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INTEGRAL NEUTRON SCATTERING MEASUREMENTS ON CARBON FROM 1 TO 20 MeV

G. L. Morgan
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MASTER

OAK RIDGE NATIONAL LABORATORY

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Neutron Physics Division

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ABSTRACT

The spectrum of neutrons above ~1 MeV scattered by a thick (3.81 cm) carbon ring has been measured as a function of the incident neutron energy over the range from 1 to 20 MeV. Data were taken for three angles: 60° , 90° , and 135° . A linac (ORELA) was used as a neutron source with a 47-m flight path. Incident energy was determined by time of flight while scattered spectra were determined by pulse-height unfolding techniques. The results of the measurement are presented in forms suitable for comparison to calculations based on the evaluated neutron data files.

I. INTRODUCTION

Measurements have been made of the scattered neutron spectra from a thick (3.81 cm) carbon ring for incident neutron energies between 1 and 20 MeV. These measurements were performed as part of a program underway at ORNL to test the evaluated neutron cross sections for those elements which are important in the analysis of problems related to reactor and weapons shielding.

It is intended that the experimental data be compared to the results of a transport calculation employing those codes commonly applied to shielding problems. In this fashion not only may confidence be gained in the proper handling of the cross-section processing codes and the transport codes themselves, but also the cross section will be checked after having been processed in the way transport codes use them. Because of this constraint of using the transport codes to extract the information from the experiments, these are properly called integral measurements, but their primary purpose still remains the validation of evaluated cross-section files. In order to test evaluated neutron cross-section files, the most important criterion for the type of integral experiment is that it have a sensitivity which can be localized. That is, if the calculation and experiment are not in agreement, it must be unambiguously clear which cross section is in error and by how much. This requires an experiment with adequate resolution in direction and energy for both the incident and secondary neutrons. The term 'adequate' as used here means resolution comparable to the group cross sections commonly employed in shielding analysis. The best way to achieve this type of sensitivity was to use an experimental configuration similar to that commonly employed

for differential cross-section measurements - the ring geometry. However, because comparison to cross-section libraries can be made using sophisticated transport codes, the sample thickness and detector geometry may be adjusted to give optimum count rate and signal-to-background ratio rather than the low multiple scattering and good angular definition normally needed for a differential cross-section measurement. The dimensions of the ring scatterer may be adjusted in any given situation to approach a differential cross-section measurement or emphasize the neutron transport problem depending upon the degree of confidence which one has in the evaluated data file.

Although carbon is not an element of prime importance in shielding applications, it was chosen for a first experiment for several reasons. Next to hydrogen, carbon is the simplest, best understood, and most thoroughly investigated element. Its cross section exhibits sharp structure even at the higher energies and the angular dependence also varies rapidly in this region. These properties make it an ideal test case for investigating the adequacy of the experimental technique and calculational methods. For this test case a ring with radial thickness of about one mean-free path was selected as a compromise which would not emphasize the neutron transport problem.

In the work reported here, data were taken at three points corresponding to average laboratory scattering angles of 60° , 90° , and 135° . The results are presented in several different ways, each designed to provide the maximum amount of information about a particular feature of the cross section.

II. EXPERIMENTAL PROCEDURE

The Oak Ridge Electron Linear Accelerator (ORELA) was used as the neutron source. The neutron production target¹ consisted of a tantalum electron-bremstrahlung converter followed by a beryllium block. Neutrons were created by the $\text{Be}(\gamma, n)$ reaction. The neutron source was 'defined' by a 7.6-cm-diameter collimator located about 3 m from the Be block. A depleted uranium filter (2.54 cm thick) located 9 m from the Be block was used to reduce the intensity of the gamma flash. The depleted uranium filter was followed by a 1.9-cm-thick ^{10}B filter which removed 'all neutrons' below 1 keV, preventing an overlap of low-energy neutrons from one neutron burst to the next. In the work reported here an electron burst width of 12 ns was used. The repetition rate was 800 sec^{-1} with a total beam power of about 10 kW. The schematic of neutron source and flight path is shown in Figure 1.

The scattering sample was positioned approximately 47 m from the neutron source. It consisted of a graphite ring 25.37 cm in outer diameter, 15.24 cm in inner diameter with a thickness of 3.81 cm. The density was 1.741 g/cm^3 . Secondary neutrons emitted from the ring were detected by a 4.22-cm-long by 4.65-cm-diameter NE-213 scintillation counter positioned on the axis of the ring (see Figure 2). In this arrangement, the average scattering angle was selected by the position of the detector along the axis of the scattering sample. A shadow bar of polyethylene shielded the detector from the direct neutron beam. Because the source was 47 m away, the incident flux was essentially a parallel neutron beam.

As indicated in Figure 2, a fast signal from the detector was used to determine the time after the linac pulse at which a scattered neutron

Fig. 1. Neutron source and flight path at ORELA used for this experiment.

Fig. 2. Schematic diagram of the scatterer-detector geometry, electronics, and data handling system.

was detected. Since the flight time of neutrons from the source to the sample was long compared to any transport time in the sample-detector system, the time at which the scattered neutron was detected provided an accurate measure of the incident neutron energy (about 780 ns are required for a 20 MeV neutron to traverse 47 m, but a 1 MeV neutron needs only 12 ns to cover the maximum dimension of the sample-detector system).

The pulse height of the event was obtained from a linear charge signal from the detector which was recorded simultaneously. Neutron-gamma-ray discrimination was used to reject gamma-ray events. Pile-up detection logic served to insure that only those events were recorded for which pulse-height information had not been distorted by the occurrence of more than one event. The average counting rate was about 0.5 events/beam burst.

Valid events were selected on-line by a PDP-9 computer. These events were then transferred to the ORELA SEL-810B computer for storage on magnetic disk (see Figure 2) in a two-parameter array of time of flight versus pulse height. The pulse-height data had been digitized to 4096 channels by the ADC and were "crunched" for storage in this array in such a way as to retain a dispersion adequate for the pulse-height resolution of the detector over the complete neutron energy range. In addition, a high-resolution (4 ns/channel) time-of-flight spectrum was stored for all neutron events which produced valid time-of-flight, pulse-height, and pulse-shape discrimination data without pile-up.

As illustrated in Figure 2 the logic of data acquisition was as follows: the linac beam burst signal generated an observation gate which was put in coincidence with the fast signal from the detector. This gate was 8 μ s long (1 MeV neutrons traverse 47 m in 3.5 μ s) and started soon enough to overlap

events in the detector due to the gamma flash. The first event in the detector which triggered the fast discriminator during this gate period was the event which would be analyzed. Any other event triggering the fast discriminator inside the gate period and within 4.0 μ s of the first signal was used to indicate that a pile-up had occurred. The time-of-flight data generated by the first event were stored in a separate spectrum (TOF #1) regardless of the condition of the other parameters which categorized the event (i.e., pulse height, pile-up, etc.). If the pile-up logic indicated that pile-up had occurred, the time-of-flight data were also stored in a second (TOF #2) spectrum reserved for those events spoiled by pile-up. In this case the other two parameters were ignored and no data were stored in either the two-parameter array or the high resolution neutron TOF spectrum. If no pile-up was detected, the pulse-height and pulse-shape discrimination data were examined to determine whether the event was due to a neutron or gamma ray. If due to a neutron, the event was stored in the two-parameter array and the high-resolution TOF spectrum. If due to a gamma ray, the event was stored in a separate one-parameter spectrum (512 channels for gamma-ray TOF).

Gamma flash events occurred in about 5% of the beam bursts. They were, of course, easily separated by TOF but reduced the number of useful linac bursts by 5%.

Special care was given to the scaling of various pulses in the electronics and of all events which had to be discarded by the computer for any reason. This was done to verify that all events were accounted for in the data reduction and obtain a check on the corrections performed.

The total TOF spectrum (#1) was used for deadtime corrections. For events occurring at a given time, T , after the linac burst, the sum of all events at earlier times gave that number of bursts for which the electronics were already busy and thus unavailable to events occurring at time T . The "pile-up" TOF spectrum (#2), when compared to TOF #1, allowed determination of the fraction of events in each time channel which were lost because of pile-up. These two spectra thus provided the information necessary to correct the two parameter array and neutron time-of-flight spectrum for deadtime and pile-up losses.

The neutron flux incident on the scattering sample was measured using the same detector as was used to detect the scattered neutrons. The flux measurements were made with the detector axis perpendicular to the beam at eight equally spaced points around the position of the scattering ring (neither sample nor shadow bar were in place during these measurements). Because the same detector was used to measure both incident flux and scattered neutrons, any uncertainties in the absolute efficiency cancelled out and only uncertainties in the relative efficiency affect the measurements. The efficiencies used with this detector are discussed at the end of this section.

Additional flux measurements were made at the sample position with and without the shadow bar in place. Within the limits of statistical accuracy ($\approx 1\%$) no effect of the shadow bar on the incident flux was observed.

Using a 2-cm-long by 2-cm-diameter NE-213 detector, measurements were made of the neutron beam profile at the sample position. By time-of-flight restrictions only neutrons whose energy was between 1 and 20 MeV were

observed. These results are shown in the inset of Figure 3. As indicated here, the profile is not constant. This is due to the fact that the beryllium block is an extended source of nonuniform brightness. The overall diameter of the beam was about 40 cm.

Careful comparison of the flux measurements for various points around the scattering ring showed that there was no variation in the shape of the incident spectrum with location in the beam (see Figure 3). The flux used in analyzing the scattering data was obtained by averaging the eight measurements made around the ring.

Flux normalization between the flux measurements and actual scattering measurements was made using a monitor counter (6 mm x 6 mm NE-110 scintillator) located in the edge of the neutron beam 30 m from the neutron source. Time of flight was used to insure that the monitor counted only neutrons in the energy range from 3 to 20 MeV. As a check, a second monitor was also recorded. This monitor consisted of a fission chamber located in the linac target vault. Over the course of the experiment, the ratio of these two monitors remained constant to within 4%.

The shadow bar (≈ 75 cm long), the sample, and the detector were suspended by strings or thin wire from a light aluminum frame which was entirely out of the neutron beam. Measurements were made at three detector positions corresponding to average scattering angles of about 60° , 90° , and 135° (a detailed schematic diagram of the sample-detector geometries is contained in the appendix). Each measurement consisted of a run with the sample in place and a background measurement with the sample removed. The scattering sample was located at least 2 m from any large mass and the background was assumed to be independent of the sample. Overall backgrounds were about 4% at 60° and 90° and about 9% at 135° .

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Fig. 3. The neutron flux measured at several points on the scattering sample position. The inset shows the profile of the neutron beam. The scattering ring is shown for reference along with labels indicating where the flux measurements were made.

In order to determine the spectrum of the scattered neutrons, it is necessary to unfold the pulse-height spectra. This requires a knowledge of the pulse-height response of the detector as a function of the incident neutron energy. The relative pulse-height distributions were measured as a function of neutron energy by placing the detector in the direct beam and recording pulse height versus time of flight. This was done for two detector orientations: detector axis parallel to the incident beam and detector axis perpendicular to the beam. It should be emphasized that the response functions were measured immediately after the scattering measurements and using the same electronic configuration and average counting rate. Thus these data represent the response of the entire data acquisition system.

Comparison of the pulse-height distributions for the two different detector orientations showed no difference in shape even for incident energies as high as 20 MeV, or as low as 1 MeV. Absolute efficiencies as a function of detector orientation were determined in a separate experiment made with the same detector. This experiment showed that the pulse-height spectrum for neutrons from an Am-Be source placed 1.4 m from the center of the detector was independent of detector orientation in both shape and absolute counting rate (to within 5%). Therefore, the response of the detector when expressed in terms of neutrons per square cm is independent of orientation with respect to the neutron flux.

The efficiencies used in this work are based on a set of absolute integral efficiencies given in Ref. 2. These efficiencies are given in terms of counts/incident neutron for neutrons laterally incident on a detector 4.65 cm in diameter and 4.60 cm long. The integral efficiencies

used here (see Table 1 in the appendix) were obtained by multiplying the values given in Ref. 2 for the appropriate bias level, by the lateral cross-sectional area (19.6 cm^2) of the present detector. The system of units for pulse height as well as the curves for light output versus proton recoil energy given in Ref. 2 were adopted for the present work.

III. DATA REDUCTION

The results of the sample-in and background runs at each angle were a two-parameter array of the pulse-height distribution due to the scattered neutrons versus the time of flight of the incident neutron ($\approx 275,000$ channels) and a high resolution spectrum of time of flight only (≈ 2048 channels). The background spectra were subtracted from the sample-in data after corrections were made for deadtime and pile-up losses. Uncertainties associated with these corrections and background subtraction were propagated through all the succeeding analysis.

The integral high resolution time-of-flight spectrum at each angle was reduced to provide the integral counting rate in the detector as a function of the energy of the incident neutrons. This counting rate was normalized to unit incident neutron using the measured flux. Because these data make maximum use of the available statistics to get the best time resolution, they provide the greatest amount of information about structure in the cross section. The data are shown in Figure 4. Detailed values are given in Tables 2, 3, and 4 of the appendix. Comparison of these data to cross-section sets would require that the calculation include the detector and its efficiency as a function of neutron energy (Table 1 of the appendix).

The spectra of scattered neutrons were obtained by unfolding the pulse-height spectra resulting from integration of the two parameter data over a

Fig. 4. The integral counting rate due to scattered neutrons as a function of incident neutron energy.

range of times of flight. The extent of these TOF intervals was determined by the need to get adequate statistics in the pulse-height spectra. Below 10 MeV incident energy the data were integrated over 1 MeV intervals. Above 10 MeV, 2.5 MeV wide intervals were used.

The pulse-height unfolding was done using the code FERD (see Ref. 3 for a detailed description of the technique and its application) and the response functions whose measurements were described in the previous section. The response functions were formed by dividing the two-parameter data of the response function measurements into 76 time-of-flight intervals to provide pulse-height responses at comparison point³ energies covering the neutron energy range from 0.76 to 24.0 MeV. Sixty-eight pulse-height bins were used to cover the pulse-height range from 0.1 to 14.0 light units. The incident energy and pulse-height bin widths were varied so as to be small compared to the pulse-height resolution of the detector. The pulse-height response distributions were converted to absolute responses by normalizing the integral over the region of the proton recoil plateau to the integral efficiencies of Ref. 2.

The neutron energy spectra produced by this unfolding procedure still retain the effects of the finite pulse-height resolution of the detector. In the present data the resolution (in neutron energy) varied from about 30% at 1 MeV to 17% at 20 MeV according to the following formula

$$R = \sqrt{250 + 670/E_n}$$

where R = FWHM in percent and E_n = neutron energy in MeV.

As a test and illustration of the technique, the neutron flux from the beryllium block through a 5-cm uranium filter was measured by both time-of-flight and pulse-height unfolding over the energy range 1 to 20 MeV (a

description of a similar type of test can be found in Ref. 4). The results are shown in Figure 5. Since the pulse-height spectrum was allowed to contain only events due to 1 to 20 MeV neutrons (by means of a restriction on time of flight), the ends of the pulse-height unfolded spectrum show the effects of the finite pulse-height resolution whereas the ends of the TOF results are sharp due to the comparatively good TOF energy resolution. These resolution differences account for the only non-negligible differences between the two approaches.

Several of the secondary neutron spectra resulting from the application of the above described procedure to the carbon scattering data are shown in Figures 6 to 8. These data have been normalized to unit incident neutron using the integral of the incident flux over the same incident energy range from which each pulse-height spectrum was formed. The units in which these data are defined are the secondary neutron flux per MeV at the detector averaged over the projected area of the detector as seen from a point on the scattering ring. The results are tabulated in the appendix (Tables 5, 6, and 7).

Because of the demands on statistical accuracy, the unfolding technique requires rather large bins in the incident energy. Thus while this approach provides a great deal of information about the secondary neutron spectra, the effects of sharp structure in the cross section are washed out by the need for large incident energy bin widths. As a compromise between the unfolding procedure and the integral count-rate analysis, a third method of data reduction, pulse-height weighting, was undertaken. This approach, which is a pulse-height unfolding technique, may be used to provide general information about the secondary neutron spectrum (e.g., total number of scattered

Fig. 5. Comparison of the neutron flux through 5 cm of uranium as determined by time-of-flight and pulse-height unfolding techniques.

Fig. 6. Scattered neutron spectra at 60° for several different incident energies. The abscissa is secondary neutron energy; the ordinate is neutron flux, at the detector, averaged over the projected area of the detector as seen from a point on the ring.

Fig. 7. Scattered neutron spectra at 90° for several different incident energies.

Fig. 8. Scattered neutron spectra at 135° for several incident energies.

neutrons, average secondary neutron energy) allowing the use of smaller bin widths on the incident energy and thus permitting the effects of sharp structure in the cross section to be seen. The method will be described briefly.

Let $\bar{I}$ denote some integral quantity of the scattered neutron spectrum. This quantity is then a function of the scattered neutron flux at each energy, $\phi(E)$ and the value of I at each neutron energy. That is,

$$\bar{I} = \int_0^{\infty} \phi(E) I(E) dE \quad (1)$$

Let $g(p)$ be a "weight factor" determined such that

$$\int_0^{\infty} g(p) R(p,E) dp = I(E) \text{ for all } E, \quad (2)$$

where $R(p,E)$ is the response of the detector to neutrons of energy E ; i.e., $R(p,E)dp$ is the probability that unit flux at energy E will produce an event in the pulse height interval dp about p . Using equation (2) in (1) gives

$$\begin{aligned} \bar{I} &= \int_0^{\infty} \phi(E) dE \int_0^{\infty} g(p) R(p,E) dp \\ &= \int_0^{\infty} g(p) dp \int_0^{\infty} \phi(E) R(p,E) dE \\ &= \int_0^{\infty} g(p) L(p) dp \end{aligned} \quad (3)$$

where

$$dp L(p) = dp \int_0^{\infty} \phi(E) R(p,E) dE$$

is just the number of events in dp about p produced by the spectrum, $\phi(E)$. In practice, the pulse-height distribution is a series of bins (possibly of varying width) so one can redefine the quantities as follows:

$$L_i = \int_{P_i - \Delta P_i}^{P_i + \Delta P_i} L(p) dp$$

$$R_i(E) = \int_{P_i - \Delta P_i}^{P_i + \Delta P_i} R(p, E) dp.$$

Then equations (2) and (3) become

$$I(E) = \sum_i g_i R_i(E)$$

and

$$\bar{I} = \sum_i g_i L_i.$$

If one determines a set of g_i using the condition

$$\sum_i g_i R_i(E) = 1$$

then

$$\bar{I} = \int_0^{\infty} \phi(E) dE = N_T, \text{ the total neutron flux.}$$

Or, using the condition

$$\sum_i g_i R_i(E) = E$$

gives

$$\bar{I} = \int_0^{\infty} \phi(E) E dE = E_T = \text{total neutron energy carried by the flux.}$$

Using the N_T derived above this can be converted to the average neutron energy

$$\bar{E} = E_T/N_T.$$

Because of the finite bias in pulse-height with any practical detector, only events are sampled which are due to neutrons with energies greater than some minimum value. Then the N_T and $\bar{E}$ are quantities representative only of that part of the scattered spectrum above this minimum energy.

For the present analysis 20 weight factors were used to cover the pulse-height range from 0.1 to 12.0 light units. The response functions were determined as has been previously described, except that in this instance the available two-parameter data were partitioned to produce response functions at 34 energies between 0.76 and 20.0 MeV. The incident energy and pulse-height bin widths were varied so as to be compatible with the pulse-height resolution of the detector. To arrive at a set of g_i it was necessary to satisfy 34 normalization equations which contained 20 unknowns. Such an over determination allows use of a least squares method which helps to prevent instabilities in the solution. A solution was deemed satisfactory if all the normalization equations were satisfied to within 2%. The normalization constants were given weights in the fit in such a manner to provide uniform goodness of fit over the whole energy range.

For the pulse-height weighting analysis, the scattered neutron pulse-height spectra were formed over 0.10 MeV intervals in incident neutron energy for incident energies less than 9 MeV. Between 9 and 12 MeV, intervals of 0.30 MeV were used and above 12 MeV the intervals were 1.0 MeV. The total flux and average secondary neutron energy derived from the data are shown in Figures 9 to 11. Tables 8, 9, and 10 in the appendix give the numerical values.

Fig. 9. Total secondary neutron yield and average secondary energy at 60° . Minimum energy sampled is 0.76 MeV.

Fig. 10. Total secondary neutron yield and average secondary energy at 90° . Minimum energy sampled is 0.76 MeV.

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Fig. 11. Total secondary neutron yield and average secondary energy at 135° . Minimum energy sampled is 0.76 MeV.

As a test of this technique, those pulse-height distributions which were prepared for the unfolding procedure were also analyzed by the pulse-height weighting technique. The results for N_T and $\bar{E}$ were then compared with the same quantities calculated directly from the unfolded secondary neutron spectra. All differences between the two approaches were less than 1% for N_T and less than 2% for $\bar{E}$.

IV. SUMMARY

An integral neutron scattering experiment has been performed on carbon to provide the necessary data for checking the cross-section libraries. The prime criterion was that the experiment have localized sensitivity thus requiring adequate resolution in energy and direction for both incident and scattered neutrons.

Reference to Figure 4 shows the experiment had quite good resolution in incident energy. The structure in the carbon cross section is clearly seen. Only a few very narrow resonances are missed in this data. Likewise the strong variation in structure with angle indicates adequate angular resolution for the scattered neutrons.

The secondary neutron spectra shown in Figures 6 to 8 clearly indicate the measurement had adequate resolution in scattered neutron energy. There is clear separation between the elastically scattered neutrons and those inelastically scattered from the first excited state in ^{12}C (4.43 MeV). The data at the higher incident energies clearly show the presence of lower energy secondary neutrons due to inelastic scattering to higher levels. These data provide a great deal of information about the secondary neutron spectra and are very useful in resolving or evaluating any differences between the calculation and experiment for the integral count rate versus incident energy (Figure 4).

The pulse-height weighting results provide a convenient compromise which permits good resolution on the incident energy and provides rough information about the secondary neutron spectrum.

Cross section tests based on this experiment are presently underway at this laboratory⁵ and will be released shortly.

V. ACKNOWLEDGMENTS

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APPENDIX

CARBON RING TABULATED DATA

Fig. 12. Details of the sample-detector geometry.

TABLE 1.

DETECTOR EFFICIENCY, IN COUNTS/INCIDENT NEUTRON/SQ. CM, AS A FUNCTION OF NEUTRON ENERGY
FOR A BIAS LEVEL OF 0.030 LIGHT UNITS (SEE TEXT).

NEUTRON ENERGY (MEV)	EFFICIENCY (CTS/N/SQ. CM)	NEUTRON ENERGY (MEV)	EFFICIENCY (CTS/N/SQ. CM)	NEUTRON ENERGY (MEV)	EFFICIENCY (CTS/N/SQ. CM)
0.200	0.00000	1.900	6.75395	5.250	4.65066
0.225	0.00000	2.000	6.66590	5.500	4.54745
0.250	0.00000	2.100	6.57948	5.750	4.44382
0.275	0.00000	2.200	6.49470	6.000	4.33978
0.300	0.05444	2.300	6.41216	6.250	4.22193
0.325	0.35728	2.400	6.33139	6.500	4.11150
0.350	0.90540	2.500	6.25207	6.750	4.00978
0.375	1.44580	2.600	6.17420	7.000	3.91677
0.400	1.97847	2.700	6.09778	7.250	3.83727
0.425	2.50341	2.800	6.02281	7.500	3.76396
0.450	3.12497	2.900	5.94929	7.750	3.69573
0.475	3.71528	3.000	5.87722	8.000	3.63256
0.500	4.26389	3.100	5.80660	8.250	3.58104
0.525	4.77082	3.200	5.73744	8.500	3.53345
0.550	5.23606	3.300	5.66914	8.750	3.48882
0.575	5.65962	3.400	5.60210	9.000	3.44713
0.600	6.04149	3.500	5.53672	9.250	3.40840
0.650	6.45163	3.600	5.47298	9.500	3.37261
0.700	6.69044	3.700	5.41089	9.750	3.33977
0.750	6.89883	3.800	5.35045	10.000	3.30989
0.800	7.07679	3.900	5.29166	11.000	3.25065
0.850	7.22434	4.000	5.23452	12.000	3.17461
0.900	7.34147	4.100	5.17904	13.000	3.05634
1.000	7.39508	4.200	5.12520	14.000	2.97848
1.100	7.41231	4.300	5.07506	15.000	2.94440
1.200	7.39870	4.400	5.02689	16.000	2.94436
1.300	7.30695	4.500	4.97932	17.000	3.00324
1.400	7.21288	4.600	4.93236	18.000	3.02739
1.500	7.11979	4.700	4.88602	19.000	3.01408
1.600	7.02767	4.800	4.84029	20.000	3.00395
1.700	6.93498	4.900	4.79517	21.000	2.99699
1.800	6.84364	5.000	4.75345	22.000	2.99320

TABLE 2.

INTEGRAL NEUTRON COUNTING RATE AS A FUNCTION OF INCIDENT NEUTRON ENERGY
FOR AN APPROXIMATE SCATTERING ANGLE OF 60 DEGREES. AT THE LOWER ENERGIES,
THE DATA ARE AVERAGES OVER 0.020 MEV INTERVALS.

ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC.NEUT.	ERROR (ABS)	ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC.NEUT.	ERROR (ABS)
1.0017	0.0211	0.298E-02	0.167E-04	2.9630	0.0119	0.167E-02	0.327E-04
1.0219	0.0193	0.296E-02	0.173E-04	2.9810	0.0241	0.145E-02	0.214E-04
1.0415	0.0199	0.295E-02	0.168E-04	2.9991	0.0121	0.128E-02	0.284E-04
1.0616	0.0204	0.293E-02	0.161E-04	3.0174	0.0245	0.123E-02	0.195E-04
1.0810	0.0184	0.285E-02	0.165E-04	3.0421	0.0248	0.122E-02	0.193E-04
1.1010	0.0216	0.273E-02	0.153E-04	3.0607	0.0125	0.125E-02	0.271E-04
1.1215	0.0194	0.270E-02	0.162E-04	3.0796	0.0253	0.123E-02	0.190E-04
1.1412	0.0199	0.279E-02	0.161E-04	3.0987	0.0128	0.123E-02	0.267E-04
1.1614	0.0205	0.279E-02	0.159E-04	3.1179	0.0258	0.126E-02	0.191E-04
1.1821	0.0210	0.284E-02	0.158E-04	3.1438	0.0261	0.129E-02	0.191E-04
1.2019	0.0185	0.284E-02	0.167E-04	3.1635	0.0132	0.132E-02	0.274E-04
1.2205	0.0189	0.283E-02	0.164E-04	3.1833	0.0266	0.134E-02	0.195E-04
1.2413	0.0226	0.277E-02	0.149E-04	3.2033	0.0134	0.136E-02	0.276E-04
1.2625	0.0199	0.273E-02	0.159E-04	3.2236	0.0271	0.141E-02	0.198E-04
1.2826	0.0208	0.270E-02	0.157E-04	3.2439	0.0187	0.166E-02	0.283E-04
1.3015	0.0193	0.267E-02	0.171E-04	3.2576	0.0138	0.168E-02	0.285E-04
1.3208	0.0213	0.266E-02	0.154E-04	3.2784	0.0278	0.152E-02	0.203E-04
1.3405	0.0181	0.261E-02	0.166E-04	3.2993	0.0140	0.154E-02	0.287E-04
1.3607	0.0222	0.261E-02	0.148E-04	3.3205	0.0283	0.156E-02	0.204E-04
1.3813	0.0190	0.246E-02	0.155E-04	3.3418	0.0143	0.157E-02	0.286E-04
1.4004	0.0194	0.245E-02	0.153E-04	3.3633	0.0289	0.158E-02	0.204E-04
1.4200	0.0198	0.243E-02	0.151E-04	3.3851	0.0146	0.159E-02	0.286E-04
1.4400	0.0207	0.240E-02	0.147E-04	3.3997	0.0147	0.157E-02	0.286E-04
1.4604	0.0206	0.243E-02	0.146E-04	3.4216	0.0296	0.160E-02	0.207E-04
1.4812	0.0211	0.239E-02	0.142E-04	3.4441	0.0150	0.161E-02	0.287E-04
1.5025	0.0215	0.237E-02	0.140E-04	3.4591	0.0151	0.159E-02	0.286E-04
1.5220	0.0195	0.233E-02	0.155E-04	3.4818	0.0298	0.159E-02	0.200E-04
1.5420	0.0224	0.234E-02	0.138E-04	3.5047	0.0154	0.155E-02	0.279E-04
1.5623	0.0182	0.230E-02	0.151E-04	3.5201	0.0155	0.159E-02	0.284E-04
1.5807	0.0186	0.225E-02	0.149E-04	3.5356	0.0156	0.157E-02	0.281E-04
1.5994	0.0189	0.222E-02	0.148E-04	3.5591	0.0314	0.155E-02	0.196E-04
1.6209	0.0241	0.225E-02	0.131E-04	3.5828	0.0159	0.154E-02	0.277E-04
1.6428	0.0197	0.220E-02	0.144E-04	3.5987	0.0160	0.150E-02	0.274E-04
1.6627	0.0200	0.221E-02	0.144E-04	3.6228	0.0323	0.153E-02	0.192E-04
1.6829	0.0204	0.222E-02	0.145E-04	3.6471	0.0163	0.151E-02	0.270E-04
1.7009	0.0155	0.217E-02	0.165E-04	3.6634	0.0164	0.146E-02	0.265E-04
1.7192	0.0211	0.218E-02	0.145E-04	3.6799	0.0165	0.148E-02	0.266E-04
1.7404	0.0215	0.215E-02	0.145E-04	3.6965	0.0166	0.149E-02	0.267E-04
1.7621	0.0219	0.215E-02	0.146E-04	3.7216	0.0336	0.143E-02	0.184E-04
1.7814	0.0167	0.207E-02	0.164E-04	3.7469	0.0170	0.137E-02	0.253E-04
1.8010	0.0226	0.203E-02	0.138E-04	3.7639	0.0171	0.137E-02	0.253E-04
1.8209	0.0172	0.195E-02	0.155E-04	3.7811	0.0172	0.133E-02	0.250E-04
1.8412	0.0234	0.189E-02	0.132E-04	3.7983	0.0173	0.133E-02	0.248E-04
1.8618	0.0178	0.185E-02	0.149E-04	3.8157	0.0174	0.134E-02	0.249E-04
1.8797	0.0181	0.180E-02	0.148E-04	3.8421	0.0353	0.131E-02	0.174E-04
1.9010	0.0245	0.178E-02	0.129E-04	3.8686	0.0178	0.130E-02	0.244E-04
1.9226	0.0187	0.180E-02	0.150E-04	3.8865	0.0179	0.125E-02	0.239E-04
1.9414	0.0190	0.182E-02	0.150E-04	3.9045	0.0181	0.124E-02	0.238E-04
1.9605	0.0192	0.185E-02	0.150E-04	3.9226	0.0182	0.122E-02	0.234E-04
1.9799	0.0195	0.185E-02	0.150E-04	3.9408	0.0183	0.120E-02	0.231E-04
1.9996	0.0198	0.184E-02	0.150E-04	3.9592	0.0184	0.117E-02	0.229E-04
2.0195	0.0201	0.185E-02	0.151E-04	3.9777	0.0186	0.116E-02	0.227E-04
2.0398	0.0204	0.190E-02	0.150E-04	3.9961	0.0187	0.113E-02	0.224E-04
2.0604	0.0207	0.197E-02	0.156E-04	4.0151	0.0188	0.116E-02	0.225E-04
2.0813	0.0211	0.199E-02	0.150E-04	4.0240	0.0190	0.113E-02	0.224E-04
2.1025	0.0214	0.190E-02	0.154E-04	4.0530	0.0191	0.114E-02	0.224E-04
2.1204	0.0194	0.183E-02	0.145E-04	4.0772	0.0192	0.115E-02	0.224E-04
2.1386	0.0219	0.180E-02	0.150E-04	4.1013	0.0389	0.121E-02	0.162E-04
2.1607	0.0223	0.177E-02	0.150E-04	4.1305	0.0197	0.130E-02	0.238E-04
2.1831	0.0226	0.176E-02	0.151E-04	4.1503	0.0198	0.138E-02	0.245E-04
2.2021	0.0153	0.171E-02	0.182E-04	4.1701	0.0199	0.151E-02	0.255E-04
2.2213	0.0232	0.168E-02	0.148E-04	4.1901	0.0201	0.168E-02	0.269E-04
2.2408	0.0157	0.170E-02	0.183E-04	4.2103	0.0202	0.185E-02	0.283E-04
2.2605	0.0238	0.170E-02	0.151E-04	4.2306	0.0204	0.201E-02	0.295E-04
2.2805	0.0161	0.171E-02	0.186E-04	4.2510	0.0205	0.209E-02	0.299E-04
2.3008	0.0245	0.170E-02	0.151E-04	4.2716	0.0207	0.213E-02	0.302E-04
2.3213	0.0165	0.171E-02	0.187E-04	4.2924	0.0208	0.209E-02	0.300E-04
2.3421	0.0251	0.171E-02	0.153E-04	4.3133	0.0210	0.205E-02	0.297E-04
2.3632	0.0170	0.168E-02	0.186E-04	4.3343	0.0211	0.200E-02	0.294E-04
2.3803	0.0172	0.167E-02	0.187E-04	4.3555	0.0213	0.194E-02	0.287E-04
2.4019	0.0261	0.165E-02	0.152E-04	4.3769	0.0214	0.186E-02	0.281E-04
2.4238	0.0176	0.164E-02	0.185E-04	4.3984	0.0216	0.185E-02	0.281E-04
2.4415	0.0178	0.163E-02	0.187E-04	4.4201	0.0218	0.181E-02	0.276E-04
2.4594	0.0180	0.161E-02	0.186E-04	4.4419	0.0219	0.174E-02	0.271E-04
2.4776	0.0182	0.153E-02	0.183E-04	4.4639	0.0221	0.169E-02	0.270E-04
2.5006	0.0277	0.152E-02	0.152E-04	4.4861	0.0222	0.158E-02	0.259E-04
2.5238	0.0187	0.153E-02	0.191E-04	4.5084	0.0224	0.154E-02	0.255E-04
2.5427	0.0190	0.152E-02	0.189E-04	4.5309	0.0226	0.147E-02	0.248E-04
2.5617	0.0192	0.150E-02	0.189E-04	4.5536	0.0228	0.144E-02	0.244E-04
2.5810	0.0194	0.149E-02	0.191E-04	4.5764	0.0229	0.148E-02	0.238E-04
2.6005	0.0196	0.153E-02	0.196E-04	4.5994	0.0231	0.134E-02	0.235E-04
2.6202	0.0198	0.148E-02	0.192E-04	4.6226	0.0233	0.130E-02	0.232E-04
2.6402	0.0201	0.143E-02	0.191E-04	4.6460	0.0235	0.126E-02	0.227E-04
2.6603	0.0203	0.144E-02	0.198E-04	4.6695	0.0236	0.124E-02	0.225E-04
2.6808	0.0205	0.141E-02	0.205E-04	4.6933	0.0238	0.123E-02	0.224E-04
2.7014	0.0208	0.145E-02	0.219E-04	4.7172	0.0240	0.121E-02	0.219E-04
2.7223	0.0210	0.149E-02	0.230E-04	4.7412	0.0242	0.118E-02	0.216E-04
2.7434	0.0213	0.154E-02	0.232E-04	4.7655	0.0244	0.116E-02	0.213E-04
2.7648	0.0215	0.156E-02	0.235E-04	4.7900	0.0246	0.111E-02	0.208E-04
2.7810	0.0108	0.159E-02	0.239E-04	4.8144	0.0247	0.110E-02	0.207E-04
2.7973	0.0219	0.161E-02	0.239E-04	4.8395	0.0249	0.107E-02	0.204E-04
2.8193	0.0221	0.161E-02	0.235E-04	4.8645	0.0251	0.108E-02	0.204E-04
2.8416	0.0224	0.163E-02	0.234E-04	4.8897	0.0253	0.106E-02	0.202E-04
2.8641	0.0227	0.166E-02	0.234E-04	4.9152	0.0255	0.103E-02	0.200E-04
2.8812	0.0114	0.171E-02	0.235E-04	4.9408	0.0257	0.104E-02	0.201E-04
2.8985	0.0231	0.176E-02	0.240E-04	4.9666	0.0259	0.101E-02	0.198E-04
2.9217	0.0234	0.181E-02	0.241E-04	4.9926	0.0261	0.993E-03	0.194E-04
2.9452	0.0236	0.176E-02	0.238E-04	5.0189	0.0263	0.985E-03	0.195E-04

