if ISO2≠O (see ADMNSTR file description).
INIT	520	R1(I+1) - R1(I)≤O where the array R1 contains the current mesh boundaries.

C. ANISN Code

The ANISN code⁴ is a one-dimensional (slab, cylinder, or sphere) transport theory (S_{p}) code with general anisotropic scattering. A diffusion solution can also be executed for specified groups. This code can perform a fixed source, k_{eff} , or alpha calculation. It can also perform searches on concentration, zone width, outer radius, or buckling. Either a regular or adjoint model can be used. Alpha or k_{eff} can be used as a parametric eigenvalue in search calculations.

ANISN can also compute activities by interval and zone for any material in the system and can perform a group collapse of the cross sections. The collapsed cross sections can be punched in a format suitable for use in creating a standard MULTIGRP file with the INP module.

Variable dimensioning is used in ANISN. If all of the data cannot be accommodated in fast memory, selected arrays are stored on disk files.

Cross-section input to ANISN is obtained exclusively from the standard MULTIGRP (wither reference or perturbed) file. Other standard files required are ADMNSTR, SN CONS, GEO DIST, and MIX DATA. At user option, the GRP FLXS (either regular or adjoint) file is read to obtain a flux guess for the problem. Input data not available in the standard files are read from the ANI INPT file. Card input for the creation of the ANI INPT file is described in Appendix B. Standard files created are INTQUANT, ZONEDENS, and GRP FLXS (either regular or adjoint).

ANISN can be loaded by either the DRIVER module or by the CLUB code and is segmented as shown in Table XII. The ANIS segment is loaded at level I+1 where I=0 if loaded by DRIVER and I=2 if loaded by CLUB. All the level I+2 segments are loaded by subroutine MAIN. A simplified flow diagram for the ANISM code is shown in Fig. 3, and a brief description of each subvoutine in the code is given in Table XIII. Causes of error stops are explicitly identified in the output.

D. DOT2DB Code

The DOT2DB code⁵ is a combination of the 2DB code¹⁵ and the DOT code.¹⁶ It can solve either the transport theory (s_n) or the diffusion theory multigroup equations in two space dimensions. If diffusion theory is specified, selected groups can be treated by s_n . Solutions can be obtained in slab (X-Y) and cylindrical (R-Z or R- θ) geometries. An additional option, triangular geometry, is available if diffusion theory is specified for all groups. Anisotropic scattering cf any order is allowed in the s_n option. In the diffusion theory option, anisotropic scattering is treated in the transport approximation. That is, the P_1 scattering matrix, when provided, is used to calculate the transport cross section.

Either direct or adjoint fluxes can be computed for fixed volume-distributed source, k_{eff} , alpha, concentration search, delta search, or fixed boundary source problems. Alpha or k_{eff} can be used as a parametric eigenvalue in search calculations. In addition, activities for any material in the system can be computed by interval and zone.

Cross-section input to DOT2DB is obtained exclusively from the MULTIGRP file. Although the calculational portion of the code car *reat anisotropic scattering, subroutine S860, w..ich reads the NULTIGRP file, cannot do so at present. Other standard data files required are ADMNSTR, SN CONS, GEO DIST, MIX DATA, and GRP FiXS (either regular or adjoint). The flux guess is always read from GRP FLXS, and SN CONS is always read because of the

TABLE XII

SEGMENTATION STRUCTURE OF ANISN CODE

Segment Level ^a	Segment <u>Name</u>	Subroutines Included
I+1 .	ANIS	MAIN, WOT, TIME, SNIFF
1+2	OVLAY1	PLSNT, TP, ADJNT, S805, S804, S814, WOT8
I+2	OVLAY2	GUTS, HEART, S807, S810, S821, DT, S824, S833, S851
1+2	OVLAY3	FINPR1, FINPR, PUNSH, DTFPUN, FLTFX, BT, SUMARY, WRITE, FEWG, WATE

^aI=0 1f ANISN is loaded by DRIVER, I=2 if loaded by CLUB.



Fig. 3. Simplified flow diagram for ANISN code.

TABLE XIII

DESCRIPTION OF SUBROUTINES IN ANISN CODE

Subroutine	Description	Subroutine	Description
MAIN	Controls overall flow of ANISN calcula- tion and loads segments OVLAY1, OVLAY2, and OVLAY3. Called by DRIVER and CLUB	WATE	Performs group collapse of cross sec- tions. Called by FEWG and calls WOT and PUNSH.
	and calls BUFOPEN, TIME, FLSNI, GUIS, and FINPR1. BUFOPEN is included in the DRIVER module.	PUNSH	Sets up each card to be punched by DTFPUN. Called by WATE and calls DTFPUN.
WOT	Prints one-, two-, or three-dimensional arrays. Called by PLSNT, S807, FINPR, SUMARY, and WATE.	DTFPUN	Punches collapsed cross sections in DTF format. Called by PUNSH and calls
TIME	Obtains and prints elapsed CP time for the job. Called by MAIN.	FLTFX	Converts a floating point number to an integer. Called by DTFPUN.
SNIFF	Assigns a file number corresponding to a given file name and assigns a parti- cular I/O buffer to the file. Called by PLSNT, TP, and SUMARY and calls	S821	Computes and normalizes fission source Also normalizes fluxes and currents. Called by HEART.
PLSNT	BUFOPEN (see DRIVER module). Controls the reading of all the input	DT	Performs diffusion theory inner itera- tion. Called by HEART.
	data and computes pointers for vari- able dimension arrays. Called by MAIN and calls SNIFF, TP, S804, ADJNT, S814, WOT8. and WOT.	S824	Computes total source (exclusive of self-scatter) for each interval. Called by HEART.
TP	Reads cross sections, source, and flux	S833	Performs transport theory (S _n) inner iteration. Called by HEART.
	or fission guess and copies the data to scratch files if it does not all fit in core. Called by PLSNT and calls SNIFF, S805, and ADJNT.	S851	Performs outer iteration convergence tests and computes new parameters for search calculations. Called by HEART.
ADJNT	Performs group reversal of specified arrays. Called by PLSNT and TP.	F'INPR1	Prints final monitor line and sets up pointers for activity and collapsing
S805	Rearranges cross-section matrices for adjoint calculations. Called by TP.		FINPR, BT, and FEWG.
S804	Checks S_n constants and computes P_ℓ constants. Called by PLSNT.	FINPR	Computes and prints activities and prints scalar flux and sources. Calle by FINPR1 and calls WOT.
S814	Computes areas and volumes, performs source normalization, and computes to- tal fixed source by group. Called by PLSNT.	BT	Determines storage needed for balance tables. Called by FINPR1 and calls SUMARY.
WOT8	Prints up to eight one-dimensional ar- rays. Called by FLSNT.	SUMARY	Computes and prints summary tables, prints angular fluxes, and writes INTOLIANT, GRP FLXS, and ZONEDENS data
GUTS	Called by MAIN and calls HEART.		files. Called by BT and calls WRITE,
HEART	Controls the outer iteration loop. Called by GUTS and calls S807, S810, S821, S824, DT, S833, and S851.	WRITE	Wol, and SNIFF. Writes one-dimensional arrays on a scratch file. Called by SUMARY.
S807	Mixes cross sections. Called by HEART and calls WOT.	FEWG	Performs preliminary calculations for cross-section collapsing. Called by
S810	Computes geometry-dependent arrays. Called by HEART.		FINFRI and Calls WATE.

• •

possibility of mixed diffusion- S_n calculations. Input data not available in the standard files are read from the DOT INPT file. Card input for creating this file with the INP module is described in Appendix B. Standard files created by DOT2DB are INTQUANT, ZONEDENS, and GRP FLXS (either regular or adjoint).

The DOT2DB code can be loaded by either the DRIVER module or the CLUB code and is segmented as shown in Table XIV. The DOTS segment is loaded at level I+1, where I=0 if loaded by DRIVER and I=2 if loaded by CLUB. All of the level I+2 segments are loaded by subroutine MAIN. Level I+3 segments DOTTY2 and DOTTY3 are loaded by subroutine INP, and the remaining level I+3 segments are loaded by subroutine OUTER.

TABLE XIV

SEGMENTATION STRUCTURE FOR DOT2DB CODE

Segment Level ^a	Segment Name	Subroutines Included in Segment			
I +1	DOTS	MAIN, SNIFF, BUFCL, BUFOP, CLEAR CLOCK, ERRO2, WOT			
I+2	DOTTY1	INP			
I+2	DOTTY4	S8830, OUTER, INIT, WOT8, FISCAL, S8847, CNNP			
I+2	DOTTY5	58850, SUMRY, ACTVTY			
I+3	DOTTY2	S860			
I+3	DOTTY3	S862, S863, RSTD, PCON, MAPR			
I+3	DOT4A	INNER, GRIND			
I+3	DOT4B	DTC, IFLUXN, DTP, DTJ, DTI			

^aI=0 if DOT2DB loaded by DRIVER, I=2 if loaded by CLUB.

Variable dimensioning is used in DOT2DB, and the code has been processed with the DYNBUF code¹⁷ to simulate dynamic buffer allocation on the CDC 6600. A simplified flow diagram is shown in Fig. 4. Subroutines BUFOP and BUFCL, which provide dynamic buffer allocation and are called by many of the DOT2DB subroutines, are not shown. A brief description of each subroutine is given in Table XV.

Most error stops in DOT2).B are identified only by the message "ERROR(H,I) DETECTED," where H is a four-character Hollerith word and I is an integer. Conditions resulting in this type of error message are summarized in Table XVI. Causes of all other error stops are explicitly identified in the output.



Fig. 4. Simplified flow diagram for DOT2DB code.

TABLE XV

DESCRIPTION OF SUBROUTINES IN DOT2DB CODE

Subroutine	Description	Subroutine	Description
MAIN	Controls the flow of the DOT2DB calcu- lation and loads segments DOTTY1, DOTTY4, and DOTTY5. Called by DRIVER and CLUB and calls BUFOP, INP, INIT, FISCAL, S8830, ERRO2, OUTER, S8847, CNNP, BUFCL, S8850, SUMRY, and ACTVTY.	INIT	Computes S _n constants, mixes cross sec- tions, performs adjoint reversals on fission fractions and velocities, modi- fies meshes for delta calculations, normalizes distributed and boundary sources, and calculates the fission
BUFOP	Assigns a buffer to a specified logi- cal unit. Calls BUFOPEN (see DRIVER routines) and is called by MAIN, INP,		source by interval. Called by MAIN and calls CLEAR, ERRO2, BUFOP, WOT, BUFCL, and WOT8.
	SNIFF, INIT, FISCAL, OUTER, S8847, S8850, SUMRY, ACTVTY, S860, S862, S863, RSTD, MAPR, S8830, WOT8, INNER, GRIND, DTC, DTP, ERRO2, VOT, DTJ, and DTI.	WOT	Prints one-, two-, and three-dimensional arrays. Calls BUFOP and is called by INIT, S8850, SUMRY, ACTVTY, and GRIND.
BUFCL	Rewinds a specified logical unit and releases the buffer assigned to the	WOT8	Prints up to eight one-dimensional ar- rays. Called by INIT and calls BUFOP.
	unit. Called by MAIN SNIFF, INP, S860, S862, S863, RSTD, OUTER, INIT, FISCAL, S8847, S8850, SUMRY, and ACTVTY.	FISCAL	Calculates the total fission source and normalizes the fluxes and fission sources. Called by MAIN and calls BUFOP, BUFCL, and ERRO2.
ERRO2	Prints error messages. Calls BUFOP and is ~alled by MAIN, INP, S860, INIT, FISCAL, and CNNP.	S8830	Prints the iteration monitor line. Called by MAIN and S8847 and calls CLOCK and BUFOP.
INP	Coordinates reading of the input data, loads segments DOTTY2 and DOTTY3, cal- culates pointers for array variables,	CLOCK	Obtains the current CPU time in sec- onds. Called by S8850, S8830, INNER, DTP, DTJ, DTI, and ACTVTY.
	and reads ADMNSTR file and control parameters from DOT INPT file. Called by MAIN and calls CLEAR, SNIFF, BUFOP, BUFCL, ERRO2, S860, S862, S863, RSTD, PCON, and MAPR.	OUTER	Coordinates the outer iteration loop and loads segments DOT4A and DOT4B. Called by MAIN and calls CLEAR, BUFOP, BUFCL, INNER, DTC, DTP, DTJ, DTI, and IFLUXN.
CLEAR	Sets a specified numer of elements in a given array to a specified value. Called by INP, S860, OUTER, INIT, SUMBY, and ACTYTY.	INNER	Coordinates the S _n inner iteration loop. Called by OUTER and calls GRIND, BUFCL, CLOCK, and BUFOP.
SNIFF	Assigns a file number corresponding to a specified file name. Called by INP S860, S863, RSTD, S8847, and SUMRY and	GRIND	Solves the S _n equations for a group. Called by INNER and calls BUFOP and WOT.
S860	calls BUFOP and BUFCL. Reads cross sections from the MULTIGRP	DTC	Calculates coefficients for the diffu- sion theory flux equations. Called by OUTER and calls BUFOP.
	rearranges the cross-section balance, and rearranges the cross sections for the adjoint case. Called by INP and calls SNIFF, ERRO2, BUFCL, BUFOP, and CLEAR.	DTP	Performs the diffusion theory inner it- eration calculation on horizontal mesh lines for problems with periodic bound-
S86 2	Reads the distributed fixed source from the DOT INPT file. Called by INP and	DT I	calls CLOCK and BUFOP.
\$863	Reads the fixed boundary source from the DOT INPT file. Called by INP and calle SNIFE FUEOD	015	eration calculation on vertical mesh lines. Called by OUTER and calls CLOCK and BUFOP.
RSTD	Reads SN CONS, GEO DIST, and GRP FLXS files and the remainder of the DOT INPT file. Called by INP and calls SNIFF, BUFOP, and BUFCL.	DTI	Performs the diffusion theory inner it- eration calculation on horizontal mesh lines for nonperiodic boundary condi- tions. Called by OUTER and calls CLOCK and BUFOP.
PCON	Calculates P_{ℓ} constants. Called by INP.	IFLUXN	Calculates leakages, reaction rates, and neutron balance and normalizes
MAPR	Prints zone and material maps. Called by INP and calls BUFOP.		fluxes after diffusion theory inner iteration. Called by OUTER.

میشود. میشود: ایرانیکی از این ایرانی ایرانی و میروان در ایرانی میکردی در میکرد. میشود: ایرانیکی ایرانی در ایرانی در ایرانی در میکرد در ایرانی در ایرانی ایرانیکی در ایرانیکی در ایرانی در ایران

	TABLE XV	(cont.)	
Subroutine	Description	Subroutine	Description
58847	Sums the reaction rates and leakages and prints the final monitor line and balance tables. Also writes first record of INTQUANT file. Called by MAIN and calls BUFOP, S8830, SNIFF, and BUFCL.	SUMRY	Calculates and prints the summary table by group and zone, writes the remainder of the INTQUANT file, and writes the GRP FLXS and ZONEDENS files. Called by MAIN and calls BUFCL, CLEAR, BUFOP, SNIFF, and WOT.
CNNP	Performs outer iteration convergence tests on fission and scatter source ratios and adjusts search parameters. Called by MAIN and calls ERRO2.	ACTVTY	Calculates and prints activities by interval and zone. Called by MAIN and calls BUFOP, BUFCL, CLEAR, WOT, and CLOCK.
S8850	Prints the final boundaries, total flux, fission source density, and flux moments. Called by MAIN and calls BUFOP, WOT, BUFCL, and CLOCK.		

TABLE XVI

SUMMARY OF "ERROR (H, I) DETECTED" MESSAGES

H	<u> </u>	Condition Causing Error Stop	Н	1	Condition Causing Error Stop
DOT 2	106	Program logic error encountered in	*MU	m	μ = 0 for angle m.
		MAIN.	*ETA	m	η = 0 for angle m.
*B01	0	IB01 = 2 or IB02 = 2 and IB01 \neq IB02. If either the left- or right-boundary	-ETA	m	η <0 for angle number m > MM/2.
		condition is periodic, the other must	+ETA	m	$n>0$ for angle number $m \leq MM/2$.
		also be periodic.	MU	m	No μ found that mates with $\neg \mu_m$.
*B03	0	IBO3 = 2 or IBO4 = 2 and IBO3 ≠ IBO4. If either the top- or bottom-boundary condition is periodic, the other must also be periodic.	*WT	0	$\Big \sum_{m=1}^{MM} w_m\Big + \Big \sum_{m=1}^{MM} w_m \mu_m\Big $
*A04	n	Order n of S _n quadrature is not even.			
*S03	1502	$ISO2 \neq 0$ and $SO3 \approx 0.0$ where $ISO2$ is the parametric eigenvalue type and $SO3$			$+ \left \sum_{m=1}^{\infty} w_m \eta_m \right \neq 1.0$
IBSS	IBSS	Insufficient core storage available for boundary source that uses IBUS	*MT	MT	Mixture number or component in mixing table exceeds MT where MT is the total number of materials.
		words. Increase dimension of common array A in DRIVER.	TMAX	0	$\theta_{max} > 1.0$ (0-mesh is measured in revolutions, i.e., in units of 2π).
SIZE	LAST	Insufficient core storage available for variable dimension arrays. Amount cf storage required is LAST words.	*R	I	R(I+1)-R(I) < 0.0. Radial mesh is not in ascending order.
		Increase size of common array A in DRIVER.	*2	J	Z(J+1)-Z(J) < 0.0. Axial mesh is not in ascending order.
S860	161	MTP≤0 where MTP is the number of iso- topes in the MULTIGRP file.	*EV	I	Calculated eigenvalue for outer iter- ation I differs from input estimate
S860	160	Insufficient storage for reading			by more than 1/I.
		MULTIGRP file. Increase size of com- mon array A in DRIVER.	*FS	IEVT	Calculated fission source is zero. Not allowed except for fixed-source problems.

E. CLUB Code

The CLUB code is a time-dependent fueldepletion and fission-product-buildup program based on the CITATION code.⁸ CLUB uses zone-averaged fluxes to compute time-dependent changes in atom densities on a subzone and zone scale. Fuel management is not available.

Zone-averaged fluxes are obtained from the INTQUANT interface file. At user option, this file can be provided by (1) the user, (2) infinitemedium calculations in CLUB, or (3) any one of the flux codes. For Option (1), the user creates the INTQUANT file from card input to the INP module. This file is then used for every burnup step. For Option (2), CLUB performs an infinite-medium flux calculation and writes the INTQUANT file at the beginning of each burnup step. For Option (3), CLUB loads the specified flux code (IDX, ANISN, or DOT2DB) at the beginning of each time step. CLUB provides the flux code with current compositions through the MIX DATA file, and the flux code provides CLUB with zone-averaged fluxes through the INTQUANT file. In concentration search problems. the flux code also provides CLUB with a new ZONEDENS file.

CLUP uses variable dimensioning and has been processed with the DYNBUF code¹⁷ to provide dynamic buffer allocation on the CDC 6600.

Interface files always required by CLUB are ZONEDENS, MULTIGRP, INTQUANT. and CLB INPT. Card input for creating CLB INPT with the INP module is described in Appendix B. It is recommended that the isotopes on the MULTIGRP file be ordered such that the fission products follow the fissile isotopes. If a flux code is used, CLUB also requires the ADMNSTR, MIX DATA, and GEO DIST files.

The ZONEDENS file is always created after each depletion step. If a flux code is used, CLUB also creates the MIX LATA file after each burnup step. The INTQUANT and GRP FLXS (both regular and adjoint) files are created by CLUB if an infinite-medium flux calculation is specified. The segmentation structure for the CLUB code is shown in Table XVII. Level 1 segment CLBS is loaded by the DRIVER module; level 2 segments DEPINP, FLXCON, and DEPLET are loaded by subroutine CLUB; and level 3 segment PTSPEC is loaded by subroutine NFLX if an infinite-medium flux calculation is specified. However, if a finite-medium flux calculation is specified, NFLX loads (at level 3) the ODXS, ANIS, or DOTS segment of the lDX, ANISN, or DOT2DB code, respectively. Level 3 segments BURNDTA and BURNUP are loaded by subroutine BURN.

TABLE XVII

SEGMENTATION STRUCTURE FOR CLUB CODE

Segment Level	Segment Name	Subroutines Included in Segment		
1	CLBS	CLUB, ITTIME, ICLOCK, GRIT, XION, BUFCL, BUFOP		
2	DEPINP	INPT		
2	FLXCON	NFLX		
2	DEPLET	BURN		
3	PTSPEC	SPEC, SPTC, SPTQ, SPTP		
3	BURNDTA	BRNA, BRNB, BRNC, BRND, BRNH, BRNI, BRNJ		
3	BURNUF	BRNP, BRNQ, BRNR, MIXDAT, NUCX		

A simplified flow diagram for the CLUB code is shown in Fig. 5. Not shown are subroutines BUFOP and BUFCL, which are used in connection with dynamic buffer allocation and are called by many of the CLUB subroutines. A brief description of each subroutine is given in Table XVIII.

Many of the error stops in CLUB are explained adequately in printed messages. However, one of the error messages is in the form

***DATA ERROR STOP NUMBER N

where N is an integer. The possible values of N, the routine in which the message is printed, and the conditions leading to the error message are summarized in Table XIX.

TABLE XVIII

Culmant-	Decontration of SUBN	Cuba-ut-	Docent-ti
Subroutine	Description	Subroutine	Description
CLUB	Controls the overall flow of the CLUB calculation and loads segments DEPINP, FLXCON, and DEPLET. Called by DRIVER module and calls subroutines INPT,	SPTP	Performs perturbation calculations using infinite-medium fluxes. Called by SPTQ and calls BUFOP.
INPT	NFLX, ICLOCK, BUFOP, BURN, and ITTIME. Reads title and neutronics selection from the CLB INPT file. Called by CLUB and calls BUFOP ICLOCK and ITTIME	BURN	controls flow of burning calculation for the depletion step and loads segments BURNDTA and BURNUP. Called by CLUB and calls ICLOCK, ITTIME, XION, BUFCL, CDTT BENK BUFCD BENK and
BUFOP	Assigns an available buffer to a speci- fied logical unit. Calls BUFOPEN (see DRIVER module) and is called by CLUB, INPT, NFLX, XION, SPEC, GRIT, SPTC, SPTQ, SPTP, BURN, BRNA, BRNB, BRNC, BRND, BRNH, BRNI, BRNJ, BRNP, BRNQ, WCX BPNP, cad MINDAT	BRNA	Controls reading of input data from CLI INPT file and computes pointers for burnup calculation. Called by BURN and calls XION, BUFCL, BUFOP, BRNC, BRND, BRNB, BRNH, BRNI, and BRNJ.
BUFCL	Rewinds a specified logical unit and releases the buffer assigned to the	BRNC	Reads depletion history and editing options from CLB INPT file. Called by BRNA and calls XION and BUFOP.
	SPTC, SPTQ, BURN, BRNA, BRNB, BRNJ, BRNP, BRNQ, BRNR, and MIXDAT.	BRND	Reads zone-classification data from CL INPT file. Called by BRNA and calls BUFOP and XION.
ICLOCK	Obtains current CPU time in units of 0.01 sec. Called by CLUB, INPT, NFLX, and BURN.	BRNB	Reads cross sections from MULTIGRP file. Called by BRNA and calls XION, BUFCL, and BUFOP.
ITTIME	Obtains current real time in units of 0.01 sec. Called by CLUB, INPT, NFLX, and BURN.	BRNH	Reads yield data for fission products from CLB INPT file. Called by BRNA and calls XION and BUFOP.
NFLX	Loads the appropriate segment (PTSPEC, ODXS, ANIS, or DOTS) for the flux cal- culation at each depletion step. Called by CLUB and calls ITTIME, ICLOCK,	BRNI	Reads nuclide-chain specifications from CLB INPT file, Called by BRNA and calls BUFOP and XION.
	SPEC, BUFOP, GRIT, BUFCL, and MAIN. Depending on the flux module celected, MAIN is the main subroutine of the 1DX, ANISN, or DOT2DB module.	BRNJ	Reads nuclide densities by subzone from CLB INPT file. If subzone densities are specified, writes the ZONEDENS file. If subzone densities are to be set event to the zone densities reads
SPEC	Reads infinite-medium data from the CLB INPT file and computes pointers for the infinite-medium flux calculation.		the ZONEDENS file. Called by BRNA and calls XION, BUFCL, and BUFOP.
YTON	Called by NFLX and calls XION, BUFOP, BUFCL, GRIT, SPTC, and SPTQ.	BRNP	Performs initialization for each deple- tion step and reads INTQUANT and ZONEDENS files. Called by BURN and
	file name. Calls BUFOP and is cal) ad by SPEC, SPTC, SPTQ, BURN, BRNA, BRNB, BRNC, BRND, BRNH, BRNI, BRNJ, BRNP, and BRNR,	BRNQ	calls XION, BUFCL, and BUFOP. Performs the depletion calculation for the time step. Called by BURN and calls BUFOP, GRIT, NUCX, and BUFCL.
GRIT	Performs block-data transfers to or from a specified logical unit. Calls	NUCX	Solves the chain equations. Called by BRNQ and calls BUFOP.
	BUFCL and BUFOP and is called by NFLX, SPEC, SPTC, SPTQ, BURN, BRNQ, and BRNR.	BRNR	Performs after-depletion-step calcula- tions and edits and writes ZONEDENS
SPIC	Reads ZONEDENS and MULTIGRP files and computes constants for the infinite- medium flux calculation. Called by SPEC and calls XION, GRIT, BUFCL, and	MTYDAT	ADMNSTR file. Called by BURN and calls BUFCL, BUFOP, GRIT, XION, and MIXDAT.
SPTQ	BUFOP. Performs the infinite-medium flux cal- culation. Writes INTQUANT and GRP FLXS (both regular and adjoint) files. Called by SPEC and calls XION, BUFCL, BUFOP, SPTP, and GRIT.		pletion step. Reads old MIX DATA file to obtain mix numbers and mix commands and GEO DIST to obtain material numbers by zone.



Fig. 5. Simplified flow diagram for CLUB code.

F. DAC Code

The DAC code is a perturbation theory program for use with one-, two-, and three-dimensional geometries. Geometries treated are slab (X, X-Y, and X-Y-Z), cylinder (R, R-Z, R- θ , and R- θ -Z), sphere (R), and hexagon (H and H-Z). At user option, DAC can use either angular fluxes from S_n calculations or scalar fluxes and currents from either diffusion or S_n calculations.

Reactivity worths of perturbations in microscopic cross sections and/or atom densities are computed by mesh interval and reaction type (fission, scattering, and total cross-section components). The prompt-neutron generation time and effective delayed-neutron fractions are also computed for the reference (unperturbed) system. The theory and numerical methods are the same as those used in the one-dimensional DAC1 code.⁶ Only isotropic scattering is treated. The code is variably dimensioned.

TABLE XIX								
SUMMARY	OF	"***DATA	ERI	ROR	STO	90	NUMBER	N"
	000	URRENCES	IN	CLU	JB C	:01)E	

<u>N</u>	Routine	Condition Causing Error Stop
702	SP EC	INFD(1).EQ.0 where INFD(1) is the number of energy groups for the infinite-medium flux calculation.
703	SPEC	Insufficient core storage available for infinite-medium flux calcula- tion. Increase dimension of common array A in DRIVER.
705	SPTC	No fission source in first group in infinite-medium calcula- tion. Check fission fractions (average for set) in MULTIGRP file.
706	SPTC	Total fission source is zero in infinite-medium flux calculation. See error stop 705.
710	SPTQ	No convergence in infinite-medium flux calculation, i.e., fluxes growing without bound.
802	BRNA	Insufficient core storage for burn- up arrays. Increase dimension of common array A in DRIVER.
811	BRNC	DD(1).LE.O.O where DD(1) is refer- ence core power level for first cycle.
820	BRND	NL(N).LT.NF(N) for zone set N in zone-classification data.
821	BRND	All zones were not specified in zone-classification data.
844	BRNH	A fission product specified in the fission-yield data is not in the MULTIGRP file.
854	BRNI	A nuclide in a nuclide chain is not in the MULTIGRP file.
855	BRNI	lncorrect transition type (NTYPE=O) in a nuclide chain.
857	BRNI	A fissile nuclide specified in the fission-yield data is not in the MULTIGRP file.
858	BRNI	Logic error in setting up the chain array in subroutine BRNI.
861	BRNJ	A nuclide specified in the subzone concentration data is not in the MULTIGRP file.
862	BRNJ	Logic error in setting up subzone concentrations in subroutine BRNJ.
863	BRNJ	Logic error in setting up zone con- centrations in subroutine BRNJ.
871	BRNP	Could not perform end-of-cycle re- covery for overshot conditions because start-of-step nuclide con- centrations were not saved. See input parameter ND(7) in CLB INPT file.

Interface-data files required by DAC are ADMNSTR, GEO DIST, GRP FLXS (both regular and adjoint), INTQUANT, MIX DATA, reference MULTIGRP, and DAC INPT. If worths of perturbations in microscopic cross sections are to be calculated, the perturbed MULTIGRP (PERTMTGP) file is required. If angular fluxes are to be used, the SN CONS file is also required. Card input for creation of the DAC INPT file is described in Appendix B. In the current system, there are no interface files created by DAC.

The segmentation structure of DAC consists of a single segment DACS as shown in Table XX. DACS is loaded at level 1 by the DRIVER module.

A simplified flow diagram is shown in Fig. 6, and a brief description of each subroutine is given in Table XXI. All error messages printed by DAC are self-explanatory.

TABLE XX SEGMENTATION STRUCTURE OF DAC CODE

Segment	Segment	Subroutines Included
Level	Name	in Segment
1	DACS	MAIN, SNIFF, TRSINT, PRINCS, DNSPEC, SCATCS, DNSORT, RMAVGF, MIXCX, SCRATO, PERT



Fig. 6. Simplified flow diagram for DAC code.

. .

TABLE XXI

DESCRIPTION OF SUBROUTINES IN DAC CODE

Subroutine	Description
MAIN	Controls overall flow of DAC calcula- tion, reads all input data except flux- es, and computes pointers for variable- dimension arrays. Called by DRIVER and calls BUFOPEN (see DRIVER subroutines), SNIFF, PRINCS, DNSPEC, SCATCS, DNSORT, MIXCX, TRSINT, RMAVGF, SCRATO, and PERT.
SNIFF	Assigns a file number corresponding to a specified file name and assigns a buffer to the file. Calls BUFOPEN (see DRIVER subroutines) and is called by MAIN, TRSINT, RMAVGF, and PERT.
PRINCS	Stores principal cross sections from the MULTIGRP file into the cross- section matrix. Called by MAIN.
DNSPEC	Stores delayed-neutron spectrum data from MULTIGRP into the proper array. Called by MAIN.
SCATCS	Stores the scattering cross sections from MULTIGRP into the cross-section matrix. Called by MAIN.
DNSORT	Sorts delayed-neutron spectrum and abundances. Called by MAIN.
MIXCX	Mixes and prints cross sections for the reference configuration. Called by MAIN.
trs int	Reads the GRP FLXS (both regular and adjoint) file and computes the trans- port weighting integrals. Called by MAIN and calls SNIFF.
RMAVGF	Computes volume elements corresponding to the final spatial mesh. In delta and outer radius search problems, the final mesh is computed from the eigen- value and initial mesh and mesh modi- fiers. Called by MAIN and calls SNIFF.
SCRATO	Reads and writes a scratch file. Called by MAIN.
PERT	Computes the prompt-neutron generation time, effective delayed-neutron frac- tions, and perturbation reactivities. Called by MAIN and calls SNIFF.