TABLE 2. CONTINUED

ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC. NEUT.	ERROR (ABS)	ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC. NEUT.	ERROR (ABS)
5.0453	0.0265	0.972E-03	0.194E-04	8.9134	0.0625	0.448E-03	0.111E-04
5.0720	0.0268	0.945E-03	0.193E-04	8.9762	0.0632	0.447E-03	0.109E-04
5.0988	0.0270	0.930E-03	0.191E-04	9.0398	0.0639	0.431E-03	0.107E-04
5.1259	0.0272	0.922E-03	0.191E-04	9.1040	0.0646	0.427E-03	0.105E-04
5.1532	0.0274	0.938E-03	0.190E-04	9.1689	0.0653	0.410E-03	0.103E-04
5.1807	0.0276	0.944E-03	0.191E-04	9.2345	0.0660	0.412E-03	0.103E-04
5.2085	0.0279	0.938E-03	0.189E-04	9.3008	0.0667	0.410E-03	0.103E-04
5.2364	0.0281	0.956E-03	0.190E-04	9.3679	0.0674	0.404E-03	0.102E-04
5.2646	0.0283	0.985E-03	0.193E-04	9.4356	0.0681	0.424E-03	0.105E-04
5.2930	0.0285	0.103E-02	0.197E-04	9.5041	0.0689	0.433E-03	0.107E-04
5.3217	0.0288	0.115E-02	0.207E-04	9.5730	0.0696	0.466E-03	0.111E-04
5.3506	0.0290	0.120E-02	0.212E-04	9.6434	0.0704	0.479E-03	0.113E-04
5.3797	0.0292	0.117E-02	0.210E-04	9.7142	0.0712	0.502E-03	0.117E-04
5.4091	0.0295	0.107E-02	0.198E-04	9.7858	0.0720	0.508E-03	0.118E-04
5.4387	0.0297	0.978E-03	0.188E-04	9.8582	0.0728	0.493E-03	0.115E-04
5.4685	0.0300	0.919E-03	0.184E-04	9.9314	0.0736	0.504E-03	0.117E-04
5.4986	0.0302	0.912E-03	0.183E-04	10.0050	0.0744	0.491E-03	0.116E-04
5.5289	0.0305	0.905E-03	0.181E-04	10.0800	0.0753	0.476E-03	0.114E-04
5.5595	0.0307	0.904E-03	0.181E-04	10.1560	0.0761	0.478E-03	0.115E-04
5.5904	0.0310	0.896E-03	0.181E-04	10.2330	0.0770	0.488E-03	0.116E-04
5.6215	0.0312	0.892E-03	0.181E-04	10.3100	0.0779	0.490E-03	0.118E-04
5.6529	0.0315	0.845E-03	0.181E-04	10.3880	0.0788	0.498E-03	0.118E-04
5.6845	0.0318	0.887E-03	0.180E-04	10.4680	0.0797	0.519E-03	0.120E-04
5.7164	0.0320	0.883E-03	0.178E-04	10.5480	0.0806	0.520E-03	0.120E-04
5.7486	0.0323	0.881E-03	0.175E-04	10.6290	0.0815	0.547E-03	0.121E-04
5.7810	0.0326	0.861E-03	0.175E-04	10.7110	0.0825	0.578E-03	0.124E-04
5.8137	0.0329	0.858E-03	0.173E-04	10.7940	0.0835	0.588E-03	0.127E-04
5.8467	0.0331	0.850E-03	0.171E-04	10.8780	0.0844	0.581E-03	0.124E-04
5.8800	0.0334	0.847E-03	0.170E-04	10.9630	0.0854	0.600E-03	0.125E-04
5.9136	0.0337	0.837E-03	0.171E-04	11.0490	0.0864	0.616E-03	0.128E-04
5.9474	0.0340	0.831E-03	0.169E-04	11.1360	0.0875	0.625E-03	0.131E-04
5.9816	0.0343	0.831E-03	0.168E-04	11.2240	0.0885	0.639E-03	0.133E-04
6.0160	0.0346	0.827E-03	0.167E-04	11.3130	0.0896	0.624E-03	0.133E-04
6.0508	0.0349	0.826E-03	0.167E-04	11.4030	0.0907	0.618E-03	0.132E-04
6.0858	0.0352	0.805E-03	0.168E-04	11.4940	0.0918	0.623E-03	0.134E-04
6.1212	0.0355	0.813E-03	0.164E-04	11.5860	0.0929	0.626E-03	0.136E-04
6.1569	0.0358	0.823E-03	0.166E-04	11.6800	0.0940	0.663E-03	0.140E-04
6.1928	0.0361	0.834E-03	0.166E-04	11.7740	0.0952	0.673E-03	0.141E-04
6.2291	0.0365	0.869E-03	0.170E-04	11.8700	0.0963	0.665E-03	0.142E-04
6.2657	0.0368	0.917E-03	0.173E-04	11.9670	0.0975	0.647E-03	0.138E-04
6.3027	0.0371	0.918E-03	0.173E-04	12.0650	0.0987	0.606E-03	0.132E-04
6.3400	0.0374	0.885E-03	0.169E-04	12.1640	0.1000	0.561E-03	0.127E-04
6.3776	0.0378	0.839E-03	0.164E-04	12.2650	0.1012	0.556E-03	0.128E-04
6.4155	0.0381	0.807E-03	0.162E-04	12.3670	0.1025	0.546E-03	0.129E-04
6.4538	0.0385	0.787E-03	0.160E-04	12.4700	0.1038	0.552E-03	0.132E-04
6.4924	0.0388	0.770E-03	0.158E-04	12.5740	0.1051	0.582E-03	0.137E-04
6.5314	0.0391	0.703E-03	0.150E-04	12.6800	0.1064	0.588E-03	0.139E-04
6.5707	0.0395	0.638E-03	0.143E-04	12.7870	0.1078	0.596E-03	0.142E-04
6.6104	0.0399	0.590E-03	0.136E-04	12.8960	0.1092	0.605E-03	0.142E-04
6.6504	0.0402	0.571E-03	0.134E-04	13.0050	0.1106	0.614E-03	0.144E-04
6.6908	0.0406	0.584E-03	0.137E-04	13.1170	0.1120	0.613E-03	0.143E-04
6.7316	0.0410	0.584E-03	0.136E-04	13.2290	0.1135	0.600E-03	0.142E-04
6.7728	0.0413	0.578E-03	0.135E-04	13.3440	0.1149	0.611E-03	0.143E-04
6.8143	0.0417	0.598E-03	0.138E-04	13.4590	0.1164	0.608E-03	0.144E-04
6.8563	0.0421	0.593E-03	0.138E-04	13.5770	0.1180	0.614E-03	0.146E-04
6.8986	0.0425	0.589E-03	0.137E-04	13.6950	0.1195	0.581E-03	0.144E-04
6.9413	0.0429	0.618E-03	0.139E-04	13.8160	0.1211	0.597E-03	0.147E-04
6.9844	0.0433	0.631E-03	0.140E-04	13.9380	0.1228	0.608E-03	0.151E-04
7.0279	0.0437	0.646E-03	0.141E-04	14.0610	0.1244	0.616E-03	0.150E-04
7.0718	0.0441	0.678E-03	0.146E-04	14.1870	0.1261	0.619E-03	0.149E-04
7.1162	0.0445	0.733E-03	0.152E-04	14.3130	0.1278	0.600E-03	0.147E-04
7.1609	0.0450	0.799E-03	0.156E-04	14.4470	0.1295	0.601E-03	0.144E-04
7.2061	0.0454	0.907E-03	0.167E-04	14.5730	0.1313	0.596E-03	0.145E-04
7.2517	0.0458	0.102E-02	0.175E-04	14.7050	0.1331	0.611E-03	0.149E-04
7.2977	0.0463	0.113E-02	0.185E-04	14.8390	0.1349	0.610E-03	0.150E-04
7.3442	0.0467	0.119E-02	0.191E-04	14.9750	0.1368	0.617E-03	0.152E-04
7.3912	0.0472	0.116E-02	0.186E-04	15.1120	0.1387	0.620E-03	0.154E-04
7.4386	0.0476	0.103E-02	0.176E-04	15.2520	0.1407	0.636E-03	0.157E-04
7.4864	0.0481	0.887E-03	0.164E-04	15.3940	0.1427	0.623E-03	0.158E-04
7.5347	0.0485	0.799E-03	0.153E-04	15.5370	0.1447	0.624E-03	0.159E-04
7.5835	0.0490	0.720E-03	0.145E-04	15.6830	0.1467	0.616E-03	0.159E-04
7.6328	0.0495	0.689E-03	0.142E-04	15.8310	0.1488	0.617E-03	0.160E-04
7.6825	0.0500	0.682E-03	0.140E-04	15.9810	0.1510	0.598E-03	0.157E-04
7.7328	0.0505	0.680E-03	0.140E-04	16.1330	0.1531	0.576E-03	0.155E-04
7.7835	0.0510	0.679E-03	0.137E-04	16.2870	0.1554	0.569E-03	0.156E-04
7.8347	0.0515	0.677E-03	0.138E-04	16.4440	0.1576	0.538E-03	0.152E-04
7.8865	0.0520	0.710E-03	0.140E-04	16.6020	0.1599	0.550E-03	0.153E-04
7.9387	0.0525	0.744E-03	0.144E-04	16.7640	0.1623	0.529E-03	0.153E-04
7.9915	0.0530	0.768E-03	0.146E-04	16.9270	0.1647	0.535E-03	0.158E-04
8.0448	0.0536	0.765E-03	0.145E-04	17.0930	0.1671	0.532E-03	0.156E-04
8.0987	0.0541	0.740E-03	0.143E-04	17.2610	0.1696	0.526E-03	0.158E-04
8.1531	0.0547	0.680E-03	0.136E-04	17.4320	0.1722	0.522E-03	0.161E-04
8.2080	0.0552	0.624E-03	0.132E-04	17.6060	0.1748	0.526E-03	0.163E-04
8.2635	0.0558	0.578E-03	0.126E-04	17.7820	0.1774	0.542E-03	0.168E-04
8.3196	0.0564	0.536E-03	0.123E-04	17.9610	0.1801	0.545E-03	0.168E-04
8.3763	0.0569	0.510E-03	0.121E-04	18.1420	0.1829	0.546E-03	0.171E-04
8.4335	0.0575	0.506E-03	0.120E-04	18.3270	0.1857	0.537E-03	0.167E-04
8.4913	0.0581	0.478E-03	0.117E-04	18.5140	0.1886	0.547E-03	0.172E-04
8.5498	0.0587	0.473E-03	0.115E-04	18.7040	0.1916	0.525E-03	0.170E-04
8.6088	0.0593	0.466E-03	0.115E-04	18.8970	0.1944	0.532E-03	0.173E-04
8.6684	0.0600	0.457E-03	0.114E-04	19.0930	0.1976	0.572E-03	0.181E-04
8.7287	0.0606	0.460E-03	0.115E-04	19.2920	0.2008	0.575E-03	0.185E-04
8.7896	0.0612	0.459E-03	0.113E-04	19.4950	0.2040	0.577E-03	0.184E-04
8.8512	0.0619	0.463E-03	0.113E-04	19.7000	0.2072	0.573E-03	0.185E-04

TABLE 3.

INTEGRAL NEUTRON COUNTING RATE AS A FUNCTION OF INCIDENT NEUTRON ENERGY
FOR AN APPROXIMATE SCATTERING ANGLE OF 90 DEGREES. AT THE LOWER ENERGIES,
THE DATA ARE AVERAGES OVER 0.020 REV INTERVALS.

ENERGY (REV)	ENERGY SPREAD (REV)	CTS/INC. NEUT.	ERROR (ABS)	ENERGY (REV)	ENERGY SPREAD (REV)	CTS/INC. NEUT.	ERROR (ABS)
1.0486	0.0200	0.360E-02	0.266E-04	3.0112	0.0244	0.158E-02	0.312E-04
1.0690	0.0206	0.359E-02	0.255E-04	3.0295	0.0123	0.154E-02	0.447E-04
1.0885	0.0185	0.348E-02	0.267E-04	3.0481	0.0249	0.144E-02	0.313E-04
1.1087	0.0218	0.341E-02	0.253E-04	3.0668	0.0125	0.137E-02	0.452E-04
1.1294	0.0196	0.345E-02	0.269E-04	3.0858	0.0253	0.130E-02	0.293E-04
1.1493	0.0201	0.342E-02	0.263E-04	3.1113	0.0256	0.123E-02	0.258E-04
1.1696	0.0207	0.345E-02	0.264E-04	3.1306	0.0129	0.120E-02	0.362E-04
1.1890	0.0181	0.346E-02	0.274E-04	3.1501	0.0261	0.114E-02	0.252E-04
1.2090	0.0217	0.341E-02	0.251E-04	3.1697	0.0132	0.111E-02	0.359E-04
1.2294	0.0191	0.337E-02	0.263E-04	3.1896	0.0266	0.108E-02	0.251E-04
1.2487	0.0195	0.331E-02	0.257E-04	3.2097	0.0134	0.106E-02	0.361E-04
1.2684	0.0200	0.329E-02	0.255E-04	3.2300	0.0271	0.102E-02	0.255E-04
1.2887	0.0205	0.320E-02	0.249E-04	3.2504	0.0137	0.101E-02	0.370E-04
1.3094	0.0210	0.319E-02	0.251E-04	3.2710	0.0276	0.100E-02	0.220E-04
1.3288	0.0179	0.320E-02	0.279E-04	3.2918	0.0140	0.999E-03	0.312E-04
1.3487	0.0219	0.318E-02	0.245E-04	3.3058	0.0140	0.985E-03	0.315E-04
1.3690	0.0187	0.314E-02	0.265E-04	3.3271	0.0284	0.101E-02	0.227E-04
1.3879	0.0191	0.304E-02	0.260E-04	3.3484	0.0143	0.101E-02	0.327E-04
1.4072	0.0195	0.306E-02	0.258E-04	3.3700	0.0289	0.102E-02	0.238E-04
1.4269	0.0199	0.299E-02	0.249E-04	3.3918	0.0146	0.103E-02	0.349E-04
1.4470	0.0203	0.299E-02	0.246E-04	3.4064	0.0147	0.105E-02	0.360E-04
1.4675	0.0207	0.297E-02	0.241E-04	3.4286	0.0297	0.105E-02	0.242E-04
1.4884	0.0212	0.291E-02	0.236E-04	3.4509	0.0150	0.106E-02	0.314E-04
1.5099	0.0216	0.285E-02	0.233E-04	3.4644	0.0151	0.106E-02	0.316E-04
1.5295	0.0177	0.276E-02	0.253E-04	3.4887	0.0305	0.107E-02	0.228E-04
1.5496	0.0225	0.271E-02	0.225E-04	3.5117	0.0154	0.108E-02	0.338E-04
1.5700	0.0184	0.258E-02	0.236E-04	3.5271	0.0155	0.109E-02	0.338E-04
1.5886	0.0187	0.250E-02	0.233E-04	3.5505	0.0313	0.111E-02	0.250E-04
1.6074	0.0190	0.248E-02	0.237E-04	3.5740	0.0158	0.112E-02	0.369E-04
1.6266	0.0194	0.249E-02	0.228E-04	3.5898	0.0159	0.114E-02	0.383E-04
1.6486	0.0247	0.248E-02	0.205E-04	3.6058	0.0160	0.113E-02	0.319E-04
1.6685	0.0151	0.251E-02	0.272E-04	3.6300	0.0323	0.118E-02	0.232E-04
1.6863	0.0204	0.255E-02	0.236E-04	3.6543	0.0163	0.119E-02	0.336E-04
1.7069	0.0208	0.257E-02	0.241E-04	3.6707	0.0164	0.120E-02	0.339E-04
1.7279	0.0212	0.259E-02	0.239E-04	3.6872	0.0166	0.121E-02	0.346E-04
1.7493	0.0216	0.260E-02	0.244E-04	3.7038	0.0167	0.123E-02	0.358E-04
1.7698	0.0165	0.259E-02	0.284E-04	3.7289	0.0337	0.124E-02	0.265E-04
1.7878	0.0223	0.252E-02	0.231E-04	3.7543	0.0168	0.122E-02	0.386E-04
1.8074	0.0170	0.246E-02	0.269E-04	3.7713	0.0171	0.123E-02	0.324E-04
1.8275	0.0231	0.239E-02	0.220E-04	3.7885	0.0172	0.124E-02	0.327E-04
1.8478	0.0176	0.234E-02	0.263E-04	3.8058	0.0174	0.126E-02	0.334E-04
1.8685	0.0238	0.229E-02	0.213E-04	3.8232	0.0175	0.125E-02	0.336E-04
1.8895	0.0182	0.224E-02	0.265E-04	3.8496	0.0353	0.128E-02	0.246E-04
1.9079	0.0185	0.226E-02	0.260E-04	3.8762	0.0178	0.126E-02	0.357E-04
1.9265	0.0187	0.224E-02	0.250E-04	3.8941	0.0180	0.129E-02	0.373E-04
1.9485	0.0254	0.226E-02	0.217E-04	3.9121	0.0181	0.127E-02	0.382E-04
1.9709	0.0194	0.230E-02	0.257E-04	3.9303	0.0182	0.127E-02	0.395E-04
1.9904	0.0197	0.227E-02	0.250E-04	3.9486	0.0183	0.128E-02	0.228E-04
2.0102	0.0200	0.228E-02	0.253E-04	3.9670	0.0185	0.126E-02	0.324E-04
2.0303	0.0203	0.229E-02	0.254E-04	3.9855	0.0186	0.128E-02	0.331E-04
2.0508	0.0206	0.228E-02	0.254E-04	4.0042	0.0187	0.127E-02	0.337E-04
2.0715	0.0209	0.204E-02	0.239E-04	4.0230	0.0189	0.129E-02	0.343E-04
2.0890	0.0141	0.196E-02	0.207E-04	4.0419	0.0190	0.130E-02	0.356E-04
2.1067	0.0214	0.205E-02	0.241E-04	4.0610	0.0191	0.133E-02	0.377E-04
2.1283	0.0217	0.203E-02	0.244E-04	4.0802	0.0193	0.136E-02	0.394E-04
2.1502	0.0221	0.201E-02	0.238E-04	4.0995	0.0194	0.140E-02	0.417E-04
2.1687	0.0143	0.203E-02	0.249E-04	4.1190	0.0195	0.145E-02	0.346E-04
2.1875	0.0227	0.200E-02	0.245E-04	4.1386	0.0395	0.156E-02	0.267E-04
2.2104	0.0230	0.199E-02	0.243E-04	4.1783	0.0200	0.170E-02	0.407E-04
2.2296	0.0155	0.197E-02	0.302E-04	4.1983	0.0201	0.181E-02	0.437E-04
2.2492	0.0236	0.197E-02	0.256E-04	4.2185	0.0202	0.190E-02	0.472E-04
2.2690	0.0260	0.197E-02	0.305E-04	4.2389	0.0204	0.197E-02	0.505E-04
2.2891	0.0243	0.196E-02	0.257E-04	4.2594	0.0206	0.197E-02	0.531E-04
2.3095	0.0164	0.190E-02	0.301E-04	4.2800	0.0207	0.194E-02	0.417E-04
2.3301	0.0249	0.192E-02	0.242E-04	4.3008	0.0209	0.189E-02	0.411E-04
2.3510	0.0168	0.185E-02	0.300E-04	4.3217	0.0210	0.183E-02	0.410E-04
2.3679	0.0170	0.182E-02	0.287E-04	4.3428	0.0212	0.180E-02	0.417E-04
2.3894	0.0259	0.179E-02	0.253E-04	4.3640	0.0213	0.174E-02	0.423E-04
2.4110	0.0175	0.175E-02	0.295E-04	4.3855	0.0215	0.171E-02	0.427E-04
2.4286	0.0177	0.173E-02	0.282E-04	4.4070	0.0216	0.169E-02	0.448E-04
2.4464	0.0179	0.173E-02	0.287E-04	4.4287	0.0218	0.166E-02	0.465E-04
2.4689	0.0272	0.168E-02	0.248E-04	4.4506	0.0220	0.163E-02	0.370E-04
2.4917	0.0184	0.162E-02	0.297E-04	4.4727	0.0221	0.159E-02	0.367E-04
2.5102	0.0186	0.164E-02	0.291E-04	4.4949	0.0223	0.155E-02	0.366E-04
2.5289	0.0188	0.164E-02	0.291E-04	4.5172	0.0225	0.154E-02	0.373E-04
2.5477	0.0190	0.163E-02	0.295E-04	4.5398	0.0226	0.149E-02	0.377E-04
2.5668	0.0192	0.162E-02	0.293E-04	4.5625	0.0228	0.147E-02	0.387E-04
2.5862	0.0194	0.162E-02	0.294E-04	4.5854	0.0230	0.147E-02	0.400E-04
2.6057	0.0196	0.162E-02	0.294E-04	4.6084	0.0231	0.143E-02	0.413E-04
2.6255	0.0199	0.155E-02	0.285E-04	4.6317	0.0233	0.140E-02	0.330E-04
2.6454	0.0201	0.151E-02	0.285E-04	4.6551	0.0235	0.139E-02	0.334E-04
2.6657	0.0203	0.149E-02	0.296E-04	4.6787	0.0237	0.138E-02	0.337E-04
2.6861	0.0206	0.146E-02	0.307E-04	4.7024	0.0239	0.138E-02	0.344E-04
2.7068	0.0208	0.147E-02	0.328E-04	4.7264	0.0240	0.134E-02	0.349E-04
2.7277	0.0210	0.149E-02	0.337E-04	4.7505	0.0242	0.133E-02	0.363E-04
2.7489	0.0213	0.148E-02	0.339E-04	4.7748	0.0244	0.130E-02	0.374E-04
2.7703	0.0215	0.146E-02	0.340E-04	4.7993	0.0246	0.128E-02	0.306E-04
2.7920	0.0218	0.150E-02	0.354E-04	4.8240	0.0248	0.125E-02	0.304E-04
2.8084	0.0110	0.148E-02	0.535E-04	4.8489	0.0250	0.124E-02	0.311E-04
2.8250	0.0222	0.143E-02	0.314E-04	4.8740	0.0252	0.125E-02	0.320E-04
2.8471	0.0224	0.134E-02	0.303E-04	4.8993	0.0254	0.122E-02	0.327E-04
2.8698	0.0227	0.127E-02	0.297E-04	4.9247	0.0256	0.120E-02	0.340E-04
2.8927	0.0230	0.125E-02	0.306E-04	4.9504	0.0258	0.119E-02	0.352E-04
2.9100	0.0116	0.122E-02	0.461E-04	4.9763	0.0260	0.115E-02	0.283E-04
2.9275	0.0234	0.121E-02	0.276E-04	5.0024	0.0262	0.111E-02	0.283E-04
2.9510	0.0237	0.132E-02	0.303E-04	5.0286	0.0264	0.109E-02	0.287E-04
2.9689	0.0120	0.147E-02	0.516E-04	5.0551	0.0266	0.110E-02	0.299E-04
2.9869	0.0241	0.157E-02	0.321E-04	5.0818	0.0268	0.107E-02	0.307E-04

TABLE 3. CONTINUED

ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC. NEUT.	ERROR (ABS)	ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC. NEUT.	ERROR (ABS)
5.1088	0.0270	0.106E-02	0.320E-04	8.9931	0.0633	0.562E-03	0.170E-04
5.1359	0.0272	0.106E-02	0.272E-04	9.0568	0.0640	0.554E-03	0.154E-04
5.1632	0.0275	0.107E-02	0.273E-04	9.1211	0.0647	0.552E-03	0.158E-04
5.1908	0.0277	0.109E-02	0.283E-04	9.1861	0.0654	0.538E-03	0.167E-04
5.2186	0.0279	0.108E-02	0.289E-04	9.2519	0.0661	0.522E-03	0.149E-04
5.2466	0.0281	0.109E-02	0.305E-04	9.3183	0.0668	0.510E-03	0.157E-04
5.2748	0.0284	0.112E-02	0.323E-04	9.3855	0.0675	0.501E-03	0.145E-04
5.3033	0.0286	0.115E-02	0.278E-04	9.4533	0.0683	0.494E-03	0.148E-04
5.3320	0.0288	0.119E-02	0.289E-04	9.5220	0.0690	0.485E-03	0.159E-04
5.3610	0.0291	0.119E-02	0.295E-04	9.5914	0.0698	0.497E-03	0.148E-04
5.3901	0.0293	0.116E-02	0.300E-04	9.6615	0.0705	0.507E-03	0.159E-04
5.4195	0.0295	0.111E-02	0.304E-04	9.7324	0.0713	0.522E-03	0.151E-04
5.4492	0.0298	0.106E-02	0.311E-04	9.8041	0.0721	0.526E-03	0.158E-04
5.4791	0.0300	0.105E-02	0.263E-04	9.8767	0.0729	0.519E-03	0.173E-04
5.5092	0.0303	0.105E-02	0.266E-04	9.9500	0.0737	0.501E-03	0.153E-04
5.5396	0.0305	0.105E-02	0.274E-04	10.0240	0.0746	0.506E-03	0.164E-04
5.5703	0.0308	0.103E-02	0.281E-04	10.0990	0.0754	0.490E-03	0.150E-04
5.6012	0.0310	0.103E-02	0.293E-04	10.1750	0.0763	0.507E-03	0.162E-04
5.6324	0.0313	0.104E-02	0.313E-04	10.2520	0.0771	0.508E-03	0.153E-04
5.6638	0.0316	0.104E-02	0.259E-04	10.3290	0.0780	0.522E-03	0.163E-04
5.6955	0.0318	0.102E-02	0.263E-04	10.4080	0.0789	0.525E-03	0.158E-04
5.7274	0.0321	0.102E-02	0.270E-04	10.4870	0.0798	0.532E-03	0.163E-04
5.7597	0.0324	0.103E-02	0.283E-04	10.5670	0.0807	0.527E-03	0.153E-04
5.7922	0.0326	0.101E-02	0.297E-04	10.6490	0.0817	0.553E-03	0.163E-04
5.8250	0.0329	0.100E-02	0.249E-04	10.7310	0.0826	0.559E-03	0.180E-04
5.8580	0.0332	0.990E-03	0.253E-04	10.8140	0.0836	0.568E-03	0.165E-04
5.8914	0.0335	0.985E-03	0.259E-04	10.8980	0.0846	0.564E-03	0.179E-04
5.9250	0.0338	0.981E-03	0.270E-04	10.9830	0.0856	0.578E-03	0.166E-04
5.9589	0.0341	0.980E-03	0.285E-04	11.0690	0.0866	0.567E-03	0.183E-04
5.9931	0.0344	0.974E-03	0.241E-04	11.1560	0.0876	0.588E-03	0.171E-04
6.0276	0.0347	0.971E-03	0.247E-04	11.2440	0.0887	0.602E-03	0.193E-04
6.0624	0.0350	0.975E-03	0.255E-04	11.3340	0.0897	0.598E-03	0.175E-04
6.0976	0.0353	0.961E-03	0.266E-04	11.4240	0.0908	0.597E-03	0.170E-04
6.1330	0.0356	0.982E-03	0.284E-04	11.5150	0.0919	0.608E-03	0.183E-04
6.1687	0.0359	0.105E-02	0.252E-04	11.6080	0.0930	0.636E-03	0.177E-04
6.2047	0.0362	0.113E-02	0.272E-04	11.7010	0.0942	0.671E-03	0.200E-04
6.2411	0.0365	0.125E-02	0.302E-04	11.7960	0.0953	0.696E-03	0.191E-04
6.2778	0.0368	0.144E-02	0.356E-04	11.8920	0.0965	0.704E-03	0.211E-04
6.3148	0.0372	0.147E-02	0.392E-04	11.9890	0.0977	0.680E-03	0.189E-04
6.3521	0.0375	0.133E-02	0.294E-04	12.0870	0.0989	0.651E-03	0.206E-04
6.3898	0.0378	0.114E-02	0.273E-04	12.1870	0.1001	0.633E-03	0.186E-04
6.4278	0.0382	0.996E-03	0.263E-04	12.2870	0.1014	0.632E-03	0.178E-04
6.4662	0.0385	0.930E-03	0.268E-04	12.3900	0.1026	0.641E-03	0.201E-04
6.5049	0.0389	0.867E-03	0.222E-04	12.4930	0.1039	0.648E-03	0.191E-04
6.5439	0.0392	0.800E-03	0.213E-04	12.5970	0.1053	0.657E-03	0.221E-04
6.5833	0.0396	0.773E-03	0.218E-04	12.7030	0.1066	0.664E-03	0.206E-04
6.6231	0.0399	0.804E-03	0.233E-04	12.8110	0.1080	0.677E-03	0.196E-04
6.6632	0.0403	0.846E-03	0.216E-04	12.9190	0.1093	0.668E-03	0.218E-04
6.7037	0.0407	0.867E-03	0.224E-04	13.0290	0.1108	0.667E-03	0.203E-04
6.7445	0.0410	0.868E-03	0.233E-04	13.1410	0.1122	0.661E-03	0.194E-04
6.7858	0.0414	0.871E-03	0.246E-04	13.2540	0.1136	0.644E-03	0.212E-04
6.8274	0.0418	0.842E-03	0.214E-04	13.3680	0.1151	0.636E-03	0.198E-04
6.8694	0.0422	0.820E-03	0.216E-04	13.4840	0.1166	0.628E-03	0.191E-04
6.9118	0.0426	0.808E-03	0.219E-04	13.6010	0.1182	0.607E-03	0.221E-04
6.9546	0.0430	0.795E-03	0.228E-04	13.7200	0.1197	0.635E-03	0.206E-04
6.9977	0.0434	0.793E-03	0.206E-04	13.8410	0.1213	0.650E-03	0.201E-04
7.0413	0.0438	0.781E-03	0.209E-04	13.9630	0.1230	0.646E-03	0.233E-04
7.0853	0.0442	0.785E-03	0.216E-04	14.0870	0.1246	0.633E-03	0.214E-04
7.1297	0.0446	0.782E-03	0.228E-04	14.2120	0.1263	0.629E-03	0.201E-04
7.1746	0.0450	0.809E-03	0.208E-04	14.3400	0.1280	0.618E-03	0.193E-04
7.2198	0.0455	0.843E-03	0.216E-04	14.4680	0.1297	0.601E-03	0.215E-04
7.2655	0.0459	0.886E-03	0.235E-04	14.5990	0.1315	0.603E-03	0.205E-04
7.3117	0.0463	0.917E-03	0.257E-04	14.7310	0.1333	0.599E-03	0.198E-04
7.3582	0.0468	0.928E-03	0.226E-04	14.8660	0.1352	0.600E-03	0.195E-04
7.4053	0.0472	0.909E-03	0.228E-04	15.0020	0.1370	0.601E-03	0.195E-04
7.4527	0.0477	0.879E-03	0.237E-04	15.1400	0.1390	0.615E-03	0.226E-04
7.5007	0.0482	0.861E-03	0.253E-04	15.2800	0.1409	0.641E-03	0.226E-04
7.5491	0.0486	0.876E-03	0.218E-04	15.4220	0.1429	0.628E-03	0.217E-04
7.5979	0.0491	0.890E-03	0.228E-04	15.5660	0.1449	0.609E-03	0.206E-04
7.6473	0.0496	0.939E-03	0.254E-04	15.7120	0.1470	0.623E-03	0.209E-04
7.6971	0.0501	0.983E-03	0.229E-04	15.8600	0.1491	0.613E-03	0.208E-04
7.7474	0.0506	0.971E-03	0.234E-04	16.0100	0.1512	0.573E-03	0.196E-04
7.7983	0.0511	0.918E-03	0.241E-04	16.1620	0.1534	0.573E-03	0.225E-04
7.8496	0.0516	0.835E-03	0.202E-04	16.3160	0.1556	0.564E-03	0.218E-04
7.9014	0.0521	0.805E-03	0.205E-04	16.4730	0.1579	0.564E-03	0.216E-04
7.9538	0.0526	0.830E-03	0.220E-04	16.6320	0.1602	0.544E-03	0.210E-04
8.0067	0.0531	0.868E-03	0.245E-04	16.7940	0.1625	0.541E-03	0.210E-04
8.0601	0.0537	0.865E-03	0.211E-04	16.9570	0.1650	0.540E-03	0.211E-04
8.1140	0.0542	0.846E-03	0.218E-04	17.1240	0.1674	0.529E-03	0.213E-04
8.1685	0.0548	0.764E-03	0.220E-04	17.2920	0.1699	0.527E-03	0.210E-04
8.2236	0.0553	0.711E-03	0.186E-04	17.4630	0.1725	0.538E-03	0.219E-04
8.2792	0.0559	0.673E-03	0.189E-04	17.6370	0.1751	0.531E-03	0.220E-04
8.3353	0.0565	0.648E-03	0.198E-04	17.8140	0.1777	0.540E-03	0.225E-04
8.3921	0.0570	0.626E-03	0.175E-04	17.9930	0.1804	0.566E-03	0.237E-04
8.4494	0.0576	0.613E-03	0.179E-04	18.1750	0.1832	0.555E-03	0.238E-04
8.5074	0.0582	0.611E-03	0.191E-04	18.3590	0.1860	0.547E-03	0.240E-04
8.5659	0.0588	0.592E-03	0.168E-04	18.5470	0.1889	0.570E-03	0.256E-04
8.6250	0.0594	0.589E-03	0.175E-04	18.7370	0.1919	0.569E-03	0.224E-04
8.6848	0.0601	0.588E-03	0.188E-04	18.9300	0.1949	0.589E-03	0.237E-04
8.7452	0.0607	0.583E-03	0.167E-04	19.1270	0.1979	0.596E-03	0.242E-04
8.8062	0.0613	0.584E-03	0.174E-04	19.3260	0.2011	0.585E-03	0.248E-04
8.8679	0.0620	0.576E-03	0.161E-04	19.5290	0.2043	0.574E-03	0.256E-04
8.9302	0.0626	0.571E-03	0.163E-04	19.7350	0.2076	0.583E-03	0.275E-04

TABLE 4.

INTEGRAL NEUTRON COUNTING RATE AS A FUNCTION OF INCIDENT NEUTRON ENERGY
FOR AN APPROXIMATE SCATTERING ANGLE OF 135 DEGREES. AT THE LOWER ENERGIES,
THE DATA ARE AVERAGES OVER 0.020 MEV INTERVALS.

ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC.NEUT.	ERROR (ABS)	ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC.NEUT.	ERROR (ABS)
1.0164	0.0190	0.150E-02	0.242E-04	2.9754	0.0119	0.123E-02	0.510E-04
1.0369	0.0220	0.151E-02	0.223E-04	2.9934	0.0240	0.957E-03	0.319E-04
1.0580	0.0202	0.151E-02	0.226E-04	3.0176	0.0243	0.961E-03	0.316E-04
1.0771	0.0181	0.152E-02	0.236E-04	3.0359	0.0123	0.109E-02	0.475E-04
1.0969	0.0213	0.149E-02	0.218E-04	3.0544	0.0248	0.120E-02	0.344E-04
1.1171	0.0192	0.152E-02	0.237E-04	3.0794	0.0251	0.130E-02	0.358E-04
1.1365	0.0197	0.155E-02	0.233E-04	3.0983	0.0127	0.140E-02	0.513E-04
1.1564	0.0202	0.153E-02	0.228E-04	3.1174	0.0256	0.146E-02	0.369E-04
1.1768	0.0207	0.156E-02	0.228E-04	3.1366	0.0129	0.152E-02	0.531E-04
1.1963	0.0182	0.156E-02	0.240E-04	3.1561	0.0260	0.159E-02	0.381E-04
1.2163	0.0218	0.153E-02	0.216E-04	3.1756	0.0131	0.161E-02	0.542E-04
1.2367	0.0191	0.152E-02	0.230E-04	3.1955	0.0265	0.164E-02	0.386E-04
1.2561	0.0196	0.150E-02	0.228E-04	3.2154	0.0134	0.167E-02	0.550E-04
1.2759	0.0200	0.149E-02	0.228E-04	3.2356	0.0270	0.178E-02	0.392E-04
1.2962	0.0205	0.144E-02	0.221E-04	3.2560	0.0136	0.178E-02	0.554E-04
1.3170	0.0210	0.143E-02	0.219E-04	3.2766	0.0275	0.181E-02	0.397E-04
1.3364	0.0119	0.141E-02	0.230E-04	3.2972	0.0134	0.184E-02	0.555E-04
1.3564	0.0220	0.143E-02	0.212E-04	3.3183	0.0281	0.187E-02	0.400E-04
1.3767	0.0187	0.137E-02	0.222E-04	3.3394	0.0142	0.189E-02	0.559E-04
1.3956	0.0191	0.135E-02	0.220E-04	3.3536	0.0143	0.189E-02	0.557E-04
1.4169	0.0235	0.135E-02	0.198E-04	3.3751	0.0288	0.192E-02	0.400E-04
1.4387	0.0200	0.134E-02	0.211E-04	3.3968	0.0145	0.190E-02	0.555E-04
1.4568	0.0163	0.137E-02	0.234E-04	3.4188	0.0294	0.194E-02	0.397E-04
1.4758	0.0208	0.135E-02	0.208E-04	3.4408	0.0146	0.193E-02	0.560E-04
1.4963	0.0212	0.133E-02	0.202E-04	3.4557	0.0149	0.194E-02	0.563E-04
1.5178	0.0217	0.131E-02	0.198E-04	3.4782	0.0301	0.194E-02	0.396E-04
1.5375	0.0177	0.130E-02	0.221E-04	3.5009	0.0152	0.194E-02	0.559E-04
1.5576	0.0225	0.127E-02	0.192E-04	3.5162	0.0153	0.193E-02	0.549E-04
1.5780	0.0184	0.123E-02	0.209E-04	3.5315	0.0154	0.188E-02	0.550E-04
1.5966	0.0187	0.123E-02	0.211E-04	3.5488	0.0311	0.189E-02	0.386E-04
1.6155	0.0190	0.124E-02	0.210E-04	3.5782	0.0157	0.182E-02	0.536E-04
1.6371	0.0243	0.121E-02	0.182E-04	3.5940	0.0158	0.184E-02	0.535E-04
1.6592	0.0198	0.121E-02	0.202E-04	3.6179	0.0320	0.182E-02	0.376E-04
1.6792	0.0202	0.119E-02	0.201E-04	3.6420	0.0161	0.180E-02	0.526E-04
1.6970	0.0154	0.118E-02	0.231E-04	3.6582	0.0163	0.175E-02	0.520E-04
1.7151	0.0208	0.117E-02	0.202E-04	3.6745	0.0164	0.173E-02	0.514E-04
1.7361	0.0212	0.117E-02	0.203E-04	3.6909	0.0165	0.171E-02	0.506E-04
1.7575	0.0216	0.117E-02	0.204E-04	3.7157	0.0333	0.163E-02	0.349E-04
1.7766	0.0165	0.116E-02	0.232E-04	3.7408	0.0168	0.157E-02	0.486E-04
1.7960	0.0223	0.115E-02	0.199E-04	3.7576	0.0169	0.155E-02	0.486E-04
1.8157	0.0170	0.114E-02	0.225E-04	3.7746	0.0170	0.153E-02	0.474E-04
1.8357	0.0231	0.114E-02	0.194E-04	3.7917	0.0172	0.149E-02	0.473E-04
1.8560	0.0176	0.114E-02	0.224E-04	3.8176	0.0347	0.146E-02	0.329E-04
1.8768	0.0239	0.114E-02	0.193E-04	3.8437	0.0175	0.142E-02	0.451E-04
1.8978	0.0182	0.114E-02	0.226E-04	3.8613	0.0176	0.140E-02	0.446E-04
1.9161	0.0185	0.117E-02	0.227E-04	3.8789	0.0177	0.136E-02	0.445E-04
1.9347	0.0191	0.117E-02	0.227E-04	3.8968	0.0178	0.136E-02	0.441E-04
1.9567	0.0254	0.119E-02	0.194E-04	3.9167	0.0180	0.133E-02	0.436E-04
1.9791	0.0194	0.119E-02	0.228E-04	3.9327	0.0181	0.131E-02	0.434E-04
1.9987	0.0197	0.117E-02	0.225E-04	3.9509	0.0182	0.132E-02	0.444E-04
2.0185	0.0200	0.118E-02	0.227E-04	3.9692	0.0184	0.132E-02	0.434E-04
2.0386	0.0203	0.127E-02	0.234E-04	3.9970	0.0371	0.132E-02	0.305E-04
2.0590	0.0206	0.123E-02	0.215E-04	4.0244	0.0184	0.131E-02	0.432E-04
2.0797	0.0209	0.185E-02	0.280E-04	4.0438	0.0189	0.131E-02	0.425E-04
2.0972	0.0141	0.144E-02	0.306E-04	4.0627	0.0190	0.128E-02	0.423E-04
2.1149	0.0214	0.129E-02	0.237E-04	4.0818	0.0192	0.126E-02	0.422E-04
2.1385	0.0217	0.123E-02	0.232E-04	4.1014	0.0191	0.122E-02	0.407E-04
2.1584	0.0221	0.122E-02	0.233E-04	4.1204	0.0194	0.117E-02	0.401E-04
2.1769	0.0189	0.120E-02	0.229E-04	4.1399	0.0196	0.118E-02	0.393E-04
2.1956	0.0226	0.121E-02	0.235E-04	4.1595	0.0197	0.105E-02	0.380E-04
2.2185	0.0230	0.120E-02	0.233E-04	4.1793	0.0199	0.961E-03	0.365E-04
2.2377	0.0155	0.119E-02	0.266E-04	4.1993	0.0200	0.868E-03	0.348E-04
2.2573	0.0236	0.120E-02	0.233E-04	4.2193	0.0201	0.747E-03	0.332E-04
2.2771	0.0159	0.119E-02	0.289E-04	4.2395	0.0203	0.663E-03	0.309E-04
2.2972	0.0242	0.120E-02	0.236E-04	4.2599	0.0204	0.597E-03	0.296E-04
2.3175	0.0164	0.118E-02	0.288E-04	4.2804	0.0206	0.534E-03	0.282E-04
2.3381	0.0249	0.119E-02	0.235E-04	4.3010	0.0207	0.501E-03	0.273E-04
2.3589	0.0188	0.116E-02	0.207E-04	4.3219	0.0209	0.481E-03	0.270E-04
2.3758	0.0170	0.115E-02	0.283E-04	4.3428	0.0210	0.473E-03	0.268E-04
2.3973	0.0258	0.116E-02	0.233E-04	4.3639	0.0212	0.471E-03	0.265E-04
2.4189	0.0175	0.116E-02	0.289E-04	4.3852	0.0213	0.476E-03	0.269E-04
2.4365	0.0176	0.116E-02	0.289E-04	4.4066	0.0215	0.489E-03	0.273E-04
2.4542	0.0178	0.117E-02	0.294E-04	4.4282	0.0217	0.507E-03	0.272E-04
2.4767	0.0271	0.114E-02	0.238E-04	4.4499	0.0218	0.500E-03	0.274E-04
2.4994	0.0183	0.115E-02	0.299E-04	4.4718	0.0220	0.519E-03	0.277E-04
2.5179	0.0185	0.118E-02	0.308E-04	4.4939	0.0221	0.486E-03	0.267E-04
2.5365	0.0187	0.118E-02	0.309E-04	4.5161	0.0223	0.495E-03	0.267E-04
2.5554	0.0190	0.121E-02	0.313E-04	4.5385	0.0225	0.502E-03	0.267E-04
2.5744	0.0147	0.122E-02	0.317E-04	4.5610	0.0226	0.493E-03	0.260E-04
2.5937	0.0194	0.126E-02	0.330E-04	4.5838	0.0228	0.476E-03	0.237E-04
2.6132	0.0196	0.127E-02	0.326E-04	4.6067	0.0230	0.489E-03	0.257E-04
2.6329	0.0198	0.125E-02	0.327E-04	4.6297	0.0232	0.471E-03	0.255E-04
2.6529	0.0201	0.123E-02	0.333E-04	4.6530	0.0233	0.488E-03	0.261E-04
2.6730	0.0203	0.125E-02	0.351E-04	4.6764	0.0235	0.482E-03	0.255E-04
2.6934	0.0205	0.128E-02	0.370E-04	4.7000	0.0237	0.468E-03	0.256E-04
2.7141	0.0208	0.136E-02	0.397E-04	4.7238	0.0239	0.470E-03	0.253E-04
2.7350	0.0210	0.142E-02	0.409E-04	4.7477	0.0240	0.469E-03	0.251E-04
2.7561	0.0212	0.144E-02	0.411E-04	4.7719	0.0242	0.465E-03	0.251E-04
2.7774	0.0215	0.151E-02	0.421E-04	4.7962	0.0244	0.455E-03	0.242E-04
2.7990	0.0217	0.162E-02	0.432E-04	4.8207	0.0246	0.462E-03	0.247E-04
2.8209	0.0220	0.177E-02	0.448E-04	4.8454	0.0248	0.442E-03	0.245E-04
2.8375	0.0111	0.186E-02	0.648E-04	4.8703	0.0250	0.450E-03	0.243E-04
2.8542	0.0224	0.191E-02	0.458E-04	4.8954	0.0252	0.448E-03	0.242E-04
2.8767	0.0227	0.201E-02	0.466E-04	4.9207	0.0254	0.443E-03	0.242E-04
2.8995	0.0229	0.216E-02	0.485E-04	4.9461	0.0256	0.438E-03	0.244E-04
2.9167	0.0116	0.219E-02	0.676E-04	4.9718	0.0258	0.432E-03	0.236E-04
2.9342	0.0233	0.210E-02	0.464E-04	4.9977	0.0260	0.439E-03	0.239E-04
2.9577	0.0236	0.168E-02	0.420E-04	5.0238	0.0262	0.419E-03	0.236E-04

TABLE 4. CONTINUED

ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC. NEUT.	ERROR (ABS)	ENERGY (MEV)	ENERGY SPREAD (MEV)	CTS/INC. NEUT.	ERROR (ABS)
5.0501	0.0264	0.418E-03	0.236E-04	8.9429	0.0624	0.379E-03	0.186E-04
5.0765	0.0266	0.401E-03	0.230E-04	9.0056	0.0630	0.383E-03	0.182E-04
5.1032	0.0268	0.405E-03	0.229E-04	9.0689	0.0637	0.382E-03	0.180E-04
5.1302	0.0270	0.406E-03	0.231E-04	9.1330	0.0644	0.366E-03	0.176E-04
5.1573	0.0272	0.402E-03	0.233E-04	9.1977	0.0651	0.360E-03	0.173E-04
5.1846	0.0275	0.397E-03	0.230E-04	9.2631	0.0658	0.343E-03	0.171E-04
5.2122	0.0277	0.379E-03	0.226E-04	9.3293	0.0665	0.325E-03	0.168E-04
5.2400	0.0279	0.389E-03	0.225E-04	9.3961	0.0672	0.323E-03	0.166E-04
5.2680	0.0281	0.369E-03	0.217E-04	9.4637	0.0679	0.306E-03	0.162E-04
5.2962	0.0283	0.359E-03	0.219E-04	9.5320	0.0687	0.293E-03	0.159E-04
5.3247	0.0286	0.356E-03	0.212E-04	9.6010	0.0694	0.286E-03	0.159E-04
5.3534	0.0288	0.391E-03	0.223E-04	9.6708	0.0702	0.268E-03	0.159E-04
5.3823	0.0290	0.422E-03	0.229E-04	9.7414	0.0710	0.250E-03	0.153E-04
5.4115	0.0293	0.432E-03	0.234E-04	9.8128	0.0717	0.238E-03	0.149E-04
5.4409	0.0295	0.437E-03	0.231E-04	9.8849	0.0725	0.227E-03	0.149E-04
5.4705	0.0298	0.407E-03	0.227E-04	9.9578	0.0734	0.218E-03	0.143E-04
5.5004	0.0300	0.408E-03	0.230E-04	10.0320	0.0742	0.230E-03	0.149E-04
5.5305	0.0303	0.411E-03	0.227E-04	10.1060	0.0750	0.235E-03	0.148E-04
5.5609	0.0305	0.404E-03	0.225E-04	10.1820	0.0758	0.240E-03	0.152E-04
5.5915	0.0308	0.409E-03	0.224E-04	10.2580	0.0767	0.255E-03	0.158E-04
5.6224	0.0310	0.408E-03	0.222E-04	10.3350	0.0776	0.270E-03	0.163E-04
5.6535	0.0313	0.406E-03	0.220E-04	10.4130	0.0785	0.284E-03	0.168E-04
5.6849	0.0315	0.408E-03	0.226E-04	10.4920	0.0794	0.292E-03	0.167E-04
5.7166	0.0318	0.414E-03	0.225E-04	10.5720	0.0803	0.296E-03	0.167E-04
5.7485	0.0321	0.416E-03	0.226E-04	10.6530	0.0812	0.290E-03	0.163E-04
5.7807	0.0323	0.414E-03	0.223E-04	10.7340	0.0821	0.281E-03	0.161E-04
5.8132	0.0326	0.419E-03	0.223E-04	10.8170	0.0831	0.270E-03	0.156E-04
5.8460	0.0329	0.432E-03	0.229E-04	10.9000	0.0841	0.259E-03	0.156E-04
5.8790	0.0332	0.437E-03	0.221E-04	10.9850	0.0851	0.244E-03	0.151E-04
5.9123	0.0338	0.444E-03	0.226E-04	11.0710	0.0861	0.242E-03	0.149E-04
5.9459	0.0337	0.434E-03	0.224E-04	11.1570	0.0871	0.246E-03	0.151E-04
5.9798	0.0340	0.451E-03	0.227E-04	11.2450	0.0881	0.241E-03	0.151E-04
6.0139	0.0343	0.460E-03	0.230E-04	11.3330	0.0892	0.245E-03	0.157E-04
6.0484	0.0346	0.459E-03	0.229E-04	11.4230	0.0902	0.251E-03	0.157E-04
6.0832	0.0349	0.471E-03	0.232E-04	11.5140	0.0913	0.263E-03	0.161E-04
6.1182	0.0352	0.484E-03	0.231E-04	11.6060	0.0924	0.269E-03	0.161E-04
6.1536	0.0355	0.520E-03	0.238E-04	11.6990	0.0935	0.266E-03	0.164E-04
6.1893	0.0358	0.579E-03	0.252E-04	11.7930	0.0947	0.275E-03	0.164E-04
6.2253	0.0361	0.676E-03	0.271E-04	11.8880	0.0958	0.265E-03	0.161E-04
6.2616	0.0365	0.728E-03	0.279E-04	11.9840	0.0970	0.252E-03	0.159E-04
6.2982	0.0368	0.718E-03	0.276E-04	12.0820	0.0982	0.255E-03	0.155E-04
6.3352	0.0371	0.632E-03	0.262E-04	12.1810	0.0994	0.255E-03	0.160E-04
6.3725	0.0374	0.515E-03	0.238E-04	12.2810	0.1006	0.257E-03	0.161E-04
6.4101	0.0378	0.458E-03	0.225E-04	12.3820	0.1019	0.262E-03	0.166E-04
6.4480	0.0381	0.414E-03	0.212E-04	12.4850	0.1032	0.275E-03	0.172E-04
6.4863	0.0385	0.384E-03	0.205E-04	12.5880	0.1045	0.268E-03	0.170E-04
6.5249	0.0388	0.341E-03	0.195E-04	12.6940	0.1058	0.281E-03	0.176E-04
6.5639	0.0391	0.297E-03	0.182E-04	12.8000	0.1071	0.271E-03	0.176E-04
6.6032	0.0395	0.296E-03	0.180E-04	12.9080	0.1085	0.268E-03	0.175E-04
6.6429	0.0399	0.297E-03	0.179E-04	13.0170	0.1099	0.273E-03	0.174E-04
6.6830	0.0402	0.309E-03	0.185E-04	13.1280	0.1113	0.259E-03	0.170E-04
6.7234	0.0406	0.315E-03	0.186E-04	13.2400	0.1127	0.247E-03	0.167E-04
6.7641	0.0410	0.311E-03	0.185E-04	13.3530	0.1142	0.246E-03	0.164E-04
6.8053	0.0413	0.309E-03	0.185E-04	13.4680	0.1157	0.238E-03	0.169E-04
6.8468	0.0417	0.312E-03	0.183E-04	13.5850	0.1172	0.240E-03	0.169E-04
6.8887	0.0421	0.302E-03	0.179E-04	13.7030	0.1187	0.249E-03	0.174E-04
6.9310	0.0425	0.312E-03	0.182E-04	13.8220	0.1203	0.251E-03	0.176E-04
6.9737	0.0429	0.301E-03	0.181E-04	13.9430	0.1219	0.264E-03	0.184E-04
7.0168	0.0433	0.295E-03	0.177E-04	14.0660	0.1235	0.269E-03	0.182E-04
7.0603	0.0437	0.293E-03	0.176E-04	14.1900	0.1252	0.274E-03	0.180E-04
7.1042	0.0441	0.291E-03	0.175E-04	14.3160	0.1269	0.252E-03	0.178E-04
7.1485	0.0445	0.286E-03	0.174E-04	14.4440	0.1286	0.237E-03	0.169E-04
7.1932	0.0449	0.298E-03	0.178E-04	14.5740	0.1303	0.221E-03	0.165E-04
7.2384	0.0454	0.307E-03	0.183E-04	14.7050	0.1321	0.211E-03	0.164E-04
7.2839	0.0458	0.353E-03	0.188E-04	14.8380	0.1339	0.210E-03	0.165E-04
7.3299	0.0462	0.429E-03	0.207E-04	14.9730	0.1358	0.208E-03	0.170E-04
7.3764	0.0467	0.538E-03	0.233E-04	15.1090	0.1377	0.205E-03	0.167E-04
7.4233	0.0471	0.658E-03	0.253E-04	15.2480	0.1396	0.216E-03	0.175E-04
7.4706	0.0476	0.755E-03	0.271E-04	15.3890	0.1415	0.206E-03	0.174E-04
7.5184	0.0480	0.784E-03	0.272E-04	15.5310	0.1435	0.218E-03	0.172E-04
7.5667	0.0485	0.788E-03	0.273E-04	15.6760	0.1455	0.213E-03	0.176E-04
7.6154	0.0490	0.802E-03	0.273E-04	15.8220	0.1476	0.216E-03	0.178E-04
7.6646	0.0494	0.840E-03	0.273E-04	15.9710	0.1497	0.208E-03	0.175E-04
7.7143	0.0499	0.894E-03	0.281E-04	16.1220	0.1518	0.211E-03	0.175E-04
7.7645	0.0504	0.904E-03	0.285E-04	16.2750	0.1540	0.215E-03	0.178E-04
7.8151	0.0509	0.862E-03	0.276E-04	16.4300	0.1563	0.220E-03	0.182E-04
7.8663	0.0514	0.788E-03	0.264E-04	16.5870	0.1585	0.220E-03	0.178E-04
7.9180	0.0519	0.721E-03	0.254E-04	16.7470	0.1608	0.218E-03	0.183E-04
7.9702	0.0524	0.684E-03	0.251E-04	16.9090	0.1632	0.217E-03	0.185E-04
8.0229	0.0530	0.650E-03	0.243E-04	17.0730	0.1656	0.218E-03	0.187E-04
8.0761	0.0535	0.599E-03	0.233E-04	17.2400	0.1681	0.227E-03	0.193E-04
8.1299	0.0540	0.535E-03	0.220E-04	17.4090	0.1706	0.214E-03	0.189E-04
8.1842	0.0544	0.465E-03	0.208E-04	17.5810	0.1731	0.219E-03	0.194E-04
8.2390	0.0551	0.410E-03	0.195E-04	17.7560	0.1757	0.220E-03	0.200E-04
8.2944	0.0557	0.377E-03	0.188E-04	17.9330	0.1786	0.216E-03	0.199E-04
8.3504	0.0563	0.370E-03	0.187E-04	18.1120	0.1811	0.210E-03	0.194E-04
8.4069	0.0568	0.362E-03	0.187E-04	18.2950	0.1839	0.221E-03	0.200E-04
8.4641	0.0574	0.363E-03	0.185E-04	18.4800	0.1867	0.211E-03	0.200E-04
8.5218	0.0580	0.377E-03	0.189E-04	18.6680	0.1896	0.217E-03	0.201E-04
8.5801	0.0586	0.375E-03	0.186E-04	18.8590	0.1925	0.219E-03	0.204E-04
8.6390	0.0592	0.369E-03	0.188E-04	19.0530	0.1955	0.209E-03	0.203E-04
8.6985	0.0598	0.384E-03	0.189E-04	19.2500	0.1986	0.198E-03	0.203E-04
8.7586	0.0605	0.383E-03	0.189E-04	19.4510	0.2017	0.199E-03	0.202E-04
8.8194	0.0611	0.384E-03	0.190E-04	19.6540	0.2040	0.191E-03	0.196E-04
8.8808	0.0617	0.382E-03	0.186E-04	19.8610	0.2082	0.186E-03	0.198E-04

TABLE 5.

SECONDARY NEUTRON SPECTRA FOR A SCATTERING ANGLE OF APPROXIMATELY 60 DEGREES. THE VALUES GIVEN ARE THE LOWER AND UPPER LIMITS OF THE 68% CONFIDENCE INTERVAL FOR THE FLUX AT THE DETECTOR POSITION. THEY REPRESENT THE SECONDARY FLUX AS AVERAGED OVER THE AREA OF THE PROJECTION OF THE DETECTOR IN THE DIRECTION OF A POINT ON THE SCATTERING RING. THE DATA ARE NORMALIZED TO UNIT INCIDENT NEUTRON.

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 17.518 TO 20.014 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.944E-05	0.166E-04	4.885	0.177E-05	0.639E-05	9.890	-0.115E-05	0.281E-05
0.910	0.383E-05	0.160E-04	5.098	0.332E-05	0.830E-05	10.191	-0.200E-05	0.209E-05
1.011	0.820E-05	0.180E-04	5.287	0.288E-05	0.787E-05	10.586	-0.106E-05	0.258E-05
1.112	0.651E-05	0.181E-04	5.494	0.194E-05	0.700E-05	11.004	0.808E-06	0.458E-05
1.211	0.859E-05	0.181E-04	5.711	0.140E-05	0.608E-05	11.400	0.196E-05	0.679E-05
1.310	0.961E-05	0.206E-04	5.908	0.215E-05	0.690E-05	11.772	0.285E-05	0.720E-05
1.406	0.826E-05	0.185E-04	6.116	0.388E-05	0.851E-05	12.163	0.375E-05	0.715E-05
1.505	0.536E-05	0.155E-04	6.297	0.471E-05	0.917E-05	12.574	0.428E-05	0.787E-05
1.604	0.291E-05	0.121E-04	6.487	0.470E-05	0.961E-05	13.006	0.531E-05	0.963E-05
1.704	0.200E-05	0.103E-04	6.685	0.423E-05	0.942E-05	13.401	0.574E-05	0.898E-05
1.813	0.471E-05	0.117E-04	6.898	0.356E-05	0.890E-05	13.818	0.580E-05	0.916E-05
1.933	0.103E-04	0.169E-04	7.110	0.415E-05	0.898E-05	14.251	0.573E-05	0.883E-05
2.095	0.129E-04	0.200E-04	7.292	0.476E-05	0.879E-05	14.680	0.587E-05	0.880E-05
2.291	0.645E-05	0.133E-04	7.481	0.486E-05	0.862E-05	15.046	0.654E-05	0.933E-05
2.490	0.500E-05	0.115E-04	7.677	0.458E-05	0.882E-05	15.396	0.743E-05	0.101E-04
2.694	0.738E-05	0.138E-04	7.881	0.393E-05	0.869E-05	15.761	0.872E-05	0.115E-04
2.902	0.507E-05	0.120E-04	8.093	0.332E-05	0.796E-05	16.375	0.114E-04	0.139E-04
3.109	0.143E-05	0.682E-05	8.314	0.289E-05	0.739E-05	17.190	0.138E-04	0.157E-04
3.310	0.228E-05	0.858E-05	8.515	0.267E-05	0.695E-05	17.973	0.128E-04	0.142E-04
3.501	0.630E-05	0.120E-04	8.653	0.262E-05	0.666E-05	18.718	0.926E-05	0.106E-04
3.693	0.750E-05	0.127E-04	8.877	0.255E-05	0.630E-05	19.511	0.505E-05	0.615E-05
3.901	0.557E-05	0.120E-04	9.099	0.223E-05	0.561E-05	20.469	0.156E-05	0.228E-05
4.107	0.399E-05	0.987E-05	9.329	0.132E-05	0.474E-05	21.493	-0.464E-07	0.354E-06
4.309	0.268E-05	0.733E-05	9.499	0.113E-06	0.411E-05	22.473	-0.425E-06	-0.205E-06
4.503	0.513E-06	0.556E-05	9.674	-0.270E-06	0.357E-05	23.523	-0.391E-06	-0.255E-06
4.688	0.268E-06	0.518E-05						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 15.043 TO 17.518 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.243E-05	0.221E-04	4.885	0.591E-05	0.983E-05	9.890	0.762E-05	0.111E-04
0.910	0.812E-05	0.202E-04	5.098	0.533E-05	0.944E-05	10.191	0.912E-05	0.123E-04
1.011	0.135E-04	0.225E-04	5.287	0.539E-05	0.961E-05	10.586	0.928E-05	0.123E-04
1.112	0.117E-04	0.233E-04	5.494	0.567E-05	0.100E-04	11.004	0.848E-05	0.110E-04
1.211	0.122E-04	0.193E-04	5.711	0.555E-05	0.961E-05	11.400	0.659E-05	0.972E-05
1.310	0.879E-05	0.198E-04	5.908	0.505E-05	0.957E-05	11.772	0.455E-05	0.768E-05
1.406	0.793E-05	0.175E-04	6.116	0.482E-05	0.822E-05	12.163	0.298E-05	0.582E-05
1.505	0.789E-05	0.183E-04	6.297	0.381E-05	0.763E-05	12.574	0.200E-05	0.424E-05
1.604	0.820E-05	0.163E-04	6.487	0.251E-05	0.031E-05	13.006	0.308E-05	0.578E-05
1.704	0.107E-04	0.173E-04	6.685	0.130E-05	0.587E-05	13.401	0.597E-05	0.815E-05
1.813	0.117E-04	0.171E-04	6.898	0.815E-06	0.527E-05	13.818	0.848E-05	0.171E-04
1.933	0.102E-04	0.151E-04	7.110	-0.228E-06	0.429E-05	14.251	0.137E-04	0.157E-04
2.095	0.897E-05	0.144E-04	7.292	-0.178E-06	0.445E-05	14.680	0.157E-04	0.175E-04
2.291	0.116E-04	0.167E-04	7.481	-0.280E-06	0.283E-05	15.046	0.161E-04	0.178E-04
2.490	0.982E-05	0.149E-04	7.677	-0.351E-06	0.335E-05	15.396	0.152E-04	0.167E-04
2.694	0.707E-05	0.122E-04	7.881	-0.699E-08	0.403E-05	15.761	0.138E-04	0.188E-04
2.902	0.498E-05	0.104E-04	8.093	0.526E-06	0.451E-05	16.375	0.947E-05	0.107E-04
3.109	0.403E-05	0.680E-05	8.314	0.812E-06	0.465E-05	17.190	0.478E-05	0.562E-05
3.310	0.211E-05	0.748E-05	8.515	0.630E-06	0.430E-05	17.973	0.181E-05	0.237E-05
3.501	0.217E-05	0.700E-05	8.653	0.855E-06	0.391E-05	18.718	0.338E-06	0.723E-06
3.693	0.619E-05	0.841E-05	8.877	0.467E-06	0.376E-05	19.511	-0.278E-06	0.841E-08
3.901	0.488E-05	0.108E-04	9.099	0.117E-05	0.410E-05	20.469	-0.406E-06	-0.224E-06
4.107	0.467E-05	0.973E-05	9.329	0.269E-05	0.582E-05	21.493	-0.303E-06	-0.193E-06
4.309	0.512E-05	0.927E-05	9.499	0.418E-05	0.747E-05	22.473	-0.200E-06	-0.120E-06
4.503	0.568E-05	0.102E-04	9.674	0.583E-05	0.924E-05	23.523	-0.130E-06	-0.456E-07
4.688	0.625E-05	0.102E-04						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 12.522 TO 15.043 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.478E-05	0.285E-04	4.885	0.101E-05	0.451E-05	9.890	0.482E-05	0.755E-05
0.910	0.142E-04	0.253E-04	5.098	0.909E-06	0.447E-05	10.191	0.427E-05	0.673E-05
1.011	0.139E-04	0.212E-04	5.287	0.130E-05	0.527E-05	10.586	0.454E-05	0.678E-05
1.112	0.929E-05	0.197E-04	5.494	0.212E-05	0.599E-05	11.004	0.603E-05	0.792E-05
1.211	0.119E-04	0.179E-04	5.711	0.251E-05	0.666E-05	11.400	0.817E-05	0.102E-04
1.310	0.117E-04	0.201E-04	5.908	0.173E-05	0.565E-05	11.772	0.108E-04	0.129E-04
1.406	0.124E-04	0.195E-04	6.116	0.483E-06	0.422E-05	12.163	0.136E-04	0.154E-04
1.505	0.126E-04	0.188E-04	6.297	-0.665E-06	0.308E-05	12.574	0.152E-04	0.167E-04
1.604	0.122E-04	0.172E-04	6.407	-0.114E-06	0.309E-05	13.006	0.151E-04	0.164E-04
1.704	0.110E-04	0.160E-04	6.685	-0.573E-06	0.375E-05	13.401	0.135E-04	0.147E-04
1.813	0.108E-04	0.149E-04	6.898	0.813E-06	0.540E-05	13.818	0.109E-04	0.120E-04
1.933	0.111E-04	0.152E-04	7.110	0.293E-05	0.765E-05	14.251	0.783E-05	0.876E-05
2.095	0.101E-04	0.149E-04	7.292	0.506E-05	0.885E-05	14.680	0.528E-05	0.606E-05
2.291	0.888E-05	0.137E-04	7.481	0.661E-05	0.974E-05	15.046	0.311E-05	0.374E-05
2.490	0.104E-04	0.147E-04	7.677	0.772E-05	0.113E-04	15.396	0.173E-05	0.224E-05
2.694	0.982E-05	0.142E-04	7.881	0.870E-05	0.125E-04	15.761	0.745E-06	0.117E-05
2.902	0.656E-05	0.120E-04	8.093	0.962E-05	0.135E-04	16.375	-0.136E-06	0.204E-06
3.109	0.691E-05	0.108E-04	8.314	0.103E-04	0.141E-04	17.190	-0.437E-06	-0.207E-06
3.310	0.759E-05	0.121E-04	8.515	0.105E-04	0.140E-04	17.973	-0.388E-06	-0.251E-06
3.501	0.731E-05	0.119E-04	8.693	0.104E-04	0.137E-04	18.718	-0.302E-06	-0.208E-06
3.693	0.671E-05	0.101E-04	8.877	0.999E-05	0.130E-04	19.511	-0.222E-06	-0.155E-06
3.901	0.466E-05	0.981E-05	9.099	0.899E-05	0.117E-04	20.469	-0.154E-06	-0.975E-07
4.107	0.318E-05	0.805E-05	9.329	0.160E-05	0.102E-04	21.493	-0.971E-07	-0.425E-07
4.309	0.275E-05	0.633E-05	9.499	0.655E-05	0.927E-05	22.473	-0.501E-07	-0.453E-08
4.503	0.237E-05	0.595E-05	9.674	0.564E-05	0.844E-05	23.523	-0.224E-07	0.163E-07
4.688	0.170E-05	0.522E-05						