G. MC² Code

- -

The interfaced version of the MC² code¹ proccsses both Version-I and -II ENDF/B data to produce a standard multigroup file. The code has not been converted for direct access by a path call in DRIVER. Instead, it is operated as a free-standing code whose output can be accessed by the other codes in the system. This mode of operation is used because of the computation time required in generating a MULTIGRP file containing a number of materials. ${\rm MC}^2$ treats the resonance region in a detailed manner, provides for linear anisotropy (P₁) in the elastic-scattering matrix, and accounts for heterogeneity effects through a two-region cell model. Cross sections for the thermal group are supplied by the user.

 ${\rm MC}^2$ is limited to single-level Breit-Wigner representation of resolved resonances, a broadgroup lethargy mesh that is a multiple of a basic lethargy width ${\rm \Delta}u \simeq 1/120$, and downscattering. ${\rm MC}^2$ does not have the capability to update or add to an existing MULTIGRP file; therefore, all isotopes needed on MULTIGRP must be calculated in the same run. This problem can be circumvented outside of ${\rm MC}^2$ by writing a separate code to merge two MULTIGRP files into one file.

Although the code uses variable dimensioning, a container array of 40,000 words has been found necessary for some problems. For this size array, the code requires a total of 82,000 words of fast memory, including 9,500 words of buffer space.

The input ENDF/B binary tape must be arranged by files (alternate arrangement). MC² expects the first two files (general information and resonance parameters) on logical unit tape 1, and the next four files (smooth cross sections, secondary angular distributions, secondary anergy distributions, and secondary energy-angle "istributions) on tape 9. The output MULTIGRP file is written on tape 20. Card input to the code is described in Appendix B. In contrast to the INP module, card input is not in free format.

 MC^2 was originally programmed for the IBM 360 and required much effort to make it operational on the CDC 6600. Most of the problems were the result of basic differences in the two computers (e.g., small word size on the IBM 360 requiring the use of double precision), use of nonstandard FORTRAN (e.g., subscripted subscripts), and differences in file usage (e.g., I/O buffer assignment).

Difficulties in converting MC² to the CDC 6600 resulted in the development of the CONVERT code¹⁸ and the DYNBUF code.¹⁷ CONVERT looks for and flags or changes some of the FORTRAN differences in an IEM program in converting it for use on the CDC. DYNBUF makes changes to an IEM FORTRAN program to provide dynamic buffer allocation on the CDC 6600. Table XXII shows the MC² overlay structure and the subroutines in each overlay. The subroutines BUFOP, BUFCL, and BUFOPEN are used in connection with dynamic buffer allocation on the CDC 6600. For a description of the other subroutines, computational algorithms, etc., the reader is referred to Ref. 1.

An improved version of MC^2 is being developed at ANL.¹⁹ The new version, called MC^2 -2, will output multigroup cross sections in the Version-II standard interface-file format and will provide, at user option, rapid generation of cross sections by relaxing the rigor of the computational algorithms. By using standard FORTRAN to the fullest extent possible, conversion of the code to computers other than the IBM 360 should be simplified. When MC^2 -2 becomes available, it will be incorporated into the system of linked reactor physics codes at LASL.

TABLE XXII

OVERLAY STRUCTURE OF INTERFACED MC² CODE

Overlay Level	Subroutines Included in Overlay
(0,0)	DRIVER, TIMEIT, SKIPFI, SNIFF, BUFCL, BUFOP, BUFOPEN, POINTR, FREE, PUTM, PUTPNT, WIPOUT, DUMP, GETPNT, SAVPRG, GETN, PRGSET, STATUS, IGET, PRTI1, PRTI2, PRTR1, PRTR2, PRTA1, PRTA2, IPTERR, ILAST, ILASTB, ALLOC2, LOCF, FREE1, REDEFM, RED2F, TLEFT, ABSTOP, ABEND
(1,0)	CSI001, INPT, READI
(2,0)	CSCOO1, FDPRTU, QUICKW
(2,1)	TWO1, UNRES, QFJ
(2,2)	TWO2, RESRS
(3,0)	CSC002
(3,1)	CNTRL1, FDPTR1, FIGERO, SIGAVC, EF, ROMBI, INSCAT, FDINST, SOURCE
(3,2)	THREE2, CNTRL2, ALRAGO, PONE
(4,0)	CSC003
(4,1)	FOUR1, AVER
(4,2)	FOURTWO, AVER1, DTASET, FDATWT, FRVEL, RESON, INSCA, QUEZE, ELAST
(4,3)	FOUR3, BGPONE
(4,4)	FOUR4, MULTGRP, PRNTAB, PRNTI4, PRNTA, RWSCT

APPENDIX A

CONVERSION OF SYSTEM TO OTHER COMPUTERS

For computers that define the overlay structure by control cards or job control language, it is suggested that a dummy subroutine called SEGMENT be added to nullify the segment calls. In situations where other calls, such as CALL LINK, are required, it is suggested that a subroutine SEGMENT be set up to perform the required call. Such a subroutine could be as simple as the following:

SUBROUTINE SEGNENT (FILE, L, SEG, LIB, MAP) CALL LINK (SEG) RETURN .

For systems with built-in dynamic buffering, subroutines BUFOPEN, BUFCL, and BUFOP should be replaced by dummy subroutines. In other situations, local adaptations of the SNIFF subroutine may be required. In general, it is not desirable to compile the whole system of codes in a single compilation run. The binary object codes for each of the files listed in Table I (Sec. II) are normally created or updated in separate compilation runs and are stacked on a magnetic tape. In executing a particular problem, the binary files are transferred to separate disk files from the tape. The small DRIVER program is usually recompiled at execution time to adjust the size of the common container block A to the particular problem dimensions. Sample control card decks for performing these operations on the CDC 6600 will be aupplied with the source programs.

APPENDIX B

DESCRIPTION OF CARD INPUT

The card input description follows procedures used by the CCCC in defining files. The description is punched on comment cards, which provide a convenient method for updating and correcting the description. The first two columns of a card contain characters CF, CL, CR, etc., which identify the type of information contained on the card. These conventions are defined as

- CF Denotes name of a file or block of input data.
- CE Description of file or block. CF and CE normally appear together enclosed by asterisk cards.
- CR Record name.
- CL List defining array to be input.
- CD Description or definition of input parameters.
- CN Explanatory notes.
- CS Description of file structure.
- CC Condition controlling reading of record.
- C Used for spacing and delimiting records.
- CEOF End of file or block of data.

The first block of input, the CONTROL block, is always read by DRIVER at the beginning of a run. The remaining input is controlled by the path defined by the user. Whenever an INP cell is used in the path, the CARD INPUT FILE CONTROLS are read first. These controls determine which of the 16 data files are to be created or modified by card input during the given INP execution. The block entitled FILE AND RECORD PRINT CONTROLS is read whenever a PRN call occurs in the path. Additional information on card reading and file printing is given in Seco. . II.A and III.B.

The last file in the card input description contains the input description for the free-standing interfaced MC² code. This input description was provided by ANL for the MC² code.

Card input for a number of sample problems is discussed in Appendix D.

C++++ C CF CE C C++++	CONTRO DRIVER	CONTROL INPUT
(
CR	NUMBER OF PA	TH ELEMENTS -
čL	NPATH	-
CD C	NPATH	NUMBER OF PATH ELEMENTS -
C	*****	
C	PATH	
C CL	(PATH(I)+1=1	•NPATH)
000000000000000000000000000000000000000	PATH(I)	 IHREE-CHAHACTER HOLLEHITH NAME FOR THE ITH MODULE.CUL BLOCX. OR ELEMENT IN THE PATH. AVAILABLE MODULES ARE FLO HEADS STANDARD AND CUDE-DEPENDENT FILES FROM TAPE AND STORES EACH FILE AS A SEPAMATE DISK FILE. FLI STORES FILE ON AN OUTPUT TAPE. INP INPUT PROCESSOR. CREATES OF MODIFIES ANY OF THE STANDARD OR CUDE-DEPENDENT FILES FROM CARD INPUT. PHN AT USEP OPTION. PRINTS ANY OF THE FILES. ETX ETOX CODE. CONSTRUCTS MULTIGROUP CROSS- SECTION DATA IN BONDARENKO FORM USING DATA- FROM ENDF/B FILE. OUX ONED& CODE. A UNE-DIMENSIONAL DIFFUSION CUDE. CONVERTS ETOX UUTPUT INID A MULTI- GROUP CRUSS-SECTION SET. ANI ANISN CODE. A ONE-DIMENSIONAL SN CODE. DOT DOT2DB CODE. A TWO-DIMENSIONAL FLUX CODE PHOVIDING EITHER THE DIFFUSION DR SN OPTIONS. DAC PERTURATION CUDE. CALCULATES PERTUR- BATION REACTIVITIES USING FLUXES FROM ANY OF THE ABOVE FLUX CODES. CLB PERFORMS BURNUP CALCULATIONS USING ANY OF THE ABOVE FLUX CODES
C		,

:

C** С CF CARD INPUT FILE CONTROLS _ C C-----CR FILE CONTROL С CL IREAD(1)+1=1+16 С CD =1. READ FILE FROM CARDS IREAD CD =2. READ FILE FROM TAPE =3. READ FILE FROM BUTH CARDS AND TAPE CD CD =4. SKIP FILE CD CD =1. ADMNSTR FILE 1 CD =2+ SN CONS FILE =3. INTQUANT FILE CD CD =4. GEO DIST FILE CD =5+ MIX DATA #6. GRP FLUX FILE. REGULAR (GRP FLXS) CD CO. =7. GRP FLUX FILE, ADJOINT (ADJ FLXS) =8. MULTIGRP FILE.REFERENCE (MULTIGRP) CD CD =9. MULTIGRP FILE, PENTURHED (PENTMTGP) CD =10. ZUNEDENS FILE =11+ ETX INPT FILE CD =12. ODX INPT FILE =13. ANI INPT FILE =14. DUT INPT FILE =15. CLB INPT FILE CD CD CD CD =16. DAC INPT FILE CD C----CEOF

2 CF ADMNSTR FILE ADMINISTRATION AND CONTROL DATA CE _____ C----CR . ADMINISTRATION AND CONTROL DATA С ID(])+]=]+8)+ IA01+ IA11+ IA02+ IA03+ IA04+ IGE+ IZH+ IH+ JH+ KH+ 18K+ IEVT+ EV+ EVH+ EP5+TEMP1+TEMP2+ (10(])+[=]+8)+ CL . CI. TEMP3.TEMP4.TEMP5. 1801. 1802. 1803. 1804. 1805. 1806. H07. CL -IFAT. 501. MT. MOI. MCR. MSF. 12. JZ. KZ. 1502. 503. IGM. 406. 1005. 1607. 605. 606. ALAL. ALAM. POD. CL CL -EPSA. AFAC ٢L С 10 IDENTIFICATION AND TITLE. 846 CD CD IA01 UIMENSION 1. 2. UP 3 TYPE. OPDIFFUSION THEORY . INTHANSPORT THEORY -CD IALL CD 1402 (HEORY. OFREGULAR. 1=ADJOINT SCATTENING: 0=1SOTROFIC. N=ORDER OF CD 1403 ANISUTHOPIC CØ • ORDER OF SN APPROXIMATION IA04 60 • JD. IGE . GEUMETRY. OFFUNDAMENTAL MODE 1=1-0 SLAH (X) 62 6.00 2=1-0 CYL(#) • 1=1-D SPHEHE (H) • CO CD 6=2-0 SLAB(X+Y) 7=2-0 CYL (H+2) C0

CD		8=2-D CYL(R+THE7A) -
CD		9=2-D HEX(H) -
CD		11=3+D SLAB(X+Y+Z) -
CD		12=3-0 UCTL(K+INEIA+2) -
CD	1 714	
CD CD	12.11 TM	NUMBER OF FIRST DIMENSION INTERVALS -
CD	.1N	NUMBER OF SECOND DIMENSION INTERVALS -
CD	KM	NUMBER OF THIRD DIMENSION INTERVALS -
CD	IBK	NUMBER OF BUCKLING SETS -
CD		=0. NONE
CD		=+1+ ZONE DEPENDENT ONLY -
CD		
CD		TATION ZONE AND ORDER DEPENDENT -
CD CD	IEVT	FIGENVALUE TYPE. 3= SOURCE CALCULATION -
CD CD	10.41	1= N-EFFECTIVE CALCULATION -
čŏ		2= ALPHA CALCULATION -
CD		3= CONCENTRATION CALCULATION -
CD		4= ZONE WIDTH CALCULATION -
CD		5= BUCKLING SEARCH -
Cυ		67. DUTER RADIUS SEARCH -
CD	EV	INTITAL CIDENVALUE DUESS
	E 9 M	CONVERGENCE CRITERION -
	TEMPI, TEMPS	UNDEFINED -
cn	1801	LEFT BOUNDARY CONDITION -
CD	100.	-1= FLUX VANISHES AT BOUNDARY -
CD		0≈ VACUUM -
CD		I= HEFLECTIVE -
CD		2= PERIODIC -
CD		
CD		
CB	1902	TAP ROUNDARY CONDITION. SAME OFTIONS AS INC.
0	1904	HOTTOM BOUNDARY CONDITION. SAME OPTIONS AS 1801-
CD CD	1804	FRONT BOUNDARY CONDITION. SAME UPTIONS AS IBOI-
CD	1806	BACK BOUNDARY CONDITION. SAME OPTIONS AS 1901 -
CD	H07	FLUX INPUT OPTIONS.
CD		-1= NU FLUX GUESS -
CD		0= A VALUE FOR EACH GROUP+ X(G)+ WHICH -
CD		REPRESENTS THAT DRUUP IN EVERT INTER-
		VAL + 1.t.+ FLUAIU+[+J+K]=A(U), =
CD CD		HY GOODE FOR EACH INTERVALV IN DEDUKS -
CD		I.F., FLUX(G+1+J+K)=X(G+1+J+K), -
CD		2= A VALUE FOR EACH GROUP+ X(G)+ FOL
CD		LOWED BY A BLOCK OF VALUES. ONE FOR -
CD		EACH INTERVAL+ Y(I+J+K)+ I+E++ -
CD		FLUX(G+1+J+1)=X(G)+Y(1+J+K)
CD		JE A VALUE FOR EACH GROUP, X(G), A VALUE -
CU CD		TUR CAUR IST UIRA INTERVALA INTERVALA TATA A -
CD		AND & VALUE FOR FACH 3PD DING INTERVAL
čĎ		#(K) + I.E. + FLUX (G.I.J.K) = -
CD		A(G) *Y(1) *Z(J) *#(K)
CD		4= ENTER COMPLETE RESTART FOR CONTINUA
CD		TION OF PROBLEM KROM STANDARD FILES
CD	10.0	5= EXTRACT FLUXES FROM THE STANDARD FILE
CD CD	1121	USE NEGALISE SUURCE CHECK + 0=NO+ 1=YES -
co	AT .	TOTAL NUMBER OF NATERIALS INCLUSING MIXTURES
čõ	H01	NUMBER OF HATERIAL SPECIFICATIONS -
CD	HCP	NUMBER OF ISOTOPES FOR WHICH CROSS SECTIONS -
CO		ARE INPUT FRUH CARDS.
CD	MSF	SAME AS MCR. BUT INPUT FROM STANDARD FILE -
C0	IZ	NUNBER OF FIRST DIMENSION ZONE NODIFIERS -
CD	J/, #7	NUMBER OF THIRD DIMENSION ZONE MODIFIERS
CU	F1 6	HOUDER OF THIAD DIMENSION TONE MODILIERS .

.

CD	1502	PARAMETRIC EIGENVALUE TYPE FOR SEARCH CALCS
CD		US NUME
CD		IS REFFECTIVE FOR OFFECTIVAL STREM
CD		
CD		2= ALPHA FOR OFF-CRITICAL SYSTEM -
CD	_	WHERE SO3=ALPHA
CD	S03	PARAMETRIC FIGENVALUE
CD	IGM	NUMBER OF ENERGY GROUPS
CD	M06	SOURCE INPUT OPTIONS
CD		0= A VALUE FOR EACH GROUP, X(G), WHICH -
CD		WILL BE USED IN ALL INTERVALS, I.E., -
CD		$Q(G \bullet I \bullet J \bullet K) = X(G) -$
CD		1= A VALUE FOR EACH INTERVAL. IN BLOCKS -
CD		BY GROUP. EACH BLOCK IM*JM*KM* LONG
CD		I.E., Q(G,I,J,K)= X(G,I,J,K) -
ČD		2= A VALUE FOR EACH GROUP, X(G), FOLLOWED -
CD		BY A BLOCK OF VALUES, ONE FOR EACH -
ĊD		INTERVAL. Y(I.J.K). I.E
ČD		Q(G+I+J+K)+ X(G)*Y(I+J+K) -
ČĎ		3= A VALUE FOR EACH GROUP, X(G), A VALUE -
CD		FOR EACH IST DIM. INTERVAL, Y(I), A -
CD		VALUE FOR EACH 2ND DIM. INTERVAL. Z(J)
CD		AND & VALUE FOR EACH 3RD DIM. INTERVAL
ČD.		$W(K) = I_{a}F_{a} = O(G = I_{a}J_{a}K) = -$
CD.		X(G)+Y(T)+Z(J)+W(K) -
CD .		4= RIGHT (NORMAL TO IST DIM.) BOUNDARY -
cĎ		SOURCE -
CD		5= TOP (NORMAL TO 2ND DIM.) BOUNDARY -
CD .		SDURCE
CD .		6= FRONT (NORMAL TO 3RD DIM.) BOUNDARY
ČD.		SDURCE -
00	1905	MAXIMUM NUMBER OF OUTER ITERATIONS
CD CD	1002	MAXIMUM NUMBER OF INNER ITERATIONS PER GROUP
CD	605	HISE NEUTRON BALANCE TEST - A=NO. G05=LINIT
	005	FOR TEST
CD	606	USE INNER POINTWISE FLUX TEST - D=NO. G06=
CD	000	I INIT FOR TEST
00	AL AL	
cn cn		HOPER I THIT ON LANDA
		NEW DADAMETED NODTETED (OF DADAMETED OCCULATION
CD	FUU	NAMPEDI
	5004	CONVERSION PESSION USED IN CRITICALITY
	efja	CONVERCENCE FREUISION USED IN CREEICALINY =
	*540	
	APAL _	CAIRAPULATION PAULOR -
CEOF		

** C CF SN CONS FILE CE SN CONSTANTS ********************* 88 C----------CR RECORD CONTROL REAU IF IREAU(2).EQ.3 Ċ CL IRD(J) + J=1+3 C CD -READ RECORD J FROM CARDS - 1= YES. 0=NO IRD(J) -С C--____ -----

.

-2

į.

C---CR MU DIRECTIONS READ IF IRU(1).EQ.1 C CL M7(M) + M=[+MM C CD м7 MU DIRECTIONS NUMBER OF DIRECTIONS CD HM С (----ĉ CR READ IF IND(2).EQ.1 ETA DIRECTIONS С C۲ N5(M) . M=[.MM С CD MS ETA DIRECTIONS С C ----_____ WEIGHTS CR READ IF IRD(3).EQ.1 С CL WO(M). H=].HH C CD WEIGHTS NO. С CEOF C+* С C₽ INTQUANT FILE INTEGRAL QUANTITIES CE С •••••••••••••••••• C ------RECORD CONTROL NEAD IF IREAD(3).E0.3 CR C CL 19D(J)+ J≡1+3 C. HEAD RECORD(J) FROM CARDS - 1=YES+ U=NO CD (F) OBT f ---C----CR DERIVED CONSTANTS. READ IF IRD(1).EQ.1 С CL EF+EV+EVS+ICON+EVC+FLO+NOI+TSO+TLO+TLE+TBLO+TBA9 С C۵ EK. EFFECTIVE HULTIPLICATION FACTOR SEARCH EIGENVALUE CD ٤٧ CD EVS ELGENVALUE SLOPE CD [CON CUNVERGENCE CRITERION SATISFIED - 0=YES+ 1=NO CD EVC EIGENVALUE CONVERGENCE REACHED (ON FISSION CD SOURCES CD FLUR CONVERGENCE REACHED FLO NUMBER OF OUTER ITERATIONS CD NO1 CÛ TSO TOTAL NEUTRON SOURCE (INTEGRATED PHIONU CŬ SIGHA FISSION) CD TLO TOTAL NEUTRON LOSSES TOTAL LEARAGE TOTAL BUCKLING LOSS TOTAL INTERNAL BLACK ABSORPTION CØ TLE CD TBLO CD TRAB С -Ç==

```
C ----
   ZONE AVERAGED FLUXES, READ IF IRD(2).EQ.1
CR
                                          -
C
CL
   ((ZF(L+I)+L=1+IZM)+I=1+IGM)
С
              AVERAGED FLUX BY GROUP AND ZONE
CD
   ZF
С
C---
            _____
C--
      ZONE VOLUMES. READ IF IND(3).EQ.1
CR
С
                                          -
   VOL(1)+I#1+IZH
CL
С
             VOLUME OF ZONE
CD
   VOL
С
CEOF
```

```
C+4
C
ĊF
    GEO DIST FILE
CΕ
    GEOMETRY AND MATERIAL DISTRIBUTIONS
С
CR
    RECORD CONTROL
                READ IF IREAD(4).EQ.3
С
CL
   IRD(J)+J=1+4
С
CD
                HEAD RECORD J FROM CARDS - 1=YES. 9=ND
    IRD(J)
С
BOUNDARIES
                READ IF IND(1).EQ.1
CP
С
                                                 -
    XO(1)+ 1=1+1M1
CL.
                NEAD IF 1A01.GE.2
    1ML+{=L +{L>0Y
CL
    20(K) . K=1.KH1
                READ IF IA01.EQ.3
CL
С
                FIRST DIMENSION BOUNDARIES
CD
    10
CD
    `''O
                SECOND DIMENSION BOUNDARIES
                THIRD DIMENSION BOUNDARIES
                                                 •
CD
    ZO
    1#1
CD
                NUMBER OF FIRST DIMENSION BOUNDARIES
                NUMBER OF SECOND DIMENSION BOUNDAWIES
CD
    J#1
                                                 •
CD
                NUMBER OF THIND DIMENSION BOUNDARIES
    KN1
                                                 -
С
C ----
C--
        -----
                      _ _ _ _ _ _ _ _
  MATERIAL NUMBERS READ IF IND(2).EQ.1
CR
                                                 _
С
   H2(1)+I=1+1ZH
CL
C
ÇD
    H2
                MATERIAL NUMBERS BY ZONE
С
C---
      -----
```

--) Э ٥J -\$31115N30 01N01V 21¥ Э VI5(1) • I=1 • W01 כר 2 1*03*(E)0H1 31 083H \$31115N30 XIN CH --) -- 7 Э 00 39UTX1H NUMBERS INDICATING WHICH ISOTOPES MAKE UP EACH 11 00 С 10W+1=I+(I)[] כר 3 46AD IF IRD(2).60.1 UNAMMOD XIM 80 - 1 --7 С NUMBERS LABELING THE CROSS-SECTION MIXTURES 00 10 С -10#+1=1+(1)91 าว 3 HEND IF INDED. FO.I SUBBHUN XIM 80 ----) --------0 Э NEVD RECORD 7 LEON CANDS - 1=455+0=NO _ ØЭ (1)091 Э 180(1)+1=1++ 10 С RECORD CONTROL READ IF INEAD(5).E0.3 S ---. -----............................... Э ATATURE DATA AND ISOTOPE NUMBERS 30 **BULY DATA FILE** 3D Э ••0 3030 -) -----Э d) =15H+ IE IBK*01*0 981 00 1981 CD RUCKLING BY ZONE AND GROUP Q) ЯK 3 ({BK(K+1)*K=]*1HC)*1=1+1BK1) CL 0 READ IF IND(4).E0.1.AND.IBK.NE.0 BACKEING CK --) --0 -----0 0W ٥D JAVABTNI YA 2946MUN BNUL Э ٦D (WX+I=V+(WC+I=C+(WI+I=I+(X+C+I)OW)) 0 SUME NUMBERS 80 1.03.16)0HI 31 0A3H ---)

C----CR ISOTOPE NUMBERS READ IF IND(4).E0.1 c. CL IS01D(1)+1=1+ML -С CD 15010 NUCLIDE NUMBERS ON MULTIGROUP FILE GR CD NUCLIDE SEQUENCE NUMBERS ON ETOX LIBRARY HL=HCR+MSF CD ML _ С Concernation _____ CEOF С ĊF GRP FLXS FILE REGULAR TOTAL FLUXES. CURRENTS. AND ANGULAR FLUXES CE C CR RECORD CONTROL READ IF INEAD(6).E0.3 С ... CL 1RD(J)+J=1+3 С -ČD READ RECORD . - I=YES. 0=NO IRD (J) С C---C----CR FLUX GUESS С KEAD IF NOT.EU.O OF NOT.EQ.2 OF NOT.E0.3 CL X(L)+L=1+IGM -(((Y(I+J+K)+I=1+IH)+J=1+JH)+K=1+KH) HEAD IF HO7.E0.2 CL _ READ IF MOT.EQ.3 CL Y(1)+1=2+1M HEAD IF HOT.EQ.3 AND IA01.GE.2 CL ZtJ)。J=1。J庄 HEAD IF 407.E0.3 AND 1401.E0.3 CL W(K),K=],KH C C -C----HEAD IF IRD(1).EQ.1.AND.H07.ED.5 CR TOTAL FLUX C • CL {{PHI(I+L)+I=]+IM}+L=1+IGH} READ IF IA01.E0.1 С DO N L=1.IGN С 00 N 6-1-KH C CL N READ(N) ((PH1(1+J)+J=1+1H)+J=1+JH) HEAU IF IAOL.GE.2 -C CD TOTAL FLUX BY GROUP AND INTERVAL PHT • C -C---C---CR CURRENT READ IF IRD(2), E0.1. AND. (IA11.E0.0.0R. (IA11.E0.-1.AND.1A03.EQ.1) .AND.NO7.E0.5 CR -С ČL. ((X(]+[)+1×3+JM)+L=1+1GM) READ IF 1A01.E0.1 -С -90 N M=1.1A01 Ċ С 00 N L=1.IGM С 00 N K#1.KH N READ(N) ((X(3.)).I=1.1N).J=1.JM) READ IF 1A01.UE.2 CL С VECTOR CURRENT BY COMPONENT. GROUP. AND CD x CD INTERVAL С C--
CH	ANGULAR FLUX	READ IF IRD(3).EQ.1 AND IA11.EQ.1.AND.M07.E0.5
	DO N L=1+IGM DO N K=1+KM DO N J=1+JM READ(N) ((AF!M+I)	•M=L•MM) •[=]•]M)
	AF	ANGULAR FLUX AT MESH CENTERS BY GROUP, INTERVAL AND DIRECTION

```
C+++++
    С
ČF
   ADJ FLXS FILE
   ADJOINT TOTAL FLUXES. CURRENTS. AND ANGULAR FLUXES
CE
C
CR
  RECORD CONTROL READ IF IREAD(7).EQ.3
С
                                      _
CL
   IRD(J1+J=1+3
                                      .
С
            READ RECORD J - 1=YES+ 0=NO
CD
   IRD(J)
С
                                      -
C----
```

```
C+++
С
    MULTIGRP FILE
CF
CE.
    MULTIGROUP CROSS-SECTION FILE
С
CS
   FILE STRUCTURE
                                                  .
CS
CS
       FILE NAME
                                                  •
                                                  -
       RECORD CONTHOL
CS
CS
       DIMENSIONS
                                                  ISOTOPE NUMBERS
FISSION ISOTOPES
CS
CS
       AN'SOTROPIC ISOTOPES
CS
       ISUTOPE NAMES
CS
CS
       GROUP STRUCTURE
       DELAYED NEUTRON DECAY FACTORS
                                                  -
CS
CS.
       DELAYED SPECTRUM
```

CS CS CS CS CS CS CS CS CS CS CS CS	<pre>************************************</pre>	FUR NIR ISUTOPES) EADING ISSION SPECTRUM EUTRON ABUNDANCES NTAB TIMES FOR ANISOTROPIC ISOTUPES) IPAL CROSS-SECTIONS STIC CROSS-SECTIONS IC CRUSS-SECTIONS CROSS-SECTIONS N(SIGMAPO)/LN(10) FOR BUNDARENKO XS O TEMPERATURES O SELF-SHIELDING FACTORS O CROSS-SECTIONS
С С		
C		
ČR	FILE NAME	ALWAYS READ
CL	ANAME(I)+I=1+9	-
C CD C	ANAME	DESCRIPTION OF CROSS-SECTION SET, 9A6
C		
C	RECORD CONTROL	ALWAYS READ
CL	IRD(J)+J=1+12	-
CD CD	IRD (J)	READ RECORD J - 1=YES, 0=NO
C		
C	FILE DIMENSIONS A	ATAD, NTAT, NTEM, THM, THT, THC, THUE,
CL CL C	IABS. ITOT. IFIS.	IN2N+ ING, INA+ INP+ IND+ INT+ ING]
ĊD	NIR	NUMBER OF ISOTOPES
CD	NGS	NUMBER OF DELAYED GROUPS -
ČD	NTAB	NUMBER OF SCATTERING TABLES GIVEN FOR -
CD CD	NINT	ANISOTROPIC ISOTOPES - NUMBER OF SIGMAPO FACTORS AT WHICH BONDARENKO -
CD CD	NTEM	SELF-SHIELDING FACTORS ARE GIVEN - NUMBER OF TEMPERATURES AT WHICH BONDARENKO -
CD	TUM	SELF-SHIELDING FACTORS ARE GIVEN - PHINCIPAL CROSS-SECTION TABLE LENGTH -
CD CD	IM1 Tuni	POSITION OF TRANSPORT CROSS-SECTION -
ČĎ	IHS	POSITION OF IN-GROUP SCATTERING CROSS-SECTION -
CD	INUF	POSITION OF NU-FISSION CROSS-SECTION -
CD	IABS	POSITION OF ABSORPTION CROSS-SECTION -
CD	1101	POSITION OF FISSION CROSS-SECTION -
CD	IN2N	POSITION OF N2N CRUSS-SECTION -
CD	ING	POSITION OF N-GAMMA CROSS-SECTION -
CD	INA	POSITION OF N-ALPHA CROSS-SECTION
CD	INP	POSITION OF N=P CROSS-SECTION -
CD	IND	PUSITION OF N=U URUSS=SECTION
CD	INI	POSITION OF N-GAMMA TO FIRST EXCITED STATE -
СD СD С	1101	CROSS-SFCTION
C	***************************************	***************************************

· . · · ,

C---CR ISOTOPE NUMBERS ALWAYS READ С CL ISNUM(I) +K=1+NIR С CD URDER NUMBER OF ISOTOPE 1 IN COMBINED SET OF TSNUM CD FILE AND CARD INPUT CROSS-SECTIONS € **C**-C---CR FISSION ISOTOPE INDICATORS ALWAYS READ С CL IFIS(I)+I=1+NIR С FISSION ISOTOPE INDICATOR. CARD INPUT ISOTOPE CD IFIS I IS FISSIONABLE. 1=YES, 0=NO CD С ĉ. C-CP ANISOTROPIC ISOTOPE INDICATORS ALWAYS READ С CL IANI(I) + I=1+N12 С ANISOTROPIC ISOTOPE INDICATOR CD IANT NTAB PRINCIPAL CHOSS-SECTION TABLES GIVEN FOR ISOTOPE I. 1=YES. 0=NU CD CD C C C--READ IF IND(1).EQ.1 CR ISOTOPE NAMES С (ISOID(ISO) + ISONME(ISO) + TDC(ISO) + IRZM(ISO) + ISO=1 + NIK) CL С ISOTOPE ID NUMBER CD ISOID ISONME ISOTOPE NAME (A6) CD TEMPERATURE (DEGREES CENTIGRADE) CD TOC CD IRZM INDEX TO A REGIONE ZONE. OR MATERIAL IF THIS HAS BEEN AVERAGED OVER A SPECTRUM AND CD COMPOSITION FOR A PARTICULAR REACTOR REGION. CD CD ZONE. OR MATERIAL С C. C------READ IF TRD(2).EQ.1 CR GROUP STRUCTURE С JFTG.IDBF.(E(I).I=1.NGS1).(U(I).I=1.NGS1).RV(I).I=1.NGS). CL CL (EBAR(I) + I=1 + NGS) + (SEPC(I) + I=1 + NGS) С GROUP NUMBER OF FIRST THERMAL GROUP CD JFTG DETAILED BALANCE FACTOR INDICATOR. IDBF=0. CD IBBF UP-SCATTERING ELEMENTS ARE GIVEN. IDBF=1. CD CD COMPUTE UP-SCATTERING ELEMENTS FROM DETAILED BALANCE CD UPPER ENERGY LIMIT (EV) OF GROUP 1. CD E(D) (E(NGS1) IS LOWER ENERGY BOUND OF GROUP NGS) CD LOWER LETHARGY LIMIT OF GROUP I. (U(NGS1) IS CD Uth UPPER LETHARGY LIMIT OF GROUP NGS) CD AVERAGED RECIPROCAL VELOCITY FOR GROUP I CD RV(I) EBAR(I) CHARACTERISTIC ENERGY (EV) OF GROUP I USED IN CD CONJUNCTION WITH POINT-POINT MATRICES CD AVERAGED FISSION SPECTPUM FOR CROSS-SECTION SET-CD SEPC(1) CD NGS+1 NGS1 С