TABLE 5. CONTINUED

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 9.968 TO 12.522 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.100E-04	0.266E-04	8.885	0.105E-04	0.139E-04	9.890	0.138E-04	0.166E-04
0.910	0.165E-04	0.239E-04	5.094	0.130E-04	0.171E-04	10.191	0.150E-04	0.161E-04
1.011	0.179E-04	0.223E-04	5.287	0.153E-04	0.188E-04	10.586	0.150E-04	0.164E-04
1.112	0.155E-04	0.217E-04	5.494	0.155E-04	0.190E-04	11.004	0.140E-04	0.169E-04
1.211	0.168E-04	0.182E-04	5.711	0.154E-04	0.188E-04	11.400	0.114E-04	0.122E-04
1.310	0.133E-04	0.168E-04	5.908	0.157E-04	0.187E-04	11.772	0.863E-05	0.933E-05
1.406	0.138E-04	0.168E-04	6.116	0.153E-04	0.183E-04	12.163	0.583E-05	0.639E-05
1.505	0.139E-04	0.171E-04	6.297	0.143E-04	0.176E-04	12.574	0.342E-05	0.386E-05
1.604	0.125E-04	0.157E-04	6.487	0.132E-04	0.167E-04	13.006	0.166E-05	0.200E-05
1.704	0.100E-04	0.135E-04	6.685	0.118E-04	0.155E-04	13.401	0.655E-06	0.955E-06
1.813	0.784E-05	0.105E-04	6.894	0.105E-04	0.136E-04	13.818	0.619E-07	0.323E-06
1.933	0.583E-05	0.902E-05	7.110	0.860E-05	0.114E-04	14.251	-0.222E-06	-0.951E-08
2.095	0.568E-05	0.941E-05	7.292	0.684E-05	0.920E-05	14.640	-0.316E-06	-0.155E-06
2.291	0.629E-05	0.939E-05	7.481	0.491E-05	0.694E-05	15.046	-0.335E-06	-0.221E-06
2.490	0.377E-05	0.753E-05	7.677	0.330E-05	0.545E-05	15.396	-0.325E-06	-0.234E-06
2.694	0.169E-05	0.547E-05	7.881	0.236E-05	0.448E-05	15.761	-0.303E-06	-0.225E-06
2.902	0.186E-05	0.610E-05	8.093	0.242E-05	0.420E-05	16.375	-0.251E-06	-0.189E-06
3.109	0.364E-05	0.683E-05	8.314	0.283E-05	0.459E-05	17.190	-0.167E-06	-0.128E-06
3.310	0.223E-05	0.616E-05	8.515	0.344E-05	0.534E-05	17.973	-0.106E-06	-0.754E-07
3.501	0.114E-06	0.424E-05	8.693	0.417E-05	0.612E-05	18.718	-0.683E-07	-0.399E-07
3.693	0.546E-06	0.377E-05	8.877	0.513E-05	0.704E-05	19.511	-0.392E-07	-0.169E-07
3.901	0.239E-05	0.697E-05	9.099	0.655E-05	0.849E-05	20.469	-0.179E-07	-0.129E-08
4.107	0.406E-05	0.890E-05	9.329	0.846E-05	0.103E-04	21.493	-0.119E-07	0.900E-08
4.309	0.509E-05	0.807E-05	9.499	0.999E-05	0.117E-04	22.473	-0.112E-07	0.135E-07
4.503	0.545E-05	0.864E-05	9.674	0.116E-04	0.130E-04	23.523	-0.860E-08	0.223E-07
4.688	0.715E-05	0.105E-04						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 9.008 TO 9.968 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.165E-05	0.114E-04	4.885	0.222E-04	0.251E-04	9.890	0.857E-05	0.943E-05
0.910	0.311E-05	0.839E-05	5.094	0.167E-04	0.194E-04	10.191	0.579E-05	0.655E-05
1.011	0.396E-05	0.731E-05	5.287	0.119E-04	0.146E-04	10.586	0.275E-05	0.332E-05
1.112	0.464E-05	0.853E-05	5.494	0.791E-05	0.106E-04	11.004	0.747E-06	0.111E-05
1.211	0.495E-05	0.829E-05	5.711	0.488E-05	0.746E-05	11.400	-0.159E-06	0.108E-06
1.310	0.357E-05	0.688E-05	5.908	0.284E-05	0.531E-05	11.772	-0.450E-06	-0.188E-06
1.406	0.358E-05	0.663E-05	6.116	0.923E-06	0.369E-05	12.163	-0.451E-06	-0.226E-06
1.505	0.390E-05	0.722E-05	6.297	-0.296E-06	0.259E-05	12.574	-0.352E-06	-0.189E-06
1.604	0.401E-05	0.735E-05	6.487	-0.925E-06	0.181E-05	13.006	-0.277E-06	-0.149E-06
1.704	0.404E-05	0.782E-05	6.685	-0.978E-06	0.164E-05	13.401	-0.240E-06	-0.125E-06
1.813	0.461E-05	0.767E-05	6.894	-0.461E-06	0.201E-05	13.818	-0.210E-06	-0.113E-06
1.933	0.425E-05	0.804E-05	7.110	0.805E-06	0.297E-05	14.251	-0.175E-06	-0.916E-07
2.095	0.506E-05	0.927E-05	7.292	0.217E-05	0.414E-05	14.640	-0.141E-06	-0.715E-07
2.291	0.672E-05	0.104E-04	7.481	0.380E-05	0.571E-05	15.046	-0.109E-06	-0.473E-07
2.490	0.550E-05	0.958E-05	7.677	0.563E-05	0.764E-05	15.396	-0.798E-07	-0.289E-07
2.694	0.594E-05	0.972E-05	7.881	0.773E-05	0.974E-05	15.761	-0.538E-07	-0.117E-07
2.902	0.670E-05	0.110E-04	8.093	0.101E-04	0.119E-04	16.375	-0.230E-07	0.871E-08
3.109	0.682E-05	0.109E-04	8.314	0.123E-04	0.139E-04	17.190	-0.611E-08	0.257E-07
3.310	0.582E-05	0.100E-04	8.515	0.137E-04	0.152E-04	17.973	-0.228E-08	0.333E-07
3.501	0.561E-05	0.999E-05	8.693	0.145E-04	0.159E-04	18.718	-0.113E-08	0.385E-07
3.693	0.855E-05	0.122E-04	8.877	0.147E-04	0.161E-04	19.511	-0.337E-07	0.407E-07
3.901	0.142E-04	0.184E-04	9.099	0.152E-04	0.162E-04	20.469	-0.844E-11	0.444E-07
4.107	0.217E-04	0.257E-04	9.329	0.131E-04	0.143E-04	21.493	0.108E-09	0.479E-07
4.309	0.277E-04	0.306E-04	9.499	0.119E-04	0.130E-04	22.473	0.186E-09	0.526E-07
4.503	0.290E-04	0.317E-04	9.674	0.105E-04	0.115E-04	23.523	0.143E-09	0.532E-07
4.688	0.266E-04	0.296E-04						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 8.018 TO 9.008 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.109E-04	0.297E-05	4.885	0.196E-06	0.305E-05	9.890	0.156E-05	0.211E-05
0.910	0.982E-06	0.640E-05	5.094	0.970E-06	0.398E-05	10.191	0.408E-06	0.871E-06
1.011	0.372E-05	0.699E-05	5.287	0.201E-05	0.518E-05	10.586	-0.224E-06	0.140E-06
1.112	0.172E-05	0.550E-05	5.494	0.222E-05	0.528E-05	11.004	-0.345E-06	-0.816E-07
1.211	0.119E-05	0.479E-05	5.711	0.158E-05	0.444E-05	11.400	-0.339E-06	-0.109E-06
1.310	0.214E-05	0.563E-05	5.908	0.106E-05	0.396E-05	11.772	-0.322E-06	-0.866E-07
1.406	0.392E-05	0.719E-05	6.116	0.116E-05	0.403E-05	12.163	-0.280E-06	-0.921E-07
1.505	0.575E-05	0.936E-05	6.297	0.233E-05	0.556E-05	12.574	-0.246E-06	-0.104E-06
1.604	0.723E-05	0.108E-04	6.487	0.421E-05	0.750E-05	13.006	-0.218E-06	-0.110E-06
1.704	0.785E-05	0.115E-04	6.685	0.627E-05	0.931E-05	13.401	-0.187E-06	-0.881E-07
1.813	0.878E-05	0.122E-04	6.894	0.841E-05	0.111E-04	13.818	-0.144E-06	-0.609E-07
1.933	0.100E-04	0.137E-04	7.110	0.109E-04	0.132E-04	14.251	-0.100E-06	-0.172E-07
2.095	0.110E-04	0.166E-04	7.292	0.134E-04	0.154E-04	14.640	-0.664E-07	-0.105E-07
2.291	0.106E-04	0.146E-04	7.481	0.164E-04	0.180E-04	15.046	-0.392E-07	0.788E-09
2.490	0.836E-05	0.127E-04	7.677	0.196E-04	0.215E-04	15.396	-0.219E-07	0.828E-08
2.694	0.115E-04	0.155E-04	7.881	0.222E-04	0.241E-04	15.761	-0.109E-07	0.126E-07
2.902	0.208E-04	0.256E-04	8.093	0.274E-04	0.250E-04	16.375	-0.108E-08	0.155E-07
3.109	0.301E-04	0.308E-04	8.314	0.225E-04	0.239E-04	17.190	0.170E-08	0.179E-07
3.310	0.322E-04	0.362E-04	8.515	0.202E-04	0.213E-04	17.973	0.140E-08	0.192E-07
3.501	0.305E-04	0.349E-04	8.693	0.172E-04	0.183E-04	18.718	0.119E-08	0.208E-07
3.693	0.274E-04	0.305E-04	8.877	0.139E-04	0.149E-04	19.511	0.123E-08	0.215E-07
3.901	0.200E-04	0.236E-04	9.099	0.993E-05	0.109E-04	20.469	0.137E-08	0.233E-07
4.107	0.128E-04	0.164E-04	9.329	0.654E-05	0.734E-05	21.493	0.138E-08	0.248E-07
4.309	0.796E-05	0.105E-04	9.499	0.522E-05	0.526E-05	22.473	0.114E-08	0.267E-07
4.503	0.376E-05	0.628E-05	9.674	0.294E-05	0.359E-05	23.523	0.770E-09	0.264E-07
4.688	0.952E-06	0.384E-05						

TABLE 5. CONTINUED

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 7.006 TO 8.018 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.569E-05	0.229E-04	4.885	0.171E-05	0.569E-05	9.890	-0.101E-05	-0.551E-06
0.910	0.571E-05	0.123E-04	5.094	0.184E-05	0.543E-05	10.191	-0.899E-06	-0.547E-06
1.011	0.405E-05	0.795E-05	5.267	0.187E-05	0.571E-05	10.586	-0.685E-06	-0.426E-06
1.112	0.684E-05	0.112E-04	5.494	0.338E-05	0.730E-05	11.004	-0.481E-06	-0.286E-06
1.211	0.993E-05	0.141E-04	5.711	0.827E-05	0.119E-04	11.400	-0.358E-06	-0.196E-06
1.310	0.122E-04	0.164E-04	5.908	0.165E-04	0.199E-04	11.772	-0.278E-06	-0.129E-06
1.406	0.145E-04	0.186E-04	6.116	0.280E-04	0.318E-04	12.163	-0.196E-06	-0.715E-07
1.505	0.162E-04	0.206E-04	6.297	0.388E-04	0.432E-04	12.574	-0.115E-06	-0.170E-07
1.604	0.163E-04	0.204E-04	6.487	0.493E-04	0.535E-04	13.006	-0.488E-07	0.286E-07
1.704	0.153E-04	0.199E-04	6.685	0.568E-04	0.606E-04	13.401	-0.191E-07	0.608E-07
1.813	0.158E-04	0.195E-04	6.894	0.596E-04	0.628E-04	13.818	-0.260E-08	0.626E-07
1.933	0.180E-04	0.222E-04	7.110	0.573E-04	0.597E-04	14.251	0.468E-08	0.559E-07
2.095	0.261E-04	0.311E-04	7.292	0.520E-04	0.539E-04	14.640	0.551E-08	0.447E-07
2.291	0.378E-04	0.417E-04	7.481	0.444E-04	0.460E-04	15.046	0.476E-08	0.343E-07
2.490	0.415E-04	0.465E-04	7.677	0.355E-04	0.371E-04	15.396	0.354E-08	0.292E-07
2.694	0.385E-04	0.428E-04	7.881	0.265E-04	0.280E-04	15.761	0.219E-08	0.257E-07
2.902	0.285E-04	0.333E-04	8.093	0.183E-04	0.197E-04	16.375	-0.678E-09	0.229E-07
3.109	0.160E-04	0.201E-04	8.314	0.116E-04	0.126E-04	17.190	-0.295E-08	0.224E-07
3.310	0.441E-05	0.902E-05	8.515	0.707E-05	0.784E-05	17.973	-0.386E-08	0.228E-07
3.501	-0.136E-05	0.389E-05	8.693	0.419E-05	0.483E-05	18.718	-0.435E-08	0.233E-07
3.693	0.439E-06	0.408E-05	8.877	0.202E-05	0.273E-05	19.511	-0.880E-08	0.263E-07
3.901	0.295E-05	0.866E-05	9.099	0.392E-06	0.112E-05	20.469	-0.527E-08	0.284E-07
4.107	0.387E-05	0.960E-05	9.329	-0.493E-06	0.180E-06	21.493	-0.559E-08	0.293E-07
4.309	0.343E-05	0.703E-05	9.499	-0.827E-06	-0.213E-06	22.473	-0.522E-08	0.341E-07
4.503	0.205E-05	0.570E-05	9.674	-0.983E-06	-0.440E-06	23.523	-0.398E-08	0.347E-07
4.688	0.148E-05	0.570E-05						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 5.999 TO 7.006 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.109E-05	0.191E-04	4.885	0.110E-04	0.136E-04	9.890	0.130E-06	0.262E-06
0.910	0.137E-04	0.204E-04	5.094	0.196E-04	0.219E-04	10.191	0.108E-06	0.210E-06
1.011	0.192E-04	0.226E-04	5.267	0.285E-04	0.307E-04	10.586	0.696E-07	0.153E-06
1.112	0.241E-04	0.281E-04	5.494	0.376E-04	0.398E-04	11.004	0.360E-07	0.100E-06
1.211	0.327E-04	0.363E-04	5.711	0.450E-04	0.469E-04	11.400	0.150E-07	0.594E-07
1.310	0.425E-04	0.458E-04	5.908	0.419E-04	0.445E-04	11.772	0.220E-08	0.313E-07
1.406	0.497E-04	0.519E-04	6.116	0.455E-04	0.471E-04	12.163	-0.415E-08	0.129E-07
1.505	0.494E-04	0.524E-04	6.297	0.400E-04	0.414E-04	12.574	-0.808E-08	0.438E-08
1.604	0.464E-04	0.492E-04	6.487	0.321E-04	0.334E-04	13.006	-0.910E-08	0.990E-09
1.704	0.419E-04	0.448E-04	6.685	0.237E-04	0.248E-04	13.401	-0.822E-08	0.399E-09
1.813	0.361E-04	0.384E-04	6.894	0.157E-04	0.167E-04	13.818	-0.635E-08	0.166E-09
1.933	0.275E-04	0.303E-04	7.110	0.934E-05	0.101E-04	14.251	-0.432E-08	0.745E-09
2.095	0.167E-04	0.198E-04	7.292	0.550E-05	0.406E-04	14.640	-0.299E-08	0.129E-08
2.291	0.783E-05	0.104E-04	7.481	0.275E-05	0.317E-05	15.046	-0.210E-08	0.200E-08
2.490	0.301E-05	0.588E-05	7.677	0.954E-06	0.136E-05	15.396	-0.178E-08	0.251E-08
2.694	0.729E-06	0.321E-05	7.881	-0.173E-07	0.371E-06	15.761	-0.163E-08	0.290E-08
2.902	-0.110E-05	0.186E-05	8.093	-0.379E-06	-0.371E-07	16.375	-0.146E-08	0.311E-08
3.109	0.402E-06	0.365E-05	8.314	-0.378E-06	-0.100E-06	17.190	-0.136E-08	0.372E-08
3.310	0.393E-05	0.707E-05	8.515	-0.234E-06	0.201E-06	17.973	-0.135E-08	0.399E-08
3.501	0.600E-05	0.919E-05	8.693	-0.111E-06	0.125E-06	18.718	-0.139E-08	0.404E-08
3.693	0.717E-05	0.102E-04	8.877	-0.340E-07	0.234E-06	19.511	-0.145E-08	0.431E-08
3.901	0.666E-05	0.109E-04	9.099	0.866E-07	0.312E-06	20.469	-0.150E-08	0.502E-08
4.107	0.633E-05	0.992E-05	9.329	0.129E-06	0.336E-06	21.493	-0.143E-08	0.520E-08
4.309	0.560E-05	0.786E-05	9.499	0.138E-06	0.325E-06	22.473	-0.120E-08	0.581E-08
4.503	0.435E-05	0.713E-05	9.674	0.140E-06	0.301E-06	23.523	-0.822E-09	0.630E-08
4.688	0.577E-05	0.852E-05						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 5.006 TO 5.999 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.203E-04	0.360E-04	4.885	0.873E-04	0.899E-04	9.890	0.251E-06	0.336E-06
0.910	0.344E-04	0.401E-04	5.094	0.875E-04	0.896E-04	10.191	0.187E-06	0.249E-06
1.011	0.347E-04	0.375E-04	5.267	0.777E-04	0.798E-04	10.586	0.105E-06	0.155E-06
1.112	0.295E-04	0.325E-04	5.494	0.606E-04	0.622E-04	11.004	0.468E-07	0.856E-07
1.211	0.228E-04	0.251E-04	5.711	0.412E-04	0.424E-04	11.400	0.176E-07	0.450E-07
1.310	0.139E-04	0.166E-04	5.908	0.260E-04	0.269E-04	11.772	0.517E-08	0.240E-07
1.406	0.700E-05	0.949E-05	6.116	0.141E-04	0.149E-04	12.163	-0.150E-08	0.116E-07
1.505	0.320E-05	0.401E-05	6.297	0.739E-05	0.806E-05	12.574	-0.400E-08	0.556E-08
1.604	0.316E-05	0.596E-05	6.487	0.310E-05	0.365E-05	13.006	-0.433E-08	0.269E-08
1.704	0.416E-05	0.720E-05	6.685	0.670E-06	0.115E-05	13.401	-0.372E-08	0.159E-08
1.813	0.428E-05	0.693E-05	6.894	-0.609E-06	-0.170E-06	13.818	-0.312E-08	0.169E-08
1.933	0.227E-05	0.545E-05	7.110	-0.117E-05	-0.807E-06	14.251	-0.313E-08	0.213E-08
2.095	0.181E-05	0.529E-05	7.292	-0.131E-05	-0.103E-05	14.640	-0.332E-08	0.224E-08
2.291	0.346E-05	0.683E-05	7.481	-0.129E-05	-0.105E-05	15.046	-0.341E-08	0.247E-08
2.490	0.579E-05	0.933E-05	7.677	-0.116E-05	-0.920E-06	15.396	-0.339E-08	0.274E-08
2.694	0.826E-05	0.115E-04	7.881	-0.937E-06	-0.694E-06	15.761	-0.331E-08	0.298E-08
2.902	0.792E-05	0.116E-04	8.093	-0.643E-06	-0.422E-06	16.375	-0.309E-08	0.346E-08
3.109	0.960E-05	0.138E-04	8.314	-0.335E-06	-0.148E-06	17.190	-0.286E-08	0.425E-08
3.310	0.123E-04	0.161E-04	8.515	-0.939E-07	0.709E-07	17.973	-0.275E-08	0.481E-08
3.501	0.130E-04	0.168E-04	8.693	0.715E-07	0.231E-06	18.718	-0.271E-08	0.474E-08
3.693	0.144E-04	0.182E-04	8.877	0.192E-06	0.350E-06	19.511	-0.272E-08	0.545E-08
3.901	0.184E-04	0.227E-04	9.099	0.280E-06	0.429E-06	20.469	-0.281E-08	0.514E-08
4.107	0.269E-04	0.306E-04	9.329	0.313E-06	0.447E-06	21.493	-0.289E-08	0.545E-08
4.309	0.418E-04	0.445E-04	9.499	0.310E-06	0.431E-06	22.473	-0.275E-08	0.628E-08
4.503	0.603E-04	0.630E-04	9.674	0.290E-06	0.395E-06	23.523	-0.219E-08	0.672E-08
4.688	0.768E-04	0.794E-04						

TABLE 5. CONTINUED

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 4.006 TO 5.006 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.281E-05	0.163E-04	4.885	0.508E-04	0.519E-04	9.890	0.315E-08	0.166E-07
0.910	0.218E-05	0.911E-05	5.094	0.254E-04	0.263E-04	10.191	0.580E-09	0.122E-07
1.011	0.264E-05	0.616E-05	5.287	0.112E-04	0.118E-04	10.586	-0.141E-08	0.915E-08
1.112	0.323E-05	0.799E-05	5.494	0.315E-05	0.357E-05	11.004	-0.275E-08	0.826E-08
1.211	0.539E-05	0.976E-05	5.711	-0.610E-06	-0.234E-06	11.400	-0.329E-08	0.826E-08
1.310	0.485E-05	0.941E-05	5.908	-0.176E-05	-0.145E-05	11.772	-0.315E-08	0.830E-08
1.406	0.398E-05	0.843E-05	6.116	-0.193E-05	-0.168E-05	12.163	-0.272E-08	0.887E-08
1.505	0.496E-05	0.970E-05	6.297	-0.172E-05	-0.152E-05	12.574	-0.284E-08	0.990E-08
1.604	0.694E-05	0.116E-04	6.487	-0.140E-05	-0.123E-05	13.006	-0.356E-08	0.115E-07
1.704	0.866E-05	0.137E-04	6.685	-0.107E-05	-0.929E-06	13.401	-0.421E-08	0.123E-07
1.813	0.117E-04	0.170E-04	6.894	-0.777E-06	-0.653E-06	13.818	-0.447E-08	0.133E-07
1.933	0.159E-04	0.212E-04	7.110	-0.525E-06	-0.428E-06	14.251	-0.448E-08	0.140E-07
2.095	0.201E-04	0.263E-04	7.292	-0.361E-06	-0.283E-06	14.640	-0.417E-08	0.141E-07
2.291	0.262E-04	0.324E-04	7.481	-0.230E-06	-0.168E-06	15.066	-0.361E-08	0.136E-07
2.490	0.373E-04	0.438E-04	7.677	-0.129E-06	-0.750E-07	15.396	-0.305E-08	0.124E-07
2.694	0.417E-04	0.471E-04	7.881	-0.577E-07	-0.107E-07	15.761	-0.261E-08	0.111E-07
2.902	0.335E-04	0.395E-04	8.093	-0.111E-07	0.303E-07	16.375	-0.247E-08	0.963E-08
3.109	0.322E-04	0.385E-04	8.314	0.151E-07	0.504E-07	17.190	-0.296E-08	0.994E-08
3.310	0.477E-04	0.530E-04	8.515	0.254E-07	0.544E-07	17.973	-0.317E-08	0.107E-07
3.501	0.798E-04	0.855E-04	8.693	0.275E-07	0.507E-07	18.718	-0.314E-08	0.116E-07
3.693	0.125E-03	0.131E-03	8.877	0.255E-07	0.452E-07	19.511	-0.366E-08	0.114E-07
3.901	0.167E-03	0.171E-03	9.099	0.195E-07	0.380E-07	20.469	-0.398E-08	0.126E-07
4.107	0.179E-03	0.182E-03	9.329	0.135E-07	0.299E-07	21.493	-0.369E-08	0.141E-07
4.309	0.159E-03	0.161E-03	9.499	0.945E-08	0.251E-07	22.473	-0.316E-08	0.141E-07
4.503	0.123E-03	0.124E-03	9.674	0.611E-08	0.207E-07	23.523	-0.240E-08	0.143E-07
4.688	0.898E-04	0.861E-04						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 3.005 TO 4.006 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.125E-04	0.130E-04	4.885	-0.130E-05	-0.116E-05	9.890	-0.772E-08	0.216E-07
0.910	0.252E-05	0.117E-04	5.094	-0.119E-05	-0.105E-05	10.191	-0.786E-08	0.226E-07
1.011	0.443E-05	0.900E-05	5.287	-0.943E-06	-0.831E-06	10.586	-0.654E-08	0.237E-07
1.112	0.482E-05	0.111E-04	5.494	-0.694E-06	-0.605E-06	11.004	-0.433E-08	0.220E-07
1.211	0.988E-05	0.153E-04	5.711	-0.473E-06	-0.401E-06	11.400	-0.272E-08	0.260E-07
1.310	0.129E-04	0.190E-04	5.908	-0.310E-06	-0.253E-06	11.772	-0.238E-08	0.303E-07
1.406	0.156E-04	0.216E-04	6.116	-0.183E-06	-0.138E-06	12.163	-0.668E-09	0.339E-07
1.505	0.206E-04	0.264E-04	6.297	-0.104E-06	-0.693E-07	12.574	-0.250E-09	0.339E-07
1.604	0.286E-04	0.343E-04	6.487	-0.514E-07	-0.217E-07	13.006	-0.142E-09	0.277E-07
1.704	0.388E-04	0.448E-04	6.685	-0.199E-07	0.526E-08	13.401	-0.103E-09	0.215E-07
1.813	0.500E-04	0.562E-04	6.894	-0.389E-08	0.181E-07	13.818	-0.101E-09	0.185E-07
1.933	0.592E-04	0.642E-04	7.110	0.277E-09	0.212E-07	14.251	-0.936E-10	0.651E-08
2.095	0.587E-04	0.653E-04	7.292	-0.120E-08	0.213E-07	14.640	-0.741E-10	0.494E-08
2.291	0.503E-04	0.565E-04	7.481	-0.375E-08	0.210E-07	15.066	-0.104E-09	0.312E-08
2.490	0.506E-04	0.562E-04	7.677	-0.580E-08	0.198E-07	15.396	-0.119E-09	0.253E-08
2.694	0.787E-04	0.833E-04	7.881	-0.654E-08	0.183E-07	15.761	-0.358E-09	0.225E-08
2.902	0.126E-03	0.131E-03	8.093	-0.566E-08	0.160E-07	16.375	-0.902E-09	0.209E-08
3.109	0.163E-03	0.166E-03	8.314	-0.472E-08	0.142E-07	17.190	-0.114E-08	0.213E-08
3.310	0.165E-03	0.168E-03	8.515	-0.711E-08	0.132E-07	17.973	-0.124E-08	0.218E-08
3.501	0.138E-03	0.140E-03	8.693	-0.593E-08	0.131E-07	18.718	-0.131E-08	0.221E-08
3.693	0.969E-08	0.983E-08	8.877	-0.711E-08	0.142E-07	19.511	-0.138E-08	0.255E-08
3.901	0.538E-04	0.548E-04	9.099	-0.821E-08	0.149E-07	20.469	-0.149E-08	0.277E-08
4.107	0.240E-04	0.247E-04	9.329	-0.888E-08	0.155E-07	21.493	-0.157E-08	0.285E-08
4.309	0.807E-05	0.850E-05	9.499	-0.897E-08	0.161E-07	22.473	-0.148E-08	0.345E-08
4.503	0.136E-05	0.165E-05	9.674	-0.880E-08	0.197E-07	23.523	-0.116E-08	0.366E-08
4.688	-0.858E-06	-0.638E-06						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 1.996 TO 3.005 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.162E-05	0.461E-04	4.885	0.130E-07	0.375E-07	9.890	-0.361E-08	0.354E-08
0.910	0.130E-04	0.302E-04	5.094	0.257E-07	0.457E-07	10.191	-0.338E-08	0.381E-08
1.011	0.270E-04	0.353E-04	5.287	0.200E-07	0.367E-07	10.586	-0.320E-08	0.419E-08
1.112	0.389E-04	0.505E-04	5.494	0.110E-07	0.237E-07	11.004	-0.356E-08	0.462E-08
1.211	0.516E-04	0.613E-04	5.711	0.334E-08	0.125E-07	11.400	-0.383E-08	0.445E-08
1.310	0.623E-04	0.729E-04	5.908	-0.200E-10	0.659E-08	11.772	-0.378E-08	0.395E-08
1.406	0.709E-04	0.811E-04	6.116	-0.139E-08	0.481E-08	12.163	-0.329E-08	0.309E-08
1.505	0.790E-04	0.885E-04	6.297	-0.121E-08	0.537E-08	12.574	-0.248E-08	0.230E-08
1.604	0.958E-04	0.105E-03	6.487	-0.358E-09	0.653E-08	13.006	-0.155E-08	0.140E-08
1.704	0.130E-03	0.139E-03	6.685	0.317E-09	0.773E-08	13.401	-0.772E-09	0.134E-08
1.813	0.183E-03	0.191E-03	6.894	0.321E-09	0.754E-08	13.818	-0.166E-09	0.165E-08
1.933	0.232E-03	0.239E-03	7.110	0.402E-09	0.715E-08	14.251	-0.675E-10	0.194E-08
2.095	0.247E-03	0.253E-03	7.292	0.349E-09	0.654E-08	14.640	-0.662E-10	0.205E-08
2.291	0.303E-03	0.207E-03	7.481	0.291E-09	0.600E-08	15.066	-0.725E-10	0.216E-08
2.490	0.143E-03	0.145E-03	7.677	0.242E-09	0.567E-08	15.396	-0.857E-10	0.224E-08
2.694	0.863E-04	0.876E-04	7.881	0.134E-09	0.503E-08	15.761	-0.102E-09	0.229E-08
2.902	0.113E-08	0.421E-08	8.093	-0.229E-09	0.452E-08	16.375	-0.135E-09	0.240E-08
3.109	0.137E-08	0.142E-08	8.314	-0.447E-09	0.402E-08	17.190	-0.131E-09	0.266E-08
3.310	0.190E-05	0.214E-05	8.515	-0.159E-08	0.380E-08	17.973	-0.935E-10	0.289E-08
3.501	-0.146E-05	-0.126E-05	8.693	-0.218E-08	0.358E-08	18.718	-0.826E-10	0.286E-08
3.693	-0.183E-05	-0.169E-05	8.877	-0.273E-08	0.347E-08	19.511	-0.107E-09	0.312E-08
3.901	-0.135E-05	-0.125E-05	9.099	-0.324E-08	0.334E-08	20.469	-0.127E-09	0.303E-08
4.107	-0.800E-06	-0.723E-06	9.329	-0.359E-08	0.337E-08	21.493	-0.237E-10	0.332E-08
4.309	-0.401E-06	-0.346E-06	9.499	-0.371E-08	0.356E-08	22.473	0.150E-09	0.381E-08
4.503	-0.163E-06	-0.121E-06	9.674	-0.372E-08	0.357E-08	23.523	0.240E-09	0.388E-08
4.688	-0.419E-07	-0.105E-07						

TABLE 5. CONTINUED

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 1.496 TO 1.996 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.384E-04	0.116E-03	4.885	0.738E-09	0.902E-08	9.890	-0.103E-08	0.127E-08
0.910	0.885E-04	0.112E-03	5.094	0.243E-08	0.102E-07	10.191	-0.979E-09	0.145E-08
1.011	0.109E-03	0.123E-03	5.267	0.296E-08	0.110E-07	10.586	-0.109E-08	0.186E-08
1.112	0.132E-03	0.146E-03	5.494	0.258E-08	0.104E-07	11.004	-0.127E-08	0.216E-08
1.211	0.181E-03	0.194E-03	5.711	0.171E-08	0.772E-08	11.400	-0.152E-08	0.242E-08
1.310	0.268E-03	0.281E-03	5.908	0.885E-09	0.542E-08	11.772	-0.153E-08	0.251E-08
1.406	0.363E-03	0.374E-03	6.116	0.194E-09	0.336E-08	12.163	-0.138E-08	0.232E-08
1.505	0.422E-03	0.431E-03	6.297	-0.976E-10	0.210E-08	12.574	-0.107E-08	0.215E-08
1.604	0.420E-03	0.426E-03	6.487	-0.298E-09	0.142E-08	13.006	-0.668E-09	0.196E-08
1.704	0.361E-03	0.365E-03	6.685	-0.435E-09	0.997E-09	13.401	-0.298E-09	0.177E-08
1.813	0.259E-03	0.262E-03	6.894	-0.537E-09	0.837E-09	13.818	0.258E-11	0.187E-08
1.933	0.188E-03	0.150E-03	7.110	-0.634E-09	0.752E-09	14.251	0.432E-10	0.202E-08
2.095	0.493E-04	0.502E-04	7.292	-0.710E-09	0.755E-09	14.640	-0.950E-10	0.203E-08
2.291	0.513E-05	0.575E-05	7.481	-0.786E-09	0.817E-09	15.046	-0.201E-09	0.208E-08
2.490	-0.316E-05	-0.285E-05	7.677	-0.864E-09	0.832E-09	15.396	-0.747E-09	0.217E-08
2.694	-0.258E-05	-0.241E-05	7.881	-0.946E-09	0.836E-09	15.761	-0.711E-09	0.223E-08
2.902	-0.115E-05	-0.102E-05	8.093	-0.103E-08	0.870E-09	16.375	-0.207E-09	0.236E-08
3.109	-0.363E-06	-0.280E-06	8.314	-0.111E-08	0.882E-09	17.190	-0.189E-09	0.259E-08
3.310	-0.444E-07	0.981E-08	8.515	-0.119E-08	0.867E-09	17.973	-0.246E-09	0.272E-08
3.501	0.636E-07	0.104E-06	8.693	-0.124E-08	0.863E-09	18.718	-0.277E-09	0.285E-08
3.693	0.778E-07	0.110E-06	8.877	-0.127E-08	0.930E-09	19.511	-0.255E-09	0.266E-08
3.901	0.582E-07	0.785E-07	9.059	-0.129E-08	0.956E-09	20.469	-0.189E-09	0.233E-08
4.107	0.287E-07	0.465E-07	9.329	-0.125E-08	0.999E-09	21.493	-0.200E-09	0.197E-08
4.309	0.109E-07	0.271E-07	9.499	-0.119E-08	0.104E-08	22.473	-0.224E-09	0.164E-08
4.503	0.213E-08	0.161E-07	9.674	-0.112E-08	0.116E-08	23.523	-0.206E-09	0.183E-08
4.688	-0.363E-09	0.106E-07						

APPROXIMATE SCATTERING ANGLE = 60 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 0.991 TO 1.496 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.174E-03	0.283E-03	4.885	-0.148E-08	0.663E-08	9.890	-0.222E-08	0.651E-08
0.910	0.399E-03	0.437E-03	5.094	-0.275E-08	0.553E-08	10.191	-0.242E-08	0.590E-08
1.011	0.551E-03	0.565E-03	5.267	-0.408E-08	0.376E-08	10.586	-0.343E-08	0.483E-08
1.112	0.583E-03	0.593E-03	5.494	-0.502E-08	0.215E-08	11.004	-0.413E-08	0.433E-08
1.211	0.519E-03	0.526E-03	5.711	-0.510E-08	0.134E-08	11.400	-0.488E-08	0.454E-08
1.310	0.391E-03	0.396E-03	5.908	-0.452E-08	0.176E-08	11.772	-0.519E-08	0.626E-08
1.406	0.249E-03	0.252E-03	6.116	-0.357E-08	0.277E-08	12.163	-0.470E-08	0.789E-08
1.505	0.129E-03	0.131E-03	6.297	-0.266E-08	0.396E-08	12.574	-0.359E-08	0.767E-08
1.604	0.548E-04	0.559E-04	6.487	-0.170E-08	0.514E-08	13.006	-0.302E-08	0.792E-08
1.704	0.183E-04	0.189E-04	6.685	-0.677E-09	0.645E-08	13.401	-0.340E-08	0.717E-08
1.813	0.111E-05	0.343E-05	6.894	0.130E-09	0.756E-08	13.818	-0.408E-08	0.675E-08
1.933	-0.114E-05	-0.926E-06	7.110	0.540E-09	0.407E-08	14.251	-0.455E-08	0.647E-08
2.095	-0.123E-05	-0.109E-05	7.292	0.456E-09	0.792E-08	14.640	-0.440E-08	0.617E-08
2.291	-0.369E-06	-0.272E-06	7.481	0.689E-10	0.743E-08	15.046	-0.340E-08	0.802E-08
2.490	0.276E-07	0.826E-07	7.677	-0.655E-09	0.665E-08	15.396	-0.300E-08	0.895E-08
2.694	0.654E-07	0.105E-06	7.881	-0.165E-08	0.576E-08	15.761	-0.241E-08	0.937E-08
2.902	0.708E-07	0.474E-07	8.093	-0.275E-08	0.497E-08	16.375	-0.225E-08	0.105E-07
3.109	-0.783E-08	0.129E-07	8.314	0.444E-09	0.411E-08	17.190	-0.292E-08	0.105E-07
3.310	-0.154E-07	0.381E-08	8.515	-0.429E-08	0.392E-08	17.973	-0.388E-08	0.301E-07
3.501	-0.115E-07	-0.250E-08	8.693	-0.440E-08	0.388E-08	18.718	-0.371E-08	0.831E-08
3.693	-0.382E-08	0.316E-08	8.877	-0.352E-08	0.430E-08	19.511	-0.220E-08	0.841E-08
3.901	0.784E-09	0.805E-08	9.099	-0.298E-08	0.459E-08	20.469	-0.903E-09	0.757E-08
4.107	0.195E-08	0.917E-08	9.329	-0.249E-08	0.533E-08	21.493	0.181E-09	0.653E-08
4.309	0.388E-09	0.819E-08	9.499	-0.226E-08	0.593E-08	22.473	-0.544E-09	0.607E-08
4.503	-0.382E-09	0.704E-08	9.674	-0.221E-08	0.638E-08	23.523	-0.127E-08	0.681E-08
4.688	-0.837E-09	0.637E-08						

TABLE 6.

SECONDARY NEUTRON SPECTRA FOR A SCATTERING ANGLE OF APPROXIMATELY 90 DEGREES. THE VALUES GIVEN ARE THE LOWER AND UPPER LIMITS OF THE 68% CONFIDENCE INTERVAL FOR THE FLUX AT THE DETECTOR POSITION. THEY REPRESENT THE SECONDARY FLUX AS AVERAGED OVER THE AREA OF THE PROJECTION OF THE DETECTOR IN THE DIRECTION OF A POINT ON THE SCATTERING RING. THE DATA ARE NORMALIZED TO UNIT INCIDENT NEUTRON.

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 17.550 TO 20.050 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.367E-07	0.258E-04	4.885	0.405E-05	0.820E-05	9.890	-0.385E-06	0.318E-05
0.910	0.986E-05	0.216E-04	5.094	0.436E-05	0.874E-05	10.191	0.568E-06	0.424E-05
1.011	0.876E-05	0.178E-04	5.287	0.418E-05	0.886E-05	10.586	0.205E-05	0.538E-05
1.112	0.908E-05	0.199E-04	5.494	0.438E-05	0.894E-05	11.004	0.266E-05	0.605E-05
1.211	0.145E-04	0.229E-04	5.711	0.383E-05	0.826E-05	11.400	0.263E-05	0.681E-05
1.310	0.151E-04	0.251E-04	5.908	0.343E-05	0.771E-05	11.772	0.317E-05	0.686E-05
1.406	0.123E-04	0.220E-04	6.116	0.400E-05	0.816E-05	12.163	0.379E-05	0.692E-05
1.505	0.932E-05	0.184E-04	6.297	0.500E-05	0.905E-05	12.574	0.357E-05	0.675E-05
1.604	0.726E-05	0.156E-04	6.487	0.569E-05	0.101E-04	13.006	0.328E-05	0.619E-05
1.704	0.692E-05	0.145E-04	6.685	0.586E-05	0.100E-04	13.401	0.341E-05	0.612E-05
1.813	0.846E-05	0.147E-04	6.894	0.424E-05	0.901E-05	13.818	0.439E-05	0.709E-05
1.933	0.101E-04	0.163E-04	7.110	0.398E-05	0.812E-05	14.251	0.625E-05	0.901E-05
2.095	0.107E-04	0.176E-04	7.292	0.408E-05	0.759E-05	14.640	0.816E-05	0.108E-04
2.291	0.102E-04	0.162E-04	7.481	0.440E-05	0.775E-05	15.046	0.100E-04	0.125E-04
2.490	0.838E-05	0.143E-04	7.677	0.469E-05	0.838E-05	15.396	0.113E-04	0.136E-04
2.694	0.827E-05	0.142E-04	7.881	0.470E-05	0.884E-05	15.761	0.118E-04	0.142E-04
2.902	0.896E-05	0.151E-04	8.093	0.427E-05	0.821E-05	16.375	0.117E-04	0.140E-04
3.109	0.809E-05	0.130E-04	8.314	0.347E-05	0.703E-05	17.190	0.943E-05	0.111E-04
3.310	0.521E-05	0.108E-04	8.515	0.265E-05	0.609E-05	17.973	0.591E-05	0.118E-05
3.501	0.404E-05	0.923E-05	8.693	0.194E-05	0.528E-05	18.718	0.294E-05	0.398E-05
3.693	0.415E-05	0.895E-05	8.877	0.125E-05	0.438E-05	19.511	0.996E-06	0.186E-05
3.901	0.326E-05	0.947E-05	9.099	0.330E-06	0.336E-05	20.469	-0.567E-07	0.550E-06
4.107	0.265E-05	0.794E-05	9.329	-0.451E-06	0.257E-05	21.493	-0.311E-06	0.286E-07
4.309	0.256E-05	0.689E-05	9.499	-0.771E-06	0.245E-05	22.473	-0.277E-06	-0.105E-06
4.503	0.242E-05	0.717E-05	9.674	-0.745E-06	0.260E-05	23.523	-0.199E-06	-0.888E-07
4.688	0.318E-05	0.762E-05						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 14.933 TO 17.550 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.663E-05	0.309E-04	4.885	0.734E-05	0.108E-04	9.890	0.615E-05	0.912E-05
0.910	0.150E-04	0.267E-04	5.094	0.806E-05	0.117E-04	10.191	0.540E-05	0.813E-05
1.011	0.143E-04	0.227E-04	5.287	0.801E-05	0.117E-04	10.586	0.456E-05	0.721E-05
1.112	0.774E-05	0.188E-04	5.494	0.635E-05	0.100E-04	11.004	0.396E-05	0.615E-05
1.211	0.107E-04	0.172E-04	5.711	0.348E-05	0.736E-05	11.400	0.376E-05	0.649E-05
1.310	0.144E-04	0.238E-04	5.908	0.134E-05	0.499E-05	11.772	0.536E-05	0.807E-05
1.406	0.175E-04	0.262E-04	6.116	0.695E-06	0.411E-05	12.163	0.862E-05	0.108E-04
1.505	0.174E-04	0.241E-04	6.297	0.585E-06	0.403E-05	12.574	0.117E-04	0.137E-04
1.604	0.144E-04	0.211E-04	6.487	0.868E-06	0.474E-05	13.006	0.137E-04	0.154E-04
1.704	0.117E-04	0.181E-04	6.685	0.111E-05	0.508E-05	13.401	0.139E-04	0.156E-04
1.813	0.121E-04	0.168E-04	6.894	0.111E-05	0.561E-05	13.818	0.131E-04	0.147E-04
1.933	0.135E-04	0.180E-04	7.110	0.129E-05	0.553E-05	14.251	0.117E-04	0.132E-04
2.095	0.131E-04	0.180E-04	7.292	0.160E-05	0.495E-05	14.640	0.100E-04	0.118E-04
2.291	0.111E-04	0.157E-04	7.481	0.950E-06	0.390E-05	15.046	0.820E-05	0.937E-05
2.490	0.985E-05	0.145E-04	7.677	-0.683E-07	0.334E-05	15.396	0.658E-05	0.762E-05
2.694	0.740E-05	0.122E-04	7.881	-0.101E-05	0.274E-05	15.761	0.488E-05	0.590E-05
2.902	0.637E-05	0.110E-04	8.093	-0.999E-06	0.268E-05	16.375	0.259E-05	0.344E-05
3.109	0.612E-05	0.104E-04	8.314	0.239E-06	0.375E-05	17.190	0.695E-06	0.130E-05
3.310	0.441E-05	0.897E-05	8.515	0.201E-05	0.534E-05	17.973	-0.131E-07	0.384E-06
3.501	0.448E-05	0.888E-05	8.693	0.386E-05	0.691E-05	18.718	-0.222E-06	0.205E-07
3.693	0.569E-05	0.908E-05	8.877	0.850E-05	0.841E-05	19.511	-0.258E-06	0.169E-07
3.901	0.540E-05	0.109E-04	9.099	0.494E-05	0.974E-05	20.469	-0.206E-06	-0.939E-07
4.107	0.586E-05	0.108E-04	9.329	0.739E-05	0.102E-04	21.493	-0.141E-06	-0.722E-07
4.309	0.722E-05	0.109E-04	9.499	0.724E-05	0.102E-04	22.473	-0.913E-07	-0.413E-07
4.503	0.738E-05	0.111E-04	9.674	0.679E-05	0.979E-05	23.523	-0.564E-07	-0.128E-07
4.688	0.690E-05	0.104E-04						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 12.545 TO 14.933 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.106E-04	0.338E-04	4.885	0.208E-05	0.536E-05	9.890	0.768E-05	0.102E-04
0.910	0.172E-04	0.277E-04	5.094	0.981E-06	0.460E-05	10.191	0.110E-04	0.132E-04
1.011	0.200E-04	0.266E-04	5.287	0.270E-06	0.454E-05	10.586	0.151E-04	0.170E-04
1.112	0.203E-04	0.301E-04	5.494	0.121E-05	0.504E-05	11.004	0.180E-04	0.197E-04
1.211	0.215E-04	0.270E-04	5.711	0.251E-05	0.669E-05	11.400	0.186E-04	0.203E-04
1.310	0.168E-04	0.241E-04	5.908	0.321E-05	0.732E-05	11.772	0.176E-04	0.194E-04
1.406	0.140E-04	0.196E-04	6.116	0.365E-05	0.714E-05	12.163	0.158E-04	0.169E-04
1.505	0.121E-04	0.178E-04	6.297	0.331E-05	0.684E-05	12.574	0.121E-04	0.133E-04
1.604	0.137E-04	0.185E-04	6.487	0.306E-05	0.757E-05	13.006	0.844E-05	0.951E-05
1.704	0.157E-04	0.203E-04	6.685	0.319E-05	0.804E-05	13.401	0.548E-05	0.645E-05
1.813	0.167E-04	0.205E-04	6.894	0.422E-05	0.902E-05	13.818	0.309E-05	0.393E-05
1.933	0.154E-04	0.193E-04	7.110	0.587E-05	0.101E-04	14.251	0.140E-05	0.209E-05
2.095	0.136E-04	0.180E-04	7.292	0.723E-05	0.108E-04	14.640	0.454E-06	0.984E-06
2.291	0.146E-04	0.189E-04	7.481	0.803E-05	0.110E-04	15.046	-0.112E-06	0.322E-06
2.490	0.144E-04	0.186E-04	7.677	0.818E-05	0.115E-04	15.396	-0.366E-06	-0.617E-08
2.694	0.114E-04	0.159E-04	7.881	0.779E-05	0.114E-04	15.761	-0.474E-06	-0.182E-06
2.902	0.776E-05	0.130E-04	8.093	0.717E-05	0.106E-04	16.375	-0.491E-06	-0.270E-06
3.109	0.596E-05	0.970E-05	8.314	0.464E-05	0.984E-05	17.190	-0.387E-06	-0.211E-06
3.310	0.447E-05	0.896E-05	8.515	0.581E-05	0.904E-05	17.973	-0.265E-06	-0.126E-06
3.501	0.439E-05	0.892E-05	8.693	0.533E-05	0.846E-05	18.718	-0.172E-06	-0.693E-07
3.693	0.435E-05	0.774E-05	8.877	0.408E-05	0.765E-05	19.511	-0.113E-06	-0.252E-07
3.901	0.224E-05	0.698E-05	9.099	0.446E-05	0.711E-05	20.469	-0.852E-07	0.546E-08
4.107	0.145E-05	0.600E-05	9.329	0.445E-05	0.701E-05	21.493	-0.804E-07	0.227E-07
4.309	0.198E-05	0.556E-05	9.499	0.487E-05	0.750E-05	22.473	-0.755E-07	0.416E-07
4.503	0.254E-05	0.621E-05	9.674	0.587E-05	0.845E-05	23.523	-0.593E-07	0.620E-07
4.688	0.265E-05	0.614E-05						

TABLE 6. CONTINUED

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 9.987 TO 12.545 MEV.

NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.197E-04	0.384E-04	4.885	0.181E-04	0.212E-04	9.890	0.141E-04	0.152E-04
0.910	0.240E-04	0.298E-04	5.094	0.178E-04	0.207E-04	10.191	0.127E-04	0.136E-04
1.011	0.211E-04	0.246E-04	5.287	0.158E-04	0.190E-04	10.586	0.100E-04	0.108E-04
1.112	0.192E-04	0.236E-04	5.494	0.132E-04	0.163E-04	11.004	0.683E-05	0.751E-05
1.211	0.198E-04	0.229E-04	5.711	0.109E-04	0.138E-04	11.400	0.403E-05	0.462E-05
1.310	0.179E-04	0.213E-04	5.908	0.981E-05	0.124E-04	11.772	0.206E-05	0.258E-05
1.406	0.158E-04	0.187E-04	6.116	0.885E-05	0.116E-04	12.163	0.772E-06	0.120E-05
1.505	0.139E-04	0.168E-04	6.297	0.784E-05	0.109E-04	12.574	0.634E-07	0.371E-06
1.604	0.125E-04	0.153E-04	6.487	0.676E-05	0.999E-05	13.006	-0.265E-06	-0.299E-07
1.704	0.103E-04	0.136E-04	6.685	0.542E-05	0.853E-05	13.401	-0.348E-06	-0.151E-06
1.813	0.835E-05	0.108E-04	6.894	0.416E-05	0.708E-05	13.818	-0.338E-06	-0.175E-06
1.933	0.654E-05	0.964E-05	7.110	0.366E-05	0.591E-05	14.251	-0.283E-06	-0.160E-06
2.095	0.556E-05	0.933E-05	7.292	0.350E-05	0.548E-05	14.640	-0.233E-06	-0.134E-06
2.291	0.462E-05	0.732E-05	7.481	0.375E-05	0.556E-05	15.046	-0.189E-06	-0.108E-06
2.490	0.288E-05	0.651E-05	7.677	0.458E-05	0.611E-05	15.396	-0.160E-06	-0.834E-07
2.694	0.237E-05	0.580E-05	7.881	0.609E-05	0.786E-05	15.761	-0.135E-06	-0.590E-07
2.902	0.216E-05	0.615E-05	8.093	0.847E-05	0.101E-04	16.375	-0.105E-06	-0.242E-07
3.109	0.336E-05	0.650E-05	8.314	0.113E-04	0.128E-04	17.190	-0.696E-07	0.132E-08
3.310	0.386E-05	0.749E-05	8.515	0.135E-04	0.150E-04	17.973	-0.406E-07	0.915E-08
3.501	0.310E-05	0.701E-05	8.693	0.149E-04	0.164E-04	18.718	-0.238E-07	0.223E-07
3.693	0.278E-05	0.574E-05	8.877	0.158E-04	0.172E-04	19.511	-0.181E-07	0.311E-07
3.901	0.369E-05	0.792E-05	9.099	0.160E-04	0.174E-04	20.469	-0.156E-07	0.384E-07
4.107	0.695E-05	0.112E-04	9.329	0.157E-04	0.171E-04	21.493	-0.171E-07	0.468E-07
4.309	0.114E-04	0.142E-04	9.499	0.154E-04	0.166E-04	22.473	-0.177E-07	0.492E-07
4.503	0.166E-04	0.173E-04	9.674	0.149E-04	0.160E-04	23.523	-0.151E-07	0.598E-07
4.688	0.167E-04	0.197E-04						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 9.025 TO 9.987 MEV.

NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.964E-06	0.140E-04	4.885	0.722E-05	0.103E-04	9.890	0.193E-05	0.252E-05
0.910	0.621E-05	0.113E-04	5.094	0.296E-05	0.589E-05	10.191	0.800E-06	0.130E-05
1.011	0.507E-05	0.823E-05	5.287	0.828E-06	0.387E-05	10.586	0.503E-07	0.433E-06
1.112	0.268E-05	0.616E-05	5.494	0.532E-06	0.366E-05	11.004	-0.259E-06	-0.273E-08
1.211	0.397E-05	0.729E-05	5.711	0.145E-05	0.426E-05	11.400	-0.384E-06	-0.165E-06
1.310	0.598E-05	0.910E-05	5.908	0.221E-05	0.508E-05	11.772	-0.410E-06	-0.213E-06
1.406	0.591E-05	0.897E-05	6.116	0.239E-05	0.572E-05	12.163	-0.384E-06	-0.223E-06
1.505	0.423E-05	0.758E-05	6.297	0.267E-05	0.618E-05	12.574	-0.326E-06	-0.207E-06
1.604	0.342E-05	0.684E-05	6.487	0.360E-05	0.694E-05	13.006	-0.259E-06	-0.170E-06
1.704	0.360E-05	0.721E-05	6.685	0.549E-05	0.860E-05	13.401	-0.205E-06	-0.125E-06
1.813	0.411E-05	0.721E-05	6.894	0.887E-05	0.116E-04	13.818	-0.153E-06	-0.862E-07
1.933	0.416E-05	0.769E-05	7.110	0.140E-04	0.162E-04	14.251	-0.110E-06	-0.564E-07
2.095	0.534E-05	0.951E-05	7.292	0.189E-04	0.209E-04	14.640	-0.931E-07	-0.371E-07
2.291	0.693E-05	0.105E-04	7.481	0.235E-04	0.254E-04	15.046	-0.648E-07	-0.182E-07
2.490	0.828E-05	0.129E-04	7.677	0.268E-04	0.287E-04	15.396	-0.533E-07	-0.904E-08
2.694	0.115E-04	0.153E-04	7.881	0.280E-04	0.299E-04	15.761	-0.445E-07	0.939E-09
2.902	0.101E-04	0.146E-04	8.093	0.268E-04	0.285E-04	16.375	-0.332E-07	0.124E-07
3.109	0.978E-05	0.139E-04	8.314	0.237E-04	0.251E-04	17.190	-0.161E-07	0.238E-07
3.310	0.118E-04	0.158E-04	8.515	0.199E-04	0.211E-04	17.973	-0.301E-08	0.264E-07
3.501	0.150E-04	0.202E-04	8.693	0.164E-04	0.175E-04	18.718	0.213E-08	0.300E-07
3.693	0.218E-04	0.253E-04	8.877	0.129E-04	0.140E-04	19.511	0.306E-08	0.336E-07
3.901	0.263E-04	0.144E-04	9.099	0.930E-05	0.103E-04	20.469	0.389E-08	0.373E-07
4.107	0.271E-04	0.316E-04	9.329	0.630E-05	0.718E-05	21.493	0.157E-08	0.414E-07
4.309	0.242E-04	0.271E-04	9.499	0.432E-05	0.537E-05	22.473	0.232E-08	0.452E-07
4.503	0.183E-04	0.211E-04	9.674	0.319E-05	0.389E-05	23.523	0.101E-08	0.449E-07
4.688	0.125E-04	0.156E-04						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 7.980 TO 9.025 MEV.

NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CH**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.862E-05	0.231E-04	4.885	0.212E-06	0.352E-05	9.890	-0.154E-06	0.299E-06
0.910	0.535E-05	0.109E-04	5.094	0.142E-05	0.451E-05	10.191	-0.253E-06	0.114E-06
1.011	0.434E-05	0.777E-05	5.287	0.221E-05	0.528E-05	10.586	-0.307E-06	-0.218E-07
1.112	0.427E-05	0.815E-05	5.494	0.267E-05	0.576E-05	11.004	-0.278E-06	-0.791E-07
1.211	0.514E-05	0.882E-05	5.711	0.377E-05	0.665E-05	11.400	-0.248E-06	-0.669E-07
1.310	0.507E-05	0.867E-05	5.908	0.632E-05	0.927E-05	11.772	-0.218E-06	-0.393E-07
1.406	0.551E-05	0.912E-05	6.116	0.108E-04	0.143E-04	12.163	-0.178E-06	-0.224E-07
1.505	0.772E-05	0.116E-04	6.297	9.146E-06	0.708E-05	12.574	-0.127E-06	-0.133E-07
1.604	0.946E-05	0.130E-04	6.487	0.244E-04	0.277E-04	13.006	-0.101E-06	0.197E-08
1.704	0.103E-04	0.143E-04	6.685	0.315E-04	0.348E-04	13.401	-0.758E-07	0.216E-07
1.813	0.130E-04	0.164E-04	6.894	0.369E-04	0.397E-04	13.818	-0.561E-07	0.432E-07
1.933	0.152E-04	0.189E-04	7.110	0.384E-04	0.406E-04	14.251	-0.371E-07	0.528E-07
2.095	0.162E-04	0.207E-04	7.292	0.364E-04	0.381E-04	14.640	-0.222E-07	0.559E-07
2.291	0.211E-04	0.248E-04	7.481	0.319E-04	0.335E-04	15.046	-0.104E-07	0.558E-07
2.490	0.267E-04	0.315E-04	7.677	0.259E-04	0.275E-04	15.396	-0.506E-08	0.558E-07
2.694	0.305E-04	0.342E-04	7.881	0.197E-04	0.211E-04	15.761	-0.521E-08	0.563E-07
2.902	0.336E-04	0.380E-04	8.093	0.139E-04	0.152E-04	16.375	-0.676E-08	0.537E-07
3.109	0.353E-04	0.389E-04	8.314	0.921E-05	0.102E-04	17.190	-0.769E-08	0.591E-07
3.310	0.384E-04	0.341E-04	8.515	0.684E-05	0.884E-05	17.973	0.742E-08	0.674E-07
3.501	0.221E-04	0.269E-04	8.693	0.393E-05	0.473E-05	18.718	-0.823E-08	0.640E-07
3.693	0.150E-04	0.180E-04	8.877	0.236E-05	0.317E-05	19.511	-0.891E-08	0.697E-07
3.901	0.799E-05	0.123E-04	9.099	0.115E-05	0.192E-05	20.469	-0.117E-07	0.771E-07
4.107	0.350E-05	0.774E-05	9.329	0.458E-06	0.113E-05	21.493	-0.149E-07	0.746E-07
4.309	0.151E-05	0.429E-05	9.499	0.177E-06	0.767E-06	22.473	-0.155E-07	0.837E-07
4.503	0.253E-06	0.312E-05	9.674	-0.497E-08	0.540E-06	23.523	-0.137E-07	0.876E-07
4.688	-0.327E-06	0.289E-05						