C-

C---DELAYED NEUTRON DECAY CONSTANTS READ IF IRD(3).EU.1 CR AND NUGS.NE.0 CR С DNDC(N) +N=1+NDGS CL С ČD. DELAYED NEUTRON DECAY CONSTANTS DNDC C C----CR DELAYED NEUTRON SPECTRUM READ IF IRU(4).EQ.1 AND NDGS.NE.0 С ((CHIDN(N+L)+N=1+NGS)+L=1+NDGS) CL С ĊD. DELAYED NEUTRON YIELDS BY PROMPT GROUP AND CHIDN -CD DELAYED GROUP С C ------C---CR ISOTOPE HEADING READ IF IRD(5).EQ.1 С ISOID, ISONME, TUC, IRZM, IZAS, AWR, EFIS, ECAP, DCA, TEMPG, TEMP7, TEMP8, TEMP9, TEMP10, KBR, ICHI, LIN, LEL, CL CL • CL LN2N. JSKL, JSKH, JONL, JONH, NBINT, NBTEM C ISOTOPE NUMBER ISOTOPE NAME (A6) ČD ISOID ISONME CD TEMPERATURE (DEGREES CENTIGRADE) CD TOC INDEX TO REGION. ZONE, ON MATERIAL IF THIS DATA-IRZM CD CD HAS BEEN AVERAGED OVER A SPECTRUM AND COMPOSITION FOR A PARTICULAR REACTOR REGION. CD CD ZONE, OP MATERIAL INTEGER Z+A+ STATE NUMBER. IZAS=10000*Z+10*A+5-WHERE Z= ATOHIC NUMBER+ A= ATOMIC WEIGHT, -CD IZAS CD AND ST FINAL STATE NUMBER (0% GROUND STATE)-RATIO OF ISOTUPE ATOMIC WEIGHT TO THAT OF A -CD CD AWR NEUTRON CD -**EFIS** FISSION ENERGY (WATT SECONDS PER FISSION) CD CAPTURE ENERGY (WATT SECONDS PER CAPTURE) CD ECAP CD DCA DECAY CONSTANT FOR ISOTOPE (PER SECOND) UNDEFINED CD TEMP6+TEMP10 CD KBR NATERIAL TYE FLAG 0=UNDEF TINED CD CD 1=FISSILE 2≖FERTILE CD CD 3=OTHER HEAVY MATERIAL 4=STRUCTURAL CD CD S=FISSION PRODUCT FISSION FLAG FOR ISOTOPE CD IEHI CD -1=FISSIONABLE. USE SET CHIS CD 0=NONFISSIONABLE CD NGS=INCIDENT ENERGY DEPENDENT 1=NOT INCIDENT ENERGY DEPENDENT CD CD LYN HAXINUM ONDER OF INELASTIC SCATTERING 0=1.0NE CD 1=1SOTROPIC CD CD NTAB=NUMBER OF TABLES GIVEN CD LEL MAXINUM ONDER OF ELASTIC SCATTERING SAME OTIONS AS LIN CD CD LNSN MAXIMUM ORDER OF N2N SCATTERING c n SAME OPTIONS AS LIN CD JSXL LOWEST GROUP AT WHICH SECUNDARY CROSS-SECTION DATA GIVEN CD CD JSXH HIGHEST GROUP AT WHICH SECONDARY CRUSS-SECTION -UATA GIVEN 22 CD JONL LOWEST GROUP AT WHICH DELAYED NEUTRON SPECTRA -ARE GIVEN CD

```
CD
     JÐNH
                    HIGHEST GROUP AT WHICH DLAYED NEUTRON SPECTRA
CD
                       ARE GIVEN
     NBINT
                    NUMBER OF VALUES OF SIGPO AT WHICH BONDARENKO
CD
                       SELF-SHIELDING FACTORS ARE GIVEN
CD
                    NUMBER OF TEMPERATURES AT WHICH BONDARENKO
CD
     NHTEM
CD
                       SELF-SHIELDING FACTORS ARE GIVEN
С
C---
C---
CR
     ISOTOPE FISSION SPECTRUM NEAD IF IND(6).EU.1 AND IFIS(1).GT.0 -
C
CL
     ((CHI(K+J)+K=1+IFIS(1))+J=1+NGS)
С
CD
     CHI(K+J)
                   FISSION SPECTRUM INCIDENT IN GROUP K. BORN
CD
                       IN GROUP J
С
DELAYED NEUTRON ABUNDANCES HEAD IF IND (7).EQ.I AND
CR
CR
                                 IFIS.NE.0 AND NDGS.GT.0
С
CL
     DNAB(1)+I=1+NDGS
С
CD
     DNAB
                   DELAYED NEUTRON ABUNDANCES
С
    C----
PRINCIPAL CROSS-SECTIONS READ IF IND(8).EQ.1
CR
C
     ((XS(I+J)+I=1+1MM)+J=1+NGS)
CL
С
CD
                   PRINCIPAL CROSS-SECTIONS. CROSS SECTIONS
    XS
                       OCCUPYING POSITIONS X5(1.J) TO XS(INT-1.J) -
CD
                       INCLUSIVELY ARE SPECIFIED BY THE POSITION
CD
                      INDICATORS INUF. IABS. ETC.. GIVEN IN THE FILE DIMENSIONS AND CUNTHOLS RECOPD.
CD
CD
CĐ
                       POSITIONS AS(IHS+J) TO AS(IHM+J) ARE
                       OCCUPIED BY CROSS-SECTIONS FOR SCATTERING -
CD
CÐ
                       FROM GROUP K TO GROUP J. WITH K=J.J-1..... -
                       J-NDN+RESPECTIVELY, WHERE NON IS THE TOTAL
CD
                      NUMBER OF ALLOWED DOWNSCATTERING TERMS.
CD
                       IF NO UPSCATTER CROSS-SECTIONS ARE PRESENT.-
CD
CD
                       IHS= IHT+1. UPSCATTEN CROSS-SECTIONS
CD
                      K= J+NUP+J+NUP-1+...+J+I OCCUPY POSITIONS
                      XS(1HT+2) TO XS(1H5-1+J+ HESPECTIVELY+
CD
CD
                      AND X5(THT+1+J) CONTAINS THE SUN OF
                      SCATTERING CHOSS-SECTIONS FROM GHOUP J TO
CD
                      GROUPS OF HIGHER ENERGY. NUP IS THE TOTAL -
CD
                      NURBER OF ALLOWED UPSCATTER TERMS.
CD
С
C-
                  CR
    INELASTIC SCATTERING CHOSS-SECTIONS READ IF IND(9).EQ.1
C
CL.
     LS(J+NUP+J)+K5(J+NUP+1+J)++++K5(J+J)++++K5(J+NUP+J)+J+J+J+J+
CO
                   INELASTIC SCATTENING CROSS-SECTIONS
    15
C
```

____ CR ELASTIC SCATTERING CROSS-SECTIONS READ IF IND(10).E0.1 С C LIST SAME AS FOR INELASTIC CROSS-SECTIONS С C---C----N2N SCATTERING CROSS-SECTIONS READ IT IND(11).EQ.1 CR C -С LIST SAME AS FOR INELASTIC CHOSS-SECTIONS С C-C----SIGMAPO FACTORS FOR BUNDARENKO CROSS-SECTIONS CR CR HEAD IF IRD(12)_EQ.1 AND NINT.NE.0 С CL J(1).J=i.NINT C CD VALUES OF LN(SIGPO)/LN(10) FOR WHICH BONDARENKOx CROSS-SECTIONS ARE GIVEN 00 c C--C--CR BONDARENKO TEMPERATURES READ IF INDII21.E0.1 AND NTEM.NE.O C2 TB(1)+1=1+NTEM c TEMPERATURES AT WHICH SONDARENNO CHUSS-SECTIONS-CD **T**8 CD ARE GIVEN r C-C-+ BONDA JENKO SELF-SHIELDING FACTORS READ IF IRDII2).EU.I AND CR NINT.NE.O AND NTEN.NE.O CR С (IFYOT (NoK.J) OFCAP (NoK.J) OFF IS (NoK.J) OFTR (NoK.J) OFEL (NoK.J) O CL CL N=1.NINT).X=1.NTEM):J=1.HOS) С CÐ FTOT TUTAL SELF-SHIELDING FACTOR FOR GROUP J. SIGPO -FACTOR N. AND TEMPERATURE R CD CAPTURE SELF-SHIELDING FACTOR FISSION SELF-SHIELDING FACTOR CO FEAP CD -THANSPORT SELF-SHIELDING FACTOR CD ETR ELASTIC SELT-SHIELDING FACTOR CD FEL -C C--CR BONDAHENKO CROSS-SECTIONS READ IF IND(12).EU.1 AND NINT."E.0-CR AND NTEM.NE.0 С (XSPO(J)+XSIN(J)+ASE(J)+ASMU(J)+XSX1(J)+XSED(J)+J=1+NGS) CL С POTENTIAL SCATTERING INELASTIC CROSS-SECTION CD XSP0 CD **XSIN** LLASTIC CRUSS-SECTION CD XSE AVERAGE CUSTINE OF SCATTERING ANGLE CD **XS**₽U AVERAGE ELASTIC SCATTERING LETHARGY INCREMENT ELASTIC SURNSCATTERING TO ADJACENT GROUP CD ASXI • CÐ **XSED** . C C-CEOF

C • • • • C CF CÉ C	PERIMIGP FILE PERIMIGP HULTIGRE FILE	
(
C C C	SAME STRUCTURE AS MULTIGHP FILE.	-
CEOF	***************************************	

```
C.
                      .............
C
ČF
      ZONEDENS FILE
C
c •
7-
                             -----
                                                               -
(R)
     ATOM DENSITIES
С
                                                               .
CL
     ((ADEN(1+J)+1=1+ML)+J=1+12M)
                                                               •
C
                                                               •
CD
                     ATOM DENSITIES BY ISUTOPE AND ZONE
     ADEN
                                                               -
CD
                     ML=HCR+HSF
     ML
                                                               _
С
C---
                  CEOF
```

```
C•
         .....
                    .........
С
            ETX INPT
CF
                                                                           -
CE
            ETOX CODE-DEPENDENT INPUT DATA FILE
                                                                           , a
С
C*•
                     ************************
       ............
                                                 ....
(------
                                  *********
CR
            RECORD CONTRUL
                                                                           -
С
                                                                           •
            READ IF IMEAD(11).E0.3
cc
                                                                           .
С
                                                                           _
CL
      (1=0(J)+J=1+3)
c
CD
                    READ RECORD J FROM CANOS (1/0#YES/NU). IF IREAD(11)-
      (L) GAT
                    .EQ.1. ALL THE IRU(J) ANE SET INTERNALLY TO 1. IF -
IREAD(11) IS EQUAL TO 2 ON 4. OMIT ALL CARD INPUT -
CD
CD
                    FOR THE ETX INPT FILE.
CŨ
                                                                           -
                    SEE CARD INPUT FILE CONTHOLS
CD
      19EA0(11)
Ç,
č--
                           -----
```

C	**********	,	
ČR	DAT	COMMON TO ALL ISOTOPES TO BE PROCESSED	•
čC	READ) IF IRD(1).EQ.1	-
CL C	NISO.IDTAP	•NG,LN5	-
ČL C	(EG(I)+I=)	•NG1) READ IF NISO.GT.0	-
CD CD CD	NISO TO BF	TAL NUMBER OF ISOTOPES TO BE PROCESSED. ISOTOPES MUST PROCESSED IN THE SAME RELATIVE ORDER AS THEY OCCUR THE ENDEZH TAPE.	-
CD CD	IDTAP EN 99	OF/B DATA TAPE 1.D. NUMBER (IDTAP.GF.).AND.IDTAP.LE. 99). ENDF/B DATA TAPE MUST BE MODE 1 (STANDARD BINARY).	-
CD CD CD	NG NU • A MU	MBER OF ENERGY GROUPS OF OUTPUT CHOSS SECTIONS (NG.GE.S. ND.NG.LE.99). IF AN ETOX LIBRARY IS TO BE UPDATED, NG ST BE THE SAME AS THAT FOR ISOTOPES ALREADY IN THE	-
CD CD		BRARY. BRARY. MBER OF DOWNSCATTERING GROUPS (INCLUDING SELF-SCATTER) OUTPUT CHOSS SECTIONS (INS-GE-L-AND-INS-) F-NGL. IF	-
CD CD	AN AS EG(1) G9	ETOX LIBRARY IS TO BE UPDATED. LN5 MUST BE THE SAME THAT FOR ISOTOPES ALREADY IN THE LIBRARY.	-
CD CD	EG NG1 =	(1)=0 IS NOT ALLOWED. NG+1	-
C		, 	-
^			_
CR	1501	OPE-DEPENDENT DATA	-
cc c	READ	IF NISO.GT.0.AND.IRD(2).EW.1	-
ČC C	00 5 N=1+N	1150	-
ČL C	(DESCRI(I)	•I=1+12) • NAMEI • MAT1 } • NTEMP • NS 160 • LN6	-
CL CL C	S EMAXFF+EMI NTEMP;+(SI	NFF•EMN1EF•CFF•DELMAX•DELUMX•ANFMPD•EPS•(TT(I)•I=1• GO(I)•I=1•NSIGO)	-
сD	DESCRI(1)	TITLE (TWELVE 6-CHARACTER WORDS)	-
CD CD	NAMEI MATII	ISOTOPE NAME (ONE 6-CHARACTER WURD) ENDF/B ISOTOPE 1.D. NUMBER (MATII.GE.l.AND.MATII.	-
CD CD CD	NTEMP	LE.99999). NUMBER OF TEMPERATURES AT WHICH SELF-SHIELDING FACTORS ARE TO BE CALCULATED (NTEMP.EQ.1.0R.NTEMP.EQ.	_
CD CD CD	NSIGO	.3). NUMBER OF *SIGMA ZERO* VALUES AT WHICH SELF-SHIELDING FACTORS ARE TO BE CALCULATED (NSIGO.GE.1.AND.NSIGO.	-
CD CD CD	LN6 Emaxff	LE.0). 1/D=CALCULATE AND PRINT FISSION FRACTIONS/NO MAXIMUM ENERGY (EV) FOR SELF-SHIELDING FACTOR CALCULATIONS (EMAXEE OF 0)	-
CD	EMINEE	MINIMUM ENERGY (EV) FOR SELF-SHIELDING FACTOR	-
CD CD	EMNIEF	MINIMUM ENERGY (EV) FOR WHICH FISSION SPECTRUM IS -	-
CD CD	CFF	FISSION SPECTRUM CONSTANT (EV). CFF.GE.I.RE+06.AND	-
CD CD	DELMAX	NAXIMUM LETHARGY SIZE FOR FINE GROUPS (DELMAX.GE	-
ςD CD	DELUMX	MAXIMUM LETHARGY SIZE FOR UNRESULVED GROUPS (DELUMX	•
CD	ANFMPD	NUMHER OF FINE-GROUP POINTS PER ENDERS SIGMA TOTAL -	
CD CD	EPS	ACCURACY PAPAMETER FOR ROMBERG INTEGRATION (EPS.GE 0.00001.ANU.EPS.LE.0.1).	•

.

CD CD	TT(I)	TEMPERATURES (DEG K) FOR WHICH SELF-SHIELDING FACTORS- ARE TO HE CALCULATED (TT(I).GE.273.AND.TT(I).LE.5000)-
CD		VALUES MUST HE GIVEN IN INCREASING ORDER
CD	5160(1) *SIGMA ZERU* VALUES (GARNS) FOR WHICH SELF-SHIELDING -
CD		FACTORS ARE TO HE CALCULATED (SIGO(T).GT.O). VALUES -
CD		MUST BE GIVEN IN INCREASING ORDER.
C		-
C=+==		~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~
C		
ČR		ETOX LIBRARY UPDATE DATA
С		-
СС		READ IF IRD(3).EQ.1
С		-
CL	NUP+NU	PleNUP2eNPReNPRIENPR2eNLAST
С		-
CD	NUP	1/0=PRODUCE AN UPDAIED LIBRARY OF EIOX CROSS SECTIONS/NO -
CD		IF NUP=1+ CRUSS SECTIONS FOR NISO=NUP2+NUP1+1 -
CD		ISSUPPES MUST HAVE BEEN GENERATED IN THE CURRENT RUN
CD		THE OUTPUT LIBRARY IS WRITTEN ON LOGICAL UNIT TAPE46
CD	NUPI	LIBRARY SEQUENCE NUMBER OF FIRST ISUTOPE TO BE UPDATED
CD		NOT USED IF NUP=0. IF NUP=1 AND NLAST=0. SET NUP1=1
CD	NUP2	LIBRARY SEQUENCE NUMBER OF LAST ISOTOPE TO BE UPDATED
CD		NOT USED IF NUP=0. IF NUP=1 AND NLAST=0+ SET NUP2=NISO
	NPR	1/0=PRINT CRUSS SECTIONS/NO. PRINT IS ALWAYS FROM -
CD		LINRARY UN LOGICAL UNIT TAPE46.
	NPRI	LIBRARY SEQUENCE NUMBER OF FIRST ISOTOPE TO BE PRINTED
CD CD		NUT USED IF NPR#0.
0	NPRC	LIBRART SEQUENCE NUMBER OF LAST ISUTOPE TO BE PRINTED
0	NF ACT	TOTAL NUMBER OF ICOTORES ON THE INDUT CLODADY AND AN ASS
	IN ASI	AT A AND NUE FO L. INDUT LIDDADY MUST DE ACCIONED TO
cn cn		LOGICAL UNIT TAPEAT. TE NUD FO O AND NOD FO L. TNDOT
cD		LIRRARY MUST HE ASSIGNED TO LOGICAL UNIT TAREAS
č		
Č		-
CEOF		

******************* C### -С ODX INPT CF 1DX CODE-DEPENDENT INPUT DATA FILE CΕ С C## C-----CR RECORD CONTROL -С READ IF IREAD(12).EQ.3 СС С (IRD(J),J=1,7) ČL С READ RECORD J FROM CARDS (1/0=YES/NO). IF IREAD(12)-.EQ.1. ALL THE IRD(J) ARE SET INTERNALLY TO 1. IF -CD IRD(J) CD IREAD(12) IS EQUAL TO 2 OR 4, UMIT ALL CARD INPUT -CD -FOR THE ODX INPT FILE. CD SEE CARD INPUT FILE CONTROLS • CD 1READ(12) -С _____ C--------------

C			•
ČR		TITLE AND CONTROL PARAMETERS	-
C CC		PEAD IE TODILLED.	-
c		WCWD IN INDIINSCHAL	2
ČL	(10(1)	•I=1•11)•MAXI•NXCM•NPHT•NRCF•NIFF•MM0I•NCR•NTR•	-
CL	NFGM+1	PUN+MULT1+MULT2	-
C	10/1		•
	MAXT	MAXIMUM RUNNING TIME (MIN) FOR JOH (NOT USED IF ZERO)	-
ČD	NXCM	NUMBER OF DOWNSCATTER GROUPS IN CROSS SECTIONS	-
CD	NPRT	0/1/2=DELETE PRINTING OF XSEC DATA AND FLUXES(MINI PRINT)	-
CD		/DELETE PRINTING OF INPUT XSECS IN BONDARENKO	•
CD	NOCE	FORMAT(MIDI PRINT)/FULL PRINT(MAXI PRINT)	
	NHCF	XSECS. IF NRCE-GT-0. XSEC INDUT NUST HE IN THE	2
ĊD		BONDARENKO FORMAT FROM AN ETOX LIBRARY TAPE ON LOGICAL	•
CD		UNIT TAPE46. THE NRCF MIXTURES MUST BE SPECIFIED FIRST	-
CD		IN THE ID TABLE (SEE MMO1 BELOW). IF NRCF=0. XSECS ARE	-
CD CD		READ FROM MOLITORP. IN TABLE IS SPECIFIED IN MIX DATA	Ξ
ČD	NIFF	NUMBER OF SPECTRUM ITERATIONS IN THE CALCULATION OF	_
CD		ELASTIC DOWNSCATTERING. RECOMMENDED VALUE NIFF=0.	-
CD		NOT USED IF NRCF=0 AND SET TO ZERO INTERNALLY IF IA02=1	-
CD	NN01	(ADJOINT CALCULATION).	-
CD	Mei A T	NUMBER OF MIXTURE SPECIFICATIONS FOR GENERATING RESONANCE	-
ĊD		SHIELDED CHOSS SECTIONS (MMO1.LE.MO1). SET MMO1=0 IF	-
CD		NPCF=0. Mul IS SPECIFIED IN THE ADMNSTR FILE.	-
CD	NER	NUMBER OF CULLAPSED GROUPS. IF NCR.GT.IGM. NO GROUP	-
CD		WRITTEN ONLY IF NCR-LE-IGM. IGM IS SPECIFIED IN ADMNSTR	-
CD		FILES	-
CD	NTP	TYPE WEIGHTING FOR COLLAPSED SIGMA TRANSPORT (0/1=	-
CD	NEGH	NORMALIZED/RECIPROCAL)	-
CD	TPUN	0/)=PRINT COLLAPSED MATERIALS (NOT USED IF NCR.01.10M)	_
CD	MULT1	0/]=MULTIGRP/PERIMIGP TO BE READ (USED ONLY IF NRCF=0).	-
CD	MULT2	0/]=MULTIGRP/PERTMTGP TO BE WRITTEN (USED ONLY IF	-
CD		NCR.LE.IGM).	-
((2
č			
C			-
CR	F	NUCKLING MODIFIERS	-
C CC	6	FAD_IF_()FVT_FQ_5_0R_(TBK_FQ)_AND_HK_NF_0.0))_AND_TRD(2)	-
cc			-
С			•
CL	(GAM(I)	+I=1+IZM)	-
C CD	GAM(I)	BUCKLING MODIFIERS BY ZONE	
CD	BK	BUCKLING (SINGLE VALUE) FROM GEO DIST FILE	-
CD	IEVT, 1	ZM. AND IBK ARE SPECIFIED IN ADMNSTR FILE.	•
c		•	•
C		* +	•
C			•
CR C	r	ISSION FRACTIONS AND VELOCITIES	
сс	H	EAD IF WRCF.GT.0.AND.IRD(3).EQ.1	•
С		-	•
CL C	(K7(I)•	1=1(10M)+(V/(1)+1=1+16M)	
CD	K7(I)	FISSION FRACTION BY GROUP	
CD	V7(I)	VELOCITY HY GROUP	,
CD	IGM	NUMBER OF ENERGY GROUPS (SPECIFIED IN ADMNSTR FILE) -	•
C			

C----------CR ZONE MODIFIERS С READ IF 1EVT.EU.4.AND.IRD(4).EU.1 CC C CL $(P3(I) \cdot I = 1 \cdot 17M)$ С CD R3(1) ZONE MODIFIER FOR ZUNF 1. IF R3(1)=0. WIDTH OF ZONE I IS HELD CONSTANT. CÐ С C-C-----CR DATA FOR XSEC GROUP CULLAPSING С CC READ IF NCR.LE.IGM.ANU.IRD(5)_EQ.1 С CL (NPN(I) + I = 1 + NCR) + (NFP(1) + I = 1 + NFGM) + (NZN(I) + I = 1 + NFGM)С NUMBER OF ORIGINAL GROUPS IN ITH COLLAPSED GROUP REFERENCE I.D. NUMBER OF ITH MATERIAL TO BE COLLAPSED ZONE NUMBER OF FLUXES TO BE USED FOR COLLAPSING ITH CD NPN(I) CD NFP(I) CD NZN(1)MATERIAL CD С C-----CR DATA FOR RESONANCE SHIELDING CALCULATIONS С CC READ IF NRCF.GT.0.AND.IRD(6).EQ.1 С CL (NZ(I) + I=1 + NRCF) + (HETC(I) + I=1 + NRCF) + (J1(I) + I=1 + MMO1) + (ATEM(I) + I=1+MM01) + (MF(1) + I=1+MM01) + (U7(I) + I=1+IGM) CL С CD ZONE NUMBER OF FLUXES TO BE USED IN ELASTIC DOWNSCATTER -N7(1) ITERATIONS FOR ITH MIX CD HETEROGENEITY CONSTANT (CM) FOR ITH MIX HETC(I) CD MIX COMMANDS SIMILAR TO II TABLE CD JI (D TEMPERATURE (DEG K) OF NUCLIDES IN JI TABLE CD ATEM(1) CD 0/1=FUEL/MODERATOR DESIGNATION FOR NUCLIDES IN J1 TABLE -MF(I) CD U7(I) LETHARGY WIDTH BY GROUP CD MIXTURES IN THIS RECORD REFER TO THE NRCF MIXES USED IN CD GENERATING RESONANCE SHIELDED XSECS. SEE BNWL-954 FOR EXAMPLE ON HOW TO SET UP J1 TABLE. CD C C---EXTRA DATA NEEDED FOR WRITING MULTIGRP FILE AFTER CR CR GROUP COLLAPSING С READ IF NCR.LE.IGM.AND.IRD(7).EQ.1 СС С NFAM, (ISONME(I), AWR(I), DCA(I), EFIS(I), ECAP(I), TDC(I), IZAS(I), CL KBR(I)+ICHI(I)+I=1+NFGM) CL С CL $(DNDC(N) \cdot N = 1 \cdot NFAM)$ READ IF NEAM.GT.0 С ĊC. DO 5 I=1+NFGM С CL ((CHI(I+J+K)+J=)+ICH1(I))+K=1+NCR) READ IF ICHI(I).GT.0 С READ IF ICHI(I) .NE.0.AND. Cι 5 ((DNCHI(1+J+N)+J=1+NCR)+N=1+NFAM) CL. NFAM.GT.O С CD NFAM NUMBER OF DELAYED NEUTRON FAMILIES CD ISONME(I) 6-CHARACTER NAME FOR COLLAPSED ISOTOPE I AWR(I) PATIO OF COLLAPSED ISOTOPE ATOMIC WEIGHT TO THAT OF CD CD NEUTRON

CD	DCA(1)	DECAY CONSTANT (1/SEC) FOR COLLAPSED ISOTOPE I -
CD	EF15(1)	FISSION ENERGY (W-SEC/FISS) FOR COLLAPSED ISUTOPE 1 -
CD	ECAP(1)	CAPTURE ENERGY (W-SEC/CAP) FUR CULLAPSED ISOTOPE I -
CD	TDC (1)	TEMPERATURE (DEG C) FOR COLLAPSED ISDTOPE 1 -
CD	17AS(1)	1000+2+10+A+S WHERE Z=ATOMIC NO+ A=ATOMIC WI+ AND S= -
CD		FINAL STATE NO (0=00 STATE) FOR CULLAPSED ISOTOPE 1 -
CD	KBR(1)	MATERIAL TYPE FLAG FOR COLLAPSED ISUTOPE I -
CD		0/1/2/3/4/5=UNDEFINED/FISSILE/FERTILE/OTHER HEAVY -
CD		MATERIAL/STRUCTURAL/FISSION PRODUCT -
CD	ICHI(I)	FISSION FLAG FOR CULLAPSED 1SOTOPE 1 -
CD		-)=FISSIUNABLE-USE CHI FOR SET -
CD		O=NONFISSIONAULE
CD		1=FISSIONABLE-CHI NOT INCIDENT ENERGY DEPENDENT -
CD		NCR=FISSIONABLE-CHI IS INCIDENT ENERGY DEPENDENT
CD	DNDC (N)	DECAY CONSTANT (1/SEC) FOR DELAYED NEUTRON PRECURSORS -
CD		IN FAMILY N
CD	CHI(1+J+*	FISSION SPECTRUM FOR ISOTOPE I. GIVEN AS THE -
CD		FRACTION OF FISSION NEUTRONS BURN IN GROUP K WHEN -
CD		THE INCIDENT NEUTRON IS IN GROUP J. IF ICHI(1)=1
CD		CHI IS NOT INCIDENT ENERGY DEPENDENT.
CD	DNCHIII+.	IN) DELAYED NEUTRON SPECTRUM FOR ISDTOPE I. GIVEN AS -
CD		THE FRACTION OF DELAYED NEUTRONS ASSOCIATED WITH -
CD		FAMILY N WHICH ENTER GROUP J. THE SUM OF -
CD		DNCHI(I+J+N) OVER ALL J IS THE YIELD FOR FAMILY N
С		-
C		
CFOF		

ANI ; Anis	INPT N CODE-DEPENDENT INPUT DATA FILE	-
RECO	D CONTROL	
READ	IF THEAD(13).EQ.3	
(IRD(J)+J=)	•9)	-
IRD(J)	READ RECORD J FROM CARDS (1/0=YES/NO). IF IREAD(1 .EG.1. ALL THE IRU(J) ARE SET INTERNALLY TO 1. IF IREAD(13) IS EQUAL TO 2 OR 4, OMIT ALL CARD INPUT FOR THE ANI INPT FILE.	3)- - - -
IREAD(13)	SEE CARD INPUT FILE CONTROLS.	- -
CONTR	OL PARAMETERS	
READ	IF [HD(1).EQ.1	-
(T(I) = 1 + 1 IFG + IFLU + IF	2) + 10 + 14T + 14S + 14M + 10FM • 1PM + 1PP + 101 + 103 + 104 + 10AT2 • N + 1PKT + UFM1 + KYF	
T(I) TIT ID PRO IHT CPO IHS CRO IHM CRO IDFM 1/0	LE (TWELVE 5-CHARACTER WORDS) BLEM ID NUMBER SS SECTION TABLE POSITION OF SIGMA TOTAL SS SECTION TABLE POSITION OF SELF SCATTER SIGMA SS SECTION TABLE LENGTH =DENSITY FACTORS USED/NO	
	ANI 1 ANIS RECOR READ (IRD (J) + J=) IRD (J) IREAD (13) IREAD (13) CONTR READ (T (I) ~ I=1+L IFG • IFLU • IF T (I) TIT ID PRO IHS CR0 IHM CR0 IDFM 1/0	ANI INPT ANJSN CODE-DEPENDENT INPUT DATA FILE RECORD CONTROL READ IF IHEAD(13).EQ.3 (IRD(J).J=1+9) IRD(J) READ RECORD J FROM CARDS (1/0=YES/NO). IF IREAD(1 .EG.1. ALL THE IRD(J) ARE SET INTERNALLY TO 1. IF IREAD(13) IS EUDAL TO 2 OR 4. OMIT ALL CARD INPUT FOR THE ANI INPUT FILE. IREAD(13) SEE CARD INPUT FILE CONTROLS. CONTROL PARAMETERS READ IF IHD(1).EQ.1 (T(1).I=1.12).ID.IHT.IHS.IHM.IDFM.IPM.IPP.ID1.ID3.ID4.IDAT2. IFG.IFLU.IFN.IPMT.OFMI.HYF T(1) TITLE (TWELVE 6-CHARACTER WORDS) ID PROMERY ID NUMBER IMT CROSS SECTION TABLE POSITION OF SIGMA TOTAL IMS CROSS SECTION TABLE POSITION OF SIGMA TOTAL IMS CROSS SECTION TABLE LENGTH IDFM 1/0=DENSITY FACTORS USED/NO

÷

IM/1/0=ENTER SHELL SOURCE BY INTERVAL+GROUP+ AND ANGLE/ IPM CD BY GROUP AND ANGLE ONLY/NONE CD IPP INTERVAL NUMBER THAT CONTAINS SHELL SOURCE IF IPM#1 CD 1/0=PRINT ANGULAR FLUX/NO CD 101 N/O=COMPUTE N ACTIVITIES BY ZONE/NO ACTIVITIES CD 103 1/0=CONFUTE N ACTIVITIES BY INTERVAL WHERE N REFERS CD 104 TO LOJZNO CD 1/0=EXECUTE DIFFUSION SOLUTION FUR SPECIFIED GROUPS/NC 10412 CD 1/0=COLLAPSE CHOSS SECTIONS/NO -CD IFG CD IFLU 0/1/2=STEP MODEL USED WHEN LINEAR EXTRAPOLATION YIELDS -NEGATIVE FLUX (MIXED MODE)/USE LINEAR HUDEL ONLY/ CD USE STEP MODEL ONLY CD 1/0=USE FLUX GUESS/FISSION GUESS. FLUX GUESS TAKEN FROM -CD 1FN GRP FLAS FILE. FISSION GUESS MUST BE ENTERED FROM CĐ CD CARDS. CD IPRT 0/1=PRINT CHUSS SECTIONS/NO DEM1 TRANSVERSE DIMENSION FOR VOID STREAMING CORRECTION CD NORMALLY 0.5. EPS/NYF IS USED AS CUNVERGENCE CRITERION RYF _ CD ON SCATTERING (TUTAL AND UP) CD С ------<u>۰</u>-C----CR FIXED SOURCE С READ IF TEVT.EQ.O.AND.IRD(2).EQ.I CC С READ IF M06.E8.11 CL ((D)(I+J)+I=I+IM)+J=I+IGM)С READ IF IPM.GT.0 CL (((QS(M+T+J)+M=1+MM)+I=1+1PM)+J=I+1GM) -С CD QD(I+J) DISTRIBUTED SUURCE FOR INTERVAL I AND GROUP J -SHELL SOURCE FOR DIRECTION M. INTERVAL I. AND QS(M+I+J) CD GROUP J CD OD AND US CANNOT BUTH BE PRESENT IN THE SAME PROBLEM. MM NUMBER OF ANGULAR DIRECTIONS. -CD CD -IFVT. IM. 1GM, AND MOD ARE SPECIFIED IN THE ADMNSTR FILE. CD С C----FISSION GUESS CR C сс READ IF IFN.EU.D.AND.IRD(3).EU.1 -C CL (FD(I) + I=1 + IM) -C CD FD(1) FISSION DENSITY BY INTERVAL r C-CR RADIUS MODIFIERS С -CC READ IF IEVT.E0.4.AND.IRD(4).EQ.1 -С -CL (PM(1)+1=1+IZM) RM(I) RADIUS MODIFIERS BY ZONE CD CD IEVT AND IZM ARE SPECIFIED IN THE ADMISTR FILE. C С C----CR DENSITY FACTURS С _ CC READ IF IDFM.EU.1.AND.IRD(5).EQ.1 С CL (DF(I) + I=1 + IM) C CD DE(1) DENSITY FACTORS BY INTERVAL IM SPECIFIED IN THE ADMNSTR FILE. CD С C---

(-----------ACTIVITY DATA CR С CC. READ IF ID3. ST. 0. AND. IRD (6) . EQ. 1 С CL (J3(1) + I =) + I03) + (J4(I) + I = 1 + I03)С MATERIAL NUMBERS FOR ACTIVITIES CD 13(1) J4(1) CROSS SECTION TABLE POSITION FOR ACTIVITIES CD С C----CR DIFFUSION MARKERS С сс READ IF IDAT2.EQ.1.AND.IND(7).EQ.1 С (1GT(I) + I=1 + 1GM) CL C 0/1=USE TRANSPORT/DIFFUSION THEORY IN GROUP I CD IGT(I) CD IGM SPECIFIED IN ADMNSTR FILE. C C ----CR ALBEDO С сс READ IF (IB01.EQ.3.OR. IR02.EQ.3) . AND. IR0(8) . EQ.1 С (ART(I)+I=1+IGM) (ALFT(I)+I=1+IGM) READ IF IB01+EQ.3.AND. CL CL 1802-EQ.3 (ART(I),I=1+IGM) READ IF IB02.EQ.3.AND.IB01.NE.3 CL СL (ALFT(I).I='.IGM) READ IF IBUL.EQ.3.AND.IB02.NE.3 С ART(I) ALBEDO BY GROUP FOR RIGHT BOUNDARY ALFT(I) ALBEDO BY GROUP FOR LEFT BOUNDARY IR01, IB02, AND IGM SPECIFIED IN ADMNSTR FILE. CD CD CD С C---_____ GROUP COLLAPSING PARAMETERS CR C -READ IF IFG.E4.1.AND.IRD(9).EQ.1 CC C ICON+IHTF+IHSF+IHMF+IPUN+(FGG(1)+I=1+IGM) CL С ČD 1/2=COLLAPSE MICRO CROSS SECTIONS/MACRO (MINUS IMPLIES I CON CELL WEIGHTING) CD POSITION OF SIGMA TOTAL IN COLLAPSED CROSS SECTIONS CD IHTE POSITION OF SELF-SCATTER SIGMA IN COLLAPSED CROSS CÐ IMSF SECTIONS (MINUS IMPLIES UPSCATTER REMOVAL) TABLE LENGTH UF COLLAPSED CROSS SECTIONS 1/0=PUNCH COLLAPSED CROSS SECTIONS/NO CD -IHME CD CD IPUN FGG(I) FEW-GROUP NUMBER FOR EACH MULTIGROUP CD SPECIFIED IN ADMNSTR FILE. IGM -CD С CEOF

******* C 4 0 C CF DOT INPT CE DOT2DB CODE-DEPENDENT INPUT DATA FILE C ****** CR RECORD CONTRUL С СC READ IF TREAD(14).LQ.3 С CL (1RD(J)+J=1+6) r CD TRD (J) READ RECURD J FROM CARDS (1/0=YES/NO). IF IREAD(14)-.EQ.1. ALL THE IRD(J) ARE SET INTERNALLY TO 1. IF -CD IREAD(14) IS EQUAL TO 2 OR 4. UMIT ALL CARD INPUT CD FOR THE DUT INPT FILE. CD SEE CARD INPUT FILE CONTROLS CD IRFAD(14) С ***** C-----C----CR CONTROL PARAMETERS С сс READ IF IRD(1).EQ.1 С CL (1D(I), I=1.12), A01.FXT.IHT.IHS.ITL.M05.S04.IAFT.M04.M06.IB01. 1802+1803+1804 CL С TITLE (TWELVE 6-CHARACTER WORDS) CD IÐ CD PROBLEM I.D. NUMBER AO I FLUX CALCULATION MODE CD νXT. 0=LINEAR EQUATIONS ONLY CD 1=LINEAR MODE + RECOMPUTE NEG. FLUX CD 2=STEP FUNCTION EQUATIONS ONLY CD CD THT CROSS SECTION TABLE POSITION OF SIGMA TOTAL CROSS SECTION TABLE POSITION OF SELF SCATTEP CD IHS _ CROSS SECTION TABLE LENGTH CD ITL NUMBER OF REGIONWISE ACTIVITIES CD M05 INITIAL INNER ITERATION MAXIMUM PER GROUP CD 1504 TAFT CD FLUX MOMENT AND ANGULAR FLUX OUTPUT TRIGGER CD 1=NO PRINT 2=PRINT CD NUMBER OF POINTWISE ACTIVITIES (M04.LE.M05) CD M04 DISTRIBUTED FIXED SOURCE INPUT OFTIONS CD M06 CD -1=NO DISTRIBUTED FIXED SOURCE INPUT 0=READ ONE BLOCK CONTAINING IGM WORDS CD 1=READ IGM BLOCKS EACH CONTAINING IM*JM WORDS. 2=READ TWO BLOCKS WITH THE FIRST CONTAINING IGM CD CÐ CD WORDS AND THE SECOND IMAJM WORDS. 3=READ THREE BLOCKS CONTAINING IGM, IM, AND JM WORDS-CD 4=READ TWO BLOCKS FOR EACH GROUP. THE-FIRST BLOCK -CONTAINS IM WOPDS AND THE SECOND CONTAINS JM WORDS-CD CD CD LEFT BOUNDARY CONDITION (0/1/2=VACUUM/REFLECT/PERIODIC). -1601 PIGHT BOUNDARY CONDITION (0/1/2/3/4=VACUUM/REFLECT/ CD 1905 PERIODIC/WHITE/INPUT ROUNDARY SOURCE) CD CD 1803 TOP BOUNDARY CONDITION (SAME OPTIONS AS IB02) BOTTOM BOUNDARY CONDITION (0/1/2/3=VACUUM/REFLECT/ CÐ **IR**04 CD PERIODICZ#H1[E) CD IGM. IM. AND JA SPECIFIED IN ADMNSTH FILE. С C-

C----CR DISTRIBUTED FIXED SOURCE C сс READ IF IEVT.FO.O.AND.MOG.GE.O.ANU.IRU(2).EU.I С сL (Z(G)+G=1+IGM) REAU IF MO6.EQ.0 С CD F1×50R(G+1+J)=7(G) 6=1+1GM 1=1.1M J≃l,JM С С DO 1 G=1,IGM CL 1 READ(N) ((FIXSOF(G+I+J)+I=1+IM)+J=1+JM) READ IF MO6.EG.1 С CL (Z(G) * G = 1 * IGM) * ((AY(I * J) * I = 1 * IM) * J = 1 * JM)READ IF MO6.E0.2 С CD $FIXSOR(G \bullet I \bullet J) = Z(G) = XY(1 \bullet J)$ С CL (Z(G)+G=1+IGM)+(X(I)+I=1+I*)+(Y(J)+J=1+JM) READ IF MO6.E0.3 С CD $FIXSOP(G \bullet I \bullet J) = Z(G) \bullet X(I) \bullet Y(J)$ С С DO 1 G=1.IGM CL 1 READ(N) (X(G+I)+1=1+1M)+(Y(G+J)+J=1+JM) KEAD IF MO0-EQ.4 С CD $FIXSOR(G \bullet I \bullet J) = x(G \bullet I) \bullet Y(G \bullet J)$ С сэ IEVT. IGM. IM. AND JM SPECIFIED IN ADMNSTR FILE. С C-CR FIXED BOUNDARY SUURCE ... С cc READ IF (1802-EQ.4-08-1803-EQ.4) - AND-180(3) - EQ.1 С (S(I) + I=1 + IBSS) CL С CD 1H55=0 CD IF (IB02.EQ.4) IB55=(I15+1A04/2)*JM*IGM ίD IF (1803.EQ.4) IUSS=IUSS+115+IM*IGM CD 115=IA04*(IA04+4)/4 CD 1404. IM. JM. AND 1GM SPECIFIED IN ADMNSTR FILE. SEE GEAP-13537 FOR OKDER OF SPECIFICATION OF S(1). DISTRIBUTED CD CD AND BOUNDARY SOURCES CANNOT BOTH BE SPECIFIED IN THE SAME PROBLEM-С C--C-----CR MATERIAL NUMBERS AND TABLE POSITIONS FOR ACTIVITY CALC. С READ IF M05.6T.0.AND. IRD(4).EQ.1 CС Ċ (MN(I) + I=1+M05; + (NPO5(1) + I=1+M05) CL C MATERIAL NOS. FOR ACTIVITY CALCULATION CD MN(I) +N=ACTIVITY OF MATERIAL N COMPUTED IN-ALL INTERVALS IN WHICH N IS -CD CD PRESENT. CD -N=ACTIVITY OF MATERIAL N COMPUTED CD IN ALL INTERVALS. CD 0=ACTIVITY COMPUTED AT EVERY CD INTERVAL FUR MIXTURE ASSIGNED TO CD THAT INTERVAL. CD CROSS SECTION TABLE POSITIONS FOR ACTIVITY NPOS(I) -CD CD CALCULATION. С ______ C-

```
C ----
69
           ZONE WIDTH SEAFCH PARAMETERS
C
            PEAD IF IEVT.LO.4.AND.190151.EQ.1
CC
Ċ
      (NPAU(1)+1=1+14)+(-400(1)+1=1+12)+(NAAL(J)+J=1+JN)+
СĽ
CL
      (AMOD(J)+J=1+J2)
C
CD
      NHAD(1)
                         SPECIFIES WHICH RADIAL MODIFIER IS USED IN THE -
CD
                         INTERVAL.
                         NADIAL MOUIFIERS
SPECIFIES WHICH AXIAL MODIFIER IS USED IN THE
CD
      PHOD (1)
CD
      NAXL(J)
CD
                         INTERVAL.
CD
      AMOD (J)
                         AXIAL HODIFIERS
      IEVT. IN. 12. JH. AND JZ SPECIFIED IN ADMINSTR FILE.
CD
С
C--
C-----
CR
            DIFFUSION MARKERS
С
CC
            READ IF 1411.E0.0.AND.100(5).E0.1
C
CL.
      (NDM(I)+I=1+IG4)
С
                         I=USE DIFFUSION THEORY IN GROUP 1
U=USE TRANSPORT THEORY IN GROUP I
CD
      NOM (1)
CD
      IAIL AND IGH SPECIFIED IN ADMINSTR FILE.
CD
C
C---
                   CEOF
        C.
С
ĊF
            CLB INPT
            CLUR CODE-DEPENDENT INPUT DATA FILE
CE
C
C+
        _____
C----
CP
            RECORD CONTHIL
С
            READ IF INEAD(15).EQ.3
СС
С
CL
      (1RD(J)+J=1+7)
С
                    PEAD RECORD J FROM CARDS (1/0=YES/NO). IF
CD
      (L) GAL
                   IREAD(15).EO.1. ALL THE IRD(J) ARE SET INTERNALLY
TO 1. IF IREAD(15) IS EQUAL TO 2 OR 4. OMIT ALL
INPUT FOR THE CLR INPT FILE.
CD
CÐ
CD
                   SEE CARD INPUT FILE CONTROLS
      IREAD(15)
CD
r
C-
```

C++-		,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	
AD	711	LE AND NEWTHONICS SELECTION	•
0 00	RF	N 1F 160(1).EQ.)	-
ĉ			-
cu	(TITL(1))	1=1+241+LIM+5EL	-
čο	TTL	ITLE (TWENTY FOUR 6-CHARACTER WORDS)	-
CD	LĪM (PU TIME LIMIT IMENT FOR THE CALCULATION	-
C0	SEL N	FUTRONICS SELECTION (3-CHARACTER HOLLERITH WORD)	-
CD		3HZAF=NU NEUTNONICS CALCULATION+ USE ZUNE=AVENAGED	-
CD CD		THAIMSINFINITE MEDIUM CALCULATION	_
čĎ	c	THERWISE DENOTES THE PARTICULAR FLUX HODULE TO BE USED	-
CD	1	N THE FUEL CYCLE CALCULATION. SHOUX, SHANI. AND SHOOT	-
CD	c	URHENTLY AVAILABLE.	-
с. С			
C == =		~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~~	
ČR	INF	INITE MEDIUM DAIA	-
c cc	WF A	0 1F SEL FU. 3HAIN. AND. 180(2) .FU.1	-
č	ACA	ne - me - nemen a mana antes destructures a 9 dates a gran a gran a s	-
ĈL	(INFD(I).	I=1+11)+(AB(1)+I=1(4)	-
CL	(A(])+1≠}	+INFD(1)) READ IF INFU(7).EQ.1	•
C			-
00	INFD(17	I INTITING NUMBER OF TIFRATIONS	-
ČĎ	INFD (3)	EIGENVALUE TYPE 10/1/2=KEFF/BUCKLING SEARCH/	-
CD		(1/V) ABSORBER SEARCH)	-
CD	INFD(4)	1/0=SOLVE ADJOINT PROBLEM EACH TIME THE SPECTRUM	-
CD		IS CALCULATED/NU NATEORIAL AD INTERPORTEM AT START-OF-CYCLE/NO	1
CD	INFD(57	1/0=SOLVE ADJOINT PROBLEM AT START-OF-CTCLE/NO	-
čĎ	INFD(7)	1/0=READ GROUP-DEPENDENT VALUES OF 8++2/NO	-
CD	INFD(8)	NUMBER OF NUCLIDES	•
CD	INFD(9)	1/0=FRINT ITERATIVE DATA/NO	-
CD CD		1/0=PRINT MACROSCOPIC CRUSS SECTIONS/NO 1/0=PRINT MACROSCOPIC SCATTERING RERNEL/NO	-
CD	AB(1)	DESIRED FLUX CONVERGENCE	-
CD	AB (2)	DESTRED SOURCE AND FISSION RATE CONVERGENCE	-
CD	A913)	VALUE OF B**2 USED FOR ALL GROUPS UNLESS INFD(7)=1	-
00	AH(4) B(1)	GROUD-OFPENDENT R##2	
č	0(1)		-
C			
CR	DEP	LETION HISTORY AND EDITING UPTIONS	-
C CC	REA	D IF IRD(3).EQ.1	-
C CL	(HOLL(I)+	I=1+12)+(ND(I)+I=1+17)+(DD(I)+I=1+17)+(IEDT(I)+I=1+12)	-
C			-
CD	HOLL(I)	UEPLETION MISTORY TITLE (TWELVE 6-CHARACTER WORDS)	-
CD CD	ND(2)	MAX NUMBER OF DEPLETION TIME SIEPS FOR FIRST CYCLE	-
ČD	NP (3)	SAME AS ND(2) BUT FUR SECOND CYCLE	-
CD	NÐ (4)	SAME AS ND(3) BUT FOR ALL OTHER CYCLES	<i>.</i> -
CD	ND (5)	NUMBER OF SUBSTEPS FOR EACH DEPLETION STEP	-
CD	ND (6)	INFARGAVERAGED FILLY VALUES OUED SUBSTED	-
CD		INTERVALS/NO	_
CD .	ND(7)	0/1=SAVE START-OF-STEP NUCLIDE DENSITIES FOR END-OF	-
CD		CYCLE RECOVERY/NO	-
CD	ND(R)	0/1=RENOPMALIZE FLUX AFTER EACH SUBSTEP TO ACHIEVE	-
CD CD	NOTON	DESTRED POWER LEVELING 1/0=RETURN POWER LEVELING ORTGINNE SPECS (SEE OD/1)	_
CD	11.2 1 2 2	AND DD(B)) AFTER EACH NEUTRONICS CALC/ADJUST	-
CD		POWER LEVEL TO GIVE DESIRED AVERAGE OVER THE	-
CD		TIME STEPS (SEE DD(5) AND DD(6))	-

CD	ND(10)	OPTION ON LIMITING POWER DENSITY (SEE UD(4))	-
¢0		U=TERMINATE IF LIMIT EXCEEDED	-
CD		I=CONTINUE WITHOUT CHANGE IN PROCEDURE	-
CD		2-ADJUST HONER LEVEL TO LIMIT THE POWER DENSITY	1-
cn.		3=ADJUST PONER TO HAINTAIN THE POWER DENSITY	· _
rn -	ND (1))	OVERTHENDING DIMESTRY DUGA APPLIES TO ZONESZ	_
C0	34123.3.3	Children Contra Children Children arrenda na Earest	_
C() 20	101131	SUPECINGS - AUCOARE EXEMPLIED SOLDS AND FRE TO	-
0	NUTLET	UTIELING AVERAGE CAUSORE DUTAT AFFETES TO	-
CU #5			. "
CU	NHILLI	INCOME CIRCARC CALC FOR END-OF-CTULE CONDITIONS/NO)-
CD	ND (14)	OPTION ON END-OF-CTCLE EXTRAPOLATION	-
CD		-2=EXTRAPOLATE ON EXPOSURE	-
CD		-I=EXTRADULATE ON CONTROL POISON	-
CD		U=EXTHAPOLATE ON EACESS HEACTIVITY	-
co		LEUU NOT EXTRAPOLATE	-
00	ND (15)	OPTION FOR CALCULATION OF CONVERSION RATIO	-
CD		DEUSE CAPTURE HATE IN FERTILE MATERIAL	-
CD		I=USE (N+DAMMA) HATE IN FERTILE MATERIAL	-
n.	ND (16)	NUMBER OF FRENDY GROUPS	-
50	NO (17)	NUMBER OF NATESTAL ZONES	_
-n	00(1)	DEFENSION OF THE DOUBLY LEVEL LATE COD START AVAIR	_
.u .n	00(1)	NELENCING CONCENTRE LETEL INTERIOR FENDE LIGUE Datio of Theomai to sisting subdry	_
, CJ	00121	THEID OF THERTAL IN FIGURES IN THE MODEL	
.U	00(3)	TRACTION OF COME ENCLODED IN THE MODEL	.*
0	100(4)	LIMITING POWER DENSITY (W/CC), USE OF DD(4) DEPENDS	>-
.0		ON OPTIONS CHOSEN FOR ND(10) AND ND(11).	-
:D	DD (5)	DESIRED POWER LEVEL (MWT) FOR ALL CLASS 1 ZONES	-
:D	DD(6)	SAME AS DU(S) BUT FOR ALL CLASS 1 AND 2 ZONES	•
0		SUMMED. FLUX LEVEL IS ADJUSTED TO MAINTAIN EITHER	•
D		DD(5) OR DD(6) . WHICHEVER REQUIRES THE HIGHEST FLUX.	, -
D	DD(7)	MAXIMUM EXPOSURE TIME (DAYS) FUR ANY CYCLE	-
D	00(8)	COHE HOWER LEVEL (WT) FOR SECOND AND SUCCEED. CYCLES	;-
n	00(9)	LIMITING AVER HEAVY METAL EXPOSURE (MWT/NETRIC TON)	
ň	00177	TERMINATING CYCLE	-
ň	00(10)	DESTRED END-DE-CYCLE MULTIPLICATION FACTOR	_
	00(10)	NECTUEN ENDINE FORCE POLITICATION ACHTON LASS IN	_
υ	00111	DESTRED ENDIGED CONTRACTION NEULINA FOSS IN	
U	201101	CUNINUL ROBURDER OFFICIAL ROBURDER	•
U	00(12)	DEPLETION TIME (DATS) FOR FIMST TIME STEP OF FIRST	•
Ð		CYCLE	-
D	00(13)	SAME AS DD(12) HUT FOR SECOND TIME STEP	-
)	00(14)	SAME AS OD(13) BUT FOR REMAINDER OF TIME STEPS	-
D	DD (15)	SAME AS DD(12) BUT FOR REMAINDER OF CYCLES	-
D	DD(16)	SAME AS DU(13) BUT FOR REMAINDER DF CYCLES	-
)	00:17)	SAFE AS DUILA BUT FOR REMAINDER OF CYCLES	-
, I	IEDT(1)	N/0=PRINT NUCLIDE DENSITIES BY ZONE EVERY NTH	-
ń	******	TIME STEP/NO	-
,	1607 (2)	SAME AS TEDTALL BUT BY SUMPONE	_
04 10	101127	DAME AD LEVILLA DAT DI DUDLUNE Nachdutnit Googe Deaction Cates the thirthigh	_
,	TEN1 (3)	N/V-FRINI ORUSS BEAULIUN RALES IN INDIVIDUAL	-
,		NULLINES EVENT NIN TIME STEP/NU	-
U	LEDI(4)	NZU=PRINT REACTION RATES IN INDIVIDUAL NUCLIDES BY	-
D		ZONE EVERY NTH TIME STEP/NO	-
)	IEDT(5)	N/O=PRINT ZONE-AVERAGE FLUX BY GROUP EVERY NTH TIME	-
D		STEP/NO	-
D	IEDT(6)	N/O=PRINT ZUNE-AVERAGE POWER DENSITIES EVERY NTH	-
0		TIME STEP/NO	-
ก	TEDT(7)	NZO=PRINT DECAY ACTIVITY BY NUCLIDE EVERY NTH	-
ñ		TIME STEP/NU	_
ň	TEDTION	120-PRINT FULL INFORMATION USEFUL IN OFBUGGING FOR	-
	1001	FACH CYCLEVNO	_
,	TEDTION	ENUT UTULE/NV Nampuint coace centions sytoanted form will troop	_
,	1501(8)	INDERKINE CROSS SECTIONS EXTRACTED FROM MULTIGRY	-
0		FILE/NU	-
)	IEDT(10)	1/0=PRINT START-OF-CYCLE ZONE NUCLIDE DENSITIES NO	-
)	IEDT(11)	1/0=PRINT END-OF-CYCLE NUCLIDE DENSITIES BY SUBZONE -	-
		/N0 ·	-
)			
)	TEDT (12)	1/0=PRINT END-OF-CYCLE FISSILE LOADINGS/NO	-
D D	TEOT (12)	I/O=PRINT END-OF-CYCLE FISSILE LOADINGS/NO	-

•

68	ZUNE	CLASSIFICATION DATA	•
cc	#E AD	IF 160(4).EU.1	-
C			-
C	NZSEI		-
CL	(NE (N) +NL (N) •NSUB2 (N) •NZCL (N) •NDPL (N) •ZA (N) •ZB (N) •N≏1 •NZSET)	-
C CD	NZSET	NUMBER OF ZONE SETS, ZONES IN EACH SET HAVE THE	-
čĎ		SAME NUMBER OF SUBZONES. HELONG TO THE SAME ZONE	~
CD		CLASS. AND ARE ALL EITHER DEPLETING OR NONDEPLETING.	,
CD	NE (N)	LAST ZONE NUMBER (OF A CONSECUTIVE SET) IN SET N	-
CD	NSUBZ (N)	NUMBER (LE.24) OF SUBZONES IN EACH ZONE OF SET N.	•
CD		IF NSUBZ(N).GT.1. SUBZONE NUCLIDE CONCENTRATIONS	-
CD	NZCL (N)	A REFERENCE NUMBER (LE.25) IDENTIFYING THE ZONE	-
CD		CLASS.	-
CD	NOPL (N)	-1/0=20NE SET N CONTAINS NONDEPLETING/DEPLETING	•
CD		DEPLETING ZONES.	
ĊD	ZA (N)	FIRST WURD (6 CHARACTERS) OF ZONE CLASS NAME	-
CD	ZR (N)	SECOND WORD (6 CHARACTERS) OF ZONE CLASS NAME	-
с			-
-			
C====		***	
ČR	FISSI	ON YIELD DATA	-
C			-
CC C	READ	IF IRD(5)+EG+1	-
ČL	JNO+NF IPRO		-
C		N	•
C C	(NNUF (1)+1=		-
ČL	(N1 (J) + (YLD	([.J)+I=1,JNQ),J=1+NFIPRO)	-
C		TOTAL MUMBER OF MUCHTORS THAT DESCRIPTION SPACE	•
CD	NEIPRO	TOTAL NUMBER OF FISSION-PRODUCT NUCLIDES	-
CD	NNOF (1)	REFERENCE I.D. NUMBERS OF NUCLIDES THAT PRODUCE	-
CD		FISSION PRODUCTS	-
CD	NI(J) YED(T+J)	YTELD (PER EISSION) OF FISSION PRODUCT J	-
ČĎ	12011107	FRUM FISSIONABLE NUCLIDE NNOF(1)	-
C			-
C			-
C			-
CR C	NUCLI	DE CHAIN SPECIFICATIONS	-
čc	READ	IF IRD(0).EQ.1	•
C			-
C	NL		-
ČL	(LC(N) +N=1+	NC)	-
C	///ID/I N)	$(TYDE (T, N)) = T = 3 = T \frac{1}{2} (T = T + T = 2 + T = N) = 1 + (T = 1) + $	-
C	(((1))))	al the (fala) a 1 - 5 a f f) a 10 (10 a la 14) } a la - 1 a la C)	-
CD	NC	TOTAL NUMBER (GE.1.AND.LE.100) OF CHAINS TO BE	-
CD		SPECIFIED	•
čĎ	20111	TO 2*NONUC(N)-1 WHERE NONUC(N) IS THE NUMBER OF	-
CD		NUCLIDES IN CHAIN N.	•
CD	ID(I+N)	REFERENCE 1.D. NUMBER FOR 1TH NUCLIDE IN CHAIN N	•
CD	1411E11041	CHAIN N.	
CD		-I=DECAY	•
CD		I=TOTAL CAPTURE -	•
CD		3=N+ALPHA -	•

C-----

. . .