TABLE 6. CONTINUED

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 7.019 TO 7.980 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.241E-05	0.217E-04	4.885	0.647E-05	0.100E-04	9.890	0.753E-07	0.398E-06
0.910	0.888E-05	0.162E-04	5.094	0.876E-05	0.118E-04	10.191	-0.383E-07	0.186E-06
1.011	0.111E-04	0.151E-04	5.287	0.127E-04	0.159E-04	10.586	-0.999E-07	0.735E-07
1.112	0.110E-04	0.159E-04	5.494	0.195E-04	0.227E-04	11.004	-0.769E-07	0.558E-07
1.211	0.138E-04	0.182E-04	5.711	0.285E-04	0.314E-04	11.400	-0.884E-07	0.593E-07
1.310	0.185E-04	0.229E-04	5.908	0.361E-04	0.388E-04	11.772	-0.475E-07	0.619E-07
1.406	0.212E-04	0.257E-04	6.116	0.407E-04	0.439E-04	12.163	-0.509E-07	0.619E-07
1.505	0.216E-04	0.261E-04	6.297	0.413E-04	0.445E-04	12.574	-0.441E-07	0.383E-07
1.604	0.226E-04	0.267E-04	6.487	0.388E-04	0.420E-04	13.006	-0.327E-07	0.258E-07
1.704	0.263E-04	0.310E-04	6.685	0.337E-04	0.367E-04	13.401	-0.237E-07	0.280E-07
1.813	0.352E-04	0.389E-04	6.894	0.271E-04	0.296E-04	13.818	-0.278E-07	0.308E-07
1.933	0.467E-04	0.506E-04	7.110	0.204E-04	0.221E-04	14.251	-0.209E-07	0.423E-07
2.095	0.602E-04	0.649E-04	7.292	0.153E-04	0.165E-04	14.640	-0.179E-07	0.480E-07
2.291	0.674E-04	0.708E-04	7.481	0.107E-04	0.118E-04	15.046	-0.151E-07	0.515E-07
2.490	0.580E-04	0.626E-04	7.677	0.690E-05	0.789E-05	15.396	-0.122E-07	0.480E-07
2.694	0.393E-04	0.428E-04	7.881	0.397E-05	0.494E-05	15.761	-0.923E-08	0.410E-07
2.902	0.189E-04	0.227E-04	8.093	0.202E-05	0.288E-05	16.375	-0.594E-08	0.291E-07
3.109	0.686E-05	0.104E-04	8.314	0.100E-05	0.165E-05	17.190	-0.120E-08	0.189E-07
3.310	0.221E-05	0.617E-05	8.515	0.600E-06	0.111E-05	17.973	-0.158E-08	0.200E-07
3.501	-0.121E-06	0.415E-05	8.693	0.478E-06	0.947E-06	18.718	-0.287E-08	0.227E-07
3.693	-0.387E-06	0.294E-05	8.877	0.424E-06	0.927E-06	19.511	-0.343E-08	0.231E-07
3.901	0.308E-06	0.515E-05	9.099	0.371E-06	0.961E-06	20.469	-0.378E-08	0.243E-07
4.107	0.228E-05	0.718E-05	9.329	0.312E-06	0.870E-06	21.493	-0.392E-08	0.252E-07
4.309	0.853E-05	0.751E-05	9.499	0.245E-06	0.754E-06	22.473	-0.340E-08	0.286E-07
4.503	0.540E-05	0.854E-05	9.674	0.169E-06	0.599E-06	23.523	-0.244E-08	0.326E-07
4.688	0.564E-05	0.945E-05						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 6.010 TO 7.019 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.593E-07	0.233E-04	4.885	0.501E-04	0.532E-04	9.890	0.231E-06	0.423E-06
0.910	0.237E-04	0.323E-04	5.094	0.690E-04	0.717E-04	10.191	0.271E-06	0.432E-06
1.011	0.398E-04	0.437E-04	5.287	0.796E-04	0.821E-04	10.586	0.223E-06	0.362E-06
1.112	0.501E-04	0.548E-04	5.494	0.810E-04	0.834E-04	11.004	0.137E-06	0.244E-06
1.211	0.587E-04	0.626E-04	5.711	0.733E-04	0.753E-04	11.400	0.597E-07	0.148E-06
1.310	0.627E-04	0.664E-04	5.908	0.614E-04	0.632E-04	11.772	0.151E-07	0.864E-07
1.406	0.608E-04	0.647E-04	6.116	0.469E-04	0.486E-04	12.163	0.925E-08	0.567E-07
1.505	0.531E-04	0.564E-04	6.297	0.347E-04	0.363E-04	12.574	-0.202E-07	0.398E-07
1.604	0.429E-04	0.459E-04	6.487	0.234E-04	0.247E-04	13.006	-0.242E-07	0.373E-07
1.704	0.329E-04	0.362E-04	6.685	0.139E-04	0.152E-04	13.401	-0.149E-07	0.332E-07
1.813	0.244E-04	0.270E-04	6.894	0.681E-05	0.798E-05	13.818	-0.108E-07	0.267E-07
1.933	0.167E-04	0.199E-04	7.110	0.239E-05	0.324E-05	14.251	-0.651E-08	0.236E-07
2.095	0.838E-05	0.119E-04	7.292	0.385E-06	0.968E-06	14.640	-0.412E-08	0.219E-07
2.291	0.221E-05	0.527E-05	7.481	-0.666E-06	-0.196E-06	15.046	-0.408E-08	0.229E-07
2.490	0.667E-06	0.435E-05	7.677	-0.107E-05	-0.587E-06	15.396	-0.506E-08	0.239E-07
2.694	0.168E-05	0.477E-05	7.881	-0.104E-05	-0.585E-06	15.761	-0.585E-08	0.249E-07
2.902	0.163E-05	0.565E-05	8.093	-0.790E-06	-0.356E-06	16.375	-0.643E-08	0.240E-07
3.109	0.186E-05	0.615E-05	8.314	-0.088E-06	-0.126E-06	17.190	-0.659E-08	0.273E-07
3.310	0.266E-05	0.637E-05	8.515	-0.283E-06	0.361E-07	17.973	-0.653E-08	0.283E-07
3.501	0.390E-05	0.804E-05	8.693	-0.181E-06	0.153E-06	18.718	-0.625E-08	0.287E-07
3.693	0.688E-05	0.106E-04	8.877	-0.123E-06	0.222E-06	19.511	-0.594E-08	0.306E-07
3.901	0.695E-05	0.142E-04	9.099	-0.641E-07	0.275E-06	20.469	-0.589E-08	0.365E-07
4.107	0.104E-04	0.149E-04	9.329	0.176E-07	0.317E-06	21.493	-0.649E-08	0.381E-07
4.309	0.126E-04	0.154E-04	9.499	0.828E-07	0.355E-06	22.473	-0.704E-08	0.421E-07
4.503	0.186E-04	0.218E-04	9.674	0.160E-06	0.388E-06	23.523	-0.643E-08	0.457E-07
4.688	0.310E-04	0.346E-04						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 4.989 TO 6.010 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.201E-04	0.362E-04	4.885	0.971E-04	0.993E-04	9.890	0.109E-06	0.182E-06
0.910	0.370E-04	0.430E-04	5.094	0.737E-04	0.754E-04	10.191	0.819E-07	0.140E-06
1.011	0.377E-04	0.404E-04	5.287	0.512E-04	0.525E-04	10.586	0.485E-07	0.105E-06
1.112	0.266E-04	0.297E-04	5.494	0.300E-04	0.319E-04	11.004	0.235E-07	0.719E-07
1.211	0.156E-04	0.183E-04	5.711	0.157E-04	0.166E-04	11.400	0.120E-07	0.512E-07
1.310	0.891E-05	0.117E-04	5.908	0.734E-05	0.797E-05	11.772	0.676E-08	0.359E-07
1.406	0.551E-05	0.807E-05	6.116	0.238E-05	0.294E-05	12.163	-0.599E-08	0.264E-07
1.505	0.268E-05	0.548E-05	6.297	0.192E-06	0.680E-06	12.574	-0.813E-08	0.231E-07
1.604	0.107E-05	0.404E-05	6.487	-0.850E-06	-0.450E-06	13.006	-0.116E-07	0.227E-07
1.704	0.139E-05	0.457E-05	6.685	-0.123E-05	-0.869E-06	13.401	-0.117E-07	0.225E-07
1.813	0.350E-05	0.645E-05	6.894	-0.128E-05	-0.956E-06	13.818	-0.105E-07	0.201E-07
1.933	0.509E-05	0.848E-05	7.110	-0.117E-05	-0.896E-06	14.251	-0.926E-08	0.157E-07
2.095	0.444E-05	0.832E-05	7.292	-0.107E-05	-0.883E-06	14.640	-0.673E-08	0.112E-07
2.291	0.428E-05	0.720E-05	7.481	-0.855E-06	-0.684E-06	15.046	-0.445E-08	0.100E-07
2.490	0.681E-05	0.106E-04	7.677	-0.672E-06	-0.481E-06	15.396	-0.309E-08	0.105E-07
2.694	0.105E-04	0.140E-04	7.881	-0.474E-06	-0.285E-06	15.761	-0.216E-08	0.109E-07
2.902	0.123E-04	0.165E-04	8.093	-0.267E-06	-0.951E-07	16.375	-0.203E-08	0.119E-07
3.109	0.153E-04	0.201E-04	8.314	-0.860E-07	0.637E-07	17.190	-0.185E-08	0.133E-07
3.310	0.160E-04	0.204E-04	8.515	0.307E-07	0.167E-06	17.973	-0.153E-08	0.145E-07
3.501	0.170E-04	0.217E-04	8.693	0.101E-06	0.231E-06	18.718	-0.132E-08	0.151E-07
3.693	0.262E-04	0.307E-04	8.877	0.139E-06	0.267E-06	19.511	-0.136E-08	0.170E-07
3.901	0.482E-04	0.525E-04	9.099	0.156E-06	0.281E-06	20.469	-0.162E-08	0.182E-07
4.107	0.776E-04	0.813E-04	9.329	0.152E-06	0.266E-06	21.493	-0.198E-08	0.183E-07
4.309	0.103E-03	0.106E-03	9.499	0.140E-06	0.245E-06	22.473	-0.218E-08	0.212E-07
4.503	0.115E-03	0.117E-03	9.674	0.126E-06	0.218E-06	23.523	-0.192E-08	0.217E-07
4.688	0.112E-03	0.114E-03						

TABLE 6. CONTINUED

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 4.014 TO 4.989 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.887E-05	0.137E-04	4.885	0.175E-04	0.184E-04	9.890	0.491E-10	0.403E-07
0.910	0.377E-05	0.118E-04	5.094	0.605E-05	0.673E-05	10.191	-0.864E-09	0.312E-07
1.011	0.695E-05	0.110E-04	5.287	0.111E-05	0.155E-05	10.586	-0.214E-09	0.255E-07
1.112	0.566E-05	0.110E-04	5.494	-0.111E-05	-0.794E-06	11.004	-0.224E-09	0.244E-07
1.211	0.548E-05	0.105E-04	5.711	-0.181E-05	-0.153E-05	11.400	-0.196E-08	0.254E-07
1.310	0.476E-05	0.101E-04	5.908	-0.176E-05	-0.152E-05	11.772	-0.489E-08	0.254E-07
1.406	0.488E-05	0.963E-05	6.116	-0.145E-05	-0.127E-05	12.163	-0.806E-08	0.297E-07
1.505	0.575E-05	0.112E-04	6.297	-0.115E-05	-0.100E-05	12.574	-0.118E-07	0.244E-07
1.604	0.919E-05	0.146E-04	6.487	-0.864E-06	-0.737E-06	13.006	-0.152E-07	0.249E-07
1.704	0.127E-04	0.185E-04	6.685	-0.616E-06	-0.504E-06	13.401	-0.169E-07	0.235E-07
1.813	0.164E-04	0.225E-04	6.894	-0.405E-06	-0.310E-06	13.818	-0.171E-07	0.239E-07
1.933	0.204E-04	0.266E-04	7.110	-0.233E-06	-0.156E-06	14.251	-0.154E-07	0.234E-07
2.095	0.243E-04	0.315E-04	7.292	-0.128E-06	-0.612E-07	14.640	-0.136E-07	0.222E-07
2.291	0.313E-04	0.384E-04	7.481	-0.490E-07	0.634E-08	15.046	-0.976E-08	0.169E-07
2.490	0.464E-04	0.531E-04	7.677	-0.597E-09	0.487E-07	15.396	-0.675E-08	0.138E-07
2.694	0.584E-04	0.644E-04	7.881	0.248E-07	0.687E-07	15.761	-0.420E-08	0.108E-07
2.902	0.607E-04	0.669E-04	8.093	0.336E-07	0.786E-07	16.375	-0.159E-08	0.105E-07
3.109	0.710E-04	0.771E-04	8.314	0.355E-07	0.708E-07	17.190	-0.131E-08	0.119E-07
3.310	0.100E-03	0.105E-03	8.515	0.299E-07	0.641E-07	17.973	-0.144E-08	0.131E-07
3.501	0.190E-03	0.145E-03	8.693	0.253E-07	0.577E-07	18.718	-0.139E-08	0.144E-07
3.693	0.173E-03	0.177E-03	8.877	0.207E-07	0.526E-07	19.511	-0.106E-08	0.152E-07
3.901	0.178E-03	0.181E-03	9.099	0.139E-07	0.493E-07	20.469	-0.377E-10	0.167E-07
4.107	0.150E-03	0.152E-03	9.329	0.826E-08	0.477E-07	21.493	0.366E-09	0.183E-07
4.309	0.107E-03	0.109E-03	9.499	0.462E-08	0.466E-07	22.473	-0.284E-09	0.205E-07
4.503	0.611E-04	0.603E-04	9.674	0.147E-08	0.444E-07	23.523	-0.945E-09	0.205E-07
4.688	0.377E-04	0.387E-04						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 3.011 TO 4.014 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.325E-05	0.214E-04	4.885	-0.603E-06	-0.490E-06	9.890	-0.132E-08	0.166E-07
0.910	0.276E-05	0.117E-04	5.094	-0.454E-06	-0.364E-06	10.191	-0.192E-08	0.170E-07
1.011	0.615E-05	0.106E-04	5.287	-0.326E-06	-0.255E-06	10.586	-0.284E-08	0.178E-07
1.112	0.959E-05	0.157E-04	5.494	-0.217E-06	-0.160E-06	11.004	-0.326E-08	0.184E-07
1.211	0.146E-04	0.199E-04	5.711	-0.130E-06	-0.837E-07	11.400	-0.308E-08	0.197E-07
1.310	0.166E-04	0.225E-04	5.908	-0.759E-07	-0.387E-07	11.772	-0.285E-08	0.208E-07
1.406	0.182E-04	0.240E-04	6.116	-0.416E-07	-0.873E-08	12.163	-0.308E-08	0.219E-07
1.505	0.232E-04	0.288E-04	6.297	-0.236E-07	0.525E-08	12.574	-0.425E-08	0.219E-07
1.604	0.320E-04	0.374E-04	6.487	-0.118E-07	0.117E-07	13.006	-0.563E-08	0.211E-07
1.704	0.435E-04	0.492E-04	6.685	-0.508E-08	0.134E-07	13.401	-0.627E-08	0.208E-07
1.813	0.583E-04	0.640E-04	6.894	-0.222E-08	0.139E-07	13.818	-0.611E-08	0.226E-07
1.933	0.735E-04	0.788E-04	7.110	0.329E-08	0.148E-07	14.251	-0.515E-08	0.253E-07
2.095	0.834E-04	0.889E-04	7.292	-0.496E-08	0.146E-07	14.640	-0.413E-08	0.268E-07
2.291	0.901E-04	0.948E-04	7.481	-0.633E-08	0.158E-07	15.046	-0.313E-08	0.253E-07
2.490	0.103E-03	0.107E-03	7.677	-0.705E-08	0.164E-07	15.396	-0.228E-08	0.291E-07
2.694	0.109E-03	0.112E-03	7.881	-0.709E-08	0.171E-07	15.761	-0.166E-08	0.303E-07
2.902	0.105E-03	0.107E-03	8.093	-0.656E-08	0.171E-07	16.375	-0.992E-09	0.329E-07
3.109	0.949E-04	0.966E-04	8.314	-0.565E-08	0.163E-07	17.190	-0.164E-08	0.314E-07
3.310	0.770E-04	0.784E-04	8.515	-0.469E-08	0.155E-07	17.973	-0.243E-08	0.320E-07
3.501	0.534E-04	0.546E-04	8.693	-0.479E-08	0.153E-07	18.718	-0.246E-08	0.346E-07
3.693	0.306E-04	0.315E-04	8.877	-0.351E-08	0.154E-07	19.511	-0.111E-08	0.322E-07
3.901	0.132E-04	0.138E-04	9.099	-0.212E-08	0.159E-07	20.469	-0.517E-09	0.335E-07
4.107	0.412E-05	0.452E-05	9.329	-0.143E-08	0.160E-07	21.493	-0.430E-11	0.301E-07
4.309	0.505E-06	0.773E-06	9.499	-0.118E-08	0.164E-07	22.473	0.802E-10	0.251E-07
4.503	-0.536E-06	-0.353E-06	9.674	-0.118E-08	0.165E-07	23.523	0.560E-09	0.239E-07
4.688	-0.695E-06	-0.554E-06						

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 1.987 TO 3.011 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.425E-05	0.550E-04	4.885	0.158E-07	0.362E-07	9.890	-0.405E-08	0.444E-08
0.910	0.198E-04	0.412E-04	5.094	0.575E-08	0.210E-07	10.191	-0.401E-08	0.496E-08
1.011	0.342E-04	0.445E-04	5.287	-0.138E-08	0.107E-07	10.586	-0.379E-08	0.600E-08
1.112	0.463E-04	0.606E-04	5.494	-0.578E-08	0.552E-08	11.004	-0.303E-08	0.619E-08
1.211	0.639E-04	0.758E-04	5.711	-0.674E-08	0.375E-08	11.400	-0.152E-08	0.676E-08
1.310	0.842E-04	0.971E-04	5.908	-0.497E-08	0.346E-08	11.772	-0.716E-09	0.632E-08
1.406	0.110E-03	0.123E-03	6.116	-0.148E-08	0.448E-08	12.163	-0.935E-09	0.583E-08
1.505	0.147E-03	0.158E-03	6.297	-0.237E-08	0.556E-08	12.574	-0.947E-09	0.447E-08
1.604	0.195E-03	0.205E-03	6.487	-0.167E-08	0.644E-08	13.006	-0.134E-08	0.695E-08
1.704	0.247E-03	0.256E-03	6.685	-0.212E-08	0.659E-08	13.401	-0.150E-08	0.744E-08
1.813	0.291E-03	0.298E-03	6.894	-0.243E-08	0.625E-08	13.818	-0.139E-08	0.633E-08
1.933	0.303E-03	0.308E-03	7.110	-0.287E-08	0.556E-08	14.251	-0.112E-08	0.665E-08
2.095	0.259E-03	0.264E-03	7.292	-0.305E-08	0.511E-08	14.640	-0.103E-08	0.709E-08
2.291	0.168E-03	0.171E-03	7.481	-0.304E-08	0.536E-08	15.046	-0.120E-08	0.905E-08
2.490	0.888E-04	0.902E-04	7.677	-0.293E-08	0.528E-08	15.396	-0.141E-08	0.914E-08
2.694	0.377E-04	0.384E-04	7.881	-0.284E-08	0.526E-08	15.761	-0.154E-08	0.870E-08
2.902	0.114E-04	0.114E-04	8.093	-0.298E-08	0.520E-08	16.375	-0.151E-08	0.728E-08
3.109	0.859E-06	0.117E-05	8.314	-0.285E-08	0.515E-08	17.190	-0.119E-08	0.874E-08
3.310	-0.184E-05	-0.164E-05	8.515	-0.259E-08	0.491E-08	17.973	-0.167E-08	0.885E-08
3.501	-0.176E-05	-0.162E-05	8.693	-0.429E-08	0.479E-08	18.718	-0.186E-08	0.841E-08
3.693	-0.113E-05	-0.103E-05	8.877	-0.208E-08	0.394E-08	19.511	-0.169E-08	0.839E-08
3.901	-0.564E-06	-0.493E-06	9.099	-0.221E-08	0.340E-08	20.469	-0.199E-08	0.103E-07
4.107	-0.225E-06	-0.168E-06	9.329	-0.261E-08	0.340E-08	21.493	-0.225E-08	0.108E-07
4.309	-0.572E-07	-0.152E-07	9.499	-0.301E-08	0.370E-08	22.473	-0.219E-08	0.110E-07
4.503	0.670E-08	0.388E-07	9.674	-0.345E-08	0.407E-08	23.523	-0.188E-08	0.971E-08
4.688	0.203E-07	0.459E-07						

TABLE 6. CONTINUED

APPROXIMATE SCATTERING ANGLE = 90 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 1.002 TO 1.987 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.207E-03	0.327E-03	4.885	0.103E-07	0.204E-07	9.890	-0.143E-08	0.670E-09
0.910	0.388E-03	0.431E-03	5.094	0.110E-07	0.199E-07	10.191	-0.115E-08	0.641E-09
1.011	0.492E-03	0.509E-03	5.287	0.730E-08	0.160E-07	10.586	-0.701E-09	0.721E-09
1.112	0.510E-03	0.524E-03	5.494	0.218E-08	0.105E-07	11.000	-0.327E-09	0.834E-09
1.211	0.484E-03	0.495E-03	5.711	-0.135E-08	0.553E-08	11.400	-0.228E-09	0.968E-09
1.310	0.439E-03	0.448E-03	5.908	-0.231E-08	0.313E-08	11.772	-0.216E-09	0.106E-08
1.406	0.386E-03	0.393E-03	6.116	-0.204E-08	0.314E-08	12.163	-0.218E-09	0.119E-08
1.505	0.322E-03	0.327E-03	6.297	-0.153E-08	0.395E-08	12.574	-0.208E-09	0.133E-08
1.604	0.249E-03	0.253E-03	6.487	-0.106E-08	0.467E-08	13.006	-0.179E-09	0.152E-08
1.704	0.175E-03	0.177E-03	6.685	-0.692E-09	0.487E-08	13.401	-0.137E-09	0.167E-08
1.813	0.104E-03	0.105E-03	6.894	-0.311E-09	0.497E-08	13.818	-0.844E-10	0.177E-08
1.933	0.496E-04	0.503E-04	7.110	0.381E-09	0.512E-08	14.251	-0.305E-10	0.188E-08
2.095	0.129E-04	0.133E-04	7.292	0.797E-09	0.568E-08	14.640	0.385E-11	0.193E-08
2.291	0.129E-06	0.400E-06	7.481	0.139E-08	0.644E-08	15.046	0.107E-10	0.196E-08
2.490	-0.137E-05	-0.124E-05	7.677	0.205E-08	0.696E-08	15.396	-0.118E-10	0.195E-08
2.694	-0.802E-06	-0.724E-06	7.881	0.232E-08	0.705E-08	15.761	-0.613E-10	0.188E-08
2.902	-0.303E-06	-0.246E-06	8.093	0.238E-08	0.652E-08	16.375	-0.185E-09	0.170E-08
3.109	-0.849E-07	-0.485E-07	8.314	0.184E-08	0.552E-08	17.190	-0.340E-09	0.156E-08
3.310	-0.663E-08	0.190E-07	8.515	0.116E-08	0.463E-08	17.973	-0.430E-09	0.146E-08
3.501	0.224E-07	0.445E-07	8.693	0.426E-09	0.367E-08	18.718	-0.449E-09	0.162E-08
3.693	0.278E-07	0.477E-07	8.877	-0.265E-09	0.289E-08	19.511	-0.379E-09	0.169E-08
3.901	0.181E-07	0.347E-07	9.099	-0.942E-09	0.204E-08	20.469	-0.183E-09	0.209E-08
4.107	0.634E-08	0.211E-07	9.329	-0.133E-08	0.133E-08	21.493	0.508E-10	0.258E-08
4.309	0.946E-09	0.145E-07	9.499	-0.147E-08	0.962E-09	22.473	0.213E-09	0.272E-08
4.503	0.227E-08	0.147E-07	9.674	-0.151E-08	0.770E-09	23.523	0.230E-09	0.257E-08
4.688	0.613E-08	0.179E-07						

TABLE 7.

SECONDARY NEUTRON SPECTRA FOR A SCATTERING ANGLE OF APPROXIMATELY 135 DEGREES. THE VALUES GIVEN ARE THE LOWER AND UPPER LIMITS OF THE 68% CONFIDENCE INTERVAL FOR THE FLUX AT THE DETECTOR POSITION. THEY REPRESENT THE SECONDARY FLUX AS AVERAGED OVER THE AREA OF THE PROJECTION OF THE DETECTOR IN THE DIRECTION OF A POINT ON THE SCATTERING RING. THE DATA ARE NORMALIZED TO UNIT INCIDENT NEUTRON.

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 17.495 TO 19.965 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.539E-05	0.915E-05	4.885	0.116E-05	0.311E-05	9.890	0.202E-05	0.375E-05
0.910	0.513E-05	0.113E-04	5.054	0.104E-05	0.290E-05	10.191	0.136E-05	0.315E-05
1.011	0.971E-05	0.181E-04	5.287	0.998E-06	0.283E-05	10.586	0.671E-06	0.284E-05
1.112	0.562E-05	0.108E-04	5.494	0.103E-05	0.289E-05	11.004	0.536E-06	0.231E-05
1.211	0.234E-05	0.695E-05	5.711	0.106E-05	0.291E-05	11.400	0.620E-06	0.255E-05
1.310	0.215E-05	0.672E-05	5.908	0.130E-05	0.301E-05	11.772	0.959E-06	0.276E-05
1.406	0.395E-05	0.815E-05	6.116	0.126E-05	0.313E-05	12.163	0.131E-05	0.293E-05
1.505	0.558E-05	0.977E-05	6.297	0.110E-05	0.295E-05	12.574	0.152E-05	0.312E-05
1.604	0.695E-05	0.108E-04	6.487	0.741E-06	0.253E-05	13.006	0.182E-05	0.338E-05
1.704	0.744E-05	0.110E-04	6.685	0.357E-06	0.221E-05	13.401	0.217E-05	0.350E-05
1.813	0.672E-05	0.997E-05	6.894	0.236E-07	0.200E-05	13.818	0.236E-05	0.371E-05
1.933	0.505E-05	0.799E-05	7.110	0.870E-07	0.198E-05	14.251	0.245E-05	0.381E-05
2.095	0.374E-05	0.696E-05	7.292	0.116E-06	0.179E-05	14.640	0.234E-05	0.365E-05
2.291	0.415E-05	0.736E-05	7.481	-0.687E-07	0.159E-05	15.046	0.200E-05	0.298E-05
2.490	0.282E-05	0.591E-05	7.677	-0.377E-06	0.142E-05	15.396	0.157E-05	0.253E-05
2.694	0.207E-05	0.497E-05	7.881	-0.638E-06	0.120E-05	15.761	0.109E-05	0.198E-05
2.902	0.311E-05	0.610E-05	8.093	-0.792E-06	0.113E-05	16.375	0.363E-06	0.113E-05
3.109	0.392E-05	0.654E-05	8.314	-0.707E-06	0.116E-05	17.190	-0.318E-06	0.417E-06
3.310	0.232E-05	0.504E-05	8.515	-0.408E-06	0.138E-05	17.973	-0.505E-06	0.164E-06
3.501	0.333E-06	0.288E-05	8.693	0.395E-07	0.187E-05	18.718	-0.497E-06	0.138E-06
3.693	-0.136E-06	0.231E-05	8.877	0.634E-06	0.239E-05	19.511	-0.361E-06	0.153E-06
3.901	0.628E-06	0.318E-05	9.099	0.139E-05	0.303E-05	20.469	-0.220E-06	0.218E-06
4.107	0.176E-05	0.415E-05	9.329	0.201E-05	0.358E-05	21.493	-0.766E-07	0.165E-06
4.309	0.238E-05	0.451E-05	9.499	0.214E-05	0.380E-05	22.473	0.721E-09	0.132E-06
4.503	0.212E-05	0.427E-05	9.674	0.217E-05	0.391E-05	23.523	-0.941E-08	0.143E-06
4.688	0.154E-05	0.369E-05						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 15.041 TO 17.495 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.139E-05	0.189E-04	4.885	-0.609E-06	0.925E-06	9.890	-0.314E-07	0.182E-05
0.910	0.461E-05	0.102E-04	5.054	-0.120E-05	0.391E-06	10.191	-0.641E-07	0.137E-05
1.011	0.605E-06	0.953E-05	5.287	-0.870E-06	0.890E-06	10.586	0.568E-06	0.206E-05
1.112	0.665E-05	0.112E-04	5.494	-0.194E-07	0.112E-05	11.004	0.175E-05	0.319E-05
1.211	0.753E-05	0.109E-04	5.711	0.371E-06	0.221E-05	11.400	0.267E-05	0.423E-05
1.310	0.731E-05	0.108E-04	5.908	0.447E-06	0.211E-05	11.772	0.317E-05	0.474E-05
1.406	0.738E-05	0.109E-04	6.116	0.382E-06	0.209E-05	12.163	0.326E-05	0.463E-05
1.505	0.725E-05	0.105E-04	6.297	0.531E-06	0.224E-05	12.574	0.279E-05	0.392E-05
1.604	0.579E-05	0.204E-04	6.487	0.861E-06	0.257E-05	13.006	0.194E-05	0.298E-05
1.704	0.462E-05	0.750E-05	6.685	0.104E-05	0.280E-05	13.401	0.117E-06	0.700E-05
1.813	0.800E-05	0.748E-05	6.894	0.981E-06	0.280E-05	13.818	0.455E-06	0.127E-05
1.933	0.612E-05	0.851E-05	7.110	0.958E-06	0.260E-05	14.251	0.187E-06	0.749E-06
2.095	0.468E-05	0.906E-05	7.292	0.118E-05	0.279E-05	14.640	-0.418E-07	0.478E-06
2.291	0.506E-05	0.785E-05	7.481	0.154E-05	0.311E-05	15.046	-0.191E-06	0.356E-06
2.490	0.427E-05	0.645E-05	7.677	0.209E-05	0.382E-05	15.396	-0.205E-06	0.316E-06
2.694	0.312E-05	0.519E-05	7.881	0.271E-05	0.450E-05	15.761	-0.166E-06	0.303E-06
2.902	0.130E-05	0.364E-05	8.093	0.325E-05	0.499E-05	16.375	-0.107E-06	0.278E-06
3.109	0.788E-06	0.276E-05	8.314	0.348E-05	0.515E-05	17.190	-0.105E-06	0.207E-06
3.310	0.653E-06	0.295E-05	8.515	0.338E-05	0.500E-05	17.973	-0.108E-06	0.148E-06
3.501	0.129E-05	0.331E-05	8.693	0.308E-05	0.468E-05	18.718	-0.881E-07	0.851E-07
3.693	0.236E-05	0.430E-05	8.877	0.265E-05	0.415E-05	19.511	-0.578E-07	0.657E-07
3.901	0.267E-05	0.486E-05	9.099	0.194E-05	0.338E-05	20.469	-0.310E-07	0.428E-07
4.107	0.237E-05	0.445E-05	9.329	0.113E-05	0.260E-05	21.493	-0.116E-07	0.393E-07
4.309	0.107E-05	0.174E-05	9.499	0.658E-06	0.208E-05	22.473	-0.161E-08	0.418E-07
4.503	0.143E-05	0.320E-05	9.674	0.194E-06	0.172E-05	23.523	-0.714E-09	0.410E-07
4.688	0.543E-06	0.216E-05						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 12.536 TO 15.041 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.799E-06	0.113E-04	4.885	-0.249E-06	0.182E-05	9.890	0.770E-05	0.909E-05
0.910	0.533E-05	0.108E-04	5.054	0.522E-06	0.272E-05	10.191	0.837E-05	0.970E-05
1.011	0.579E-05	0.931E-05	5.287	0.183E-05	0.419E-05	10.586	0.758E-05	0.872E-05
1.112	0.606E-05	0.107E-04	5.494	0.379E-05	0.588E-05	11.004	0.535E-05	0.637E-05
1.211	0.819E-05	0.113E-04	5.711	0.537E-05	0.759E-05	11.400	0.302E-05	0.401E-05
1.310	0.688E-05	0.102E-04	5.908	0.657E-05	0.872E-05	11.772	0.142E-05	0.236E-05
1.406	0.488E-05	0.805E-05	6.116	0.728E-05	0.936E-05	12.163	0.443E-06	0.132E-05
1.505	0.380E-05	0.664E-05	6.297	0.737E-05	0.944E-05	12.574	-0.299E-08	0.757E-06
1.604	0.194E-05	0.651E-05	6.487	0.690E-05	0.919E-05	13.006	-0.156E-06	0.506E-06
1.704	0.493E-05	0.739E-05	6.685	0.810E-05	0.894E-05	13.401	-0.196E-06	0.376E-06
1.813	0.554E-05	0.779E-05	6.894	0.531E-05	0.749E-05	13.818	-0.211E-06	0.285E-06
1.933	0.529E-05	0.751E-05	7.110	0.445E-05	0.637E-05	14.251	-0.232E-06	0.217E-06
2.095	0.452E-05	0.704E-05	7.292	0.360E-05	0.530E-05	14.640	-0.226E-06	0.167E-06
2.291	0.442E-05	0.662E-05	7.481	0.256E-05	0.414E-05	15.046	-0.198E-06	0.125E-06
2.490	0.391E-05	0.629E-05	7.677	0.153E-05	0.328E-05	15.396	-0.170E-06	0.103E-06
2.694	0.276E-05	0.501E-05	7.881	0.681E-06	0.259E-05	15.761	-0.138E-06	0.850E-07
2.902	0.426E-06	0.298E-05	8.093	0.400E-06	0.226E-05	16.375	-0.930E-07	0.609E-07
3.109	-0.402E-06	0.163E-05	8.314	0.591E-06	0.233E-05	17.190	-0.506E-07	0.381E-07
3.310	-0.550E-06	0.204E-05	8.515	0.921E-06	0.265E-05	17.973	-0.284E-07	0.187E-07
3.501	-0.167E-06	0.227E-05	8.693	0.150E-05	0.313E-05	18.718	-0.178E-07	0.121E-07
3.693	0.666E-06	0.287E-05	8.877	0.214E-05	0.371E-05	19.511	-0.140E-07	0.109E-07
3.901	0.716E-06	0.349E-05	9.099	0.314E-05	0.464E-05	20.469	-0.143E-07	0.109E-07
4.107	0.352E-06	0.304E-05	9.329	0.445E-05	0.591E-05	21.493	-0.179E-07	0.115E-07
4.309	0.115E-06	0.209E-05	9.499	0.552E-05	0.697E-05	22.473	-0.196E-07	0.136E-07
4.503	-0.405E-06	0.192E-05	9.674	0.659E-05	0.804E-05	23.523	-0.167E-07	0.168E-07
4.688	-0.441E-06	0.171E-05						

TABLE 7. CONTINUED

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 9.995 TO 12.536 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.632E-05	0.144E-04	4.885	0.897E-05	0.107E-04	9.890	0.253E-05	0.327E-05
0.910	0.746E-05	0.110E-04	5.094	0.700E-05	0.868E-05	10.191	0.121E-05	0.188E-05
1.011	0.628E-05	0.813E-05	5.287	0.504E-05	0.690E-05	10.586	0.254E-06	0.820E-06
1.112	0.551E-05	0.787E-05	5.494	0.333E-05	0.505E-05	11.004	-0.112E-06	0.355E-06
1.211	0.648E-05	0.841E-05	5.711	0.163E-05	0.324E-05	11.400	-0.208E-06	0.195E-06
1.310	0.640E-05	0.822E-05	5.908	0.593E-06	0.209E-05	11.772	-0.235E-06	0.124E-06
1.406	0.537E-05	0.705E-05	6.116	-0.315E-07	0.163E-05	12.163	-0.241E-06	0.459E-07
1.505	0.406E-05	0.585E-05	6.297	-0.232E-07	0.177E-05	12.574	-0.238E-06	-0.727E-08
1.604	0.299E-05	0.470E-05	6.487	0.494E-06	0.227E-05	13.006	-0.212E-06	-0.408E-07
1.704	0.172E-05	0.371E-05	6.685	0.119E-05	0.285E-05	13.403	-0.168E-06	-0.955E-07
1.813	0.103E-05	0.262E-05	6.894	0.187E-05	0.339E-05	13.818	-0.118E-06	-0.329E-07
1.933	0.797E-06	0.270E-05	7.110	0.275E-05	0.396E-05	14.251	-0.737E-07	-0.174E-07
2.095	0.132E-05	0.305E-05	7.292	0.365E-05	0.475E-05	14.640	-0.467E-07	-0.508E-08
2.291	0.125E-05	0.305E-05	7.481	0.482E-05	0.595E-05	15.046	-0.289E-07	0.538E-08
2.490	-0.102E-07	0.221E-05	7.677	0.623E-05	0.739E-05	15.396	-0.199E-07	0.113E-07
2.694	0.653E-07	0.223E-05	7.881	0.767E-05	0.879E-05	15.761	-0.141E-07	0.149E-07
2.902	0.637E-06	0.286E-05	8.093	0.889E-05	0.995E-05	16.375	-0.821E-08	0.138E-07
3.109	0.194E-05	0.398E-05	8.314	0.968E-05	0.106E-04	17.190	-0.343E-08	0.108E-07
3.310	0.299E-05	0.519E-05	8.515	0.978E-05	0.107E-04	17.973	-0.949E-09	0.105E-07
3.501	0.345E-05	0.587E-05	8.693	0.952E-05	0.104E-04	18.718	-0.108E-08	0.123E-07
3.693	0.521E-05	0.706E-05	8.877	0.889E-05	0.974E-05	19.511	-0.100E-08	0.134E-07
3.901	0.819E-05	0.106E-04	9.099	0.767E-05	0.849E-05	20.469	0.581E-10	0.108E-07
4.107	0.113E-04	0.136E-04	9.329	0.113E-05	0.700E-05	21.493	-0.321E-09	0.160E-07
4.309	0.129E-04	0.105E-04	9.499	0.494E-05	0.579E-05	22.473	-0.107E-08	0.166E-07
4.503	0.124E-04	0.139E-04	9.674	0.377E-05	0.457E-05	23.523	-0.104E-08	0.177E-07
4.688	0.108E-04	0.125E-04						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 8.974 TO 9.995 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.286E-06	0.916E-05	4.885	-0.191E-05	0.844E-07	9.890	-0.292E-06	0.215E-06
0.910	0.332E-05	0.692E-05	5.094	-0.918E-06	0.106E-05	10.191	-0.225E-06	0.187E-06
1.011	0.456E-05	0.673E-05	5.287	-0.169E-06	0.194E-05	10.586	-0.119E-06	0.166E-06
1.112	0.322E-05	0.547E-05	5.494	0.366E-06	0.236E-05	11.004	-0.244E-07	0.169E-06
1.211	0.126E-05	0.357E-05	5.711	0.806E-06	0.251E-05	11.400	0.898E-09	0.165E-06
1.310	0.699E-06	0.296E-05	5.908	0.167E-05	0.356E-05	11.772	0.630E-08	0.158E-06
1.406	0.135E-05	0.355E-05	6.116	0.364E-05	0.589E-05	12.163	0.881E-08	0.134E-06
1.505	0.201E-05	0.438E-05	6.297	0.683E-06	0.894E-05	12.574	0.124E-08	0.813E-07
1.604	0.230E-05	0.453E-05	6.487	0.106E-04	0.127E-04	13.006	-0.274E-07	0.460E-07
1.704	0.207E-05	0.451E-05	6.685	0.143E-04	0.162E-04	13.403	-0.542E-07	0.308E-07
1.813	0.215E-05	0.438E-05	6.894	0.164E-04	0.182E-04	13.818	-0.627E-07	0.254E-07
1.933	0.243E-05	0.484E-05	7.110	0.163E-04	0.178E-04	14.251	-0.528E-07	0.225E-07
2.095	0.199E-05	0.479E-05	7.292	0.146E-04	0.159E-04	14.640	-0.398E-07	0.209E-07
2.291	0.195E-05	0.451E-05	7.481	0.120E-04	0.132E-04	15.046	-0.259E-07	0.225E-07
2.490	0.439E-05	0.730E-05	7.677	0.917E-05	0.104E-04	15.396	-0.158E-07	0.273E-07
2.694	0.972E-05	0.123E-04	7.881	0.656E-05	0.776E-05	15.761	-0.787E-08	0.319E-07
2.902	0.172E-04	0.200E-04	8.093	0.447E-05	0.557E-05	16.375	-0.260E-08	0.381E-07
3.109	0.220E-04	0.244E-04	8.314	0.290E-05	0.383E-05	17.190	-0.136E-08	0.442E-07
3.310	0.212E-04	0.236E-04	8.515	0.185E-05	0.268E-05	17.973	-0.729E-09	0.480E-07
3.501	0.172E-04	0.199E-04	8.693	0.115E-05	0.188E-05	18.718	-0.271E-09	0.503E-07
3.693	0.127E-04	0.147E-04	8.877	0.604E-06	0.131E-05	19.511	0.259E-09	0.569E-07
3.901	0.775E-05	0.100E-04	9.099	0.157E-06	0.817E-06	20.469	0.475E-09	0.616E-07
4.107	0.376E-05	0.598E-05	9.329	-0.118E-06	0.490E-06	21.493	-0.713E-09	0.617E-07
4.309	0.892E-06	0.258E-05	9.499	-0.224E-06	0.350E-06	22.473	-0.308E-08	0.680E-07
4.503	-0.120E-05	0.579E-06	9.674	-0.299E-06	0.263E-06	23.523	-0.483E-08	0.663E-07
4.688	-0.216E-05	-0.130E-06						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 7.996 TO 8.974 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.197E-05	0.908E-05	4.885	0.817E-06	0.297E-05	9.890	-0.257E-06	0.173E-07
0.910	0.132E-05	0.558E-05	5.094	0.120E-05	0.325E-05	10.191	-0.186E-06	0.563E-07
1.011	0.298E-05	0.556E-05	5.287	0.250E-05	0.466E-05	10.586	-0.199E-07	0.122E-06
1.112	0.332E-05	0.617E-05	5.494	0.592E-05	0.785E-05	11.004	0.615E-07	0.167E-06
1.211	0.408E-05	0.684E-05	5.711	0.112E-04	0.129E-04	11.400	0.722E-07	0.163E-06
1.310	0.386E-05	0.565E-05	5.908	0.165E-04	0.184E-04	11.772	0.532E-07	0.132E-06
1.406	0.246E-05	0.516E-05	6.116	0.211E-04	0.233E-04	12.163	0.260E-07	0.929E-07
1.505	0.133E-05	0.419E-05	6.297	0.233E-04	0.255E-04	12.574	0.413E-08	0.613E-07
1.604	0.147E-05	0.418E-05	6.487	0.233E-04	0.251E-04	13.006	-0.966E-08	0.956E-07
1.704	0.306E-05	0.598E-05	6.685	0.207E-04	0.224E-04	13.403	-0.145E-07	0.315E-07
1.813	0.696E-05	0.958E-05	6.894	0.161E-04	0.177E-04	13.818	-0.128E-07	0.266E-07
1.933	0.118E-04	0.145E-04	7.110	0.109E-04	0.123E-04	14.251	-0.891E-08	0.238E-07
2.095	0.211E-04	0.241E-04	7.292	0.717E-05	0.825E-05	14.640	-0.611E-08	0.441E-07
2.291	0.356E-04	0.381E-04	7.481	0.413E-05	0.513E-05	15.046	-0.433E-08	0.232E-07
2.490	0.410E-04	0.439E-04	7.677	0.199E-05	0.295E-05	15.396	-0.410E-08	0.245E-07
2.694	0.354E-04	0.376E-04	7.881	0.709E-06	0.164E-05	15.761	-0.477E-08	0.257E-07
2.902	0.220E-04	0.243E-04	8.093	0.639E-07	0.948E-06	16.375	-0.563E-08	0.259E-07
3.109	0.990E-05	0.121E-04	8.314	-0.194E-06	0.590E-06	17.190	-0.552E-08	0.305E-07
3.310	0.303E-05	0.544E-05	8.515	-0.262E-06	0.427E-06	17.973	-0.519E-08	0.320E-07
3.501	0.652E-07	0.240E-05	8.693	-0.287E-06	0.329E-06	18.718	-0.536E-08	0.322E-07
3.693	-0.177E-05	0.291E-06	8.877	-0.332E-06	0.258E-06	19.511	-0.594E-08	0.325E-07
3.901	-0.312E-05	-0.362E-06	9.099	-0.364E-06	0.171E-06	20.469	-0.671E-08	0.375E-07
4.107	-0.251E-05	0.299E-07	9.329	-0.374E-06	0.872E-07	21.493	-0.647E-08	0.381E-07
4.309	-0.862E-06	0.100E-05	9.499	-0.362E-06	0.548E-07	22.473	-0.538E-08	0.424E-07
4.503	0.115E-06	0.216E-05	9.674	-0.329E-06	0.260E-07	23.523	-0.389E-08	0.466E-07
4.688	0.579E-06	0.281E-05						

TABLE 7. CONTINUED

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 6.995 TO 7.996 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.126E-05	0.119E-04	4.885	0.129E-04	0.155E-04	9.890	-0.226E-07	0.122E-06
0.910	0.351E-05	0.846E-05	5.094	0.290E-04	0.262E-04	10.191	-0.242E-07	0.906E-07
1.011	0.609E-05	0.934E-05	5.287	0.380E-04	0.402E-04	10.586	-0.163E-07	0.667E-07
1.112	0.667E-05	0.100E-04	5.494	0.524E-04	0.545E-04	11.004	-0.136E-08	0.598E-07
1.211	0.520E-05	0.829E-05	5.711	0.597E-04	0.615E-04	11.400	0.951E-08	0.592E-07
1.310	0.673E-05	0.969E-05	5.908	0.560E-04	0.576E-04	11.772	0.160E-07	0.571E-07
1.406	0.138E-04	0.166E-04	6.116	0.839E-04	0.456E-04	12.163	0.208E-07	0.548E-07
1.505	0.232E-04	0.262E-04	6.297	0.311E-04	0.326E-04	12.574	0.207E-07	0.467E-07
1.604	0.306E-04	0.334E-04	6.487	0.191E-04	0.204E-04	13.006	0.158E-07	0.360E-07
1.704	0.356E-04	0.385E-04	6.685	0.100E-04	0.112E-04	13.401	0.109E-07	0.273E-07
1.813	0.403E-04	0.427E-04	6.894	0.426E-05	0.532E-05	13.818	0.665E-08	0.189E-07
1.933	0.434E-04	0.461E-04	7.110	0.126E-05	0.217E-05	14.251	0.343E-08	0.116E-07
2.095	0.401E-04	0.429E-04	7.292	0.129E-06	0.869E-06	14.640	0.133E-08	0.683E-08
2.291	0.236E-04	0.260E-04	7.481	-0.381E-06	0.259E-06	15.046	-0.130E-10	0.393E-08
2.490	0.824E-05	0.109E-04	7.677	-0.582E-06	0.375E-07	15.396	-0.732E-09	0.272E-08
2.694	0.176E-05	0.405E-05	7.881	-0.603E-06	0.158E-07	15.761	-0.127E-08	0.202E-08
2.902	-0.890E-06	0.179E-05	8.093	-0.512E-06	0.512E-07	16.375	-0.181E-08	0.153E-08
3.109	-0.116E-05	0.176E-05	8.314	-0.348E-06	0.925E-07	17.190	-0.204E-08	0.146E-08
3.310	-0.510E-06	0.233E-05	8.515	-0.209E-06	0.131E-06	17.973	-0.214E-08	0.148E-08
3.501	-0.238E-06	0.266E-05	8.693	-0.117E-06	0.166E-06	18.718	-0.222E-08	0.149E-08
3.693	-0.343E-06	0.236E-05	8.877	-0.584E-07	0.202E-06	19.511	-0.230E-08	0.185E-08
3.901	-0.113E-05	0.267E-05	9.099	-0.174E-07	0.219E-06	20.469	-0.240E-08	0.210E-08
4.107	-0.636E-06	0.262E-05	9.329	-0.527E-08	0.203E-06	21.493	-0.242E-08	0.219E-08
4.309	0.115E-05	0.328E-05	9.499	-0.636E-08	0.181E-06	22.473	-0.222E-08	0.298E-08
4.602	0.332E-05	0.580E-05	9.674	-0.110E-07	0.154E-06	23.523	-0.169E-08	0.345E-08
4.688	0.681E-05	0.940E-05						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 5.997 TO 6.995 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.891E-05	0.221E-04	4.885	0.438E-04	0.455E-04	9.890	0.271E-06	0.348E-06
0.910	0.273E-04	0.321E-04	5.094	0.359E-04	0.375E-04	10.191	0.203E-06	0.262E-06
1.011	0.372E-04	0.394E-04	5.287	0.255E-04	0.268E-04	10.586	0.119E-06	0.166E-06
1.112	0.393E-04	0.417E-04	5.494	0.149E-04	0.161E-04	11.004	0.514E-07	0.875E-07
1.211	0.364E-04	0.384E-04	5.711	0.700E-05	0.807E-05	11.400	0.135E-07	0.394E-07
1.310	0.301E-04	0.321E-04	5.908	0.287E-05	0.375E-05	11.772	-0.334E-08	0.147E-07
1.406	0.230E-04	0.248E-04	6.116	0.652E-06	0.148E-05	12.163	-0.101E-07	0.398E-08
1.505	0.153E-04	0.171E-04	6.297	-0.162E-06	0.599E-06	12.574	-0.119E-07	0.238E-08
1.604	0.852E-05	0.104E-04	6.487	-0.478E-06	0.200E-06	13.006	-0.111E-07	0.515E-08
1.704	0.424E-05	0.624E-05	6.685	-0.577E-06	0.136E-07	13.401	-0.951E-08	0.816E-08
1.813	0.224E-05	0.399E-05	6.894	-0.611E-06	-0.964E-07	13.818	-0.884E-08	0.104E-07
1.933	0.189E-05	0.353E-05	7.110	-0.632E-06	-0.204E-06	14.251	-0.693E-08	0.108E-07
2.095	0.197E-05	0.424E-05	7.292	-0.627E-06	-0.269E-06	14.640	-0.531E-08	0.986E-08
2.291	0.341E-06	0.250E-05	7.481	-0.567E-06	-0.270E-06	15.046	-0.207E-08	0.924E-08
2.490	-0.256E-05	-0.507E-06	7.677	-0.429E-06	-0.181E-06	15.396	-0.289E-08	0.938E-08
2.694	-0.547E-06	0.153E-05	7.881	-0.235E-06	-0.414E-08	15.761	-0.219E-08	0.939E-08
2.902	0.242E-05	0.555E-05	8.093	-0.444E-08	0.206E-06	16.375	-0.221E-08	0.944E-08
3.109	0.288E-05	0.537E-05	8.314	0.206E-06	0.397E-06	17.190	-0.315E-08	0.101E-07
3.310	0.123E-05	0.360E-05	8.515	0.341E-06	0.507E-06	17.973	-0.332E-08	0.107E-07
3.501	0.174E-06	0.255E-05	8.693	0.410E-06	0.558E-06	18.718	-0.308E-08	0.113E-07
3.693	0.141E-05	0.379E-05	8.877	0.435E-06	0.570E-06	19.511	-0.289E-08	0.130E-07
3.901	0.534E-05	0.790E-05	9.099	0.426E-06	0.548E-06	20.469	-0.314E-08	0.130E-07
4.107	0.137E-04	0.159E-04	9.329	0.390E-06	0.700E-06	21.493	-0.401E-08	0.135E-07
4.309	0.260E-04	0.278E-04	9.499	0.356E-06	0.457E-06	22.473	-0.480E-08	0.158E-07
4.602	0.377E-04	0.394E-04	9.674	0.319E-06	0.408E-06	23.523	-0.455E-08	0.163E-07
4.688	0.443E-04	0.460E-04						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 5.011 TO 5.997 MEV.

NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.137E-04	0.225E-04	4.885	0.122E-04	0.133E-04	9.890	0.983E-07	0.135E-06
0.910	0.104E-04	0.136E-04	5.094	0.480E-05	0.577E-05	10.191	0.521E-07	0.807E-07
1.011	0.464E-05	0.639E-05	5.287	0.159E-05	0.238E-05	10.586	0.141E-07	0.350E-07
1.112	0.235E-05	0.428E-05	5.494	0.213E-06	0.888E-06	11.004	-0.471E-08	0.106E-07
1.211	0.238E-05	0.414E-05	5.711	-0.317E-06	0.274E-06	11.400	-0.104E-07	0.248E-08
1.310	0.179E-05	0.362E-05	5.908	-0.490E-06	0.504E-08	11.772	-0.105E-07	0.533E-09
1.406	0.862E-06	0.219E-05	6.116	-0.562E-06	-0.143E-06	12.163	-0.862E-08	0.477E-09
1.505	-0.575E-06	0.134E-05	6.297	-0.560E-06	-0.211E-06	12.574	-0.633E-08	0.921E-09
1.604	-0.102E-07	0.200E-05	6.487	-0.502E-06	-0.211E-08	13.006	-0.503E-08	0.168E-08
1.704	0.143E-05	0.348E-05	6.685	-0.414E-06	-0.166E-06	13.401	-0.469E-08	0.267E-08
1.813	0.264E-05	0.454E-05	6.894	-0.306E-06	-0.880E-07	13.818	-0.460E-08	0.353E-08
1.933	0.218E-05	0.423E-05	7.110	-0.196E-06	-0.934E-08	14.251	-0.461E-08	0.441E-08
2.095	0.248E-06	0.256E-05	7.292	-0.112E-06	0.500E-07	14.640	-0.462E-08	0.516E-08
2.291	0.196E-05	0.433E-05	7.481	-0.341E-07	0.116E-06	15.046	-0.461E-08	0.570E-08
2.490	0.391E-05	0.621E-05	7.677	0.398E-07	0.188E-06	15.396	-0.455E-08	0.588E-08
2.694	0.267E-05	0.489E-05	7.881	0.118E-06	0.265E-06	15.761	-0.442E-08	0.558E-08
2.902	0.250E-05	0.484E-05	8.093	0.198E-06	0.331E-06	16.375	-0.425E-08	0.601E-08
3.109	0.538E-05	0.707E-05	8.314	0.264E-06	0.379E-06	17.190	-0.410E-08	0.599E-08
3.310	0.103E-04	0.126E-04	8.515	0.299E-06	0.394E-06	17.973	-0.408E-08	0.654E-08
3.501	0.190E-04	0.212E-04	8.693	0.307E-06	0.396E-06	18.718	-0.404E-08	0.702E-08
3.693	0.341E-04	0.364E-04	8.877	0.294E-06	0.377E-06	19.511	-0.401E-08	0.826E-08
3.901	0.513E-04	0.534E-04	9.099	0.262E-06	0.333E-06	20.469	-0.390E-08	0.825E-08
4.107	0.587E-04	0.604E-04	9.329	0.215E-06	0.274E-06	21.493	-0.375E-08	0.842E-08
4.309	0.526E-04	0.541E-04	9.499	0.177E-06	0.229E-06	22.473	-0.355E-08	0.927E-08
4.602	0.386E-04	0.399E-04	9.674	0.140E-06	0.184E-06	23.523	-0.301E-08	0.932E-08
4.688	0.241E-04	0.253E-04						

TABLE 7. CONTINUED

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 3.997 TO 5.011 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.841E-05	0.301E-05	4.885	-0.525E-06	-0.216E-07	9.890	-0.310E-08	0.121E-07
0.910	-0.190E-05	0.220E-05	5.094	-0.535E-06	-0.142E-06	10.191	-0.378E-08	0.117E-07
1.011	0.428E-06	0.254E-05	5.287	-0.407E-06	-0.124E-06	10.586	-0.651E-08	0.123E-07
1.112	0.149E-05	0.407E-05	5.494	-0.285E-06	-0.825E-07	11.004	-0.722E-08	0.132E-07
1.211	0.319E-05	0.565E-05	5.711	-0.210E-06	-0.415E-07	11.400	-0.790E-08	0.143E-07
1.310	0.321E-05	0.578E-05	5.908	-0.141E-06	-0.742E-08	11.772	-0.752E-08	0.149E-07
1.406	0.222E-05	0.67E-05	6.116	-0.789E-07	0.235E-07	12.163	-0.651E-08	0.153E-07
1.505	0.207E-05	0.665E-05	6.297	-0.388E-07	0.411E-07	12.574	-0.611E-08	0.140E-07
1.604	0.318E-05	0.573E-05	6.487	-0.202E-07	0.524E-07	13.006	-0.393E-08	0.102E-07
1.704	0.515E-05	0.793E-05	6.685	-0.113E-07	0.501E-07	13.401	-0.252E-08	0.962E-08
1.813	0.822E-05	0.112E-04	6.894	-0.106E-07	0.450E-07	13.818	-0.203E-08	0.107E-07
1.933	0.979E-05	0.129E-04	7.110	-0.112E-07	0.375E-07	14.251	-0.203E-08	0.118E-07
2.095	0.767E-05	0.113E-04	7.292	-0.106E-07	0.313E-07	14.640	-0.215E-08	0.115E-07
2.291	0.999E-05	0.135E-04	7.481	-0.769E-08	0.254E-07	15.046	-0.238E-08	0.116E-07
2.490	0.186E-04	0.220E-04	7.677	-0.382E-08	0.213E-07	15.396	-0.259E-08	0.118E-07
2.694	0.371E-04	0.399E-04	7.881	-0.473E-09	0.198E-07	15.761	-0.270E-08	0.119E-07
2.902	0.741E-04	0.767E-04	8.093	0.979E-09	0.195E-07	16.375	-0.259E-08	0.126E-07
3.109	0.999E-04	0.102E-03	8.314	0.760E-09	0.201E-07	17.190	-0.261E-08	0.147E-07
3.310	0.942E-04	0.942E-04	8.515	-0.118E-09	0.207E-07	17.973	-0.267E-08	0.166E-07
3.501	0.738E-04	0.755E-04	8.693	-0.883E-09	0.207E-07	18.718	-0.245E-08	0.171E-07
3.693	0.502E-04	0.519E-04	8.877	-0.211E-08	0.199E-07	19.511	-0.164E-08	0.193E-07
3.901	0.278E-04	0.291E-04	9.099	-0.309E-08	0.184E-07	20.469	0.184E-09	0.199E-07
4.107	0.126E-04	0.137E-04	9.329	-0.359E-08	0.158E-07	21.493	-0.424E-09	0.231E-07
4.309	0.469E-05	0.552E-05	9.499	-0.365E-08	0.141E-07	22.473	-0.216E-08	0.265E-07
4.503	0.127E-05	0.197E-05	9.674	-0.342E-08	0.128E-07	23.523	-0.302E-08	0.258E-07
4.688	-0.775E-07	0.522E-06						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 3.005 TO 3.997 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.769E-05	0.239E-04	4.885	0.180E-08	0.116E-04	9.890	-0.102E-08	0.513E-08
0.910	0.104E-06	0.115E-04	5.094	0.159E-07	0.657E-07	10.191	-0.121E-08	0.533E-08
1.011	0.627E-05	0.120E-04	5.287	-0.449E-07	0.289E-07	10.586	-0.123E-08	0.569E-08
1.112	0.763E-05	0.156E-04	5.494	-0.469E-07	0.117E-07	11.004	-0.131E-08	0.621E-08
1.211	0.848E-05	0.155E-04	5.711	-0.301E-07	0.152E-07	11.400	-0.134E-08	0.636E-08
1.310	0.947E-05	0.174E-04	5.908	-0.107E-07	0.244E-07	11.772	-0.135E-08	0.659E-08
1.406	0.112E-04	0.191E-04	6.116	0.516E-08	0.309E-07	12.163	-0.135E-08	0.682E-08
1.505	0.117E-04	0.197E-04	6.297	0.111E-07	0.306E-07	12.574	-0.135E-08	0.687E-08
1.604	0.123E-04	0.205E-04	6.487	0.111E-07	0.269E-07	13.006	-0.143E-08	0.709E-08
1.704	0.150E-04	0.240E-04	6.685	0.785E-08	0.213E-07	13.401	-0.155E-08	0.691E-08
1.813	0.216E-04	0.311E-04	6.894	0.396E-08	0.148E-07	13.818	-0.166E-08	0.730E-08
1.933	0.349E-04	0.440E-04	7.110	0.876E-09	0.916E-08	14.251	-0.172E-08	0.760E-08
2.095	0.774E-04	0.872E-04	7.292	-0.103E-08	0.503E-08	14.640	-0.174E-08	0.806E-08
2.291	0.168E-03	0.196E-03	7.481	-0.260E-08	0.423E-08	15.046	-0.172E-08	0.848E-08
2.490	0.289E-03	0.296E-03	7.677	-0.365E-08	0.373E-08	15.396	-0.167E-08	0.874E-08
2.694	0.296E-03	0.300E-03	7.881	-0.407E-08	0.354E-08	15.761	-0.158E-08	0.907E-08
2.902	0.208E-03	0.211E-03	8.093	-0.395E-08	0.363E-08	16.375	-0.144E-08	0.878E-08
3.109	0.101E-03	0.102E-03	8.314	-0.343E-08	0.417E-08	17.190	-0.137E-08	0.905E-08
3.310	0.327E-04	0.339E-04	8.515	-0.270E-08	0.433E-08	17.973	-0.137E-08	0.101E-07
3.501	0.501E-05	0.592E-05	8.693	-0.215E-08	0.457E-08	18.718	-0.132E-08	0.989E-08
3.693	-0.317E-05	-0.241E-05	8.877	-0.153E-08	0.474E-08	19.511	-0.117E-08	0.114E-07
3.901	-0.374E-05	-0.318E-05	9.099	-0.984E-09	0.479E-08	20.469	-0.910E-09	0.988E-08
4.107	-0.232E-05	-0.192E-05	9.329	-0.826E-09	0.494E-08	21.493	-0.666E-09	0.105E-07
4.309	-0.105E-05	-0.782E-06	9.499	-0.853E-09	0.519E-08	22.473	-0.525E-09	0.105E-07
4.503	-0.385E-06	-0.151E-06	9.674	-0.928E-09	0.517E-08	23.523	-0.416E-09	0.704E-08
4.688	-0.624E-07	0.856E-07						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 2.002 TO 3.005 MEV.

NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLUX (N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	-0.106E-04	0.310E-04	4.885	0.627E-08	0.175E-07	9.890	-0.196E-08	0.360E-08
0.910	0.114E-04	0.266E-04	5.094	0.306E-08	0.119E-07	10.191	-0.149E-08	0.373E-08
1.011	0.160E-04	0.240E-04	5.287	0.167E-08	0.929E-08	10.586	-0.878E-09	0.400E-08
1.112	0.151E-04	0.260E-04	5.494	0.411E-09	0.767E-08	11.004	-0.524E-09	0.442E-08
1.211	0.267E-04	0.359E-04	5.711	-0.959E-09	0.578E-08	11.400	-0.483E-09	0.482E-08
1.310	0.571E-04	0.670E-04	5.908	-0.184E-08	0.413E-08	11.772	-0.552E-09	0.495E-08
1.406	0.111E-03	0.120E-03	6.116	-0.271E-08	0.288E-08	12.163	-0.708E-09	0.513E-08
1.505	0.176E-03	0.184E-03	6.297	-0.313E-08	0.236E-08	12.574	-0.101E-08	0.521E-08
1.604	0.225E-03	0.231E-03	6.487	-0.323E-08	0.202E-08	13.006	-0.134E-08	0.544E-08
1.704	0.243E-03	0.249E-03	6.685	-0.317E-08	0.184E-08	13.401	-0.147E-08	0.585E-08
1.813	0.236E-03	0.240E-03	6.894	-0.302E-08	0.199E-08	13.818	-0.141E-08	0.598E-08
1.933	0.210E-03	0.213E-03	7.110	-0.273E-08	0.232E-08	14.251	-0.132E-08	0.650E-08
2.095	0.158E-03	0.160E-03	7.292	-0.249E-08	0.241E-08	14.640	-0.139E-08	0.677E-08
2.291	0.447E-04	0.617E-04	7.481	-0.216E-08	0.337E-08	15.046	-0.158E-08	0.681E-08
2.490	0.234E-04	0.242E-04	7.677	-0.188E-08	0.350E-08	15.396	-0.171E-08	0.698E-08
2.694	0.813E-06	0.129E-05	7.881	-0.179E-08	0.357E-08	15.761	-0.175E-08	0.697E-08
2.902	-0.315E-05	-0.279E-05	8.093	-0.193E-08	0.353E-08	16.375	-0.168E-08	0.669E-08
3.109	-0.238E-05	-0.215E-05	8.314	-0.221E-08	0.383E-08	17.190	-0.168E-08	0.682E-08
3.310	-0.128E-05	-0.112E-05	8.515	-0.277E-08	0.454E-08	17.973	-0.174E-08	0.673E-08
3.501	-0.556E-06	-0.444E-06	8.693	-0.271E-08	0.510E-08	18.718	-0.163E-08	0.779E-08
3.693	-0.152E-06	-0.714E-07	8.877	-0.219E-08	0.544E-08	19.511	-0.109E-08	0.802E-08
3.901	0.289E-07	0.819E-07	9.099	-0.199E-08	0.553E-08	20.469	-0.150E-08	0.854E-08
4.107	0.609E-07	0.101E-06	9.329	-0.181E-08	0.533E-08	21.493	-0.163E-08	0.864E-08
4.309	0.432E-07	0.735E-07	9.499	-0.243E-08	0.393E-08	22.473	-0.854E-09	0.808E-08
4.503	0.243E-07	0.457E-07	9.674	-0.224E-08	0.369E-08	23.523	-0.359E-09	0.942E-08
4.688	0.126E-07	0.274E-07						

TABLE 7. CONTINUED

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 1.486 TO 2.002 MEV.

NEUT. ENERGY (MEV)	FLOX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.648E-05	0.626E-04	4.885	-0.257E-08	0.261E-08	9.890	-0.310E-08	0.152E-08
0.910	0.462E-04	0.658E-04	5.094	-0.272E-08	0.269E-08	10.191	-0.258E-08	0.162E-08
1.011	0.118E-03	0.127E-03	5.287	-0.311E-08	0.219E-08	10.586	-0.176E-08	0.210E-08
1.112	0.228E-03	0.235E-03	5.494	-0.362E-08	0.151E-08	11.004	-0.106E-08	0.293E-08
1.211	0.312E-03	0.318E-03	5.711	-0.368E-08	0.977E-09	11.400	-0.699E-09	0.371E-08
1.310	0.324E-03	0.329E-03	5.908	-0.390E-08	0.113E-08	11.772	-0.133E-09	0.423E-08
1.406	0.274E-03	0.278E-03	6.116	-0.275E-08	0.176E-08	12.163	-0.621E-10	0.451E-08
1.505	0.189E-03	0.192E-03	6.297	-0.195E-08	0.252E-08	12.574	-0.201E-09	0.452E-08
1.604	0.108E-03	0.109E-03	6.487	-0.135E-08	0.329E-08	13.006	-0.466E-09	0.445E-08
1.704	0.499E-04	0.509E-04	6.685	-0.964E-09	0.405E-08	13.401	-0.693E-09	0.412E-08
1.813	0.162E-04	0.169E-04	6.894	-0.734E-09	0.444E-08	13.818	-0.107E-08	0.393E-08
1.933	0.200E-05	0.250E-05	7.110	-0.647E-09	0.452E-08	14.251	-0.152E-08	0.374E-08
2.095	-0.184E-05	-0.150E-05	7.292	-0.554E-09	0.442E-08	14.640	-0.156E-08	0.306E-08
2.291	-0.115E-05	-0.960E-06	7.481	-0.392E-09	0.428E-08	15.046	-0.138E-08	0.259E-08
2.490	-0.319E-06	-0.200E-06	7.677	-0.537E-09	0.416E-08	15.396	-0.116E-08	0.233E-08
2.694	0.183E-07	0.919E-07	7.881	-0.725E-09	0.404E-08	15.761	-0.877E-09	0.215E-08
2.902	0.434E-07	0.101E-06	8.093	-0.109E-08	0.393E-08	16.375	-0.485E-09	0.198E-08
3.109	0.167E-07	0.587E-07	8.314	-0.152E-08	0.369E-08	17.190	-0.390E-09	0.206E-08
3.310	0.208E-07	0.535E-07	8.515	-0.190E-08	0.324E-08	17.973	-0.504E-09	0.224E-08
3.501	0.388E-07	0.623E-07	8.693	-0.228E-08	0.273E-08	18.718	-0.530E-09	0.254E-08
3.693	0.440E-07	0.623E-07	8.877	-0.252E-08	0.227E-08	19.511	-0.500E-09	0.267E-08
3.901	0.315E-07	0.722E-07	9.099	-0.294E-08	0.190E-08	20.469	-0.443E-09	0.301E-08
4.107	0.149E-07	0.266E-07	9.329	-0.304E-08	0.174E-08	21.493	-0.538E-09	0.330E-08
4.309	0.375E-08	0.117E-07	9.499	-0.322E-08	0.163E-08	22.473	-0.737E-09	0.355E-08
4.503	-0.114E-08	0.513E-08	9.674	-0.325E-08	0.159E-08	23.523	-0.876E-09	0.354E-08
4.688	-0.243E-08	0.297E-08						

APPROXIMATE SCATTERING ANGLE = 135 DEGREES.