CD 4=N.2N CD 5≂N•P B=NONDEPLETION--RESERVED FOR ONE-NUCLIDE CHAIN CU INDICATING THE NUCLIDE CAPTURES BACK TO CD CD ITSFIF. -J=PARTS PER 100+000 FISSION TREATED AS CAPTURE -CD 10=PARTS PER 1+000+000 TOTAL CAPTURE CD CD I1=NONUC(N)-1CD 12=11+1 С SECONDARY SOURCE ROUTES TO NUCLIDES MUST BE INDICATED FOR CORRECT-CC CC CALCULATIONS. A SEPARATE CHAIN DESCRIPTION IS REQUIRED FOR SECONDARY ROUTES AND SECONDARY-ROUTE CHAINS ARE FLAGGED BY СC ADDING 200+000 TO ALL NUCLIDE I.D. NUMBERS IN THE CHAIN. СС FOR A NONDEPLETING ONE-NUCLIDE CHAIN. THE CHAIN CONTAINS ONLY сс CC THE TWO ENTRIES ID(1+N) AND NTYPE(1+N)=8 WITH LC(N)=2. C C -C--------CR SUBZONE NUCLIDE CONCENTRATIONS С READ IF IND(7).EQ.1.AND.NSUBZ(N).GT.1 FOR ANY N (SEE ZONE CC CLASSIFICATION DATA). СС С IFLG+NZSET CL С С DO 5 N=1+NZSET READ IF IFLG.EQ.1 READ(M) MF(N) . ML(N) CL CL READ(M) (NUCL(I \bullet N) \bullet I=1 \bullet NSUBZ(N)) READ IF IFLG.EQ.1 C D0 5 I=1+NSUBZ(N)5 READ(M) (IB(J+I+N)+C(J+I+N)+J=I+NUCL(I+N)) READ IF IFLG.E4.1 CL C 1/0=PRUCEED WITH REMAINDER OF INPUT/SUBZONE CONCENTR-CD IFLG ATIONS ARE SET EQUAL TO ZONE CONCENTRATIONS IN CD ZONEDENS FILE AND REMAINING INPUT IS NOT READ. CD CD TOTAL NUMBER OF ZONE SETS. ZONES IN EACH SET MUST NZSET. HAVE THE SAME NUMBER OF SUBZUNES AND THE SAME SUR-CD ZONE CONCENTRATIONS. NOT USED IF IFLG.EG.0 FIRST ZONE NUMBER (OF A CONSECUTIVE SET) IN SET N CD MF (N) CD CD ML (N) LAST ZONE NUMBER (OF A CONSECUTIVE SET) IN SET N TOTAL NUMBER OF NUCLIDES IN ITH SUBZONE OF ZONES IN -CD NUCL (I+N) SET N CĐ NUMBER OF SUBZONES IN EACH ZONE OF ZONE SET N CÐ NSUBZ (N) (MUST AGREE WITH DATA GIVEN IN ZONE CLASSIFICATION) CD -REFERENCE I.D. NUMBER OF JTH NUCLIDE IN SUBZONE I CD IP(J+I+N) CD OF ZONE SET N -ATOM DENSITY (10**24 ATOMS/CC) OF JTH NUCLIDE IN CD $C(J_{I}N)$ -SUBZONE I OF ZONE SET N. CD С сс IF SUBZONE CONCENTRATIONS ARE READ. A NEW ZONEDENS FILE IS -WPITTEN USING ZONE-AVERAGE CONCENTRATIONS COMPUTED FROM THE SUBZONE CONCENTRATIONS. THE ZONE-AVERAGE CUNCENTRATION OF ANY NUCLIDE IS THE SUM OF THE SUBTONE CONCENTRATIONS DIVIDED BY THE СС СС сс -NUMBER OF SUBZONES. CC С _____ C-CEOF

С CF DAC INPT CE DAC CODE-DEPENDENT INPUT DATA FILE С CR RECORD CONTROL С CC READ IF IREAU(16).EQ.3 С (IRD(J)+J=1+3) CL С CD IRD (J) READ RECORD J FROM CARDS (1/0=YES/NO). IF IREAD(16)-.EQ.1. ALL THE IPD(J) ARE SET INTERNALLY TO 1. IF CD CD IREAD(16) IS EQUAL TO 2 OR 4, UMIT ALL CARD INPUT CD FOR THE DAC INPT FILE. SEE CARD INPUT FILE CONTROLS. CD IREAD(16) С C=-CR. CONTROL PARAMETERS С сс READ IF IRD(1).EQ.1 C CL ITP+ID1+MAXLP+MAXLD+NFI+NCAS С CD ITP CROSS SECTION CONTROL (1/2=READ REFERENCE MULTIGRP FILE ONLY/READ BOTH REFERENCE AND PERTURBED MULTIGRP FILES). CD FLUX CONTROL (1/0=USE ANGULAR FLUXES/SCALAR FLUXES AND CD IDI CD CURRENTS) . CD MAXLP MAXIMUM NUMBER OF GROUPS TO WHICH THERE CAN BE UPSCATTER -MAXIMUM NUMBER OF GROUPS TO WHICH THERE CAN BE CD MAXLD DOWNSCATTER CD ЗÜ NET NUMBER OF F'SSIONABLE ISOTOPES NUMBER OF PERTURBATION CASES TO BE PROCESSED CD NCAS C C--C-----CR MESH MODIFIERS AND MESH MODIFIER NUMBERS С CC READ IF IEVT.EQ.4.AND.IRD(2).EQ.1 -С CL (XM(I) + I = I + IZ) + (NXM(I) + I = I + IM)С CL (YM(J) * J=1 * JZ) * (NYM(J) * J=1 * JM)READ IF IA01.GT.I С (ZM(K)+K=1+KZ)+(NZM(K)+K=1+KM) CL READ IF 1A01.GT.2 С CD XM(I) MESH MODIFIERS FOR 1ST DIMENSION NXM(I) MESH MODIFIER NUMBERS FOR IST DIMENSION. THESE NUMBERS CD CD SPECIFY WHICH OF THE XM(I) ARE TO BE USED IN EACH CD INTERVAL. CD (L) MY MESH MODIFIERS FOR 2ND DIMENSION CD MESH MODIFIER NUMBERS FOR 2ND DIMENSION NYM(J) MESH MODIFIERS FOR JRD DIMENSION CD ZM(K) CD NZM(K) MESH MODIFIER NUMBERS FOR 3RD DIMENSION CD IZ+JZ+KZ+IM+JM+KM++IAU1+ AND IEVT DEFINED IN ADMNSTR. С C---

C		***************************************
ČR	PERTU	RBATION CASE DATA -
cc	READ	IF NCAS.(T.U.AND.IRU(3).EQ.1
с сс	DO 5 N=1.NC	- AS
C CL	ITPP+NPM+NP	Ř -
CL	(((MPA(I+J+	K)+I=1+IM)+J=1+JM)+K=1+KM) REAU IF NPM+GT+0
CL.	(MPZ(I)+I=1	•NPR) • (MP(I) • I=1 • NPM) • (MPC(I) • I=1 • NPM) •
C		
CL 5 C	(XDEP(I),I=	INPM) READ IF NPM.GI.U.ANU.IIPP.NE.2
CD CD	ITPP	TYPE PERTURBATION (1/2/3=DENSITY/CROSS SECTION/BOTH) - SET ITP9.6T.1 ONLY IF ITP.EQ.2 -
CD	NPM	NUMBER OF PERTURBATION MIXTURE SPECIFICATIONS -
CD	NPR	NUMBER OF PERTURBATION ZONES (NPR.GT.O IF NPM.GT.O) -
CD	MPA(I+J+K)	PERTURBATION ZONE NUMBERS BY INTERVAL -
CD	MPZ(I)	PERTURBATION MATERIAL NUMBERS BY ZONE -
CD	MP(I)	PERTURBATION MIX NUMBERS -
CD	MPC(I)	PERTURBATION MIX COMMANDS -
CD	XDE(I)	PERTURBATION MIA DENSITIES -
CD	XDEP(I)	PERTURBED MIX DENSITIES .
С		a
C	**********	
CEOF		

C+++++++++++++++++++++++++++++++++++++			
C		-	
CF	FILE AND RECORD P	RINT CONTROLS	
С		•	
C * * * *	*******	******	
C		***************************************	
CR	FILE PRINT CONTRO	LS -	
С		-	
CL	IPRIN(I)+I=1+16	-	
С		-	
CD	IPRIN	=I, SKIP FILE -	
CD		=2, PRINT ENTIRE FILE -	
CD		=3, PRINT SELECTED RECORDS IN FILE -	
С		-	
CD	I	=1. ADMNSTR FILE -	
CD		=2, SN CONS FILE -	
CD		= 3, INTQUANT FILE -	
CD		=4, GEO DIST FILE -	
CD		=5; MIX DATA FILE -	
CD		=6, GRP FLUX FILE, REGULAR	
CD		=7. GRP FLUX FILE, ADJOINT -	
CD		=H, MULTIGRP FILE, REFERENCE -	
CD		=9, MULTIGRP FILE, PERTURBED -	
CD		=10, ZONE DENS FILE -	
CD		=11, ETX INPT FILE -	
CD		=12+ OUX INPT FILE -	
CD		=13, ANI INPT FILE -	
CD		=14, DOT INPT FILE -	
CD		=15, CLB INPT FILE -	
CD		=16, DAC INPT FILE -	
С		•	
C		, ************************************	

وروا والألاحية

C---CR ADMNSTR FILE RECORD PRINTING CONTROLS С NO RECORD CONTROLS - PRINT ENTIRE FILE IF IPRIN(1) = 2 OR = 3 CC C _____ C---SN CONS FILE RECORD PRINTING CONTROLS CR Ĉ. сc NO RECORD CONTROLS - PRINT ENTIRE FILE IF IPRIN(2) = 2 OR = 3 ________ C----CR INTQUANT FILE RECORD PRINTING CONTROLS READ IF IPRIN(3) = 3 С CL IPR(1)+1=1+3 С PRINT RECORD I. 1=YES, 0=NO CD IPR(I) C =1. DERIVED CONSTANTS CD I =2. ZONE AVERAGED FLUXES =3. ZONE VOLUMES CD CD С C • C______ GEO DIST FILE RECORD PRINTING CUNTROLS READ IF IPRIN(4) = 3 CR C CL IPR(]) + I=1+4 С PRINT RECORD I. 1=YES. 0=NO IPR(I) CD С ĊD =I . BOUNDARIES I =2. MATERIAL NUMBERS CD =3+ ZONE NUMBERS CD CD =4+ BUCKLING С C------C----------CR MIX DATA FILE RECORD PRINTING CONTROLS READ IF IPRIN(5) = 3 C CL IPR(1)+1=1+4 C CD IPR(I) PRINT RECORD I, 1=YES, 0=NO С CD =1. MIX NUMBERS 1 CÐ =2. MIX COMMAND =3, MIX DENSITIES CD =4. ISOTOPE NUMBERS CD С C. C----CR REGULAR GRP FLUX FILE RECORD PRINTING CUNTRULS CR PEAD IF IPRIN(6) = 3С IPP(I) + I = 1 + 3CL C CÐ IPR(I) PRINT RECORD I. 1=YES. 0=NO С =1, TOTAL FLUX CD I =2, CURRENTS CD CD =3. ANGULAR FLUX С

0

C---CR ABJOINT GRP FLUX FILE RECORD PRINTING CONTROLS CR READ IF IPRIN(7) = 3С CL IPP(I) + I = 1,3C PRINT RECORD I. 1=YES. 0=NO CD IPR(I) C CD I =1, TOTAL FLUX CD =2+ CURRENTS =3+ ANGULAR FLUX CÐ С -----C٠ C-MULTIGRP FILE RECORD PRINTING CONTROLS READ IF IPRIN(8) = 3 CR €. CL IPR(I)+I=1+13 С IPR(I) PRINT RECORD I. 1=YES, 0=NO CD С CD =1. FILE TITLE AND PARAMETERS 1 CD =2, ISOTOPE IDENTIFICATION CD =3, GROUP STRUCTURE CD =4. DELAYED NEUTRON DECAY CONSTANTS =5. DELAYED NEUTRON SPECTRUM CD CD =6+ DELAYED NEUTRON ABUNDANCES =7, ISOTOPE PARAMETERS CD =8. ISOTOPE FISSION SPECTRUM CD CD =9, PRINCIPAL CROSS-SECTIONS CD =10. INELASTIC SCATTERING CROSS-SECTIONS =11. ELASTIC SCATTERING CROSS-SECTIONS CD =12, N2N SCATTERING CROSS-SECTIONS CD CD =13, BONDARENKO С C-------C-PERTURBED MULTIGRP FILE RECORD PRINTING CONTROLS CR CR READ IF IPRIN(9) = 3 С CL IPR(1), I=1,13 С IPR(I) PRINT RECORD RECORD I: I=YES; 0=NO CD С CD I =1, FILE TITLE AND PARAMETERS CD =2, ISOTOPE IDENTIFICATION CD =3, GROUP STRUCTURE =4, DELAYED NEUTRON DECAY CONSTANTS CD =5, DELAYED NEUTRON SPECTRUM CD =6, DELAYED NEUTRON ABUNDANCES CD =7, ISOTOPE PARAMETERS CD =8, ISOTOPE FISSION SPECTRUM CD CD =9. PRINCIPAL CROSS-SECTIONS CD =10. INELASTIC SCATTERING CROSS-SECTIONS CD =11+ ELASTIC SCATTERING CROSS-SECTIONS =12, N2N SCATTERING CROSS-SECTIONS CD CD =13. BONDARENKO DATA С C---CR ZONE DENSITIES FILE RECORD PRINTING CONTROLS С CC NO RECORD CONTROLS. PRINT ENTIRE FILE IF IPRIN(10) = 2 OR =3 С C---

```
0.00
     ETXIN FILE RECORD PRINTING CONTROLS - READ IF IPRIN(11) = 3
CR
С
CL
     IPR(I), I=1,3
С
                    PRINT RECORD I. 1=YES. 0=NO
CD
     IPR(I)
C
                    =1. DATA COMMON TO ALL ISUTOPES
CD
     1
                    =2+ ISOTOPE DEPENDENT DATA
CD
CD
                    =3. LIBRARY UPDATE DATA
С
                _____
C---
C====
     OBXIN FILE RECORD PRINTING CONTROLS - READ IF IPRIN(12) = 3
CR
С
     IPR(I) + 1=1,7
CL
C
                    PRINT RECORD I. 1=YES. 0=NO
CD
     IPR(I)
C.
CD
                    =1. TITLE AND CONTROL PARAMETERS
     I
CD
                    =2. BUCKLING MODIFIERS
CD
                     =3, FISSION FRACTIONS AND VELOCITIES
СD
                     =4, ZONE MODIFIERS
                                                             -
CD
                     =5. XSEC GROUP COLLAPSING DATA
                    =6, RESONANCE SHIELDING DATA
CD
                    =7. MULTIGRP FILE DATA
CD
С
C---
                CR
     ANIIN FILE RECORD PRINTING CONTROLS - READ IF IPRIN(13) = 3
С
CL
    IPR(I),I=1,9
С
CD
     IPR(I)
                    PRINT RECORD I. 1=YES. 0=NO
                                                             -
С
CD
     I
                    =1. CONTROL PARAMETERS
CD
                    =2, FIXED SOURCE
                    =3. FISSION GUESS
CD
CD
                    =4+ RADIUS MODIFIERS
CD
                    =5, DENSITY FACTORS
CD
                    =6. ACTIVITY DATA
CD
                    =7, DIFFUSION MARKERS
CD
                    =8, ALBEDU
                    =9. XSEC GROUP COLLAPSE DATA
CD
С
C---
     C----
    _________
CR
     DOTIN FILE RECORD PRINTING CONTROLS - READ IF IPRIN(14) = 3
С
CD
     IPR(I)
                    PRINT RECORD I, 1=YES, 0=NO
С
CD
     I
                    =1, CONTROL PARAMETERS
                    =2. DISTRIBUTED FIXED SOURCE
CD
                    =3. FIXED BOUNDARY SOURCE
CD
СD
                    =4. ACTIVITY DATA
CD
                    =5, ZONE WIDTH SEARCH PARAMETERS
CD
                    =6. DIFFUSION MARKERS
С
C---
        ********************
```

***** C--CLBIN FILE RECORD PRINTING CONTROLS - READ IF IPRIN(15) = 3 CR С IPR(I) + 1=1+7 CL С PRINT RECORD I. 1=YES= 0=NO I66(J) CD С =1. TITLE AND NEUTRONICS SELECTION CD I =2+ INFINITE MEDIUM DATA =3+ DEPLETION HISTORY AND EDITING OPTIONS CD CD =4, ZONE CLASSIFICATION DATA CD =5. FISSION YIELD DATA CÐ =6, NUCLIDE CHAIN SPECIFICATIONS CD =7, SUBZONE NUCLIDE CONCENTRATIONS CD С C--______ C----DACIN FILE RECORD PRINTING CONTROLS - READ IF IPRIN(16) = 3 CR -С IPR(I)+1=1+3 CL С PRINT RECORD I. 1=YES. 0=NO CÐ IPR(I) С =1+ CONTROL PARAMETERS ĊD I =2. MESH MODIFIERS AND MESH MODIFIER NUMBERS CD =3+ PERTURBATION CASE DATA CD С C----CEOF

C				
СR —	PRO	DBLEM TITLE -		
C		-		
CC	CAF	RD TYPE OI MUST ALWAYS BE PRESENT -		
С		-		
Շև	FORMAT(12	FORMAT(12+4X+9A8) .		
С		-		
CD	COLUMNS	CONTENTS -		
CD	=======	*======================================		
CD	1-2			
CD	7-78	ANY ALPHANUMERIC CHARACTERS -		
С		-		
C	********			
_				
C				

Cΰ	COLUMNS	CONTENTS	•
CD	======	***************************************	=-
CD	1-2	02	
CD	19-24	PRINT OPTION FLAG FOR CONTAINER ARRAY, TYPICALLY 0.	-
		DEFAULIEU.	-
		U=NU TRACE OR DUMPS 1=DUMPS ONLY	-
CD		2=TRACE ONLY	-
CD		3=TRACE AND DUMPS	_
Ċ			-
C			
(>>>***********************************	~ -
CK C	GEI	NERAL PRODUCT SPECIFICATIONS	-
čc	CA	PD TYPE 03 MUST ALWAYS HE PRESENT	Ξ
č	0.1		_
ČL	FORMAT(I	2,10X,616,E12.5,16)	-
С			-
CD	COLUMNS	CONTENTS	-
CD			E
CD	1=2		-
	13-18	FUNDAMENTAL MODE TYPE	-
CD CD		2=CONSISTENT BI FOR ISOTROPIC NEUTRONICS	-
CD		3=CONSISTENT PL FOR ISOTROPIC NEUTRONICS	_
CD		4=CONSISTENT B1 FUR ANISOTROPIC NEUTRONICS	-
CD		5=CONSISTENT P1 FOR ANISOTROPIC NEUTRONICS	-
CD	19-24	FUEL PIN GEOMETRY (1/2=SLAB/CYLINDER)	a
CD	25-30	LIBRARY IDENTIFICATION NUMBER OF ISOTOPE TO BE USED AS	-
CD		FISSION SOURCE.	-
CD	31-36	0/1=CALCULATE BROAD GROUP CROSS SECTIONS/NO	-
CD	37-42	OVITALL FINE GROUP PROBLEM/ULTRAFINE GROUP PROBLEM	-
	43-48	FINE GROUP WEIGHTING SPECTRUM OPTION	-
		2=CONSTANT SPECTRUM	-
CD		3=F SPECTRUM	_
CD	49-60	FINE GROUP LETHARGY WIDTH, TYPICALLY 0.25. MUST BE	-
CD		ONE OF THE VALUES LISTED BELOW. THE CORRESPONDING	-
CD		NUMBER N OF ULTRAFINE GROUPS PER FINE GROUP IS ALSO	6 53
CD		GIVEN FOR EACH LETHARGY WIDTH.	-
CD		LETHARGY WIDTH N	-
			-
		0+33335C-3 I	-
CD		2.50000E-2 3	_
ĊD		3.33333E-2 4	-
CD		4•16667E-2 5	-
CD		5.00000E-2 6	-
CD		P•33333E-2 10	-
CD			-
CD CD		1+CDUVUE=1 15 1-66667E=1 20	-
CD CD		2.500005=1 30	-
CD		5,00000E-1 60	-
CD	61-66	CALCULATION PATH OPTION	-
CD		1=COMPLETE MC**2 PROBLEM	-
CD		2=CALCULATE MESOLVED AND UNRESOLVED RESONANCE CROSS	-
CD		SECTIONS ONLY	-
Ç		•	-
(·		,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	-
•			
c			-
CR	BRO	AD GROUP SPECIFICATIONS -	•
с		•	-
CC	CAR	D TYPE 04 MUST ALWAYS BE PRESENT -	•
C Cl		-104-3(16-612-5))	•
ιL	rumma (UC)	+10443(10+112+3))	
-		-	

CONTENTS CD COLUMNS CD -----CD 1-2 n 4 CD 13-18 BROAD GROUP NUMBER UPPER ENERGY OF GROUP (EV) CD 19-30 CD 31-36 BROAD GROUP NUMBER CD 37-48 UPPER ENERGY OF GROUP (EV) BROAD GROUP NUMBER 49-54 CD CD 55-66 UPPER ENERGY OF GROUP (EV) C. AS MANY TYPE 04 CARDS ARE USED AS ARE NECESSARY TO SPECIFY THE сc ENERGY BOUNDARIES. GROUP 1 IS THE GROUP OF HIGHEST ENERGY. cc С c C-CR FUEL PIN SPECIFICATIONS Ċ CC CARD TYPE 05 15 OPTIONAL £ FORMAT(12,10X,2E12.5) CL C CD COLUMNS CONTENTS CD z2==2=3 CD 1-2 05 CD 13-24 RADIUS OF FUEL PIN IF COLS. 19-24 UN CARD TYPE 03 EQUAL-2. HALF THICKNESS OF FUEL SLAB IF COLS. 19-24 ON CARD CD CD TYPE 03 EQUAL 1. CD 25-36 RADIUS OF OUTER CLAD-COOLANT REGION IF COLS. 19-24 ON CARD TYPE 03 EQUAL 2. OUTER BOUND OF CLAD-COOLANT CD REGION RELATIVE TO CENTER OF FUEL SLAB IF COLS.19-24 CD ON CARD TYPE 03 EQUAL 1. CD С cc CARD TYPE 05 IS PERTINENT ONLY FOR A HETEROGENEOUS PROBLEM. С C-C ---PROBLEM COMPOSITION SPECIFICATIONS CR С CC CARD TYPE 26 MUST ALWAYS BE PRESENT С CL FORMAT (12+4X+2A6+16+4E12+5) С CD COLUMNS CONTENTS CD =2==2== CD 1-2 06 NUCLIDE IDENTIFICATION LABEL ON LIBRARY. LABEL MUST CD 7-12 BE LEFT JUSTIFIED IN FORMAT FIELD WITH IMBEDDED BLANKS -CÐ CD PRESERVED. ISOTOPE NAME. THIS NAME CAN BE ANY ALIAS NAME USER CD 13-18 CD WISHES. CĐ 19-24 LEGENDRE TREATMENT SPECIFICATIONS 0=NON-LEGENDRE TREATMENT OF ELASTIC SCATTERING CD CD 1=ELASTIC SCATTERING FOR NUCLIDE USES THE LEGENDRE CD TREATMENT. CD 25-36 NUCLIDE CONCENTRATION USED TO COMPUTE HOMOGENIZED CD MACROSCOPIC CROSS SECTIONS FOR USE IN THE FUNDAMENTAL CD MODE CALCULATION (ATOMS/CC*E-24). NUCLIDE TEMPERATURE (DEGREES K). CD 37-48 NUCLIDE CONCENTRATION IN THE FUEL PIN (ATOMS/CC*E-24). CD 49-60 CD IF BLANK. THE VALUES GIVEN IN COLS. 25-36 WILL BE USED -CD PROVIDED ALL D6 CARDS USED ARE BLANK IN CULS. 49-60. CD 61-72 NUCLIDE CONCENTRATION IN THE CLAD-COOLANT GUTER REGION -CD (ATOMS/CC#E-24). IGNORED IF CARD TYPE 05 IS NOT CD SUPPLIED. с сс AS MANY TYPE 06 CARDS ARE USED AS ARE NECESSARY TO SPECIFY THE cc COMPOSITION. C

C-CR FUNDAMENTAL MODE ITERATION SPECIFICATIONS С сс CARD TYPE 07 IS OPTIONAL С CL FORMAT(12.10X.3E12.5) C CD COLUMNS CONTENTS CD 2222222 1-2 07 CD CD 13-24 FIRST GUESS FOR BUCKLING (1/CH**2) CD 25-36 SECOND GUESS FOR BUCKLING (1/CM##2) CONVERGENCE CRITERION (EPS) FOR KEFF I.E. ABS(KEFF-1). CD 37-48 CO LE.EPS. C сс IF EPS=0.0. FIRST BUCKLING GUFSS IS USED AND NO ITERATION IS PERFORMED. CC C C-THERMAL CROSS SECTION DATA CR С СС CARD TYPE 08 IS OPTIONAL С FORMAT(12+10X+A6+6X+4E12.5) CL С CD COLUMNS CONTENTS CD ******* CD 1-2 08 NUCLIDE IDENTIFICATION LABEL CD 13-18 CD 25-36 MICROSCOPIC THERMAL GROUP CAPTURE CROSS SECTION (BARNS)-MICROSCOPIC THERMAL GROUP FISSION CROSS SECTION (BARNS)-CD 37-48 49-60 NUMBER OF NEUTRONS FMITTED PER FISSION IN THE THERMAL CD CD GROUP MICROSCOPIC THERMAL GROUP TRANSPORT CROSS SECTION CD 61-72 (BARNS). CD С AS MANY TYPE OB CARDS ARE USED AS ARE NECESSARY TO SPECIFY THE CC THERMAL CROSS SECTIONS FOR THE NUCLIDES IN THE PROBLEM. СС C C٠ C-THERMAL CROSS SECTION DATA CR С CARD TYPE 09 IS OPTIONAL CC С FORMAT(12+10X+A6+6X+2E12-3) CL С CD COLUMNS CONTENTS CD ====== 09 CD 1-2 13-18 NUCLIDE IDENTIFICATION LABEL CD MICROSCOPIC THERMAL N-ALPHA CROSS SECTION (BARNS). MICROSCOPIC THERMAL N-P CROSS SECTION (BARNS). CD 25-36 CD 37-48 C CC AS MANY TYPE 09 CARDS ARE USED AS ARE NECESSARY TO SPECIFY THE THERMAL CROSS SECTIONS FOR THE NUCLIDES IN THE PROBLEM. СС С C-C-WEIGHTING SPECTRUM DATA CR С CARD TYPE 10 IS OPTIONAL CC C. FORMAT(I2+10X+3(I6+E12+5)) CL С CONTENTS CD COLUMNS ______ CD ====== 1~2 10 CD FINE GROUP NUMBER 13-18 CD FINE GROUP WEIGHTING SPECTRUM 19-30 CD FINE GROUP NUMBER CD 31-36

CD 37-48 FINE GROUP WEIGHTING SPECTRUM 49-54 FINE GROUP NUMBER CD 55-66 FINE GROUP WEIGHTING SPECTRUM CD С сс AS MANY TYPE 10 CARUS ARE USED AS ARE NECESSARY TO SPECIFY THE WEIGHTING SPECTRUM. CC C C---PRINT SELECTION OPTIONS CR С СС CAPD TYPE 11 IS OPTIONAL C FORMAT(12,4X,716) CL C CD COLUMNS CONTENTS CD ====== 1-2 CD 11 CD 7-12 IF NON-ZERO, EDIT AVERAGE MICROSCOPIC GROUP CROSS CD SECTIONS SUMMED OVER ALL CONTRIBUTING RESUNANCES IN THE-CD RESOLVED RESONANCE REGION. ALSO EDIT POINT VALUES OF -MICROSCOPIC CROSS SECTIONS AVERAGED OVER PORTER-THOMAS -CD CD DISTRIBUTIONS IN THE UNRESOLVED REGION. CD 13-13 IF NON-ZERU, EDIT AVERAGE MICKOSCOPIC GROUP CROSS CD SECTIONS FOR EACH RESONANCE IN THE RESOLVED RESONANCE CD REGION. CD 19-24 IF NON-ZERG+ EDIT MICROSCOPIC AVERAGE FINE GROUP CROSS -CD SECTIONS COMPUTED FROM LINEAR-SEGMENT DATA. IF NON-ZERO. EDIT HOMOGENIZED MACROSCOPIC FINE GROUP CD 25-30 AND ULTRAFINE GROUP CROSS SECTIONS. ALSO EDIT FISSION -CD CD SPECTRUM. IF NON-ZERO, EDIT FINE AND ULTRAFINE GROUP FLUXES FOR CD 31-36 CD EACH BUCKLING ITERATION. CD IF NON-ZERO, EDIT ULTRAFINE GROUP MATERIAL MACROSCOPIC -37-42 ELASTIC REMOVAL CROSS SECTIONS, ELASTIC TRANSPORT CROSS-SECTIONS, AND ELASTIC TRANSFER CROSS SECTIONS. -CÐ CD IF NON-ZERO, A COMPLETE EDIT OF THE MULTIGRP CROSS CD 43-46 SECTION JATA SET. CD С СC ALL DEFAULT VALUES ARE ZERO. С ADDITIONAL INFORMATION NEEDED FOR WRITING MULTIGRP FILE CR C сс CARD TYPE 12 MUST ALWAYS BE PRESENT C. CL FORMAT(12+4X+A6+316+3E12+5) С CONTENTS COLUMNS CD CD ====== CO. 1-2 12 NUCLIDE IDENTIFICATION LABEL ON LIBRARY. 7-12 CD NUCLIDE IDENTIFICATION NUMBER ON MULTIGRP FILE. 13-18 CD MATERIAL TYPE FLAG (0/1/2/3/4/5=UNDEFINED/FISSILE/ CD 19-24 FERTILE/OTHER HEAVY MATERIAL/STRUCTURAL/FISSION CD FRODUCT) . CD NUCLIDE ATOMIC NUMBER CD 25-30 DECAY CONSTANT FOR NUCLIDE (1/SEC). CD 31-42 NUCLIDE FISSION ENERGY (W-SEC/FISSION). 43-54 CD NUCLIDE CAPTURE ENERGY (W-SEC/CAPTURE). CD 55-66 С A TYPE 12 CARD IS REQUIRED FOR EACH NUCLIDE THAT IS SPECIFIED ON -CC A TYPE 06 CARD. CC С

1. The second se

The logical unit n were of the interfacedata files are assigned by the codes that create the files. These logical unit numbers are available to the other codes through the COMMON array NFILES.

An input and an output version of a file can exist concurrently. Logical unit numbers of input files are stored in (NFILES(I),I=1,NSTD) and logical unit numbers of output files are stored in (NFILES(NSTD+I),I=1,NSTD). NSTD is the number of interface-data files. It is currently set equal to 16 in DRIVER and is transmitted to all the codes through NFILES(100).

The values of I corresponding to the 16 interface files are:

<u> I </u>	<u>File</u>	Ī	<u> </u>
1	ADMNSTR	9	ADJ FLXS
2	SN CONS	10	PERIMTGP
3	GEO DIST	11	ETX INPT
4	GRP FLXS	12	ODX INPT
5	INTQUANT	13	ANI INPT
6	MIX DATA	14	DOT INPT
7	MULTIGRP	15	CLB INPT
8	ZONEDENS	16	DAC INPT

Logical unit numbers for output files are assigned by

NFILES(NSTD+I)=NSCT+NSTD+I

```
1f
```

NFILES(I).LE.NSCT+NSTD

or by

NFILES (NSTD+1)=NSCT+1

1f

NFILES(1).GT.NSCT+NSTD ,

where NSCT is the maximum number of scratch files. NSCT is currently set equal to 50 in DRIVER and is transmitted to all the codes throug'n NFILES(99). Because the output file from one code becomes the input file for the next code in the path, NFILES(I) is set equal to NFILES(NSTD+I) after file I is created. This procedure may become clearer if a particular example is considered. Assume that the ADMNSTR file on logical unit NFILES(1)=51 is to be modified by the INP module. The output ADMNSTR file is then written on logical unit NFILES(17)=67, and NFILES(1) is set equal to 67. However, if the input ADMNSTR file is on logical unit NFILES(1)=67, the output file is written on NFILES(17)=51, and NFILES(1) is set equal to 51.

When an FL1 call is made to copy interface files from disk to magnetic tape, (NFILES(I),I=1, NSTD) is written as the first record on the tape. This is followed by the current versions of the interface files, i.e., those on logical units NFILES(I),I=1,NSTD. When the files are copied from the magnetic tape to disk files in an FLO call, the first record on the tape is read into the NFILES array to restore the logical unit numbers associated with the disk files.

The ENDF/B and ETOX cross-section library files are not included in the file-handling scheme described above. These files must be assigned by the user to specific logical units by control cards.

Because dynamic buffer allocation is not a built-in feature of the CDC 6600, a scheme was developed to rotate a few buffers among the many files used by the codes. This scheme involves the use of a COMPASS language routine, BUFOPEN. Whenever a buffer is to be assigned to a particular file, a call is made to

BUFOPEN(I, FNAME, BUF, L) .

This routine initializes the circular buffer parameters (CBP) for file FNAME where FNAME is a seven-character Hollerith name (7HTAPE1^,7HTAPE2^, ...,7HTAPE99^). The CBP are the first five words of the file environment table (FET), which occupies the first 17 words of the buffer area. The remainder of the buffer area is used to transmit data between core and the file. I is the relative location in core of the CBP pointer for the file, BUF is the first word address of the buffer area, and L is the length of the buffer. The reader is referred to Ref. 17 for a listing of the BUFOPEN routine and for further details.

65

APPENDIX D

SAMPLE PROBLEMS

Sample problems involving the various interfaced codes are discussed in this Appendix. These problems illustrate the card input required for the creation and modification of interface files, printing of files, execution of calculational codes in some typical sequences, and continuation of a sequence of calculations from files saved in a previous run.

Sample Problem 1. The first sample problem shows how ETOX could be used to update an isotope in an existing ETOX library. Card input for this sample problem is shown below.

CARD INPUT FOR SAMPLE PROBLEM 1

3 T NUMBER OF PATH ELEMENTS	DRIVEROI
INP PRN ETX T PATH	DRIVER02
R10(4) 1 R5(4) T CARD INPUT FILE CONTROLS	INPCNT01
1 203 26 11 T DATA COMMON TO ALL ISOTOPES TO BE PROCESSED	ETXINP01
2.26+1 3.727+1 6.144+1 1.013+2 1.67+2 2.754+2 4.54+2 7.485+2 1.234+3	ETXINP02
2,035+3 3,355+3 5,531+3 9,119+3 1,503+4 2,479+4 4,087+4 6,738+4	ETXINP03
1.111+5 1.832+5 3.02+5 4.979+5 8.208+5 1.353+6 2.231+6 3.679+6	ETXINP04
6.065+6 1.0+7 T ENERGY BOUNDARIES	ETXINP05
* FE IN 26 GROUP STRUCTURE FOR UPDATED ETOX LIBRARY	#ETXINP06
FE 1122 1 3 0 T ISOTOPE-DEPENDENT DATA1	ETXINP07
1.0+5 1.0 2.6+6 1.4+6 0.1 0.1 0.5 1.0-3 3.0+2 5.0 5.0+1	ETXINPOS
5.0+2 T ISOTOPE-DEPENDENT DATA 2	ETXINP09
1 3 3 1 1 3 3 T ETOX LIBRARY UPDATE DATA	ETXINP10
R10(1) 2 R5(1) T FILE PRINT CONTROLS	PRNCNT01

DRIVER01 and DRIVER02 are the control input cards for the DRIVER program. In this example, the number of path elements is three, and the path is INP PRN ETX. The next card, INPCNT01, contains the card input file controls for the INP module. These controls instruct INP to read the ETX INPT file from cards ETXINPO1 through ETXINP10 and to skip all the other interface files. Finally, PRNCNTO1 instructs the PRN module to print the ETX INPT file (see below) and to skip all of the other interface files.

OUTPUT FROM PRN MODULE--SAMPLE PROBLEM 1

ETOX INPUT FILE FILE 61

COMMON	FILE PARAMETERS	
NISO	NUMBER OF ISOTOPES	1
IDTAP	ENDF/B DATA TAPE I.D. NUMBER	203
NG	NUMBER OF ENERGY GROUPS OF OUTPUT X-SECTIONS	26
LN5	NUMBER OF DOWNSCATTERING GROUPS OUTPUT	11

GROUP ENERGY BOUNDS

I	BOUND
1	.22600E+02
2	°345405+05
3	.61440E+02
4	.10130E+03
5	.16700E+03
6	.27540E+03
7	45400E+03
Ŕ	.74850E+03
9	+2340E+04
1Ó	20350E+04
1 7	13550E+04
12	• JJJJJUL + 04
12	01100E+04
13	+911906+04
19	.15030E+05
12	+24/90E+05
16	.40870E+05
17	+67380E+05

18	• 1 1110E+06
19	.18320E+06
20	•30500E+06
21	+9790E+06
22	\$2080E+06
23	.13530E+07
24	22310E+07
25	.36790E+07
26	.60650E+07
27	.10000E+08

DATA

TEATADE

ISUIVE	1 DATA	
FE IN 26	GROUP STRUCTURE FOR UPDATED ETOX LIBRARY	
NAMEI	ISOTOPE NAME	FE
MAT11	ENDF/B ISOTOPE I.D. NUMBER	1122
NTEMP	NUMBER OF TEMPERATURES OF SELF-SHIELD FACTORS	1
NSIG0	NUMBER OF SIGO VALUES OF SELF-SHIELD FACTORS	3
LN6	1/0=CALCULATE AND PRINT FISSION FRACTIONS -	0
EMAXEF	MAX ENERGY FOR SS FACTOR CALCULATIONS	•10000E+06
EMINFF	MIN ENERGY FOR SS FACTOR CALCULATIONS	10000E+01
EMN1EF	MIN ENERGY WEIGHT SPECTRUM=FISSION SPECTRUM-	26000E+07
CFF	FISSION SPECTRUM CONSTANT (EV)	14000E+07
DELMAX	MAX FINE GROUP LETHARGY	10000E+00
DELUMX	MAX UNRESOLVED GROUP LETHARGY	10000E+00
ANFMPD	NUMBER FINE-GP POINTS/ENDF/B SIGMA TOT POINTS	•20000E+00
EPS	ROMBERG INTEGRATION ACCURACY PARAMETER	-10000E-02
	TEMPERATUPES SIGMA ZERO	
1	3.00000E+02 5.00000E+00	
2	5+00000E+01	
3	5,0000E+02	
ETOX LIBR	ARY CONTROLS	
NUP	1/0=PRODUCE UPDATED LIB OF ETOX X-SECTIONS/NO	1
NUP1	LIB SEQUENCE NO. OF 1ST ISOTOPE UPDATED	3
NUP2	LIB SEQUENCE NO. OF LAST ISOTOPE UPDATED	3
NPR	1/0=PRINT X-SECTIONS/NO	1
NPR1	LIB SEQUENCE NO. OF 1ST ISOTOPE PRINTED	1
NPR2	LIB SEQUENCE NO. OF LAST ISOTOPE PRINTED	3
NIACT	TATAL NO. OF ISATORS ON INDUT LIDRARY	3

The input ETOX library and the ENDF/B tape are assigned by control cards to logical units 47 and 48, respectively. In this example, the input ETOX library contains 26-group cross sections for three isotopes in the order 235 U, 238 U, and Fe. Execution of the path element ETX causes the ETOX code to be loaded and executed. ETOX calculates 26-group cross sections for Fe using input from the ETX INPT file and the Fe data on the ENDF/B tape. The output ETOX library containing the new Fe cross sections is written on logical unit 46.

Sample Problem 2. In this sample problem, the 26-group ETOX library created in Sample Problem 1 is used by 1DX in a one-dimensional calculation to create a 4-group standard MULTIGRP file containing the three isotopes 235 U, 238 U, and Fe. As a check on the group collapsing, the one-dimensional 1DX calculation is repeated with the 4-group MULTIGRP file.

Card input for this sample problem is shown on the next page. The first two cards define the number of path elements (6) and the path (INF PRN ODX INP ODX FL1) for the DRIVER program. The next card, INPCNTO1, contains the controls for the first call to INP. This card instructs the INP module to create the ADMNSTR, GEO DIST, MIX DATA, and ODX INPT files from cards and to skip the other interface files. Cards for the four files created by INP are ADMNSTO1 through ADMNSTO4, GEODISO1 through GEODISO4, MIXDATO1 through MIXDATO5, and ODXINPO1 through ODXINP15 as shown.

6 T NUMBER OF PATH ELEMENTS DRIVER01 *INP PRN ODX INP ODX FLI* T PATH 1 R2(4) ⁶2(1) R6(4) 1 R4(4) T CARD INPUT FILE CONTROLS DRIVER02 INPCNT01 * ADMNSTR FILE FOR 26-GROUP 1DX CALCULATION */ TITLE ADMNST01 R3(6) 4 R2(2) 50 K2(1) -1 1 R2(0.0) 1.0-4 R5(0.0) 1 R5(0)ADMNST02 1 1.0 5 8 0 3 R4(0) 0.0 26 0 30 10 1.0-4 1.0-3 1.0-2 ADMNST03 1 -1 1 1.0 5 8 0 3 R4(0) 0.5 1:0 1.0-3 1.0 T ADMNSTR PARAMETERS ADMNST04 0.0 129(29.64) 119(64.2) T MESH BOUNDARIES GEODIS01 4 5 T MATERIAL NUMBERS GEODIS02 R30(1) R20(2) T ZONE NUMBERS GEODIS03 1.9579-3 T BUCKLING R4(4) R4(5) T MIX NUMBERS GEOD1504 MIXDAT01 0 1 2 3 0 1 2 3 T MIX COMMANDS MIXDAT02 0.0 4:567-3 3.4392-2 7.167-03 0.0 8.9-5 4.0025-2/ MIXDAT03 6.121-3 T MIX DENSITIES MIXDAT04 1 2 3 T ISOTOPE NUMBERS MIXDAT05 #ODXINP01 * 1DX 26-GROUP REGULAR PROBLEM USING ETOX LIBRARY TAPE 2 10 0 I 0 R2(4) 0 3 R3(0) T TITLE AND CONTROL PARAMETERS R2(1.07 T BUCKLING MODIFIERS ODX1NP02 ODXINP03 2.175-2 9.994-2 1.9746-1 2.2688-1 1.8522-1 1.2228-1 7.085-2 3.789-2 ODX1NP04 1.93-2 9.55-3 4.64-3 2.23-3 1.06-3 5.0-4 2.4-4 1.1-4 5.0-5 3.0-5 2.0-5 ODXINPO5 R7(0.0) R26(1.0) 1 FISSION FRACTIONS AND VELOCITIES ODX1NP06 R4(6) 1 2 3 R3(1) T DATA FUR XSEC GROUP COLLAPSING ODXINP07 1 1.0 0 1 2 3 R4(300.0) R4(0) R25(0.5) T DATA FUR RESUNANCE-SHIELD CALCODXINP08 6 *U-235* 233.02 0.0 3.I-11 C.0 27.0 94350 1 -1 *U-238* 236.0 0.0 ODXINP09 3.1-11 0.0 27.0 94380 2 -1 *FE* 55.37 R3(0.0) 27.0 26558 4 0 T EXTRA ODXINPIC 1.24-2 3.05-2 0.111 0.301 1.14 3.01 T DELAY NEUT DECAY CONSTANTS ODXINP11 0.0 2.2258-4 R3(0.0) 1.4821-3 R3(0.0) 1.3341-3 R3(0.0) 2.6784-3 R3(0.0) ODXINP12 7.8311+4 R3(0.0) 2.8569-4 R2(0.0) T DELAY NEUT YIELD SPECTRUM FOR U-2350DXINP13 0.0 2.2258-4 R3(0.0) 1.4821-3 R3(0.0) 1.3341-3 R3(0.0) 2.6784-3 R3(0.0)ODXINP14 7.8311=4 R3(0.0) 2.8569-4 R2(0.0) T DELAY NEUT YIELD SPECTRUM FOR U-2380DXINP15 2 R2(1) R2(2) R6(1) 2 R4(1) T FILE PRINT CONTROLS PRNCNT01 3 R10(4) 1 R4(4) T CARD INPUT FILE CONTROLS INPCNT02 ADMNSTR FILE FOR 4-GROUP 1DX CALCULATION S38 4 T ADMNSTR2 * 1DX 4-GROUP REGULAR PROBLEM USING MULTIGRP FROM PREVIOUS CALC **#ODXIN 01** TITLE AND CONTROL PARAMETERS 3 R4(0) 5 R5(0) 1 ODXIN 02 2 R2(1.07 T BUCKLING MODIFIERS ODXIN 03

The next input card, PRNCNT01, instructs the PRN module to print the four files created in the preceding call to INP. This printed output is shown on the following pages. The next element in the path, ODX, causes the 1DX code to be loaded and executed. 1DX reads the four interface files available at this point and also the ETOX cross-section library. The ETOX library is expected on logical unit 46, which is the logical unit number for the output library from the ETOX code. 1DX performs the 26-group flux calculation and group collapse specified by the interface-data files. New files created by 1DX at this point are GRP FLXS and INTQUANT for the 26-group structure, MULTIGRP containing 235 U, 238 U, and Fe in the collopsed and resonance-shielded 4-group structure, and ZONEDENS.

Execution of the second call to INP sets up the ADMNSTR and ODX INPT files for repetition of the 1DX flux calculation using the MULTIGRP file created in the previous ODX call. Input card INPCETO2 instructs the EMP module to overlay the existing ADMNSTR file with card ADMNSTR2 and to create a new ODX INPT file from cards ODXINO1 through ODXINO3. Note that only the title and parameter IGM are changed in the ADMNSTR file. Note also that the input for the ODX INPT file is considerably shorter when cross sections are read from the MULTIGRP file and no group collapsing is to be done.

In the second call to ODX, the 4-group flux calculation is performed by IDX. New files created by IDX at this point are GRP FLXS and INTQUANT for the 4-group structure. Finally, all the current interface files are copied onto a magnetic tape in the call to FL1. The user must assign the tape to logical unit 50 by a control card. This tape is used in Sample Problem 3 to continue the calculations. 1

ADMINISTRATION FILE FILE 51

ADMINISTRATION AND CONTROL PARAMETERS

ADMNS1	IR FILE FOR 26-GROUP 1DX CALCULATION				
IA01	1/2/3=DIMENSION	1			
IA11	TYPE. 0=DIFFUSION+ 1=TRANSPORT	0			
IA02	THEORY. 0=REGULAR. 1=ADJOINT	0			
IA03	ANISOTROPIC SCITTERING ORDER	0			
IA04	ORDER OF SN APPROXIMATION	4			
IGE	0/1/2/3/6/7/8/9/11/12/13=GEOMETRY	2			
IZM	NUMBER OF ZONES	2			
IM	NUMBER OF 1ST DIMENSION INTERVALS	50			
JM	NUMBER OF 2ND DIMENSION INTERVALS	1			
KM	NUMBER OF 3RD DIMENSION INTERVALS	1			
IBK	NUMBER OF BUCKLING SETS	-1			
IEVT	0/1/2/3/4/5/6 = EIGENVALUE TYPE	1			
£٧	EIGENVALUE GUESS	0.			
EVM	EIGENVALUE MODIFIER	0.			
EPS	CONVERGENCE CRITERION	.10000E-03			
TEMP1	UNDEFINED.	0.			
TEMP2	UNDEFINED.	0 ა			
ТЕМРЭ	UNDEFINED.	0.			
TEMP4	UNDEFINED.	0.			
TEMP5	UNDEFINED.	0.			
1801	-1/0/1/2/3/4 = LEFT BORY CONDITION	1			
1802	-1/0/1/2/3/4 = RIGHT BDRY CONDITION	ŏ			
1803	+1/0/1/2/3/4 = TOP BORY CONDITION	Ő			
1804	-1/0/1/2/3/4 = BOTTOM BDRY CONDITION	Ő			
1805	-1/0/1/2/3/4 = FRONT BDRY CONDITION	0			
1806	-1/0/1/2/3/4 = BACK BDRY CONDITION	ŏ			
M07	0/1/2/3/4/5 = FLUX INPUT OPTION	-1			
IFXT	NEGATIVE SOURCE CHECK. 0=NO.I=YES	ī			
501	SOURCE NORMALIZATION FACTOR	.10000E+01			
MT	NUMBER OF MATERIALS.	5			
M01	NUMBER OF MATERIAL SPECIFICATIONS	B			
MCR	NUMBER OF ISOTOPE XS SETS FROM CARDS	Ō			
MSE	NUMBER OF ISOTOPE XS SETS FROM TAPE-	3			
17	NUMBER OF 1ST DIMENSION ZONE MODEYRS	Ő			
17	NUMBER OF 2ND DIMENSION ZONE MODEYRS	ů.			
ĸŹ	NUMBER OF 3RD DIMENSION ZONE MODEYRS	ŏ			
1502	PARAMETRIC FIGENVALUE TYPE	Ő			
503	PARAMETRIC FIGENVALUE.	0.			
IGM	NUMBER OF ENERGY GROUPS.	26			
MD6	0/1/2/3/4/5/6 = SOURCE INPUT OPTION-				
1005	MAXIMUM NUMBER OF OUTER ITERATIONS	30			
1007	MAXIMUM NUMBER OF INNER ITERATIONS	10			
605	NEUTRON BALANCE TEST. 0=N0.605=1 1MTT	.10000F-03			
606	POINTWISE FLUX TEST. 0=N0.G06=LIMIT-	10000F-02			
	LANDA LANFR LIMIT	.10000F-01			
	I ANDA LIPPER I INTT	-500005+00			
	NEW DADAMETER MODIFIER CONCERNING	. 1000000.00			
FDCA	CONV. CRITERION+CRIT. SEADCHES	- 100000E+01			
LF 3M	EXTRADOLATION FACTOR.	-10000E-02			

	BOUNDARIES		BOUNDARIES		BOUNDARIES
----------------	-------------	----	-------------	------	-------------
	1ST DIMENSN		1ST DIMENSN		1ST DIMENSN
1	0	19	1.77840E+01	37	4.00080E+01
2	9.88000E-01	20	1.87720E+01	- 38	4.17360E+01
3	1.97600E+00	21	1.97600E+01	39	4.34640E+01
4	2.96400E+00	22	2.07480E+01	40	4.51920E+01
5	3.95200E+00	23	2.17360E+01	41	4.69200E+01
6	4.94000E+00	24	2.27240E+01	42	4-86480E+01
7	5.92800E+00	25	2.37120E+01	43	5.03760E+01
8	€_01600E+00	26	2.47000E+01	44	5-21040E+01
9	7.90400E+00	27	2.56880E+01	45	5.38320E+01
10	8.89200E+00	28	2.66760E+01	46	5+55600E+01
¥1	9.88000E+00	29	2.76640E+01	47	5.72880F+01
1 2	1.08680E+01	30	2.86520E+01	48	5-90160F+01
13	1.18560E+01	31	2.96400E+01	49	6.07440E+01
14	1.28440E+01	32	3.13680E+01	50	6.24720E+01
15	1.38320E+01	33	3.30960E+01	51	6.42000E+01
16	1.46200E+01	34	3.48240E+01	5.	
17	1.58080E+01	35	3.65520E+01		
18	1.67960E+01	36	3.82800E+01		

MATERIAL NUMBERS

	ZONE	MATERIAL NO.
1	1	4
2	2	5

ZONE NUMBERS

	1	26	1
THINEYTONE	1	20	1
1	1	21	1
2	1	28	1
3	1	29	1
4	1	30	1
5	1	31	2
6	1	32	2
7	1	33	2
8	1	34	2
9	1	35	2
10	1	36	2
11	1	37	2
17	1	38	2
13	ī	39	ž
14	ī	40	2
15	1	41	2
16	ī	42	2
17	ī	43	2
18	1	44	2
19	1	45	2
20	1	46	2
21	1	47	2
22	ī	48	2
23	ī	40	2
2/1	1	47 50	2
25	1	50	e
20	1		

,

GROUP AND ZONE INDEPENDENT BUCKLING= .19579E-02

MIXTURE DATA FILE FILE 56

MIXTURE SPECIFICATIONS

MIX	NUMBERS	MIX COMMAND	MIX DENSITY
1	4	0	0
2	4	3	4.56700E-03
3	4	2	3,43920E-02
4	4	3	7.16700E-03
5	5	0	0
6	5	1	8.90000E-05
7	5	2	4.002502-02
8	5	3	6.12100E-03

ISOTOPE NUMBERS

150	TOPE NO.
1	1
2	2
3	3

ONEDX INPUT FILE FILE 62

TITLE A	ND CONTROL PARAMETERS	
1DX 264	GROUP REGULAR PROBLEM USING ETOX LIBRARY TAPE	
MAXT	MAX RUNNING TIME (MIN)	2
NXCM	NUMBER OF DOWNSCATTER GROUPS	10
NPRT	PRINT. 0/1/2=NO X-SEC OR FL/NO IN X-SEC/FULL	0
NRCF	NUMBER OF MIXES USED IN GENERATING X-SECTIONS	1
NIFF	NO. SPECTRUM ITERATIONS-ELASTIC DOWNSCATTER -	0
MM01	NO. MIX SPECS-RESONANCE SHIELDED X-SECTIONS -	4
NCR	NUMBER COLLAPSED GROUPS	4
NTR	0/1=NORMALIZED/RECIPROCAL WGT. SIGMA TRANSPT	0
NFGM	NUMBER COLLAPSED MATERIALS	3
IPUN	0/1=PRINT COLLAPSED XSECS/NO	0
MULT1	0/1=READ MULTIGRP/READ PERTURBED MULTIGRP	0
MULT2	0/l=WRITE MULTIGRP/WRITE PERTURBED MULTIGRP -	0

BUCKLING MODIFIERS

1	MODIFIERS
1	\$10000E+01
2	\$10000E+01

FISSION FRACTIONS AND VELOCITIES

FISSION			FISSION	
FRACTION	VELOCITY	GROUP	FRACTION	VELOCITY
2.17500E-02	1.00000E+00	14	5.00000E-04	1.00000€+00
9.99400E-02	1.00000E+00	\$5	2.40000E-04	1.000005+00
1.97460E+01	1.00000E+00	16	1.10000E-04	1.00000€+00
2.26880E-01	1.00000E+00	17	5.00000E-05	1.00009E+00
1.85220E-01	1.00000E+00	1 8	3.00000E-05	1.00000E+00
1.22280E-01	1.00000E+00	19	2.00000E-05	1.00000E+00
7.08500E-02	1.00000E+00	20	0	1.00000E+00
3.78900E-02	1.00000E+00	21	· 0	1.00000E+00
1.930005-02	1.00000E+00	82	· 0	1.00000E+00
9.55000E-03	1.00000E+00	23	0	1.00000E+00
4.64000E-03	1.00000E+00	24	0	1.00000E+00
2.23000E-03	1.00000E+00	• 25	0	1.00000E+00
1.06000E-03	1.00000E+00	26	0	1.00000E+00
	FISSION FRACTION 2.17500E-02 9.99400E-02 1.97460E-01 2.26880E-01 1.85220E-01 1.85220E-01 1.85200E-02 3.78900E-02 3.78900E-02 9.55000E-03 4.64000E-03 2.23000E-03 1.06000E-03	FISSION FRACTION VELOCITY 2.17500E-02 1.00000E+00 9.99400E-02 1.00000E+00 1.97460E-01 1.00000E+00 1.85220E-01 1.00000E+00 1.85220E-01 1.00000E+00 1.22280E-01 1.00000E+00 3.78900E-02 1.00000E+00 1.93000E+02 1.00000E+00 4.64000E-03 1.00000E+00 2.2300E-03 1.00000E+00 1.05000E+03 1.00000E+00	FISSION VELOCITY GROUP 2.17500E-02 1.00000E+00 14 9.99400E-02 1.00000E+00 15 1.97460E-01 1.00000E+00 16 2.26880E-01 1.00000E+00 17 1.85220E-01 1.00000E+00 18 1.22280E-01 1.00000E+00 19 7.08500E-02 1.00000E+00 20 3.78900E-02 1.00000E+00 21 1.93000E-02 1.00000E+00 23 4.64000E-03 1.00000E+00 23 4.64000E-03 1.00000E+00 25 1.06000E+03 1.00000E+00 25 1.06000E+03 1.00000E+00 25	FISSION FISSION FRACTION VELOCITY GROUP FRACTION 2.17500E-02 1.00000E+00 14 5.00000E-04 9.99400E-02 1.00000E+00 15 2.40000E-04 1.97460E-01 1.00000E+00 16 1.10000E-04 2.26880E-01 1.00000E+00 17 5.00000E-05 1.85220E-01 1.00000E+00 18 3.00000E-05 1.22280E-01 1.00000E+00 19 2.00000E-05 7.08500E-02 1.00000E+00 20 0 3.78900E-02 1.00000E+00 21 0 9.55000E-03 1.00000E+00 23 0 4.64000E-03 1.00000E+00 25 0 2.23000E-03 1.00000E+00 25 0 1.06000E+00 25 0 1.06000E+00 26

.

INCLUDE I.D. NU ZONE NO	S NUMBER MBER OF 1 OF FLL	R OF FINE ITH MATE IX FOR CO	GROUPS IN I RIAL TO BE C LLAPSING ITH	TH BROAD GROUN COLLAPSED,AND MATERIAL.	,		
1	NO. FI 1 2 3 4	NE GRPS 6 6 6 6	MAT I.D. 1 2 3	ZONE NOS. 1 1 1			
RESONAN INCLUDE ITH MIX MIX COM (SEE BNW TEMPERA 0/1=FUE LETHARS	CE SHIEL S ZONE N HETEROG MANDS CO L-954,00 TURES OF L/MODERA WIDTH	DING SPEC D. DF FL ENEITY C NTAINING C-32). NUCLIDE TOR DESI BY GROUP	CIFICATIONS UXES FOR ELA ONSTANT ADSOLUTE RE S IN MIX COM GNATION FOR	STIC DOWNSCAT FERENCES TO I MAND TABLE, MIX COMMAND TA	TER ITERATIONS Sotopes used in Able,	FOR ITH MI) N Forming Ri	() ESONANT MIXTURES
	I ZONE 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 24 25 26 27 28 29 20 20 20 20 20 20 20 20 20 20	NOS. 1	HET CONST 1.00000E+00	MIX COMMAND 0 1 2 3	TEMPERATURE 3.00000E+02 3.00000E+02 3.00000E+02 3.00000E+02	FUEL/MOD 0 0 0	LETHARGY WDTH 5.00000E-01
DATA NEE (SEE DES	DED FOR CRIPTION	WRITING N OF MULT	COLLAPSED MU IGRP FILE)	ULTIGRP FILE			
I ISON 10-235 20-238 3FE	ME / •230 •230 •550	9WR 302E+03 500E+03 370E+02	DCA 0. 3. 0.	EFIS •31000E-10 •31003E-10 0•	ECAP 0. 0. 0.	TDC •27000E+02 •27000E+02 •27000E+02	IZAS KBR ICHI 94350 1 -1 94380 2 -1 26558 4 0
DECAY CO	NSTANTS 1 1.24 2 3.05	(1/SEC) 000E-02 000E-02	BY DELAYED	NEUTRON FAMILY	· ·		

3 1.11000E-01 4 3.01000E-01 5 1.14000E+00 6 3.01000**E**+00

XSEC COLLAPSING SPECIFICATIONS

(

DELAYED NEUTRON SPECTRUM FOR U-235

PAMLY	GRP	1	GRP	2	GRP	3	GRP	4
1	0.		2.2258	0E-04	0.		0.	
2	0.		1.4821	0E-03	0.		0.	
3	0.		1.3341	0E-03	6.		0.	
4	0.		2.6784	0E-03	C.		0.	
5	0.		7.8311	0E-04	0.		0.	
6	0.		2.8569	0E-04	0.		0.	

DELAYED NEUTRON SPECTRUM FOR U-238

FAMLY	GRP	1	GRP	2	GRP	3	GRP	4
1	0.		2.22580)E-04	0.		0.	
2	0.		1.48210)E-03	0.		0.	
3	0.		1.33410)E-03	0.		0.	
4	0.		2.67840)E-03	0.		0.	
5	0.		7.83110)E-04	0.		0.	
6	0 +		2185690)E-04	0.		0.	

Sample Problem 3. Some of the interface files seved on magnetic tape in Sample Problem 2 are used in ANISN to obtain regular and adjoint flux solutions in S_4 approximation. The angular fluxes from ANISN are then used in a DAC perturbation calculation. Card input for this sample problem is given below. The first two cards, DRIVERO1 and DRIVERO2, instruct the DRIVER program to execute the path (FLO INP PRN ANI INP ANI DAC).

In the call to FLO, the interface files on the magnetic tape are copied to separate disk files.

The user must assign the magnetic tape to logical unit 49. The next input card, INPCNTO1, instructs the INP module to overlay the ADMNSTR file with input card ADMNSTO1 and to create the SN CONS, ANI INPT, and DAC INPT files from cards labeled SNCONSO1 through SNCONSO4, ANISNO1 through ANISNO3, and DACINPO1 through DACINPO6, respectively. Note that the only modification to the ADMNSTR file is in the title and in the diffusion/transport theory flag (IA11).

CARD INPUT FOR SAMPLE PROBLEM 3

7 T NUMBER OF PATH ELEMENTS DRIVER01 DACH T PATH DRIVER02 *FLO INP FRN ANI INP ANI 3 1 PIO(4) 1 R2(4) 1 T INPUT FILE CONTROLS (FIRST INP CALL) INPCNT01 * ADMNSTR FILE FOR 4-GROUP S4 ANISN CALCULATION * 51 I T ADMNST01 -0.471405 -0.333333 0.333333 -0.942809 -0.881917 -0.333333 SNCONS01 0.333333 0.881917 T MU DIRECTIONS SNCONS02 R8(0.0) T ETA DIRECTIONS SNCONS03 0.0 R2(0.166667) 0.0 R4(0.166667) T WETCHTS SNCONS04 • ANISN 54 4-GP REGULAR PROBLEM USING FILES SAVED IN SAMPLE PROBLEM 2 #ANISN 01 1 3 4 7 R11(0) 0.0 0.5 T ANISN CONTROL PARAMETERS ANISN 02 R30(1.0) R20(0.0) T FISSION GUESS 1 1 0 3 2 1 T DAC CONTROL PARAMETERS 1 8 2 T PERTURNATION CASE DATA ANISN 03 DACINPOI DACINP02 DACINP03 R30(1) ►20(2) I DACINP04 4 5 P4(4) R4(5) 0 1 2 3 0 1 2 3 0.0 4.567-3 3.4392-2 7.167-3 0.0 A.Y-5 4.0025-2 6.121-3 T DACINP05 0.0 5:4804-3 3.4392-2 7.167-3 0.0 8.9-5 4.0025-2 5.121-3 T R2(2) R5(1) 2 R4(1) 2 R2(1) 2 T FILE PRINT CONTROLS 3 R15(4) F INPUT FILE CONTROLS (SECOND INP CALL) DACINP06 PRNCNT01 INPCNT02 STO 1 T RESET TA02 FOR ADJUINT CALCULATION ADMNST02 Input card PRNCNTOl instructs the PRN module to print the ADMNSTR, SN CONS, MULTIGRP, ANI INPT, and DAC INPT files. Output from the PRN module, except for the ADMNSTR file, is shown below.

Execution of the next path element, ANI, causes the ANISN code to be loaded and executed. ANISN performs the regular flux calculation and creates new GRP FLXS and INTQUANT files. The next input card, INPCNT02, instructs the INP module to overlay the ADMNSTR file with input card ADMNST02. At this point, only the regular/adjoint flag (IAO2) is changed. In the second call to ANI, ANISN performs the adjoint calculation and creates the ADJ FLXS file and a new INTQUANT file.

The last path element causes the DAC code to be loaded and executed. DAC performs the perturbation calculation specified by the DAC INPT file using the current versions of the ADMNSTR, GEO DIST, GRP FLXS: ADJ FLXS, INTQUANT, MIX DATA, and MULTIGRP files. In this example, the perturbation is a 20% increase in the 235 U atom density in the core.

OUTPUT FROM PRN MODULE--SAMPLE PROBLEM 3

₽₩SN	CONSTANTS	FILE##	FILE 52	
	•	4	WEIGHT	MU
	1	1	0	-4.71405E-01
	2	5	1.66667E-01	-3.333338-01
	3	3	1.66667E-01	3.33333E-01
	4	4	0	-9.42809E-01
	5	5	1.666607E-01	-8.81917E-01
	6	6	1.66667E-01	-3.33333E-01
	7	7	1.66667E-01	3.33333E-01
	8	8	1.6666675-01	8.81917E-01

REFERENCE MULTIGROUP FILE FILE 57

FILE TITLE

CROSS SECTIONS FROM IDX (STANDARD FILE MULTIGRP)

FILE PARAMFTERS

NISO	NUMBER OF ISOTOPES	3
NEG	NUMBER OF GROUPS	4
NFAM	NUMBER OF DELAYED NEUTRON FAMILIES.	
NBON	BONDARENKO DATA GIVEN. 1=YES.0=ND.	ő
MAXUP	NUMBER OF UPSCATTER GROUPS	ŏ
MAXDN	NUMBER OF DOWNSCATTER GROUPS	3
MSEC	SECONDARY X-SECTNS GIVEN. I=YES.0=NO	õ
MORD	NUMBER OF SCATTERING TABLES	, i
MBINT	NUMBER OF SIGMAPO VALUES.	õ
MBTEM	NUMBER OF TEMPERATURES	ů
1SL	SCATTERING TABLE LENGTH	4
1 HM	CROSS-SECTION TABLE LENGTH	11

ISOTOPE IDENTIFICATION

NUMBER	NAME	TEMPERATURE	INDEX
1	U-235	•27000E*02	1
2	11-238	.27000E+02	1
3	FE	•54000E+05	1

GROUP	STRU	CTURE				
	ENE	RGY LIMITS	LETHARGIES	1/VELOCITY	AV ENERGY	FISSN SPECTR
	1	1.0000UE+0	2 0	3.14014E-10	5.24894E+0	6 8.535302-01
	2	4.97871E+0	5 J.COUDOE+UO	1.40731E-04	2.61329E+0	5 1.44460F-01
	3	2.47875E+04	+ 6.00000E+00	6.30715E-09	1.30108E+0	4 1.99000F-03
	4	1.23410E+03	3 9.00000E+00	2.82667E-08	6.47770E+0	2.0000000-05
	5	6.14421E+01	1.20000E+01			

DELAYED NEUTRON DECAY CONSTANTS

DELAY	GPOUP	DECAY CONSTANT
	1	.124008-01
	2	•30500F-01
	3	.11100£+00°
	4	.30100E+00
	5	.11400r+01
	6	.30100E+01

DELAYED NEUTRON SPECTRUM

Ρ	GRP		DGRP	1	DGRP	2	DGRP	3	DGRP	4	DGRP	5	DGRP	6
	1		0.		0.		0.		0.		0.		0.	
	г		1.00000	E+00	1.0000	0E+00	1.0000	0E+00	1,0000	0E+00	1.0000	0E+00	1.0000	0E+00
3 T	0	4	0.		0.		0.		ΰ.		0.		0.	