INCIDENT NEUTRON ENERGY RANGE = 1.007 TO 1.486 MEV.

NEUT. ENERGY (MEV)	FLOX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CM**2/MEV)		NEUT. ENERGY (MEV)	FLOX(N/CM**2/MEV)	
	LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT		LOWER LIMIT	UPPER LIMIT
0.810	0.337E-03	0.389E-03	4.885	0.568E-08	0.156E-07	9.890	-0.262E-08	0.359E-09
0.910	0.447E-03	0.467E-03	5.094	0.828E-08	0.170E-07	10.191	-0.260E-08	0.263E-09
1.011	0.402E-03	0.408E-03	5.287	0.724E-08	0.149E-07	10.586	-0.222E-08	0.441E-09
1.112	0.265E-03	0.268E-03	5.494	0.401E-08	0.111E-07	11.004	-0.161E-08	0.820E-09
1.211	0.133E-03	0.135E-03	5.711	0.748E-10	0.666E-08	11.400	-0.102E-08	0.122E-08
1.310	0.518E-04	0.527E-04	5.908	-0.256E-08	0.272E-08	11.772	-0.583E-09	0.157E-08
1.406	0.162E-04	0.169E-04	6.116	-0.422E-08	0.279E-09	12.163	-0.295E-09	0.198E-08
1.505	0.329E-05	0.378E-05	6.297	-0.443E-08	-0.716E-09	12.574	-0.971E-10	0.236E-08
1.604	-0.183E-06	0.170E-06	6.487	-0.416E-08	-0.745E-09	13.006	0.622E-10	0.280E-08
1.704	-0.652E-06	-0.416E-06	6.685	-0.338E-08	-0.204E-09	13.401	0.179E-09	0.313E-08
1.813	-0.382E-06	-0.232E-06	6.894	-0.228E-08	0.931E-09	13.818	0.269E-09	0.333E-08
1.933	-0.113E-06	-0.123E-07	7.110	-0.108E-08	0.219E-08	14.251	0.321E-09	0.352E-08
2.095	0.359E-08	0.637E-07	7.292	-0.124E-09	0.310E-08	14.640	0.338E-09	0.354E-08
2.291	0.140E-07	0.544E-07	7.481	0.744E-09	0.430E-08	15.046	0.336E-09	0.337E-08
2.490	0.226E-07	0.492E-07	7.677	0.142E-08	0.514E-08	15.396	0.320E-09	0.356E-08
2.694	0.187E-07	0.363E-07	7.881	0.181E-08	0.574E-08	15.761	0.311E-09	0.352E-08
2.902	0.441E-08	0.221E-07	8.093	0.169E-08	0.597E-08	16.375	0.255E-09	0.337E-08
3.109	-0.681E-09	0.139E-07	8.314	0.141E-08	0.585E-08	17.190	0.991E-10	0.317E-08
3.310	0.723E-09	0.158E-07	8.515	0.905E-09	0.539E-08	17.973	-0.143E-09	0.284E-08
3.501	0.279E-08	0.174E-07	8.693	0.323E-09	0.476E-08	18.718	-0.391E-09	0.293E-08
3.693	0.105E-08	0.152E-07	8.877	-0.332E-09	0.395E-08	19.511	-0.591E-09	0.283E-08
3.901	-0.387E-08	0.744E-08	9.099	-0.110E-08	0.288E-08	20.469	-0.536E-09	0.308E-08
4.107	-0.892E-08	0.421E-08	9.329	-0.176E-08	0.182E-08	21.493	-0.448E-09	0.348E-08
4.309	-0.867E-08	0.303E-08	9.499	-0.215E-08	0.117E-08	22.473	-0.262E-09	0.395E-08
4.503	-0.439E-08	0.632E-08	9.674	-0.244E-08	0.680E-09	23.523	-0.846E-10	0.368E-08
4.688	0.123E-08	0.115E-07						

TABLE 8.

TOTAL SECONDARY NEUTRON FIELD, NORMALISED TO UNIT INCIDENT NEUTRON, AND AVERAGE SECONDARY ENERGY AS A FUNCTION OF INCIDENT NEUTRON ENERGY. THE VALUES GIVEN REPRESENT THE AVERAGES OVER THE PROJECTED AREA OF THE DETECTOR AS SEEN FROM THE SCATTERING RING. VALUES IN PARENTHESES ARE STANDARD DEVIATIONS. THE APPROXIMATE SCATTERING ANGLE IS 60 DEGREES. THE MINIMUM NEUTRON ENERGY SAMPLED IS 0.760 MEV.

INC. WT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY NEUTRON FLUX (N/CM**2)	AVERAGE ENERGY (MEV)	INC. WT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY NEUTRON FLUX (N/CM**2)	AVERAGE ENERGY (MEV)
1.652	0.089	0.312E-03 (0.474E-05)	0.160E 01 (0.243E-01)	6.241	0.109	0.150E-03 (0.515E-05)	0.360E 01 (0.132E 00)
1.750	0.108	0.306E-03 (0.428E-05)	0.148E 01 (0.233E-01)	6.352	0.112	0.163E-03 (0.490E-05)	0.399E 01 (0.128E 00)
1.852	0.094	0.286E-03 (0.433E-05)	0.159E 01 (0.268E-01)	6.447	0.077	0.156E-03 (0.557E-05)	0.403E 01 (0.154E 00)
1.949	0.102	0.284E-03 (0.409E-05)	0.166E 01 (0.263E-01)	6.544	0.118	0.131E-03 (0.423E-05)	0.419E 01 (0.144E 00)
2.048	0.096	0.284E-03 (0.419E-05)	0.173E 01 (0.280E-01)	6.643	0.080	0.108E-03 (0.453E-05)	0.419E 01 (0.188E 00)
2.147	0.103	0.273E-03 (0.397E-05)	0.180E 01 (0.287E-01)	6.745	0.123	0.112E-03 (0.357E-05)	0.455E 01 (0.156E 00)
2.246	0.094	0.256E-03 (0.411E-05)	0.187E 01 (0.326E-01)	6.848	0.084	0.121E-03 (0.431E-05)	0.459E 01 (0.177E 00)
2.343	0.100	0.256E-03 (0.398E-05)	0.196E 01 (0.330E-01)	6.955	0.129	0.129E-03 (0.339E-05)	0.477E 01 (0.136E 00)
2.447	0.107	0.254E-03 (0.393E-05)	0.204E 01 (0.341E-01)	7.063	0.088	0.139E-03 (0.426E-05)	0.498E 01 (0.165E 00)
2.548	0.095	0.248E-03 (0.435E-05)	0.212E 01 (0.400E-01)	7.152	0.090	0.165E-03 (0.463E-05)	0.492E 01 (0.149E 00)
2.646	0.100	0.241E-03 (0.445E-05)	0.221E 01 (0.438E-01)	7.243	0.091	0.204E-03 (0.493E-05)	0.523E 01 (0.137E 00)
2.750	0.106	0.254E-03 (0.505E-05)	0.225E 01 (0.479E-01)	7.335	0.093	0.248E-03 (0.528E-05)	0.554E 01 (0.128E 00)
2.848	0.090	0.267E-03 (0.561E-05)	0.238E 01 (0.524E-01)	7.429	0.095	0.283E-03 (0.485E-05)	0.563E 01 (0.123E 00)
2.952	0.118	0.270E-03 (0.473E-05)	0.246E 01 (0.459E-01)	7.549	0.146	0.173E-03 (0.357E-05)	0.518E 01 (0.117E 00)
3.061	0.100	0.197E-03 (0.443E-05)	0.254E 01 (0.608E-01)	7.672	0.100	0.142E-03 (0.417E-05)	0.493E 01 (0.158E 00)
3.150	0.078	0.215E-03 (0.510E-05)	0.258E 01 (0.650E-01)	7.773	0.102	0.139E-03 (0.410E-05)	0.487E 01 (0.157E 00)
3.244	0.109	0.238E-03 (0.435E-05)	0.277E 01 (0.545E-01)	7.850	0.052	0.141E-03 (0.590E-05)	0.502E 01 (0.228E 00)
3.356	0.115	0.260E-03 (0.428E-05)	0.282E 01 (0.492E-01)	7.928	0.105	0.158E-03 (0.406E-05)	0.512E 01 (0.145E 00)
3.459	0.090	0.267E-03 (0.487E-05)	0.287E 01 (0.556E-01)	8.033	0.107	0.159E-03 (0.407E-05)	0.503E 01 (0.141E 00)
3.551	0.094	0.261E-03 (0.478E-05)	0.293E 01 (0.568E-01)	8.141	0.109	0.149E-03 (0.381E-05)	0.464E 01 (0.132E 00)
3.647	0.098	0.255E-03 (0.456E-05)	0.296E 01 (0.561E-01)	8.251	0.111	0.120E-03 (0.350E-05)	0.511E 01 (0.164E 00)
3.746	0.102	0.241E-03 (0.432E-05)	0.303E 01 (0.576E-01)	8.364	0.113	0.106E-03 (0.344E-05)	0.516E 01 (0.183E 00)
3.850	0.106	0.221E-03 (0.408E-05)	0.315E 01 (0.615E-01)	8.449	0.058	0.105E-03 (0.472E-05)	0.523E 01 (0.258E 00)
3.958	0.110	0.206E-03 (0.386E-05)	0.315E 01 (0.625E-01)	8.537	0.117	0.982E-04 (0.320E-05)	0.520E 01 (0.187E 00)
4.052	0.076	0.199E-03 (0.450E-05)	0.320E 01 (0.769E-01)	8.655	0.119	0.940E-04 (0.310E-05)	0.547E 01 (0.200E 00)
4.149	0.119	0.237E-03 (0.392E-05)	0.337E 01 (0.589E-01)	8.745	0.061	0.928E-04 (0.449E-05)	0.539E 01 (0.288E 00)
4.249	0.082	0.166E-03 (0.543E-05)	0.351E 01 (0.555E-01)	8.837	0.123	0.942E-04 (0.319E-05)	0.550E 01 (0.205E 00)
4.343	0.106	0.363E-03 (0.470E-05)	0.370E 01 (0.509E-01)	8.962	0.126	0.890E-04 (0.315E-05)	0.559E 01 (0.217E 00)
4.451	0.110	0.322E-03 (0.434E-05)	0.376E 01 (0.540E-01)	9.155	0.260	0.858E-04 (0.220E-05)	0.545E 01 (0.154E 00)
4.551	0.091	0.270E-03 (0.440E-05)	0.385E 01 (0.670E-01)	9.455	0.341	0.962E-04 (0.190E-05)	0.547E 01 (0.121E 00)
4.644	0.094	0.245E-03 (0.408E-05)	0.386E 01 (0.689E-01)	9.770	0.287	0.108E-03 (0.220E-05)	0.572E 01 (0.131E 00)
4.751	0.121	0.224E-03 (0.342E-05)	0.400E 01 (0.652E-01)	10.063	0.300	0.106E-03 (0.213E-05)	0.602E 01 (0.137E 00)
4.862	0.100	0.205E-03 (0.357E-05)	0.414E 01 (0.773E-01)	10.370	0.314	0.106E-03 (0.229E-05)	0.639E 01 (0.155E 00)
4.951	0.077	0.200E-03 (0.390E-05)	0.417E 01 (0.874E-01)	10.649	0.245	0.113E-03 (0.283E-05)	0.631E 01 (0.178E 00)
5.042	0.106	0.187E-03 (0.333E-05)	0.432E 01 (0.826E-01)	10.942	0.340	0.121E-03 (0.272E-05)	0.637E 01 (0.158E 00)
5.150	0.109	0.174E-03 (0.324E-05)	0.436E 01 (0.973E-01)	11.245	0.264	0.130E-03 (0.362E-05)	0.672E 01 (0.205E 00)
5.247	0.084	0.173E-03 (0.370E-05)	0.435E 01 (0.100E 00)	11.516	0.276	0.132E-03 (0.363E-05)	0.681E 01 (0.207E 00)
5.347	0.116	0.204E-03 (0.332E-05)	0.438E 01 (0.774E-01)	11.846	0.384	0.137E-03 (0.321E-05)	0.698E 01 (0.180E 00)
5.449	0.089	0.172E-03 (0.360E-05)	0.441E 01 (0.100E 00)	12.506	0.936	0.121E-03 (0.202E-05)	0.721E 01 (0.134E 00)
5.540	0.092	0.163E-03 (0.394E-05)	0.406E 01 (0.106E 00)	13.499	1.051	0.130E-03 (0.210E-05)	0.773E 01 (0.142E 00)
5.648	0.126	0.160E-03 (0.401E-05)	0.397E 01 (0.106E 00)	14.479	0.908	0.135E-03 (0.242E-05)	0.810E 01 (0.167E 00)
5.760	0.097	0.151E-03 (0.510E-05)	0.401E 01 (0.142E 00)	15.433	1.001	0.145E-03 (0.254E-05)	0.907E 01 (0.185E 00)
5.858	0.100	0.152E-03 (0.530E-05)	0.388E 01 (0.143E 00)	16.487	1.106	0.134E-03 (0.258E-05)	0.945E 01 (0.213E 00)
5.959	0.102	0.141E-03 (0.524E-05)	0.410E 01 (0.160E 00)	17.561	1.043	0.136E-03 (0.290E-05)	0.100E 02 (0.255E 00)
6.063	0.105	0.145E-03 (0.517E-05)	0.394E 01 (0.148E 00)	18.555	0.945	0.138E-03 (0.328E-05)	0.109E 02 (0.316E 00)
6.151	0.072	0.147E-03 (0.624E-05)	0.386E 01 (0.173E 00)	19.539	1.022	0.151E-03 (0.349E-05)	0.114E 02 (0.325E 00)

TABLE 9.

TOTAL SECONDARY NEUTRON YIELD, NORMALIZED TO UNIT INCIDENT NEUTRON, AND AVERAGE SECONDARY ENERGY AS A FUNCTION OF INCIDENT NEUTRON ENERGY. THE VALUES GIVEN REPRESENT THE AVERAGE OVER THE PROJECTED AREA OF THE DETECTOR AS SEEN FROM THE SCATTERING RING. VALUES IN PARENTHESES ARE STANDARD DEVIATIONS. THE APPROXIMATE SCATTERING ANGLE IS 90 DEGREES. THE MINIMUM NEUTRON ENERGY SAMPLED IS 0.760 MEV.

INC. NT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY NEUTRON FLOY (N/CM**2)	AVERAGE ENERGY (MEV)	INC. NT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY NEUTRON FLOY (N/CM**2)	AVERAGE ENERGY (MEV)
1.657	0.099	0.388E-03 (0.471E-05)	0.134E 01 (0.187E-01)	6.151	0.143	0.196E-03 (0.462E-05)	0.369E 01 (0.919E-01)
1.750	0.086	0.385E-03 (0.505E-05)	0.141E 01 (0.210E-01)	6.259	0.073	0.251E-03 (0.713E-05)	0.389E 01 (0.117E 00)
1.852	0.118	0.363E-03 (0.407E-05)	0.150E 01 (0.189E-01)	6.333	0.075	0.272E-03 (0.699E-05)	0.410E 01 (0.111E 00)
1.909	0.076	0.357E-03 (0.492E-05)	0.157E 01 (0.241E-01)	6.448	0.153	0.193E-03 (0.416E-05)	0.411E 01 (0.940E-01)
2.041	0.109	0.339E-03 (0.402E-05)	0.162E 01 (0.213E-01)	6.564	0.079	0.145E-03 (0.513E-05)	0.408E 01 (0.154E 00)
2.155	0.118	0.317E-03 (0.378E-05)	0.172E 01 (0.225E-01)	6.643	0.080	0.166E-03 (0.487E-05)	0.422E 01 (0.133E 00)
2.262	0.095	0.308E-03 (0.425E-05)	0.180E 01 (0.272E-01)	6.724	0.082	0.176E-03 (0.475E-05)	0.452E 01 (0.131E 00)
2.360	0.102	0.289E-03 (0.407E-05)	0.187E 01 (0.286E-01)	6.849	0.168	0.170E-03 (0.327E-05)	0.468E 01 (0.967E-01)
2.456	0.090	0.275E-03 (0.431E-05)	0.192E 01 (0.326E-01)	6.976	0.086	0.162E-03 (0.437E-05)	0.472E 01 (0.137E 00)
2.548	0.095	0.263E-03 (0.433E-05)	0.200E 01 (0.355E-01)	7.063	0.088	0.154E-03 (0.405E-05)	0.473E 01 (0.147E 00)
2.646	0.100	0.247E-03 (0.430E-05)	0.204E 01 (0.383E-01)	7.152	0.090	0.158E-03 (0.446E-05)	0.456E 01 (0.139E 00)
2.750	0.106	0.239E-03 (0.478E-05)	0.206E 01 (0.444E-01)	7.243	0.091	0.168E-03 (0.474E-05)	0.433E 01 (0.132E 00)
2.848	0.090	0.207E-03 (0.506E-05)	0.215E 01 (0.565E-01)	7.335	0.093	0.177E-03 (0.494E-05)	0.410E 01 (0.124E 00)
2.952	0.118	0.222E-03 (0.432E-05)	0.221E 01 (0.461E-01)	7.429	0.095	0.161E-03 (0.481E-05)	0.383E 01 (0.124E 00)
3.061	0.100	0.234E-03 (0.448E-05)	0.232E 01 (0.478E-01)	7.525	0.097	0.159E-03 (0.476E-05)	0.369E 01 (0.120E 00)
3.150	0.078	0.193E-03 (0.471E-05)	0.229E 01 (0.601E-01)	7.623	0.099	0.167E-03 (0.469E-05)	0.407E 01 (0.124E 00)
3.244	0.109	0.163E-03 (0.377E-05)	0.236E 01 (0.584E-01)	7.722	0.101	0.181E-03 (0.469E-05)	0.432E 01 (0.121E 00)
3.356	0.115	0.162E-03 (0.369E-05)	0.238E 01 (0.578E-01)	7.824	0.103	0.165E-03 (0.432E-05)	0.403E 01 (0.115E 00)
3.459	0.090	0.173E-03 (0.413E-05)	0.240E 01 (0.612E-01)	7.928	0.105	0.154E-03 (0.427E-05)	0.389E 01 (0.118E 00)
3.551	0.094	0.178E-03 (0.415E-05)	0.252E 01 (0.626E-01)	8.033	0.107	0.166E-03 (0.433E-05)	0.413E 01 (0.117E 00)
3.647	0.098	0.186E-03 (0.410E-05)	0.267E 01 (0.623E-01)	8.197	0.220	0.152E-03 (0.267E-05)	0.476E 01 (0.914E-01)
3.746	0.102	0.204E-03 (0.398E-05)	0.268E 01 (0.557E-01)	8.478	0.116	0.132E-03 (0.331E-05)	0.511E 01 (0.142E 00)
3.850	0.106	0.217E-03 (0.390E-05)	0.276E 01 (0.527E-01)	8.596	0.118	0.129E-03 (0.326E-05)	0.511E 01 (0.143E 00)
3.958	0.110	0.215E-03 (0.374E-05)	0.289E 01 (0.536E-01)	8.685	0.060	0.133E-03 (0.465E-05)	0.493E 01 (0.191E 00)
4.052	0.076	0.223E-03 (0.456E-05)	0.301E 01 (0.459E-01)	8.745	0.061	0.122E-03 (0.446E-05)	0.520E 01 (0.212E 00)
4.144	0.119	0.269E-03 (0.396E-05)	0.313E 01 (0.488E-01)	8.837	0.123	0.124E-03 (0.319E-05)	0.510E 01 (0.145E 00)
4.249	0.082	0.345E-03 (0.519E-05)	0.328E 01 (0.524E-01)	8.962	0.126	0.119E-03 (0.311E-05)	0.516E 01 (0.148E 00)
4.350	0.127	0.319E-03 (0.394E-05)	0.341E 01 (0.447E-01)	9.155	0.260	0.116E-03 (0.221E-05)	0.3260 01 (0.111E 00)
4.462	0.088	0.291E-03 (0.455E-05)	0.347E 01 (0.577E-01)	9.455	0.341	0.110E-03 (0.184E-05)	0.545E 01 (0.102E 00)
4.551	0.091	0.279E-03 (0.425E-05)	0.351E 01 (0.569E-01)	9.770	0.287	0.117E-03 (0.209E-05)	0.558E 01 (0.111E 00)
4.644	0.094	0.262E-03 (0.409E-05)	0.360E 01 (0.597E-01)	10.063	0.300	0.113E-03 (0.211E-05)	0.557E 01 (0.117E 00)
4.739	0.096	0.250E-03 (0.389E-05)	0.370E 01 (0.613E-01)	10.370	0.314	0.110E-03 (0.225E-05)	0.572E 01 (0.131E 00)
4.837	0.100	0.234E-03 (0.367E-05)	0.384E 01 (0.641E-01)	10.649	0.245	0.108E-03 (0.272E-05)	0.552E 01 (0.156E 00)
4.938	0.103	0.228E-03 (0.353E-05)	0.390E 01 (0.643E-01)	10.942	0.340	0.115E-03 (0.248E-05)	0.542E 01 (0.131E 00)
5.042	0.106	0.212E-03 (0.338E-05)	0.396E 01 (0.675E-01)	11.245	0.266	0.122E-03 (0.322E-05)	0.561E 01 (0.163E 00)
5.150	0.109	0.204E-03 (0.339E-05)	0.404E 01 (0.725E-01)	11.516	0.276	0.124E-03 (0.344E-05)	0.591E 01 (0.181E 00)
5.261	0.113	0.202E-03 (0.321E-05)	0.422E 01 (0.719E-01)	11.846	0.384	0.133E-03 (0.325E-05)	0.625E 01 (0.166E 00)
5.346	0.058	0.212E-03 (0.460E-05)	0.430E 01 (0.100E 00)	12.506	0.936	0.136E-03 (0.200E-05)	0.670E 01 (0.110E 00)
5.435	0.119	0.187E-03 (0.309E-05)	0.433E 01 (0.770E-01)	13.499	1.051	0.138E-03 (0.207E-05)	0.699E 01 (0.119E 00)
5.555	0.123	0.179E-03 (0.324E-05)	0.417E 01 (0.809E-01)	14.479	0.908	0.131E-03 (0.238E-05)	0.739E 01 (0.153E 00)
5.648	0.063	0.191E-03 (0.520E-05)	0.386E 01 (0.112E 00)	15.433	1.001	0.141E-03 (0.247E-05)	0.808E 01 (0.164E 00)
5.744	0.129	0.191E-03 (0.400E-05)	0.380E 01 (0.848E-01)	16.487	1.106	0.133E-03 (0.251E-05)	0.796E 01 (0.177E 00)
5.841	0.066	0.185E-03 (0.612E-05)	0.382E 01 (0.134E 00)	17.561	1.043	0.129E-03 (0.285E-05)	0.861E 01 (0.229E 00)
5.942	0.136	0.182E-03 (0.460E-05)	0.377E 01 (0.100E 00)	18.555	0.945	0.134E-03 (0.322E-05)	0.943E 01 (0.279E 00)
6.045	0.070	0.179E-03 (0.655E-05)	0.380E 01 (0.146E 00)	19.539	1.022	0.138E-03 (0.343E-05)	0.101E 02 (0.312E 00)

TABLE 10.

TOTAL SECONDARY NEUTRON YIELD, NORMALIZED TO UNIT INCIDENT NEUTRON, AND AVERAGE SECONDARY ENERGY AS A FUNCTION OF INCIDENT NEUTRON ENERGY. THE VALUES GIVEN REPRESENT THE AVERAGE OVER THE PROJECTED AREA OF THE DETECTOR AS SEEN FROM THE SCATTERING RING. VALUES IN PARENTHESES ARE STANDARD DEVIATIONS. THE APPROXIMATE SCATTERING ANGLE IS 135 DEGREES. THE MINIMUM NEUTRON ENERGY SAMPLED IS 0.760 MEV.

INC. NT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY NEUTRON FLUX (N/CM**2)	AVERAGE ENERGY (MEV)	INC. NT. ENERGY (MEV)	ENERGY SPREAD (MEV)	SECONDARY NEUTRON FLUX (N/CM**2)	AVERAGE ENERGY (MEV)
1.646	0.108	0.173E-03 (0.327E-05)	0.121E 01 (0.270E-01)	6.262	0.109	0.129E-03 (0.416E-05)	0.388E 01 (0.132E 00)
1.748	0.096	0.176E-03 (0.345E-05)	0.126E 01 (0.287E-01)	6.354	0.075	0.110E-03 (0.476E-05)	0.391E 01 (0.188E 00)
1.848	0.105	0.175E-03 (0.312E-05)	0.132E 01 (0.271E-01)	6.448	0.114	0.834E-04 (0.357E-05)	0.336E 01 (0.155E 00)
1.951	0.101	0.168E-03 (0.309E-05)	0.141E 01 (0.298E-01)	6.544	0.078	0.567E-04 (0.401E-05)	0.347E 01 (0.268E 00)
2.049	0.095	0.207E-03 (0.340E-05)	0.149E 01 (0.275E-01)	6.643	0.120	0.549E-04 (0.312E-05)	0.352E 01 (0.216E 00)
2.148	0.102	0.170E-03 (0.308E-05)	0.159E 01 (0.319E-01)	6.744	0.081	0.641E-04 (0.361E-05)	0.345E 01 (0.211E 00)
2.246	0.094	0.171E-03 (0.323E-05)	0.163E 01 (0.341E-01)	6.847	0.125	0.600E-04 (0.266E-05)	0.353E 01 (0.173E 00)
2.343	0.100	0.175E-03 (0.315E-05)	0.169E 01 (0.336E-01)	6.952	0.085	0.611E-04 (0.324E-05)	0.360E 01 (0.210E 00)
2.446	0.106	0.185E-03 (0.315E-05)	0.173E 01 (0.325E-01)	7.039	0.087	0.610E-04 (0.316E-05)	0.341E 01 (0.197E 00)
2.546	0.094	0.189E-03 (0.362E-05)	0.183E 01 (0.383E-01)	7.149	0.134	0.543E-04 (0.261E-05)	0.346E 01 (0.188E 00)
2.643	0.100	0.196E-03 (0.379E-05)	0.189E 01 (0.398E-01)	7.261	0.091	0.589E-04 (0.354E-05)	0.328E 01 (0.218E 00)
2.746	0.106	0.219E-03 (0.441E-05)	0.196E 01 (0.428E-01)	7.353	0.093	0.885E-04 (0.358E-05)	0.384E 01 (0.169E 00)
2.855	0.112	0.289E-03 (0.468E-05)	0.206E 01 (0.359E-01)	7.447	0.095	0.139E-03 (0.393E-05)	0.428E 01 (0.132E 00)
2.958	0.094	0.247E-03 (0.457E-05)	0.214E 01 (0.425E-01)	7.543	0.086	0.156E-03 (0.395E-05)	0.455E 01 (0.125E 00)
3.055	0.099	0.182E-03 (0.395E-05)	0.217E 01 (0.506E-01)	7.640	0.098	0.160E-03 (0.380E-05)	0.463E 01 (0.119E 00)
3.157	0.104	0.243E-03 (0.407E-05)	0.226E 01 (0.406E-01)	7.739	0.100	0.178E-03 (0.388E-05)	0.485E 01 (0.116E 00)
3.249	0.082	0.273E-03 (0.476E-05)	0.236E 01 (0.440E-01)	7.841	0.102	0.165E-03 (0.369E-05)	0.475E 01 (0.116E 00)
3.347	0.114	0.302E-03 (0.409E-05)	0.240E 01 (0.347E-01)	7.944	0.104	0.132E-03 (0.355E-05)	0.415E 01 (0.123E 00)
3.449	0.089	0.315E-03 (0.465E-05)	0.247E 01 (0.390E-01)	8.050	0.106	0.117E-03 (0.360E-05)	0.380E 01 (0.130E 00)
3.540	0.093	0.312E-03 (0.448E-05)	0.252E 01 (0.387E-01)	8.157	0.109	0.938E-04 (0.317E-05)	0.366E 01 (0.138E 00)
3.651	0.130	0.294E-03 (0.372E-05)	0.260E 01 (0.350E-01)	8.267	0.111	0.714E-04 (0.292E-05)	0.394E 01 (0.181E 00)
3.749	0.067	0.255E-03 (0.485E-05)	0.272E 01 (0.550E-01)	8.350	0.056	0.622E-04 (0.404E-05)	0.441E 01 (0.321E 00)
3.835	0.105	0.238E-03 (0.370E-05)	0.273E 01 (0.451E-01)	8.436	0.114	0.655E-04 (0.289E-05)	0.431E 01 (0.212E 00)
3.942	0.109	0.221E-03 (0.349E-05)	0.282E 01 (0.474E-01)	8.551	0.117	0.696E-04 (0.268E-05)	0.460E 01 (0.198E 00)
4.054	0.110	0.216E-03 (0.340E-05)	0.290E 01 (0.485E-01)	8.669	0.119	0.697E-04 (0.280E-05)	0.472E 01 (0.211E 00)
4.150	0.079	0.181E-03 (0.377E-05)	0.299E 01 (0.665E-01)	8.759	0.060	0.799E-04 (0.394E-05)	0.435E 01 (0.243E 00)
4.240	0.101	0.119E-03 (0.296E-05)	0.292E 01 (0.774E-01)	8.850	0.123	0.703E-04 (0.272E-05)	0.499E 01 (0.214E 00)
4.343	0.105	0.810E-04 (0.272E-05)	0.297E 01 (0.106E 00)	8.943	0.062	0.714E-04 (0.353E-05)	0.478E 01 (0.272E 00)
4.450	0.109	0.863E-04 (0.269E-05)	0.307E 01 (0.102E 00)	9.135	0.322	0.698E-04 (0.157E-05)	0.478E 01 (0.121E 00)
4.550	0.090	0.836E-04 (0.286E-05)	0.322E 01 (0.117E 00)	9.431	0.270	0.620E-04 (0.167E-05)	0.477E 01 (0.147E 00)
4.653	0.117	0.877E-04 (0.239E-05)	0.316E 01 (0.926E-01)	9.744	0.355	0.492E-04 (0.153E-05)	0.512E 01 (0.180E 00)
4.760	0.097	0.816E-04 (0.261E-05)	0.333E 01 (0.114E 00)	10.071	0.299	0.464E-04 (0.170E-05)	0.493E 01 (0.211E 00)
4.858	0.100	0.803E-04 (0.239E-05)	0.338E 01 (0.108E 00)	10.376	0.312	0.508E-04 (0.179E-05)	0.529E 01 (0.215E 00)
4.959	0.103	0.747E-04 (0.262E-05)	0.366E 01 (0.136E 00)	10.653	0.243	0.577E-04 (0.199E-05)	0.499E 01 (0.206E 00)
5.050	0.079	0.764E-04 (0.282E-05)	0.354E 01 (0.140E 00)	10.944	0.329	0.508E-04 (0.181E-05)	0.520E 01 (0.218E 00)
5.144	0.109	0.726E-04 (0.284E-05)	0.370E 01 (0.132E 00)	11.246	0.264	0.506E-04 (0.202E-05)	0.556E 01 (0.257E 00)
5.254	0.112	0.682E-04 (0.237E-05)	0.374E 01 (0.139E 00)	11.561	0.367	0.588E-04 (0.193E-05)	0.546E 01 (0.210E 00)
5.354	0.086	0.714E-04 (0.273E-05)	0.371E 01 (0.152E 00)	11.889	0.288	0.573E-04 (0.233E-05)	0.571E 01 (0.271E 00)
5.441	0.089	0.769E-04 (0.268E-05)	0.386E 01 (0.144E 00)	12.497	0.929	0.566E-04 (0.138E-05)	0.642E 01 (0.181E 00)
5.546	0.121	0.719E-04 (0.234E-05)	0.396E 01 (0.139E 00)	13.483	1.042	0.514E-04 (0.141E-05)	0.677E 01 (0.216E 00)
5.654	0.094	0.767E-04 (0.267E-05)	0.371E 01 (0.141E 00)	14.522	1.037	0.463E-04 (0.155E-05)	0.656E 01 (0.267E 00)
5.749	0.096	0.723E-04 (0.270E-05)	0.390E 01 (0.157E 00)	15.543	1.005	0.385E-04 (0.181E-05)	0.588E 01 (0.359E 00)
5.846	0.099	0.754E-04 (0.283E-05)	0.382E 01 (0.155E 00)	16.518	0.944	0.428E-04 (0.202E-05)	0.588E 01 (0.372E 00)
5.946	0.101	0.912E-04 (0.309E-05)	0.343E 01 (0.126E 00)	17.506	1.032	0.420E-04 (0.210E-05)	0.666E 01 (0.446E 00)
6.049	0.104	0.940E-04 (0.336E-05)	0.347E 01 (0.134E 00)	18.489	0.934	0.424E-04 (0.231E-05)	0.733E 01 (0.546E 00)
6.154	0.107	0.103E-03 (0.368E-05)	0.358E 01 (0.137E 00)	19.460	1.009	0.358E-04 (0.230E-05)	0.700E 01 (0.678E 00)

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