DELAYED NEUTRON ABUNDANCES

ft GRF	P ISO I	ISO 2	150	3
1	2.22580E-04	2.22580E-04	0.	
2	1.48210E-03	1+48510E-03	0.	
3	:.33410E-03	1.33410E-03	0.	
4	2.67840E+03	2.67840E-03	0.	
5	7.83110E-04	7.83110E-04	0.	
6	2.85690E-04	2.85690E-04	0.	

SOTOPE DATA BY ISOTOPE

ISOTOPE 1

ISOTOPE	PARAMETERS	
ISOID	ISOTOPE NUMBER	1
I SONME	ISOTOPE NAME	U-235
TDC	TEMPERATURE	.27000E÷02
IRZM	MEDIUM INDEX	1
IZAS	=10000*Z+j0*A+S	94350
AWR	ATOMIC WEIGHT	-23302E+03
EFIS	FISSION ENERGY	-31000E-10
ECAP	CAPTURE ENERGY	0.
DCA	DECA' CONSTANT	0.
KBR	MATERIAL TYPE FLAG	1
ICHI	FISSION FLAG	-1
LIN	SCATTERING ORDER: INELASTIC	1
LFL	SCATTERING ORDER. ELASTIC	0
LNSN	SCATTERING ORDER.NON	0
JSXL	SECONDARY XS-LOWEST GROUP	0
JSXH	SECONDARY AS-HIGHEST GROUP	0
JDNL	DELAYED SPECTRUM-LOWEST GROUP	1
JDNH	DELAYED SPECTRUM-HIGHEST GROUP	4
NBINT	NUMBER OF SIGPO VALUES	0
NBTEM	NUMBER OF TEMPERATURES	r

PRINCIPAL CROSS-SECTIONS

POSITION CROSS-SECTION	1 TÜTAL	A850	2 Ret10n	3 FISSION	4 NU-F15SION	5 TRANSPORT	6 N2N	7 FISSION-FRACTION
POSN	GRP	1	GRP	2	GRP 3	680 4		:
1	5.0340	7E+00	8.602	128+00	1.46333E+01	2.382505401		
2	2.4654	4E+00	3.342	178+00 6	5.44198F+00	2.305796+01		· · · · · · · · · · · · · · · · · · ·
3	1.1845	1E+00	1.490	58E+00 2	2.75076F+00	9.335686+00		
4	3.0547	4E+00	3.643	89E+00 (66963E+00	2-26204E+01		
5	5.0340	7E+00	8.602	12E+00 1	.46333E+01	2.38250F+01		
6 TO 7	0.		0.	Ċ).	0.		

POSN	GRP 1	GRP 2	GRP 3	GRP 4
1	3.17386E+00	6.73315E+00	1.09419E+01	1.01027E+01
2	0.	5.77974E-01	1.73726E-02	1.63931E-04
3	G •	0.	1.30278E-03	9.24544E=06
4	0.	0.	0.	0.

INELASTIC SCATTERING CHOSS-SECTIONS, ORDER 1

POSN	GRP	1	GRP	2	GRP	3	GRP	4
1	3.1736	6E+00	6.7331	5E+00	1.0941	9E+01	1.0102	7E+01
2	0.		5.7797	4E-01	1.7372	6E-02	1.6393	1E-04
З	0.		0.		1.3027	8E-03	9.2454	4E-06
4	0.		0.		0.		0.	

ISOTOPE 2

ISOTOPE	PARAMETERS	
ISOID	ISOTOPE NUMBER	2
ISONME	ISOTOPE NAME	U-238
TDC	TEMPERATURE	•54000E+05
IRZM	MEDIUM INDEX	1
IZAS	=10000*7+10*A+S	94380
AWR	ATOMIC WEIGHT	•23600E+03
EFIS	FISSION ENERGY	.31000E-10
ECAP	CAPTURE ENERGY	0.
DCA	DECAY CONSTANT	0.
KBR	MATERIAL TYPE FLAG	2
ICHI	FISSION FLAG	-1
LIN	SCATTERING ORDER. INELASIIC	1
LEL	SCATTERING ORDER. ELASTIC	0
LN2N	SCATTERING ORDER.N2N	0
JSXL	SECONDARY XS-LOWEST GROUP	0
JSXH	SECONDARY XS-HIGHEST GROUP	0
JDNL	DELAYED SPECTRUM-LOWEST GROUP	1
JDNH	DELAYED SPECTRUM-HIGHEST GROUP	4
NBINT	NUMBER OF SIGPO VALUES	0
NBTEM	NUMBER OF TEMPERATURES	0

PRINCIPAL CROSS-SECTIONS

POSITION 1 2 3 4 5 6 7 CROSS-SECTION TUTAL AUSORBTION FISSION NU-FISSION TRANSPORT N2N FISSION-FRACTION

POSN		GRP 1	GRP	2	GRP	3	GRP	4
1		5.24866E+6	0 8.96179	9E+00	1.20974	4E+01	9.8957	5E+00
5		3.87129E-0	1 1.88941	LE-01	5.23039	9E-01	7.3470	4E-01
3		1+39995E+(1 0.		0.		0.	
4		3.89284E-0	1 0.		0.		0.	
5		5=24866E+(0 8.96179	9E+00	1.20974	+E+01	9.8957	5E+00
5 TO	7	0•	0.		0.		0.	

TOTAL SCATTERING CROSS-SECTIONS, URDER 1

POSN	GRP 1	GRP 2	GRP 3	GRP 4
1	4.19103E+00	8.75480E+00	1.15742E+01	9.16105E+00
5	0.	8.07098E-01	1.80550E-02	1.35627E-04
Э	0.	0.	3.39506E-03	0.
4	0.	0.	0.	0.

INELASTIC SCATTERING CRUSS-SECTIONS, ORDER 1

POSN	GRP 1	GRP 2	GRP 3	GRP 4
1	4.19103E+00	8.75480E+00	1.15742E+01	9.16105E+00
2	0.	8.0709AE-01	1.80550E-02	1.35627E-04
3	0.	0.	3.39506E-03	0.
4	0.	0.	0.	0.

ISOTOPE 3

ISOTOPE	PARAMETERS	
ISOID	ISOTOPE NUMBER	3
ISONME	ISOTOPE NAME	ΨE
TDC	TEMPERATURE	•27000E+02
IRZM	MEDIUM INDEX	1
IZAS	=10000*Z+10*A+S	26558
AWR	ATOMIC WEIGHT	•224*********
EFIS	FISSION ENERGY	0.
ECAP	CAPTURE ENERGY	0.
DCA	DECAY CONSTANT	0.
KBR	MATERIAL TYPE FLAG	4
ICHI	FISSION FLAG	0
LIN	SCATTERING ORDER, INELASTIC	1
LEL	SCATTERING ORDER. ELASTIC	0
LN2N	SCATTERING OKDER.N2N	0
JSXL	SECONDARY XS-LOWEST GROUP	0
JSXH	SECONDARY XS-HIGHEST GROUP	0
JONL	DELAYED SPECTRUM-LOWEST GROUP	1
JDNH	DELAYED SPECTRUM-HIGHEST GROUP	4
NBINT	NUMBER OF SIGPO VALUES	0
NBTEM	NUMBER OF TEMPERATURES	0

PRINCIPAL CROSS-SECTIONS

POSITION1234567CROSS-SECTIONTOTALABSORBTIONFISSIONNU-FISSIONTRANSPORTN2NFISSION-FRACTION

POSN	4	GRP	1	GRP	2	GRP	3	GKP	4
1		2.1794	0E+00	3.7788	2E+00	2.2967	3E+00	3,2185	5E+00
2		6.1815	5E-03	1.0219	9E-02	2.2388	0 £-02	1.4974	6E-01
3 TO	4	0.		0.		0.		0.	
5		2.1794	0E+00	3.7788	2E+00	2,2967	3E+00	3.2185	5E+00
6 TO	7	0.		0.		0.		0.	

TOTAL SCATTERING CROSS-SECTIONS, ORDER 1

POSN	GRP	1	GRP	2	GRP	3	GRP	4
1	1.98985E	+00	3.7247	6E+00	2.2741	9E+00	3.0688	30E+00
2	0.		1.8311	1E=01	4.3845	1E-02	1.6149	01E-04
3	0.		0.		2.5620	2L-04	0.	
4	0.		0.		0.		0.	

INELASTIC SCATTERING CROSS-SECTIONS, ORDER 1

POSN	GRP 1	GRP 2	GRP 3	GRP 4
1	1.98985E+00	3.72476E+00	2.27419E+00	3.06880E+00
2	0.	1.83111E+01	4.38451E-02	1.61491E-04
Э	0.	0.	2.56202E-04	0.
4	0.	0.	0.	0.

.

ANISN CONTROL PARAMETERS

ANISN 54	4-GP REGULAR PROBLEM USING FILES SAY	VED IN SAMPLE	PROBLEM 2	3
ID	PROBLEM ID NUMBER	1		
IHT	XSEC TABLE POSN-SIGMA TOTAL	· 3		
IHS	XSEC TABLE PUSN-SIGMA SELF SCATT	4		
IHM	XSEC TABLE LENGTH	7		
IDFM	1/0=DENSITY FACTORS USED/NO	0		
IPM	IM/1/0=SHL SRC-E+G+A/G+A ONLY/NO	0		
IPP	INTRVL WITH SHL SRC IF IPM=1	0		
ID1	1/0= PRINT ANGULAR FLUX/NO	0		
ID3	N/C=N ACTVTYS BY ZONE/NO ACTVTYS	0		
ID4	1/0=N ACTVIYS BY INT (N=103)/NO	0		
IDAT2	1/0=DIFF FOR GIVEN GPS/NO	0		
IFG	1/0=COLLAPSE X-SECTIONS/NO	0		
IFLU	0/1/2=NEG FLX FXUP-MIXD/LIN/STEP	0		
IFN	1/0=FLUX GUESS/FISSION GUESS	0		
IPRT	0/l=PRINT X-SECTIONS/NO	0		
DFM1	TRNSVRS D1R-VOID STRMG CORR	0.		
RYF	NORMALLY 0.5. EPS/RYF=SCAT.CONV.	.50000E+00		

FISSION GUESS

INTERVAL	GUESS	INTERVAL	GUESS
1	.10000E+01	26	.10000E+01
2	-10000E+01	27	10000E+01
3	-10000E+01	28	10000E+01
4	+10000E+01	29	•10000E+01
5	10000E+01	30	10000E+01
6	.10000E+01	31	0.
7	10000E+01	32	0.
8	+10000E+01	33	0.
9	10000E+01	34	0.
10	.10000E+01	35	0.
11	+10000E+01	36	0.
12	10000E+01	37	0.
13	.10000E+01	38	0.
14	•10000E+01	39	0.
15	10000E+01	40	0.
16	10000E+01	41	0.
17	10000E+01	42	0.
18	•10000E+01	43	0.
19	.10000E+01	44	0.
20	•10000E+01	45	0.
21	10000E+01	46	0.
22	•10000E+01	47	0.
23	10000E+01	48	0.
24	•10000E+01	49	0.
25	.10000E+01	50	0.

CONTROL	PARAMETERS	
ITP	MLTGRP FILE. 1/2=READ REF ONLY/READ REF+PERT	1
ID1	1/0=USE ANG FLUX/USE TOT FLUX AND CURRENT	1
MAXLP	MAX NO. UPSCATTER GROUPS.	0
MAXLD	MAX NO. DOWNSCATTER GROUPS.	3
NFI	NO. FISSION ISOTOPES	2
NCAS	NO. PERTURBATION CASES	1

PERTURBATION CASE 1

PARAMET	ERS	
ITPP	TYPE PERT. 1/2/3=DENSITY/X-SEC/BOTH	1
NPM	NO. PERT. MIX SPECIFICATIONS	8
NPR	NO. PERT. ZONES	2

PERTURBATION ZONE NUMBERS

	-
1 1 26	1
2 1 27	1
3 1 28	1
4 1 29	1
5 1 30	1
6 1 31	2
7 1 32	2
8 1 33	2
9 1 34	2
10 1 35	2
11 1 36	2
12 1 37	2
13 1 38	2
14 1 39	2
15 1 40	2
16 1 41	2
17 1 42	2
18 1 43	2
19 1 44	2
20 1 45	2
21 1 46	2
22 1 47	2
23 1 48	2
24 1 49	2
25 1 50	2

PERTURBATION MIXTURE SPECIFICATIONS

	MAT NO./ZONE	MIX NUMBERS	MIX COM	MANDS DENSITIES	PT DENSITIES
1	4	4	0	0	0
S	5	4	1	4.56700E-	03 5.48040E-03
3		4	2	3.43920E-	02 3.43920E-02
4		44	3	7.16700E-	03 7.16700E-03
5		5	0	0	0
6		5	1	8.90000E-	05 8.90000E-05
7		5	2	4.00250E-	02 4.00250E-02
8		5	3	6.12100E-	03 6.121002-03

;

Sample Problem 4. In this problem, the diffusion option of DOT2DB is invoked to obtain regular and adjoint flux files that are then input to DAC to compute the reactivity of a core fuel density perturbation. The model used is a 4-group, finite cylinder containing core and blanket regions. The calculations are performed in two runs.

In run 1, the path *INP DOT INP DOT FL1* T PATH is executed, and, in run 2, the path *FL0 DAC* T PATH

is executed.

OUTPUT FROM FIRST INF CALL--SAMPLE PROBLEM 4

(Lines labeled on the far right by CARD and an integer are card images.)

****STANDARD FILE CARD INFUL****

FILE CONTROL

1 1 4 1 1 1 1 4 4 4 4 4 1 4 1 TINP FILE CONTRL CARD I TREAD 1/2/3/4=READ FILE FROM CARDS/STANDARD FILE/BOTH/SKIP FILE

FILE NAME	IPEAD
ADMNSTR.	1
SN CONS	1
INTQUANT	4
GEO DIST	Ĺ
MIX DATA	1
GRP FLXS (REGULAR)	1
GRP FLAS (AUJOINT)	1
MULTIGRE (REFURENCE)	1
MULTIGRA (AFSTURBED)	4
ZONE DENS	4
ETX INPE	4
ODX INFI	4
ANI INFT	4
DOT INPT	1
CLR INPT	4
DAC INPT	1

##ADMINISTRATION FILE##

FILES-INPUT= 0. OUTPUT= 51

ADMNSTR DATA

\$DIM+TYPE+THEOPY+SCA(+S'+SE10+ZOVESF 2 0 0 0 4 7 2 EMESH\$ 15 15 1 CARD \$BUCK+EV_TYPE+FVM+CONV2 + 1 1+0 0+0 1+0-04 \$IGNUKE\$P5(0+0) CARD \$BCS\$ 1 0 0 1 0 0 4407+1F71+S013 3 1 1+0 SHT+M01+MCR+MXF5 11 11 0 9 CARD \$ZONE_MODST_H3(0) +H2+ cVB 0 0+0 SNG55 4 \$SOURCE 0P\$ 00 CARD \$CONV_SPEC5490 10 0+0 0+0 5+0 0+0 1+0 0+0 0+0 T CARD	* IDX SAMPLE PROBLEM FOR PU CHECK OF DUT2DD. * .	CARD	1
\$BUCK+EV_TYPE+FVM+CONVED1111+0+04_\$1600HE3P5(0+0) CARD \$BCS\$1_0_0_1_0_0_\$400+1F71+5013_3_1_1+0_b4F+M01+4CR+MXFb_11_11_0_9 CARD \$ZONE_MODS\$F_43(0)_\$-24VB_0_0+0_b0655_4_5500HCE_0P5_00 CARD \$CONV_SPEC55490_10_0+0_00_0+0_50_0+0_1+0_00_0+0_0 CARD	\$DIM+TYPE+THEOPY+5C4(+5%+0610+20+65* 2 0 0 0 4 7 2 ENESH\$ 15 15 1	CARU	5
\$ECS\$ 1 0 0 1 0 0 4402+1F71+5013 3 1 1+0 5HF+M01+MCR+MXF5 11 11 0 9 CARD 6 \$ZONE MODS\$ 43(0) +44+ 245 0 0+0 5NG55 4 5500KCE 0H5 00 CARD 5 \$CONV_SPEC5490 10 0+0 0+0 0+0 0+0 0+0 0+0 0+0 0+0 CARD CARD	\$BUCK+EV TYPE+FVM+CONV2 > 1 1+0 0+0 1+0-04 \$IGNUHE\$P5(0+0)	CARU	3
\$ZONE MODS# H3(0) H4+ CVB U 0.0 BNG58 4 \$SQURCE UP\$ 00 CARD \$CONV_SPEC5490 30 0.0 0.0 0.0 0.0 0.0 1.0 0.0 0.0 T CARD	\$6CS\$ 1 0 0 1 0 0 440/+1+21+501\$ 3 1 1+0 54F+401+4CR+MXF5 11 11 0 9	CARD	4
\$CONV SPEC5490 10 0.0 0.0 0.0 0.0 1.0 0.0 0.0 T CARD CARD	SZONE MODSE H3(C) -HAH EVE U U.G DNGSE 4 DSQUECE UPE 00	CARD	5
	\$CONV SPEC5890 10 0.0 0.0 0.0 0.0 0.0 1.0 0.0 0.0 T	CARD	6

The first call to INP in run l is used to read the input shown below. This input creates the files ADMNSTR, SN CONS, GEO DIST, MIX DATA, GRP FLXS, MULTIGRP, and DOT INPT for use in the first DOT call to calculate the regular case. In addition, the input creates an ADJ FLXS file that serves as a flux guess for the second DOT call and creates a DAC INPT file that defines the perturbation specifications for eventual input to DAC.

```
**SN CONSTANTS FILE**
```

FILES-INPUT= 0. OUTPUT= 52

```
MU GIRECTNS
```

MU GIRECTNS	CARD	1
.333333394240434614173333333 .3333333 .8819174714045	CARD	5
3333333 .3333333 T	CARD	3
ETA DIRECTNS	_	
P5(3333333) P3(881417) K5(.3333333) K3(.861417) T	CARD	1
WEIGHTS		
P2(0+0+P4(+09333332)+0+0+R2(0+08333332)) T	CARD	1

GEOMETRY AND MATERIAL DISTRIBUTIONS FILE	
FILES-INPUT= 0. UUTPUT= 53	
1 DIM BORYS	
0.0 19(50.0) 14(90.0) 1	CARD
S DIW BUERS	
0.0 19(60.0) 14(90.0) T	CARD
MAT NUMBERS	
10 J1 T	CARD
ZONE NUMBERS	
R10(R10(1)+R5(2)) R5(P1=(2)) T	CARD

MIXTUPE DATA FILE		
FILFS-INPUT= 0. OUTPUT= -6		
MIX NUMPER'S		
96(10) 25(11) T	CARD	1
міх соммали		
0323450474-1	CARD	1
MTX DENSITYS		
T 5-0-5 2-0-5 2-0-5 1-4-7 0-0 2-1-1-5 1-4-7 1-5-7 5-0-5 T	CARD	1
ISOTOPE NUMS		
1234567647	CARD	1

AADEGUILAP FLUX FTLEPA FILES-THPUT= 0. DUTPUT= 54 GROUP FLUX 121.0 604.0 362.0 5.73 F FLUX DUESS. GROUP FACTURS. CARD ł 1 DIM FLUX 1.0 1.0 0.471 0.944 0.908 0.344 0.812 0.752 0.665 0.612 0.527 1 CARD 0.431 0.328 0.222 0.113 T FLUX GUESS+ CF FACTOPS-10. CARD 2 2 DIM FLUX CARD t 1.0 1.0 0.471 0.444 0.408 0.844 0.812 0.752 0.885 0.612 0.527 CARD 2 0.431 0.328 0.222 U.113 T FLUX GUESS. SPACE FACTORS-10.

ADJOINT FLUX FILE

FILES-INPUT= 0+ OUTPUT= 59

GROUP FLUX

121.0 604.0 362.0 5.73 T FLUX GUESS. GROUP FACTURS. CARD 1 1 DIM FLUX 1.0 1.0 0.971 0.944 0.908 0.864 0.812 0.752 0.685 0.612 0.527 CARD 1 0.431 0.325 0.222 0.113 1 FLUX GUESS. SPACE FACTORS-10. CARD 2

2 DIM FLUX

1.0 1.0 0.471 0.444 0.468 0.844 0.812 0.752 0.685 0.612 0.527 CARO 1 0.431 0.328 0.222 0.113 T FLUX GUESS, SPACE FACTORS-10. CARD 2

REFERENCE MULTIGROUP FILL FILES-INPUT= 0. OUTPUT= 57 FILE NAME *FOUR-GROUP+ [HM=6+3 DUAN OF-5 AS FROM ETOX-10X ₩ T CARD 1 MLTG CONTPOL 110100)1~4(u) T CARD 1 ----**DIMENSIONS** 9461008453201 H7(U) T CARD 1 ISO NUMBERS 1234567891 CARD 1 FIS ISOTOPES -1 -1 0 0 0 -1 0 0 0 T CARD 1 ANI ISOTOPES R9(0) T CARD 1 ISOTOP NAMES 1 *PU239* 0.0 0 2 *J238* 0.0 0 3 *O* 0.0 0 4 *FE* 0.0 0 CARD I 5 *NA# 0.0 0 5 *1238* 0.0 0 7 *7* 0.0 0 8 *1E+ 0.0 0 9 *NA# 0. 0T 2 CARD GROUP STRUCT 0 0 R10(0.0) 3.0213-10 -.4518-10 J.2/42-9 J.314/-8 24(0.0) .576 .411 CARD 1 .013 0.0 T CARD 2 DELAY SPECTR 1 CARD P6(0.+1.0+0.+0.) T DLY ABUNDNCS CARD 1 7.752-05 5.712-04 4.4064-04 6.0912-4 2.1012-4 7.14-5 T PRINCIPAL XS 1.98243 2.0057524 6.4106+ 4.70447 1.635527 R3(0.) CARD 1 CARD 2 1.66228 3.48326 4.6661 7.70001 4.2009378 1.02292 R2(0.) 1.89935 4.4338 - 5.45816 12.9072 A.47043631 0.0958122 0.0452701 0. CARD 3 8.11876 21.5555 23.3008 24.2078 2.6523 0.00296369 R2(0.) T CARD 4 DLY ARUNDNES 1 1.924-4 2.0274-3 2.3675-3 5.7424-3 3.33-3 1.11-3 1 CARD PRINCIPAL XS CARU 1 -543109 1.10529 1. JUNA 4.85537 1.1370641 M3(0.) CARD S .00249434 .159454 .0004 104 /.No108 7.326946 2.44899 H2(0.) 3 CARD 0. .490207 0. 12.7255 14.62326555 .1/3615 .0734659 0. CARD 4 0. 1.43541 0. 12.3745 11.44354 .0033372 0. 0. T

*	CARD	1*0 *0 2-67210*2 88490601*6 21511*6 *) 6-81590*9 *0
£	CARD	*0 *6 /\$ZZE1* AZZRATZE*E I\$5564*C *0 £-I9JUZ*\$ *U
۲	09AD	*0 *0 ZZGHEM* GOZERAM® SHIMAPE *0 MMSBB*9 *0
ι	0942	(*0)8x 4442x5*1 (*7225*) *0 E-9Ed26*1 *0
		Sx Jvaloniaa
*7	0940	T .U .U S-#1145.1 #AEURIA.DI MAAAA.CI .U S-91878.0 .O
£	CARD	*O E-EGH25*T Z-YGZ12*L T9Z4TE9*9 SL299*9 *O Z-SH9EH*L *O
S	CARD	*0 *0 \$5829* \$FT6E865*2 \$FE923*2 *0 E=90E28*5 *0
τ	0840	(*0)FH YLDEZEZA*1 ZALIE*2 *6 E-19969'8 *0
		PRINCIPAL XS
7	08AD	1 *0 *0 2*92670.5 (3*6) 24 (0*6)
٤	CARD	.0 .0 ŁBFB55. PT51500.E A5A0.E (.0)EA
2	CARD	*0 *0 T8+626* 205451*6 6266*6 (*0)68
τ	CARD	0* 1*28655-5 0* 1*584351 *0124588 #3(0*)
		PRINCIPAL XS
7	04AD	1 *0 *0 F-6290*** FELTL*OT SEVENTL *0 LI972*1 *0
ε	09AD	*0 2-15256*2 596572* 72880257*21 6652*21 *0 805865* *0
S	CARD	*O *O 91505*S 919951*L E49A0*H 5-FH97*J 495251* E+E996*1
τ	CARD	(*O)EX 6+52261*1 ++ELK*+ 12+85*1 58501*1 IIIE55*
		BBINCIBVE X2
ĩ	CARD	1 6-11*1 6-66*6 5-4247*9 5-4245*2 6-9280*2 4-486*1
		DLY ABUNUNA YJO
4	CMAD	1 *0 *0 2110510* SEARASHI'S ARUEI'F *0 E-14804*5 *0
٤	CARD	.0 .0 041151, resustry,4 2010A,4 .0 E-TAAEA.5 .0
2	09AD	*0 *0 ZZETE7* £84ZI9EZE*F TEBOS*F *0 +-ZI097*S *0
ĩ	CARD	(*0)E4 6F474/2+*T 84026*T *0 E-19806*T *0
		SX TVaIDNI88
*7	CARD	1 .0 .0 E-SSITE.8 85SICTI.01 AE05.01 .0 S-STTAR.5 .0
£	CARD	°0 E-1/R44°I vT40I40, MAQEAVEL,4 TT44E,4 .0 5-848P8I .0
Ş	QHAD	.0 .0 ESTETA, #IESTERE.5 E0058-5 .0 E-30886.8 .0
τ	CARD	(*D)EH IF#271F9*T F2F22*7 *D E-26E#L*8 *O
		SX TVALDVLA
7	CARD	L *0 *3 2~42E2**1 (9*E)28 (*U)E8
ε	CARD	●0 ●2 F425611● 2226655●6 95555●6 (*0)記者
2	САКО	.0 .0 ALCIN. THANKE I AMERICA (.0) EA
ĩ	CARD	(*B)&# EMBDAM** 24452*T *0 2-25509*T *0</td></tr><tr><td></td><td></td><td>BRINCLOVE X2</td></tr></tbody></table>

DOT2DH INPUT FILE		
FILES-INPUT= 0. UNTPUT= 64		
PARAMETERS		
* SAMPLE DOTADH HAUHLEH *	CARD	ì
* * 1 1 4 5 8 9 10 1 0 -1 1 0 0 1 T CNTRL PARAMS	CARD	2
DIFF MARKERS		-
R4(1) T DIFF MARKERS	CARD	1
CONTRL PPAMS		
1 0 0 3 3 1 T DAC PARAMETERS	CARD	1
CASE PARMINS		
1 11 2 T CASE PAREMETERS	CARD	1
PT ZONE NUS		
R10(R10(1)+H5(2)) +5(H15(2)) 1 20NE NUMHEHS	CARD	1
PT MAT SPECS		
10 11 / MATERIA, NUMRERS	CARD	1
R6(10) R5(11) Z N1X WUMBERS	CARD	2
0 1 2 3 4 5 0 6 7 H 9 / MIX CÚMMAND	CARD	3
0002 .008 .0192 .012 .011 0015 .03 .02 .005 T REF Ø	CARD	4
PT MIX DENS		
00022 .008 .0142 .012 .011 0015 .00 .00. 5500. 0	CARD	1

In the second INP call of run 1, the card inout shown below is read. This input siters the ADMENSTR file previously created to request the adjoint option in the second DOT call.

With a 10^{-4} convergence specification, k_{eff} 's of 0.96272 and 0.96279 are obtained for the regular and adjoint cases, respectively. In the DOT executions, the original flux guesses in the flux files are supplanted by the final converged fluxes. At the end of run 1, FL1 is called to store the existing set of files on tape.

In run 2, FLO is called to recover the files from tape. These existing files provide all the input needed for the DAC execution. The DAC INPT file defines a perturbation in the 239 Pu atom density from 0.002 to 0.0022. This perturbation yields a reactivity of \$10.04. An effective delayed neutron fraction of 0.003117 and a generation time of 9.71 x 10⁻⁸ sec are also obtained.

OUTPUT FROM SECOND INF CALL--SAMPLE PROBLEM 4 (Lines labeled on the far right by CARD and an integer are card images.)

####STANDARD FILE CARE INFOLMARS

FILE CONTROL

3 R15(4) T. INP. FILE CUSTADU. IREAD 1/2/3/4=PEAU FILE FROM CARUS/STANUARU FILE/SUTA/SKIP.FILE CARD 1

FTTE NAME	PE AN
ADMMS TH.	\$
SN CIMS	-•
INFORMUT	4
CEN NICT	-
MIX OTEN	••
Cbb El MA (material and	••
(APA) モデキタ イロジュ エッチャー	-•
MULTISH (Harmony (1))	4
MOFILIARE LUSEL RECEIPT	4
ZONE DENS	4
ETX 1999	4
OUX INPI	4
ANT THOL	- 4
DOT IVEL	4
CC- Thef	**
DAC THET	لم

ADMINISTRATIO : "ILT

FILES-THEOTE SEA AND LAND A

ADMNSTH FAILA

S10 1 Same T. L STATE FILE

CARD 1

<u>Sample Problem 5.</u> In this sample problem, CLUB is used in a 3-group, 9-zone, single-cycle burnup calculation involving 5 chain equations and 16 isotopes. Zone-averaged fluxes are computed at the beginning of each burnup step with the DOT2DB code. Diffusion theory is used for the X-Y, 9 x 12 mesh flux calculation.

The first two input cards, DRIVERO1 and DRIVERO2, instruct the DRIVER program to execute the path FLO INP PRN CLB. It is assumed that a 3-group MULTIGRP file containing the 16 isotopes has been previously generated and stored on tape. This tape must be assigned by the user to logical unit 49. In the call to FLO, the MULTICRP file is copied from the tape to a disk file.

The next input card, INPCNT01, instructs the INP module to create the ADMNSTR, SN CONS, GEO DIST, MIX DATA, GRP FLXS, ZONEDENS, DOT INPT, and CL3 INPT files from card input. Input cards for these files are labeled ADMNST01 through ADMNST03, SNCONS01 through SNCONSO3, GEODISO1 through GEODISO5. MIXDAT01 through MIXDAT06, GRPFLX01, ZONDEN01 and ZONDENO2, DOTINPO1 through DOTINPO3, and CLBINPO1 through CLBINP17. The SN CONS file is always required by DOT2DE because of the possibility of mixed diffusion/S computations. In this example, however, the S_n constants are not used because diffusion theory is specified for all groups. DOT2DB always obtains & flux guess from the GRP FLXS file. The GRP FLXS file created with input card GRPFLX01 is a flat flux guess for the first DOT2DB calculation.

CARD INPUT FOR SAMPLE PROBLEM 5

4 T NUMBER OF PATH ELEMENTS DRIVER01
 *FL0
 INP
 PRN
 CLB*
 T
 PATH
 DRIVER02

 R2(1)
 4
 R3(1)
 R3(4)
 1
 R2(1)
 4
 T
 CARD
 INPUT
 FILE
 CONTROLSINPCNT01
 * ADMNSTR FILE FOR CLUH CALCULATION USING DOT2DB * 2 R3(0) 2 6 ADMNST01 R2(9) 12 1 0 1 K2(0.0) 1.0-4 R5(0.0) 1 K7(0) 1.0 25 153 ADMNST02 0 16 R4(0) 0.0 3 0 20 5 1.0-4 '.0-3 0.1 0.5 0.75 1.0-3 1.5TADMNST03 R2(-0.70711) R2(0.70711) -0.70711 0.70711 T MU DIRECTIONS R3(-0.70711) R3(0.70711) T ETA DIRECTIONS SNCONS01 SNCONS02 R2(0.0.P2(0.25)) T WEIGHTS SNCONS03 0.0 13(10.5) 12(14.85) 11(26.85) T FIRST DIMENSION BOUNDARIES 0.0 19(60.0) 11(66.0) T SECOND DIMENSION BOUNDARIES 17 18 19 20 21 22 23 24 25 T MATERIAL NUMBERS BY ZONE GEODIS01 GEODIS02 GEODIS03 R18(1) R2(R4(5)+R3(9)+R2(1)) R2(P4(4)+R3(8)+R2(1)) R2(R4(3)+R3(7)+R26E0DIS04 (1)) R2(R4(2)+R3(6)+R2(1)) R18(1) T ZONE NUMBERS BY INTERVAL GE0D1505 R17(17) R17(14) R17(19) R17(20) R17(21) R17(22) R17(23) R17(24) MIXDAT01 RI7(25) 1 MIX NUMBERS MIXDAT02 R9(0+1+2+3+4+5+6+7+8+9+10+11+12+13+14+15+16) T MTX COMMANDS MIXDAT03 0+0 0+06 H7(0+0) 0+03 0+0 0+001 R5(0+0) H8(0+0+2+5+2+1+0+20+1+0+MIXDAT04 2+R3(0.0)+4-9768-4+0.0+1-5-2+0.0+1-0+2+R5(0.0)) 1 MIX DENSITIES MIXDAT05 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 T ISOTOPE NOS ON MULTIGRP FILE MIXDAT06 R3(1.0) T FLAT FLUX GUESS FOR DOT2DB GRPF1 X01 .06 R7(0.0) .03 .0 0.001 R5(0.0) R8(2.5-2+1.0-20+1.0-2+R3(0.0)+4.ZONDEN01 9768-4+0.0+1.5-2+0.0+1.0-2+R5(0.0)) T ATOM DENS BY ISOTOPE AND ZONE ZONDEN02 • 3-GRP+X-Y+9+ZONE+9X12 MESH+DUT2DB DIFFUSION PROBLEM FOR USE WITH CLUB*DOTINP01 R2(1) 4 5 6 0 5 R2(0) -1 1 R3(0) T DOT2DB CONTROL PARAMETERSDOTINPO2 DOTINP03 R3(1) T DIFFUSION MARKERS * CLUH 3-GROUP.9-ZONE.DEPLETION PROBLEM USING DOT20H TU **#CLBINP01** * COMPUTE ZONE-AVERAGE FLUXES AT BEGINNING OF EACH BURNUP STEP *CLBINP02 CLBINP03 *D0T* T TITLE AND NEUTRONICS SELECTION * SINGLE CYCLE. 5 CHAIN EQUATIONS. FND-OF-CYCLE KEFF=1.0 ***CLBINP04** 1 1C R2(0) 1 97(0) 1 R2(0) 3 9 3.6+4 R2(1.0) R3(0.0) 5.0+3 0.0 1.0+9 CLBIN-05 CLBINP06 1.0 0.0 42(20..40..60.) 1 0 1 0 K2(1) R2(0) R4(1) T DEPLET HIST CLHINP07 NUMBER OF ZUNE SETS 2 T 1 1 1 2 -1 WRETLECTOR # 2 9 1 1 0 MORE ZUNES # T ZONE CLASSIFICA CLHINP08 CLHINP09 A 4 T HUM SEN OF FISSIONABLE AND FISSION-PRODUCT NUCLIDES 3 4 5 H 7 H F I / N IMHERS OF FISSIONABLE NUCLIDES CLBINP10 12 5.39-02 5.5-02 5.62+2 5.6+2 5.17+2 6.3-2 13 2.1-3 2.2-3 2.3-3 2.3-3 2.4-3 14 2.0 2.2 2.4 2.6 2.8 3.0 15 1.0 1.1 CLBINP11 CLRIND15 1.2 1.3 1.4 1.5 T FISS-PROD IN NOS AND YIELDS FROM FISS NUCLIDES CLHINP13 CLBINP14 T NUMBER OF NUCLIDE UNAINS 5 CLBINP15 11 9 P3(3) T LENGTH OF CHAINS 3 1 4 -1 5 1 6 1 7 1 8 3 1 4 1 200006 1 200007 1 200008 12 -1 13 15 1 CLBINP16 CLHINP17 14 9 1 10 T CHAIN SPECIFICATIONS 2 82(1) 42(2) 44(1) 2 43(1) 82(2) 4 T FILE PRINT CONTROLS PRNCNT01

The last input card, PRNCNTOl, instructs the PRN module to print the ADMNSTR, GEO DIST, MIX DATA, ZONEDENS, DOT INPT, and CLB INPT files created in the preceding call to INP. This printed output is shown below. The last element in the path, CLB,

causes the CLUB code to be loaded and executed. For this particular problem, the depletion cycle is terminated when k_{eff} has been reduced to 1.0. This occurs at 135.7 days after four burnup steps.

OUTPUT FROM PRN MODULE--SAMPLE PROBLEM 5

**ADMINISTRATION FILENE FILE 51

ADMINISTRATION AND CONTROL MARABETERS

ADMNST	IN FILE FOR CLIF CALCULATION USING DUT2	95
IAOI	1/5/3=01+ENETON	2
1411	TYPE, 0=01FFUSION, 1=TRAMSPORT	Û
IA02	THEORY. DEFENDLAR. 1=40001MT	0
TA03	ANISOTROPIC SCALLE-I'NG ORDER	Û
1474	NEDER OF SN PPE-UXIMATION	2
1 GE	0/1/2/3/0/7/F/4/11/12/13=0FUMETRY	6
1ZM	NUMBER OF YONES	9
IM	NUMBER OF 1-1 DIMENSION INTERVALS	4
JM	NUMBER OF 2ND DIMENSION INTERVALS	12
KM	NUMBER OF THE LINENSIUS INTERVALS	1
тек	MUMBER OF HICKLING SETS	U
IFVT	0/1/2/3/4/-/0 = rIULINVALUE IYPE	1
EV	EIGENVALUE SUESS	0.
EVM	FIGENVALUE MUDIFIER	0.
EPS	CUNVERGENCE CHITERION	.10000E-03
TEMP1	UNDEFINED.	0.
TEMP2	UNDEFINED.	0.
TEMP3	UNDEFINED.	ΰ.
TEMP4	UNDEFINED	0.
TEMPS	UNDERINES	0.
1001	-1/0/1/2/3/4 = LEFT BURY CONDITION	1
1802	-1/0/1/2/3/4 = RIGHT HONY CONDITION	0
1803	-1/0/1/2/3/4 = TUP HORY CONDITION	0
1804	-1/9/1/2/3/4 = HOLTOM HORY CONDITION	0
1805	-1/9/1/2/3/4 = FRONT BURY CONDITION	Û
1806	-1/0/1/2/3/4 = MACK BURY CONDITION	0
MG7	0/1/2/3/4/5 = FLOX IMPUT OPTION	0
IFXT	NEGATIVE SOUPCE CHECK. 0=NO.1=YES	0
501	SOURCE NORMALIZATION FACTOR	.i0000E+01
MT	NUMPER OF MATERIALS	25
401	NUMBER OF MATERIAL SPECIFICATIONS	153
MCR	NUMBER OF ISUTIONE X5 SETS FROM CARDS	0
MSF .	NUMBER OF ISOTOPE X5 SETS FROM TAPE-	16
IZ	NUMBER OF IST DIRENSION ZONE MODEYRS	U
JZ	NUMBER OF AND DIMENSION ZONE MODEYRS	0
ĸZ	NUMBER OF SHE DIALNSION ZONE MODEYRS	0
1502	PARAMETRIC ELOENVALUE TYPE	Û
503	PAYAMETHIC EIGENVALUE.	Û.
1 GM	NUMBER OF ENERGY GROUPS	3
406	0/1/2/3/4/5/6 = SOURCE 19097 001[00-	Û
1005	MAXIMUM NUMBER OF OUTER ITERATIONS	20
1007	WIXING WINNER OF I MER LIFRATIONS	5
605	MENTHON MARANCE TEST. UPMORUBO#LIMIT	•10000E=03
606	- POINTAISE ELEA TEST. 0=40.696=LIMEE-	•10000F-05
ALAI	AND I WEN I LATE ADDRESSED	.1∪000€+0∪
ALAH	IANDA INPER I (MII)	• 5 U 0 0 0 E + 0 0
POD	THE WAR AND A STREET OF THE ADDREET AND ADDREET AND ADDREET AND ADDREET AND ADDREET AND ADDREET ADDREE	./5000E+00
FPSA	CONV. CUTTERING IT. SCHNCHES	.10006E-02
XFAC	EXTRAPOLATION FALLON,	15000E+01

BOUNDARTES

1	ST DIMENSH	200 DIDENSN	THU DIMENSIN
1	5	9	
2	2.6250000000	5.000000000000	}
3	S.25000E+00	1.20000++0]	L
4	7.475006+00	1.800000+01	l
5	1.050606+01	2.44000E+0]	L
5	1.195002+01	3.00000E+01	l
7	1.34000E+C1	3.60000F+01	
Ŗ	1.4856VE+01	4.20090E+01	L
9	2.0850VE+01	4,50v00E+V1	
10	2.68500E+01	5.400008+01	
11		6.0000E+01	ł
12		6.JUU00E+01	
13		60000E+01	Ļ

MATERIAL NUMBERS

	ZONF	MATERIAL	NÜ.
1	1	17	
5	7	18	
3	3	19	
4	4	20	
5	5	21	
6	6	26	
7	7	21	
F	×	24	
9	Ģ	ć 's	

ZONE NUMBERS

10/	90	1	5	3	4	2	6	7	8	Ŷ	10	11	12
1		1	1	5	5	4	4	3	Э	2	5	L	1
5		1	1	5	5	4	4	3	ક	2	5	1	1
3		1	1	5	5	4	4	3	3	2	Ż	1	1
4		1	1	5	Э	4	4	3	3	è	5	1	3
5		1	1	4	÷	8	н	1	1	6	6	1	1
6		ł	1	7	7	8	¥	7	7	- 6	6	1	1
7		1	1	y	4	5	8	7	1	6	5	Ł	1
8		1	1	1	1	ì	i	1	1	1	1	Ł	1
9		ł	1	1	1	1	1	1	1	i	l	1	1

MIXTURE DATA FILE FILE 56

MIXTURE SPECIFICATIONS

MIX N	UMBERS	MIX COMMAND	MIX DENSITY	MIX N	UMBERS	MIX COMMAND	MIX DENSITY
1	17	3	Ű	11	37	i u	i.
5	i (:	5.500000-02	12	17	i .	1.556604-03
3	17	۷	0	13	ł i	16	4,
4	17	3	Ú	7 /4	• 7	• :	5
~	17	4	C	15	3.4	1.4	ι,
-	• 7)	U	16	17	15	1)
· ·		+	C	17	17	15	υ U
с.		(Ú	44	1 -	5	ii -
16	17	.*	3	19	15	1	2.500002-02
• • •	1,	ų	1.06003t-02	20	14	60 C	1.600006-20

MIX	NUMBERS	MIX COMMAND	MIX DENSITY	MIX N	UMBERS	MIX COMMAND	MIX DENSITY
21	18	.5	1.000006-02	66	22	ć	1.000006-20
22	18	••	Û	ъч	22	3	1.00009E-02
23	15	5	U	<i></i>	22	4	Û
24	18	C	0	91	55	5	U
25	18	7	4.9/5398-04	92	22	t.	0
26	18	r	0	03	÷ e	1	4.476 <u>8vt-11</u> 4
27	18	ч ,	1.500008-02	92	22	. .	U
24	1.5	10	0			ų 	1.500001-02
29	15	11	1.000005-02	44	22	10	Ŭ
10	18	10	U	07	22	11	1.00000E-05
31	10	13	0	00	20	15	Ů
37	10	14	0	100	22	13	U
74	10	15	0	100	22	14	0
35	19	10 6	0 0	102	22	14	U v
36	19	ĩ	2.5000000-02	107	24	10	0
37	19	2	1.000000000000	104	23	t t	مرية = 5000 م. اس
38	19	3	1.00000E-02	105	23	, ,	2.00000E=00
39	19	4	0	106	23	ž	1.0000000-02
40	19	5	Ŭ	107	23	4	6
41	19	6	õ	108	23	5	ů
42	19	7	4.976H0E-04	109	23	6	ŭ
43	19	ь	0	110	23	7	4-97680E-04
44	19	4	1.500008-02	111	23	H	0
45	15	10	n	112	23	9	1.500008-02
46	19	11	1.000006-02	113	23	10	U
47	19	12	0	114	23	11	1.00000E-02
49	19	13	U	115	23	12	Û
49	19	14	0	116	23	13	Û
50	19	15	U	117	65	14	0
51	14	16	0	118	53	15	Ŭ
52	50	0	0	119	23	16	Û
53	20	<u>ا</u>	2.50000E-02	120	24	0	Ű
54	20	ć.	1.000001-20	151	24	1	2.50000£-02
22	20	3	1.000001-02	122	24	2	1.000001-20
<u>הק</u>	20	4	Ŭ	1:3	24	3	1.000008-02
57	20	5	0	104	24	4 6	0
70	20	ם ז		125	24 74	5	0
27	20	1	4.970802-04	127	24	7	U
- 1 - 1	20	6 L	1 600006660	128	24	, H	4.7700000-04
40	20	10	1.500002-02	129	24	с. Ц	1.500006-02
43	20	10	1 100005-62	130	24	LÓ	1.0000L-02
64	20	12	1.000000C-0r	131	24	11	1.0000000-02
65	20	13	d	132	24	12	0
66	20	14	ů 0	133	24	13	0
67	20	15	Ö	134	24	14	Ú
58	20	16	Ŭ	135	24	15	0
69	51	0	0	136	24	16	U
70	51	1	2.50000E-02	137	25	U	υ
71	51	2	1.000008-20	138	25	1	2.50000+-02
72	21	Э	1.000006-02	139	25	4	1.00000£-20
73	21	4	0	140	25	ذ	1.0000vE-02
74	21	5	U	141	25	4	Ú
75	21	ti (Û	142	25	2	Û
76	21	7	4.9/6H0E-04	143	25	6	U
77	21	н	Ú	144	25	<i>(</i>	4.976801-04
7.F	21	.4	1.500008-02	145	~5	6	0
75	21	16	U	140	22	4	1.000005-07
50 5	- 21	11	1.000008-05	147 -	25	10	U
5 J	<1	1r	0	146	25	17	1-000005-05
#2 5 3	2 C	13	0	147	25	16	U D
n ; a.,	~ 1	14	U A	151	75	14	0
-14- 46		17	9	152		 *'-	
64	24	10	U	157	25	10	ŭ
97	5.E 22	۲. ۲	V 2 606004 -6-3	• ·		- ·	-
~ /	• •	1	* * > * * * * * * * * *				

130	TOPE NO.
1	ì
2	2
,	3 ·
4	4
5	5
6	ĥ
7	7
в	5
9	9
10	10
11	11
12	15
13	13
14	14
15	15
16	ÌЬ

ZONE DENSITIES FILE FILE 58

ATOM DENSITIES

ISOTE	,	ZONE 1	ZUNE 2	ZONE 3	ZONE 4	20NE 5
1		5.00000E-02	2.500008-02	2.50000E-02	2.500008-02	2.50000E-02
2		0 •	1.00000E-20	1.00000t-20	1.00000E-20	1.00000E-20
3		0.	1.000006-05	1.00000F-05	1.00000E-02	1.00000E-02
4 TU	6	0•	U.	0.	0.	0.
7		0.	4.97680E-04	4.97680L-04	4.97680E-04	4 . 97680L-04
8		0.	U .	0.	0.	0.
9		3.000006-05	1.500006-02	1.50000E-02	1.50000L-02	1.50000E-02
10		0.	U.	0.	0.	Ű.
11		1.0000uE-u3	1.000008-02	1.000006-05	1.00000E-02	1.00000E-02
12 10	16	0.	0.	0.	0.	0.

ZONE	6	ZUNE 7	7 1	ZONE	8	ZONE	4
2.5000	0E-02	2.50000E-	-02 2	2.5000)0E-02	2.5000	06-02
1.0000	0E-20	1.00000E-	·20	1.0000	0E-20	1.0000	0E-20
1.0000	0E-02	1.00000E-	02 1	1.0000)0F-02	1.0000	0F-02
0.		0.).		0.	
4.9768	UE-04	4.97680E-	04 4	.976	0E-04	4.9768	0F-04
0.		0.	C).		0.	
1.5000	0E-02	1.500000-	02 1	.5000	0E-02	1.5000	0F-02
0.		Ú.	L).		0.	
1.0000	0E-02	1.00000E-	02 1	.0000	96-30	1.0000	0F-02
Ú.		0.	ú	•		0.	

DOT208 CONTROL PARAMETERS

3-GKP . X-Y .	9-20NE+9X12 MESH+DOT2DB DIFFUSION PROBLEM.	FOR USE	WITH CLUB
A01	PROBLEM (.). NUMBER	1	
FXT	0/1/2=LTN/LIN-HUMP NEG FLA/STP FN	1	
IHT	PUSN SIGNA TUTAL	4	
IHS	POSN SIGNA SELF-SCATTER	5	
ETI.	X-SECTION TABLE LENGTH	6	
M05	NO. REGIONWISE ACTIVITIES	0	
S04	INITIAL INNER IT MAX/GROUP	5	
IAFT	2/0=PRINT FLUA MOMENTS/NU	0	
M04	NO. PUINTWISE ACTIVITIES	0	
M06	+1/0/1/2/3/4=0151 SOURCE OPTION	-1	
1801	0/1/2/3=LEFT BOUNDARY CONVITION	1	
1902	GV1/2/3/4=k10HT BOUND CONDITION	0	
1803	UTITELOND AMAGNOOR HOLES AND HOLES AND HER VELON	υ	
1B04	0/1/2/3=90TTOM BOUND CONDITION	U	

DIFFUSION MARKERS

GROUP	MARKER
1	1
2	1
3	1

CLUB INPUT FILE FILE 65

TITLE AND NEUTRONICS SELECTION

CLUR	3-GROUP+9-ZONE+DEPLETION PROBLEM USING	001208 10
COMPL	ITE ZUNE-AVERAGE FLUAES AT BEGINNING OF	LACH BURNUP STEP
LIN	CPU TIME LIMIT (MIN)	5
SEL	NEUTRONICS SELECTION (A3)	του
	ZAF=NU FLUX CALC. USE ZONE FLUX	
	ON INTIGANT FILE.	
	AIM=INFINITE MEDIUM CALCULATION.	
	ODX=USE ONEDX CODE.	
	ANI=USE ANISH COUL.	
	DOT=USE DUT200 CODE.	
	LANY OTHER FLUX CODE CAN BE	
	ACCENSED MY PUTTING SEGMENTED	
	HINARY VERSION OF CODE ON A	
	FILE WITH 3-CHARACTER NAME.	

DEPLETION HISTORY AND EDITING UPTIONS.

.

SINGLE	CYCLE. S UNAIN EQUALIONS. EMD-OF-CYCLE KEFF=1.0	
ND (1)	MINHED DE CIULES	L
(S) GN	MAX NO. OFFICEFOR TIME STEPS FOR FIRST LYCLE	10
16) ON	MAX NO. DEPLETION TIME STEPS FOR 200 CYLLE -	0
a. 14)	MAX WO. DEPLETION TIME STEPS FOR OTHER LYCLES	0
ND (51	MUMMEN OF SUBSIEPS FOR EACH DEPLETION SIEP -	1
ND (6)	170=RFCALCHEATE ZEUX AL ENU OF TIME STER AND USE ELHEAM AVERAGED FEUX OVER SUBSTER INTERVALSZAU	0
ND (7)	GALESAVE START-OF-STER NUCLIDE DENSITIES FOR- FOD OF CYCLE RECOVERYAND	0
ND (P)	CVI=PENDRMALIZE ΕΓΙΑΧ ΑΕΙΕΡ ΕΑCΗ SUBSIER TO ΑΓΠΙΕΥΕ ΠΗΝΙΕΙΡ ΕΟΜΕΡ LEVELZNO	0

ND (9)	170=RFTURN MUNER LEVEL TO ORIGINAL SUICS (SEE DD(1) AND DU(8)) AFTER EACH FLUX CALEDLATION/ADJUST MOMER LEVEL TO GIVE DESIRFD AVERAGE OVER THE TIME STEPS (SEE DD(5) AND DU(6))	Ŭ
ND (10)	OPTION ON LIMITING POWEN DENSITY (SEE OD(4))- OFTERMINATE IN LIMIT EXCEEDED I=CONTINUT WITH UNCHANGED PROCEDURE 2=ADJUST POWEN TO LIMIT POWER DENSITY 3=ADJUST POWEN TO MAINTAIN POWER DENSITY	0
ND (11)	C/1=LIAITING POWER DENSITY DD(4) APPLIES TO ZONES/SUBZUGES	e
ND (1?)	0/1=LIMITING AVERAGE FXPOSONE DD(9) APPLIES	Û
ND (13)	1/0=DC FLUX CALC FOR END-OF-CYCLE CONDINS/ND-	1
ND(14)	OPTION UN FNU-UF-CYLLE EXTRAPOLATION	ō
••••	-2=EXTRAPOLATE UN FAPOSURE -1=EXTRAPOLATE UN CUNTRUL POISON 0=EXTRAPULATE UN EACESS REACTIVITY 1=DU NUT EATRAPULATE	
ND (15)	OPTION FOR CALCULATION OF CONVERSION RATIO	0
	0-USE CAPTURE RATE IN FERTILE MATCHIAL	
	1=JSE NOGAMMA RATE IN FERTILE MATERIAL	_
ND (15)	NUMBER OF ENERGY GROUPS	3
ND(17)	NUMBER OF WAITNIAL ZUNES	300005+05
	ANTIN OF THERMAL IN EIGENDER (NI) FOR FIRST CICLE AND	+ JOURDE+03
	EVACTION OF COME INCLUDED IN MODEL	10000E+01
00(3)	I TALTING POWER DENS. TY (W/CC) - USE OF DU(4) -	0.
00/147	DEPENDS ON NOTION AND NOTION OFTIONS.	
DD (5)	DESIRED PUNER (MWT) FOR CLASS 1 ZONES	0.
DD (6)	SAME AS DD(5) HUT FOR CLASS 1 AND CLASS 2	0.
	ZONES SUMMED. FLUX ADJUSTED TO HIGHEST	
00/71	LEVEL TO MAINTAIN DU(S) ON DU(O)	600005.04
	CODE DOMED (MT) FOR ALL HIT FIRET CYCLE	1000002+04
	I MITTAG AVEN HEAVY METAL MUTTMETRIC TON	.10000F+10
00(47	TERMINATING CYCLE	1100002.10
DD(10)	DESTRED END-OF-CYCLE MULTIPLICATION FACTOR -	•10000E+01
DD (11)	DESIPED END-OF-CYCLE FRACTION NEUTRON LUSS -	U .
00(12)	DAYS DEPLETION TIME STEP 1. CYCLE 1	.20000E+02
DD (13)	DAYS DEPLETION TIME SILE 2. CYCLE 1	+40000E+02
DD(14)	DAYS DEPLETION HEMAINING TIME STEPS. CYCLE 1-	+00000E+02
DD(15)	SAME AS UD(12) BUT FOR REMAINING CYCLES	•20000E+02
DD (16)	SAME AS DO(13) HUI FOR HEMAINING CYLLES	•40000E+02
DD(17)	SAME AS OD(14) BUT FOR REMAINING CYCLES	+00000E+02
IEDT(1)	N/O=PPINT ZONE NUCL DENS EVERY NTH TIME STEP-	1
IEDT (2)	SAME AS TEDI(1) BUT BY SUBZONE	۵
IEDT(3)	N/O=PRNT TOT NUCL REAC HATES EVERY N STEPS/NO	ĭ
IEDT(4)	SAME AS IEDT (3) BUT BY LONE	Ō
IEDT (5)	N/O=PRNT ZUNE FLUX EVERY M STEPS/NU	1
IEDT (6)	N/0=PPNT ZONE POWER EVENY N STEPS/NO	1
IEDT(7)	N/O=PENT GECAY ACTIVITIES EVERY N STEPS/NO	0
IEDT(8)	1/0=DERUGGING PHIMI/WO	0
IEDT(9)	I/D=PRNT MULTIGHP FILE A-SECTS USED/NO	1
1107(10)	LYDEMANT START OF CIULE NUCL DENS BY ZONE/NOT	I ,
1011111	TYREFAILERD OF CACLE FILCHEE LOADINGCAND	<u>г</u> 1
101/151	TARGET AL CACE LIBSICE LONDINGSING	L

ZONE CLASSIFICATION DATA

NUMBER OF ZONE SETS= 2

IST ZONE NO.	LAST ZUNE NU	NO. SUBZONES	ZONE CL NO.	DEPLETE FLAG	ZONE CLASS NAME (246)
1	1	1	2	-1	REFLECTOR
2	У	1	1	0	CORF ZONES

```
T
                 1.0.
          1
                    3
          2
                    4
          3
                    5
          4
                    6
          5
                    7
          6
                    ~
FISSION PRODUCT 1.0. NUMHERS
                 1.0.
         J
          1
                   12
          S
                   13
          3
                   14
                   15
          4
YIELD OF JTH FISSION PROJUCT FROM ITH FISSION NUCLIDE
          1
                                                            3
                             1
                                    J
                                             2
                                                    J
                                                                    J
                     J
                   5.34000E-02 2.10000E-03 2.00000E+00
                                                                  1.0000000.00
           1
                   5.50000E-02 2.20000E-03 2.20000E+00 1.10000E+00
5.62600E-02 2.20000E-03 2.40000E+00 1.20000E+00
           2
           З
                   5.H0000E-02 2.30000F-03 2.60000E+00 1.30000E+00
6.17000E-02 2.30000F-03 2.80000E+00 1.40000E+00
           4
           5
                   6.30000E-02 2.40000E-03 3.00000t+00 1.50000E+00
           6
NUCLIDE CHAIN SPECIFICATIONS
NUMBER OF CHAINS=
                        5
     CHATN
              LENGTH
          1
                  ŧ١
          2
                   9
          3
                   3
          4
                   3
          ς
                   3
CHAIN SPECIFICATIONS
TYPE TRANSITIONS
     -1=DECAY
      1=TOTAL CAPTURE
      2=N+GAMMA
     3=N+ALPHA
      4=N+2N
     5=N+P
     A=NONDEPLETION
     -3=PAPTS/100000 FISSION TREATED AS CAPTURE
     10=PARTS/1.000.000 TUTAL CAPTURF
CHAIN 1
NUCLIDE 1.D. TRANSITION TYPE
       ٦
                         1
       ć,
                        -1
       5
                         1
       4
                         Ł
       7
                         ł
       н
```

.

•

FISSION YTELP DATA

NUMBER OF FISSION NUCLIDES= 6

FISSION MUCLIPE 1.D. FUMILIES

NUMBER OF FISSION PROLUCE NUCLIDES= 4

```
CHAIN 2
NUCLIDE 1.0.
                TRANSITION TYPE
       З
                        1
       4
                        ı
 200006
                        1
 200007
                        4
 200008
CHAIN 3
NUCLIDE 1.D.
                TRANSITION TYPE
      15
                      -1
      13
CHAIN 4
NUCLIDE I.D.
               TRANSITION TYPE
     15
                       1
     1....
CHAIN 5
NUCLIDE 1.0.
               THANSITION FYPE
      G
                       1
     10
```

REFERENCES

- B. J. Toppel, A. L. Rago, and D. M. O'Shea, "MC², A Code to Calculate Nultigroup Cross Sections," ANL-7318, Argonne National Laboratory (1967).
- R. E. Schenter, J. L. Baker, and R. B. Kidman, "ETOX, A Code to Calculate Group Constants for Nuclear Reactor Calculations," BNWL-1002, Battelle Northwest Laboratory (1969).
- R. W. Hardie and W. W. Little, Jr., "1DX, A One-Dimensional Diffusion Code for Generating Effective Nuclear Cross Sections," BNML-954, Pacific Northwast Laboratory (1969).
- W. W. Engle, Jr., "A Users Manual for ANISN: A One Dimensional Discrete Ordinates Transport Code with Anisotropic Scattering," K-1693, Union Carbide Corporation (1967).
- R. Protaik and E. G. Leff, "Users Manual for DOT2DB: A Two-Dimensional Multigroup Discrete Ordinates Transport/Diffusion Code with Anisotropic Scattering," GEAP-13537, General Electric, Sunnyvale (1969).
- B. M. Carmichael, "DAC1, A One-Dimensional S_n Perturbation Code," LA-4342, Los Alamos Scientific Laboratory (1970).
- G. C. Hopkins, "DAC2, A Two-Dimensional S_n Perturbation Code," LA-4703, Los Alamos Scientific Laboratory (1971).
- T. B. Fowler, D. R. Vondy, and G. W. Cunningham, "Nuclear Reactor Core Analysis Code: CITATION," ORNL-TM-2496, Oak Ridge National Laboratory (1969).
- FORTRAN Reference Manual, Control Data 6400/6500/ 6600 Computer Systems, CDC Publication No. 60174900, Control Data Corporation (1969).

- USA Standard FORTRAN, USAS X3.9-1966, United States of America Standards Institute (1966).
- Clarice W. Cox, "GENRD: A Free-Format Card Input Processor," LA-4793, Los Alamos Scientific Laboratory (1972).
- COMPASS Reference Manual, Control Data 6400/ 6500/6600 Computer Systems, CDC Publication No. 60190900, Control Data Corporation (1969).
- Henry C. Honeck, "ENDF/B: Specifications for an Evaluated Nuclear Data File for Reactor Applications," ENL-50066 (ENDF-102), Brookhaven National Laboratory (1966).
- I. I. Bondaranko (Ed.), <u>Group Constants for</u> <u>Nuclear Reactor Calculations</u>, (Consultants Bureau, New York, 1964).
- W. W. Little, Jr. and R. W. Eardis, "2DB, A Two-Dimensional Diffusion-Burnup Code for Fast Reactor Analysis," BNWL-640, Battelle Northwest Laboratory (1968).
- F. R. Mynatt, "A User's Manual for DOT: A Two-Dimensional Discrete Ordinates Transport Code with Anisotropic Scattering," K-1694, Union Carbide Corporation (1967).
- John C. Vigil, "Dynamic Buffer Allocation on the CDC 6600: The DYNBUF Program," LA-4721, Los Alamos Scientific Laboratory (1971).
- George C. Hopkins, "CONVERT: An IBM-to-CDC Program Conversion Code," LA-4555, Los Alamos Scientific Laboratory (1970).
- "Reactor Development Program Programs Report, March 1971," ANL-7798, Argonne National Laboratory (1971).